

Program EVALPLOT  
(Version 2018-1)

by

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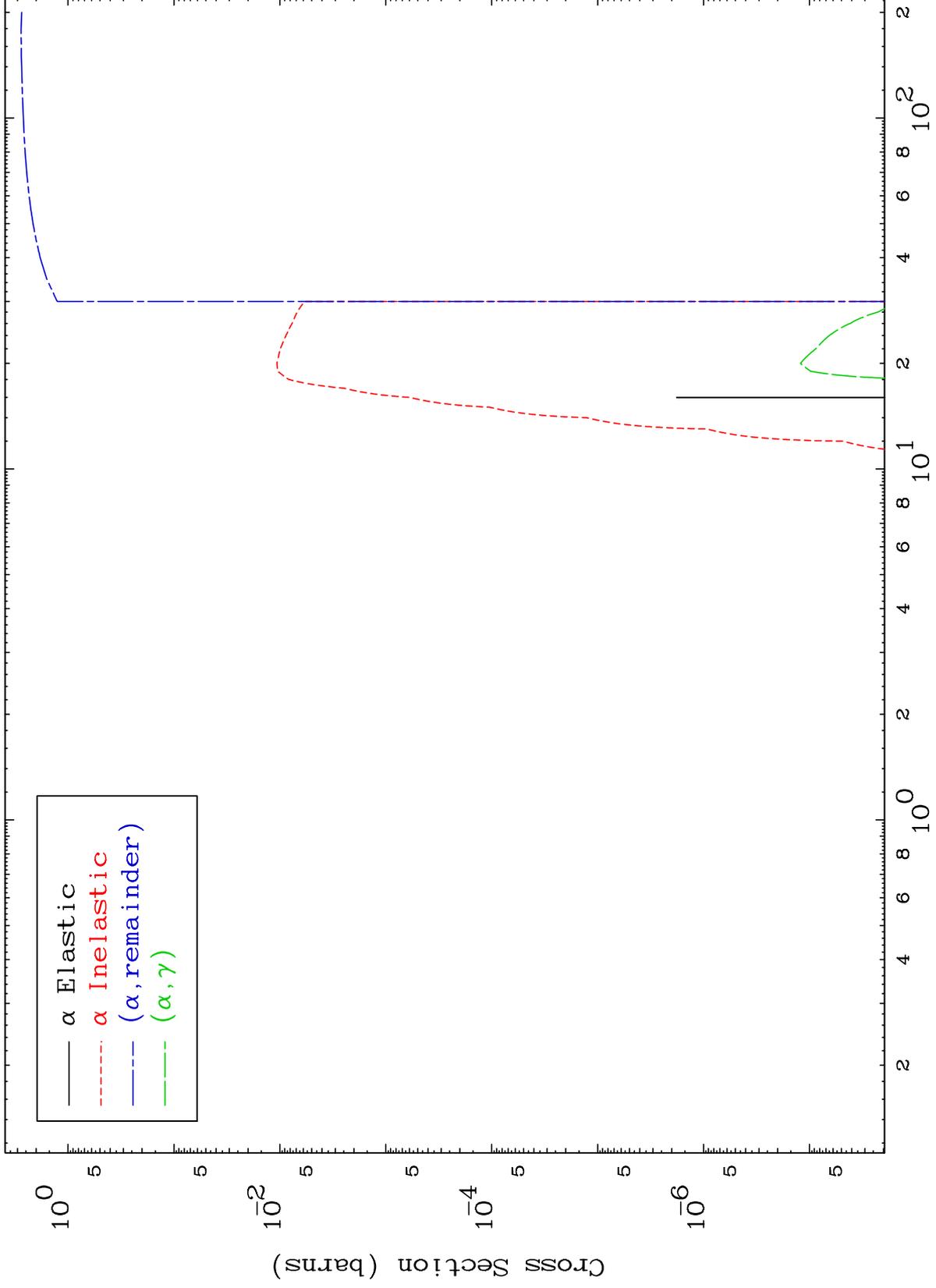
Press Mouse Button to Start

