

Program EVALPLOT  
(Version 2018-1)

by

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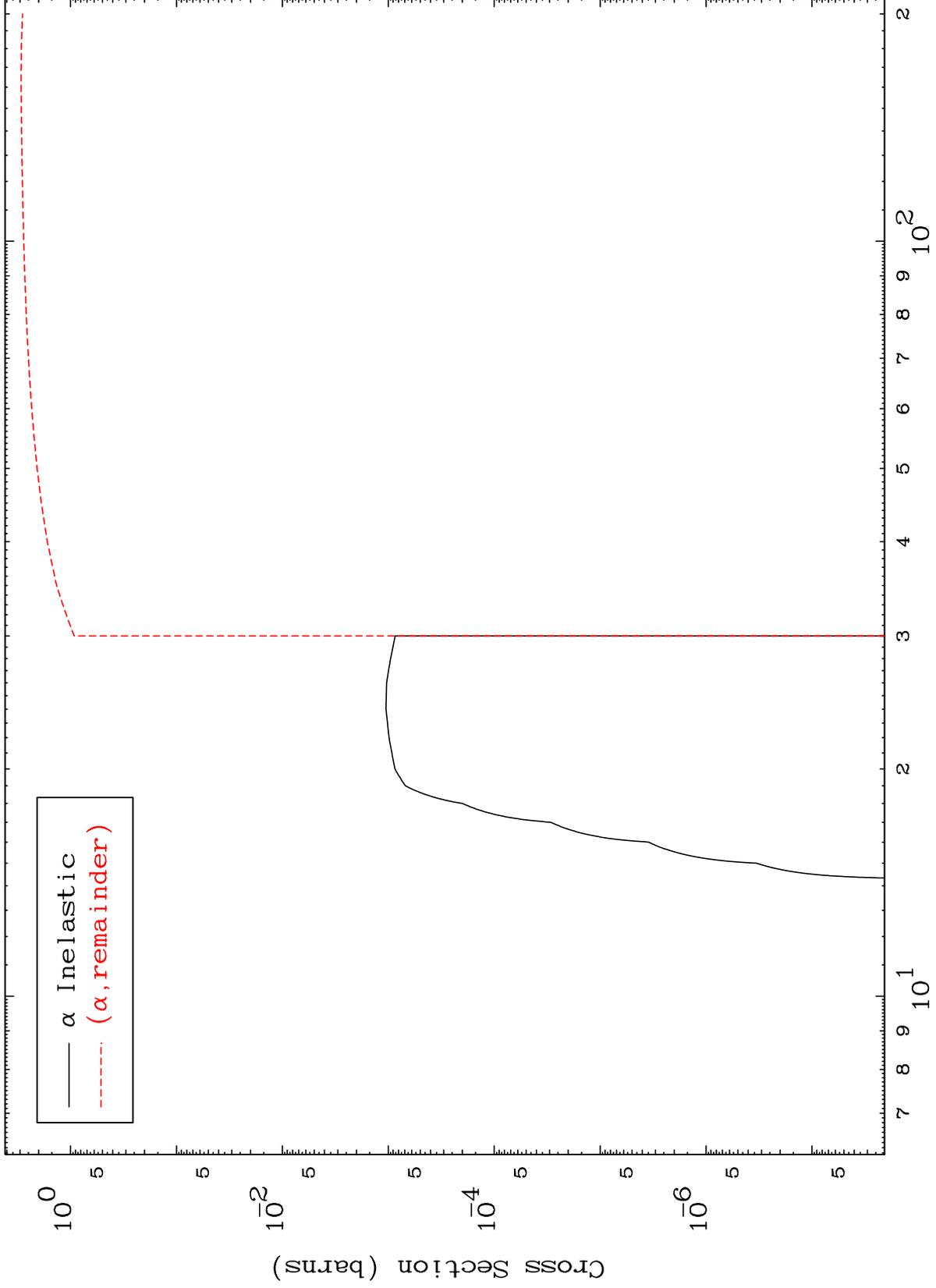
Press Mouse Button to Start

MAT 9119

$\alpha$  Major

91-Pa-227

0 Kelvin Cross Sections



1