MAT 7667

$\alpha$  Major

76-Os-198

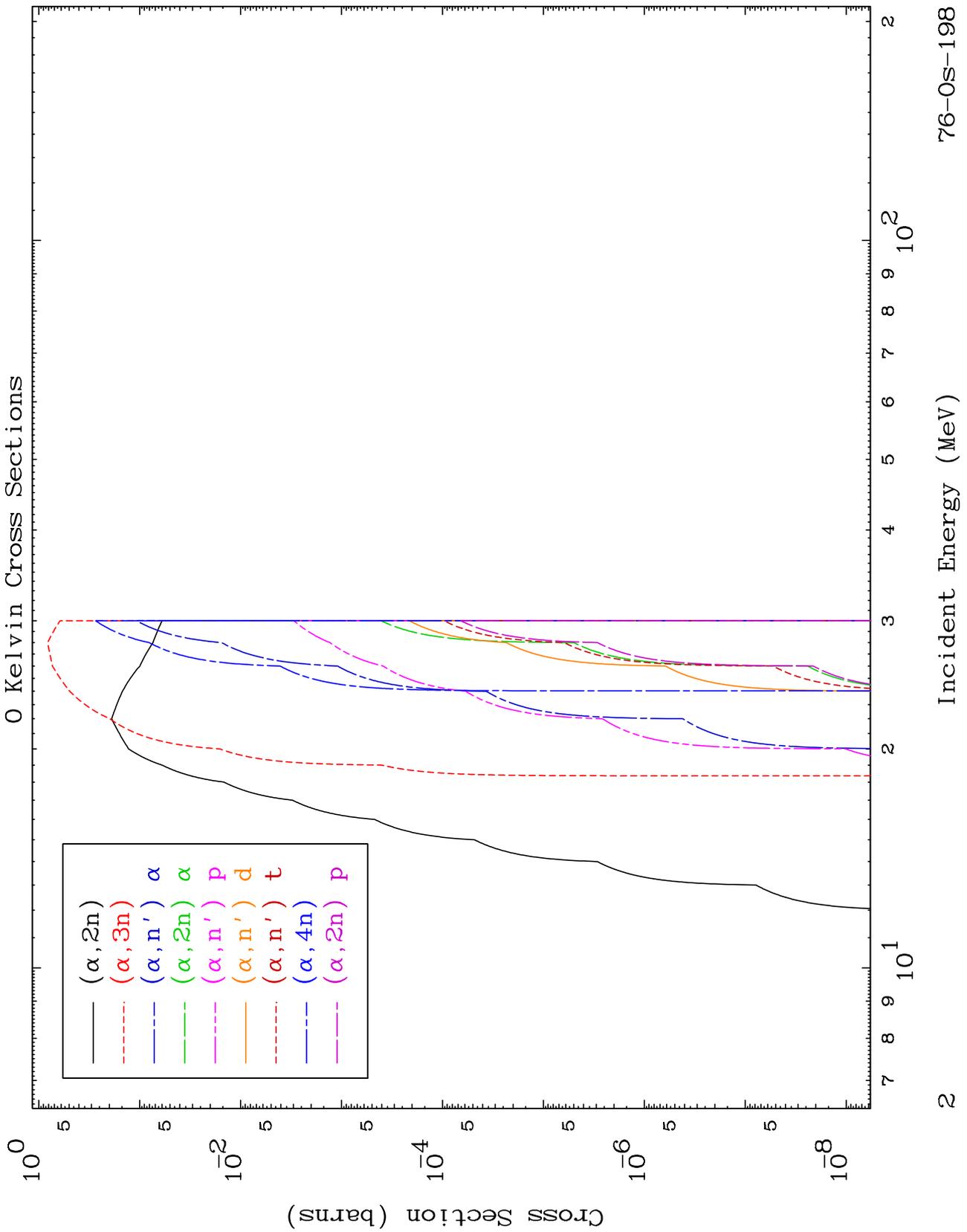
0 Kelvin Cross Sections

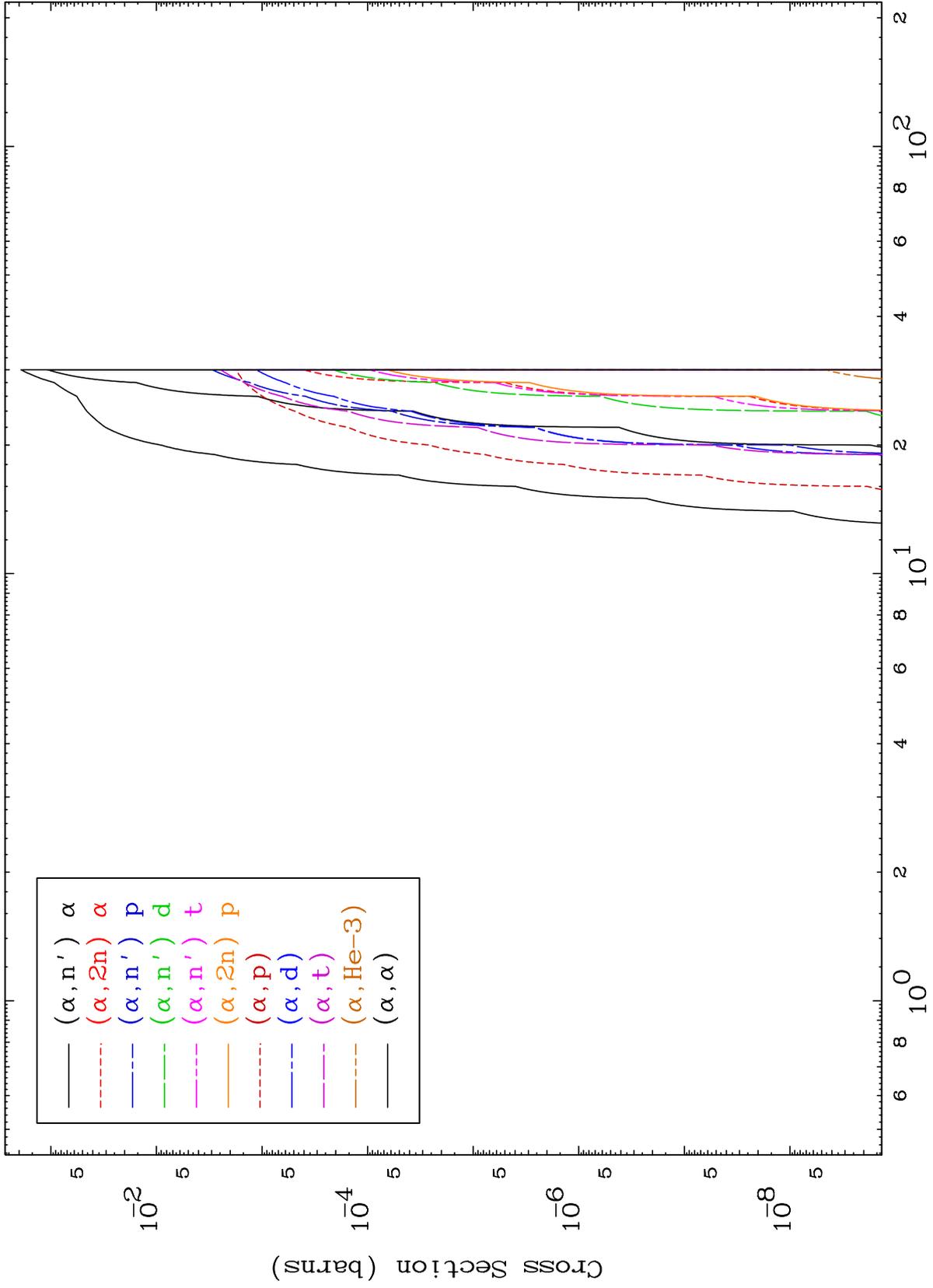


MAT 7667

$\alpha$  Neutron Production

76-Os-198



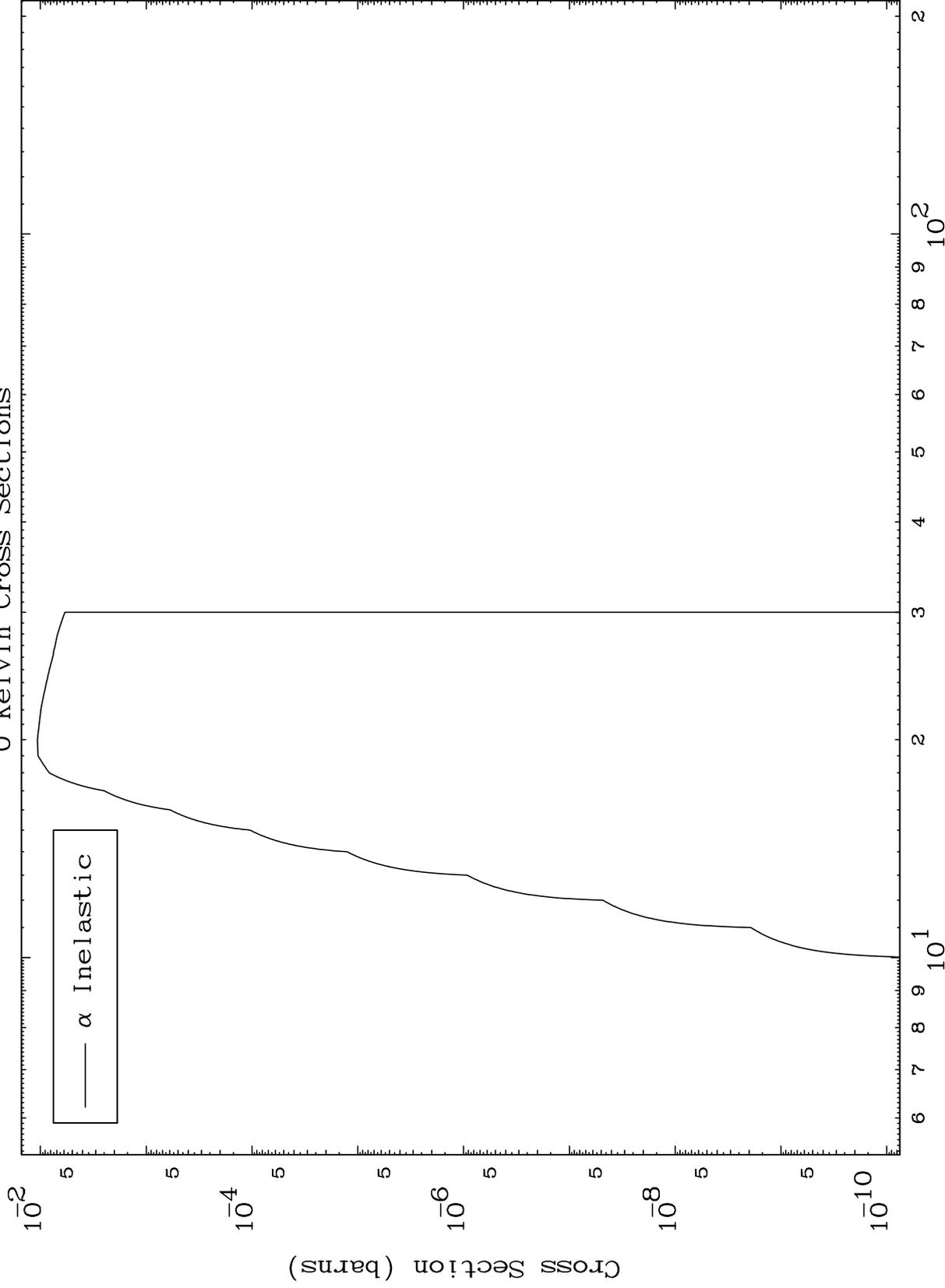


MAT 7667

( $\alpha, n'$ ) Level

76-Os-198

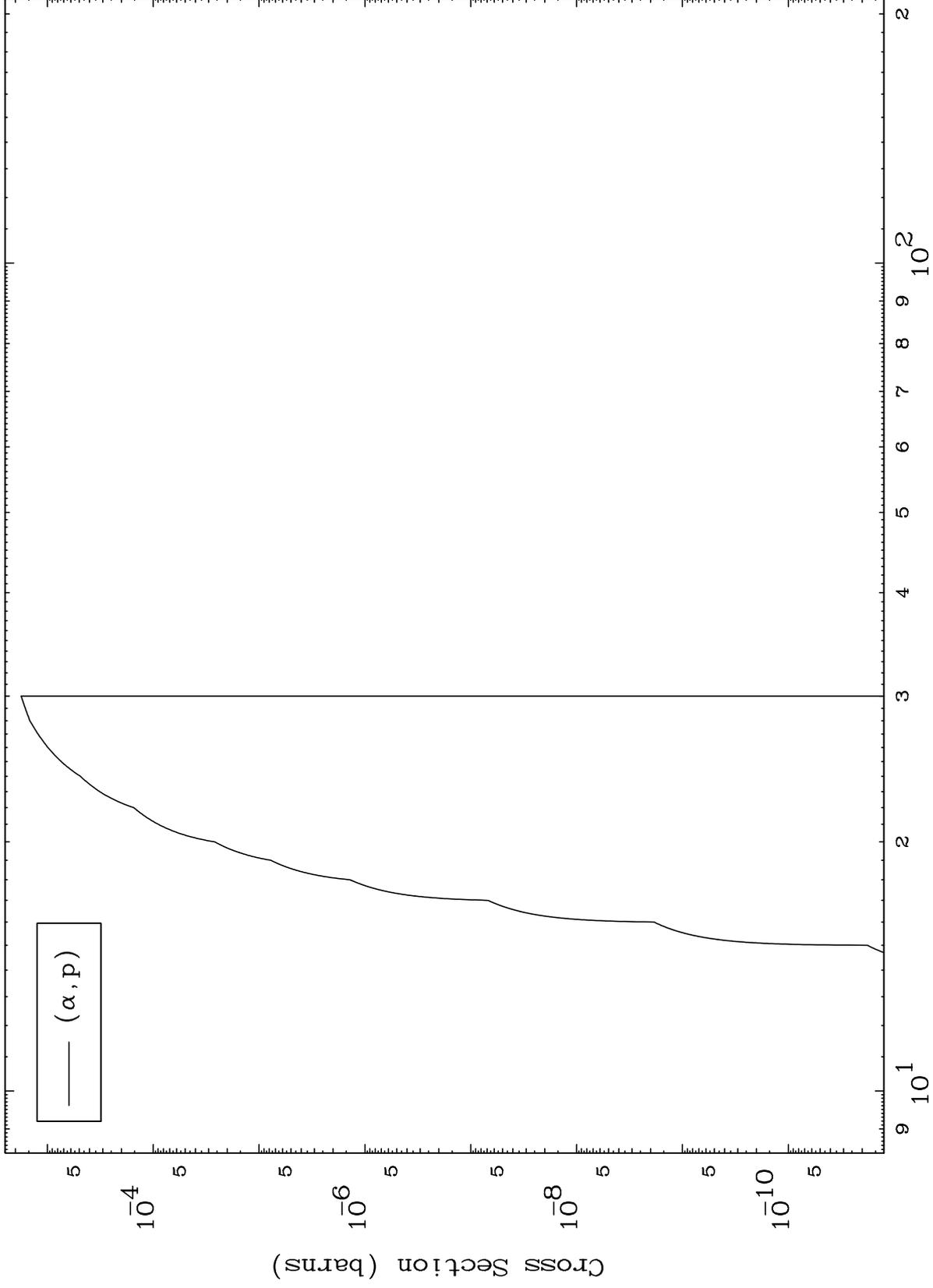
0 Kelvin Cross Sections



MAT 7667

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

76-Os-198



Incident Energy (MeV)

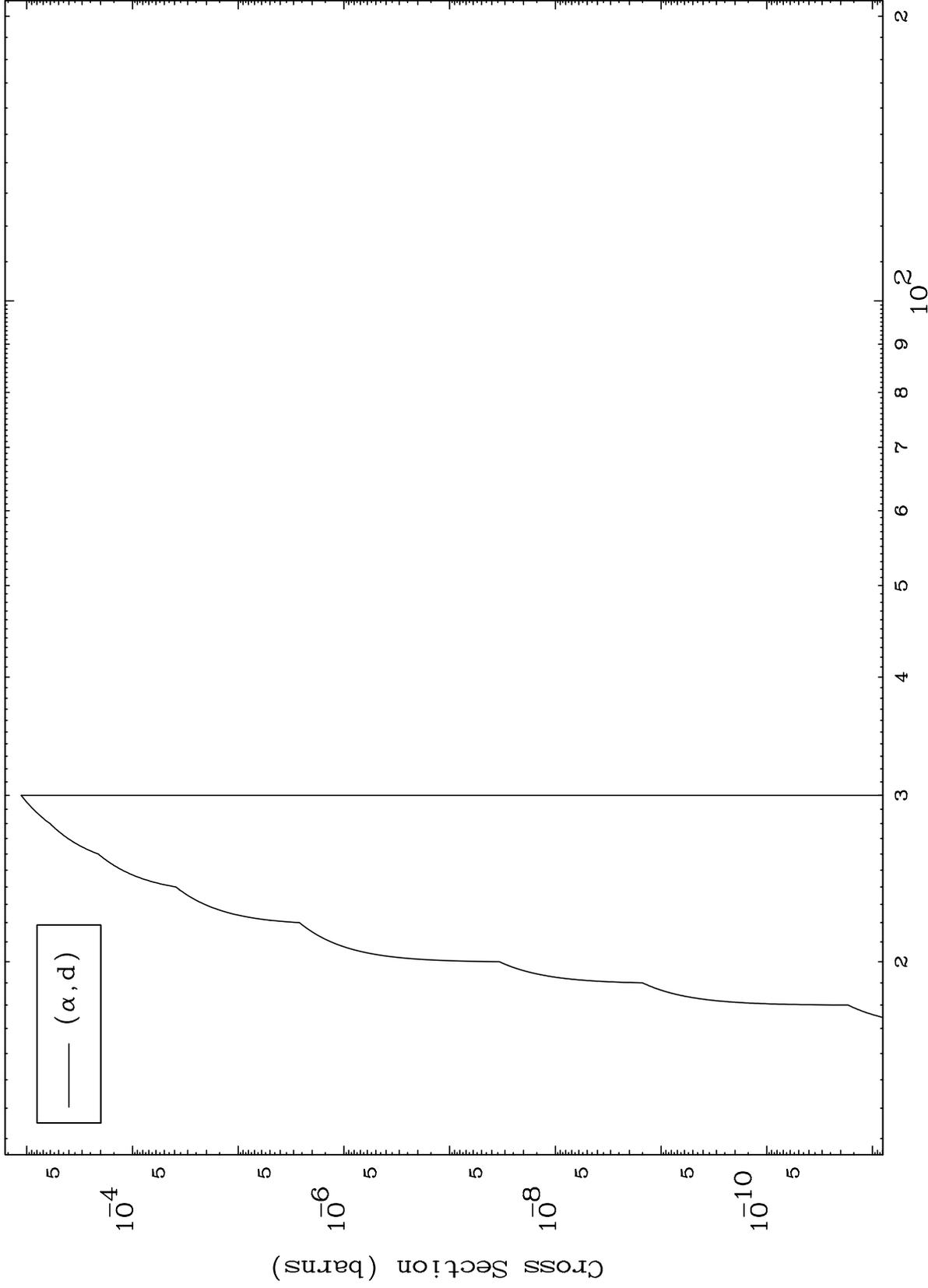
76-Os-198

5

MAT 7667

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

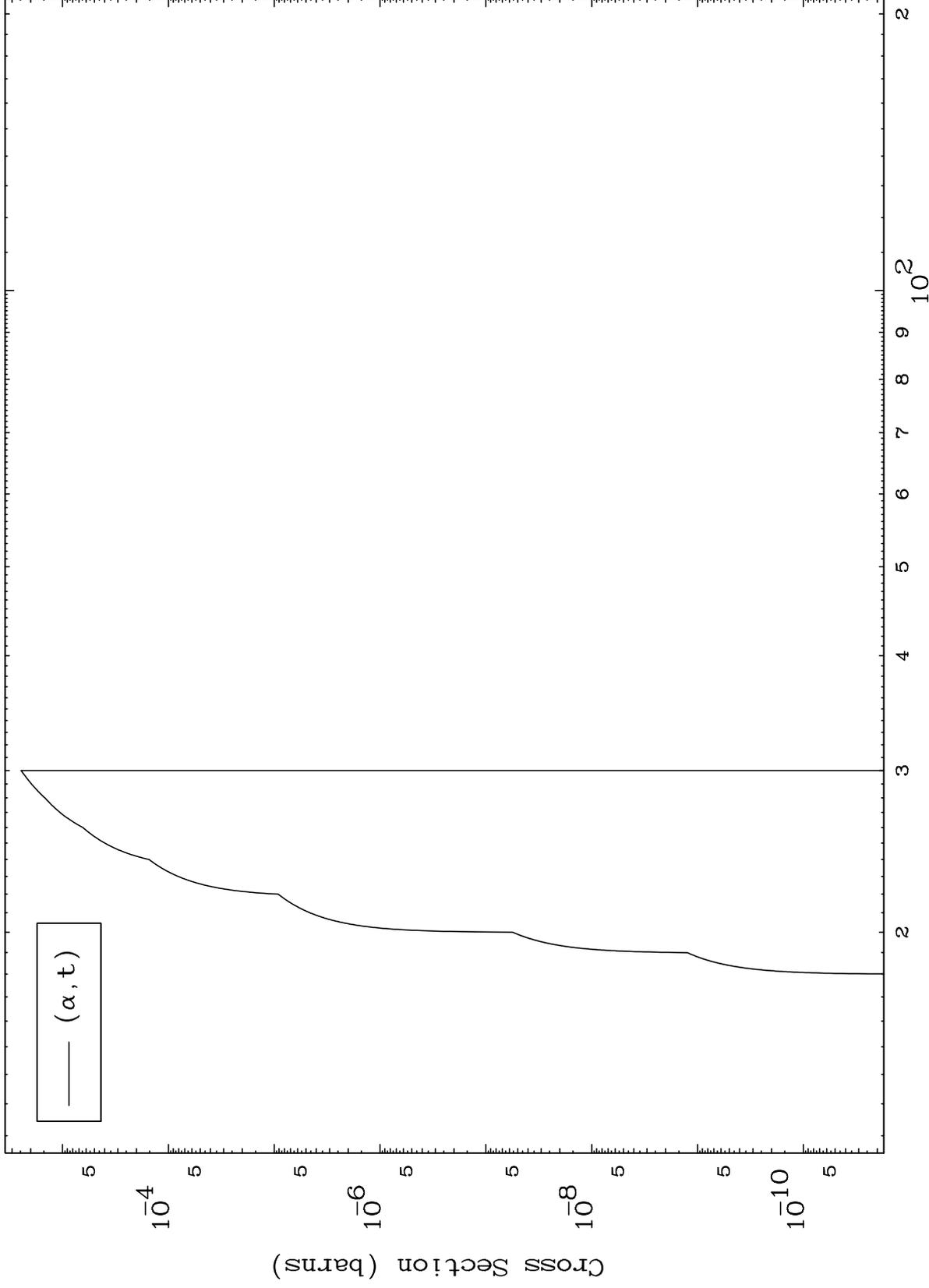
76-Os-198

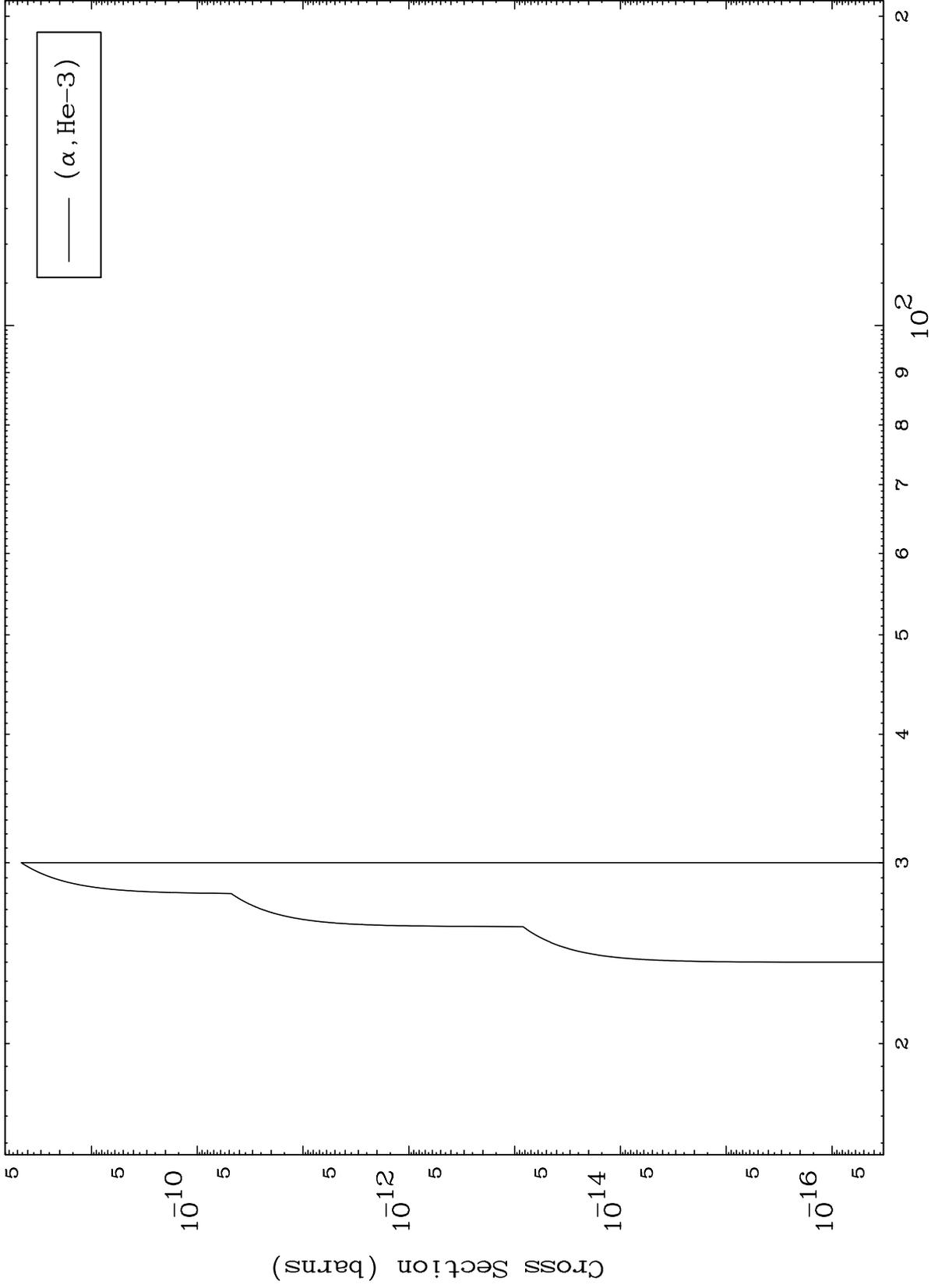


6

Incident Energy (MeV)

76-Os-198



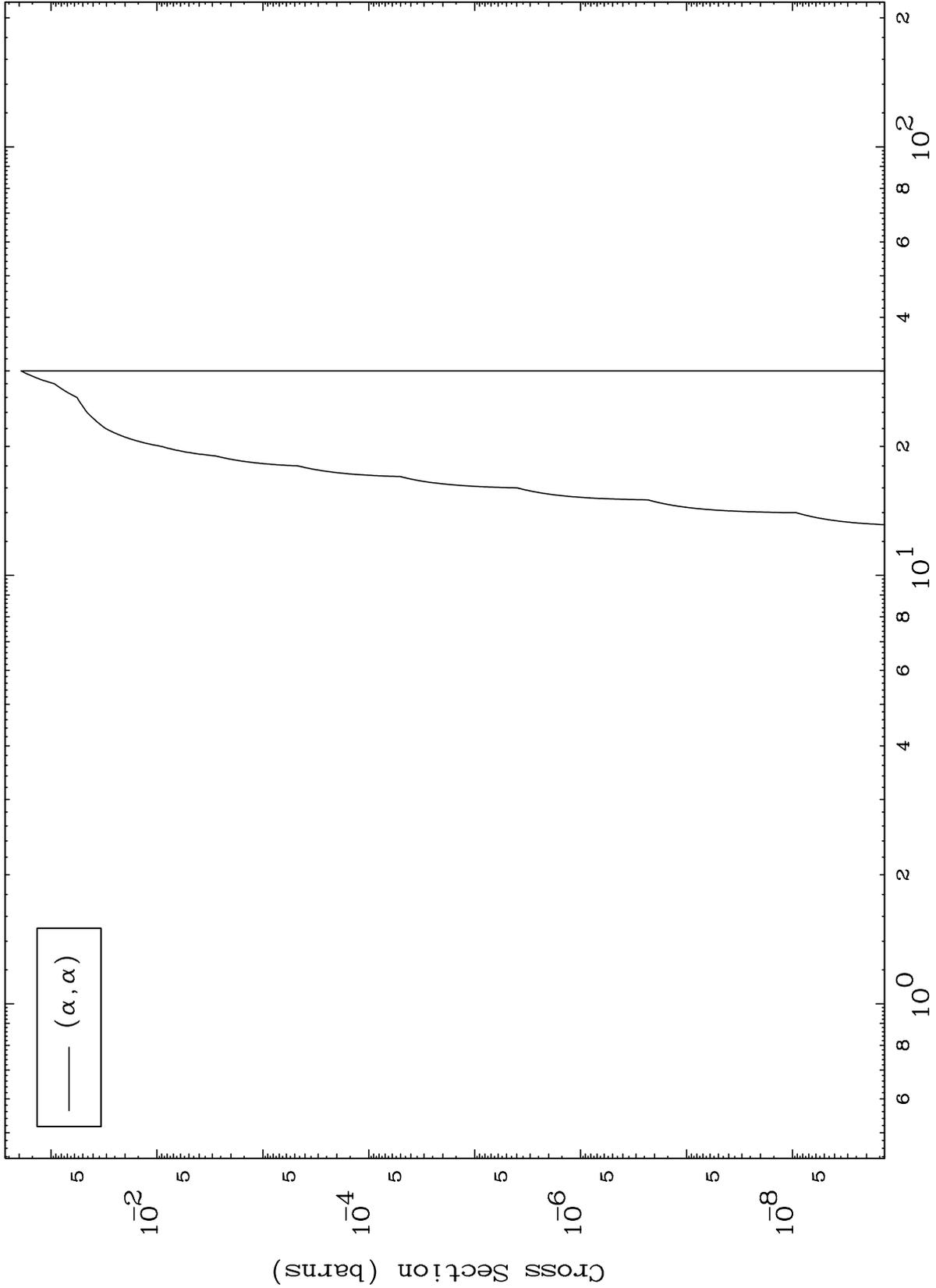


MAT 7667

( $\alpha, \alpha$ ) Levels

76-0s-198

0 Kelvin Cross Sections

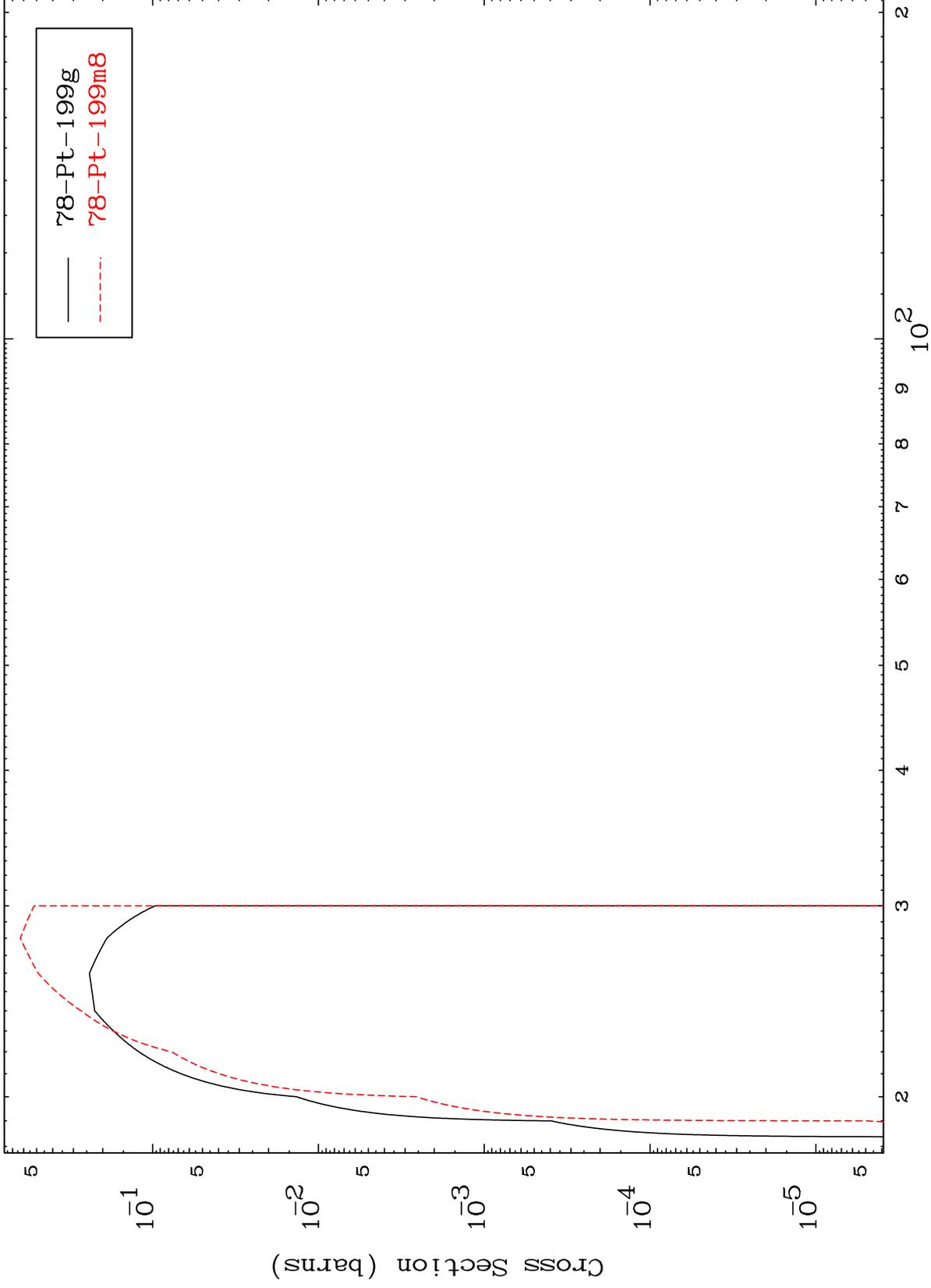


MAT 7667

( $\alpha, 3n$ )

76-Os-198

Radionuclide Production Cross Section



10

Incident Energy (MeV)

76-Os-198

MAT 7667

$\alpha$  Fission

76-Os-198

Radionuclide Production Cross Section

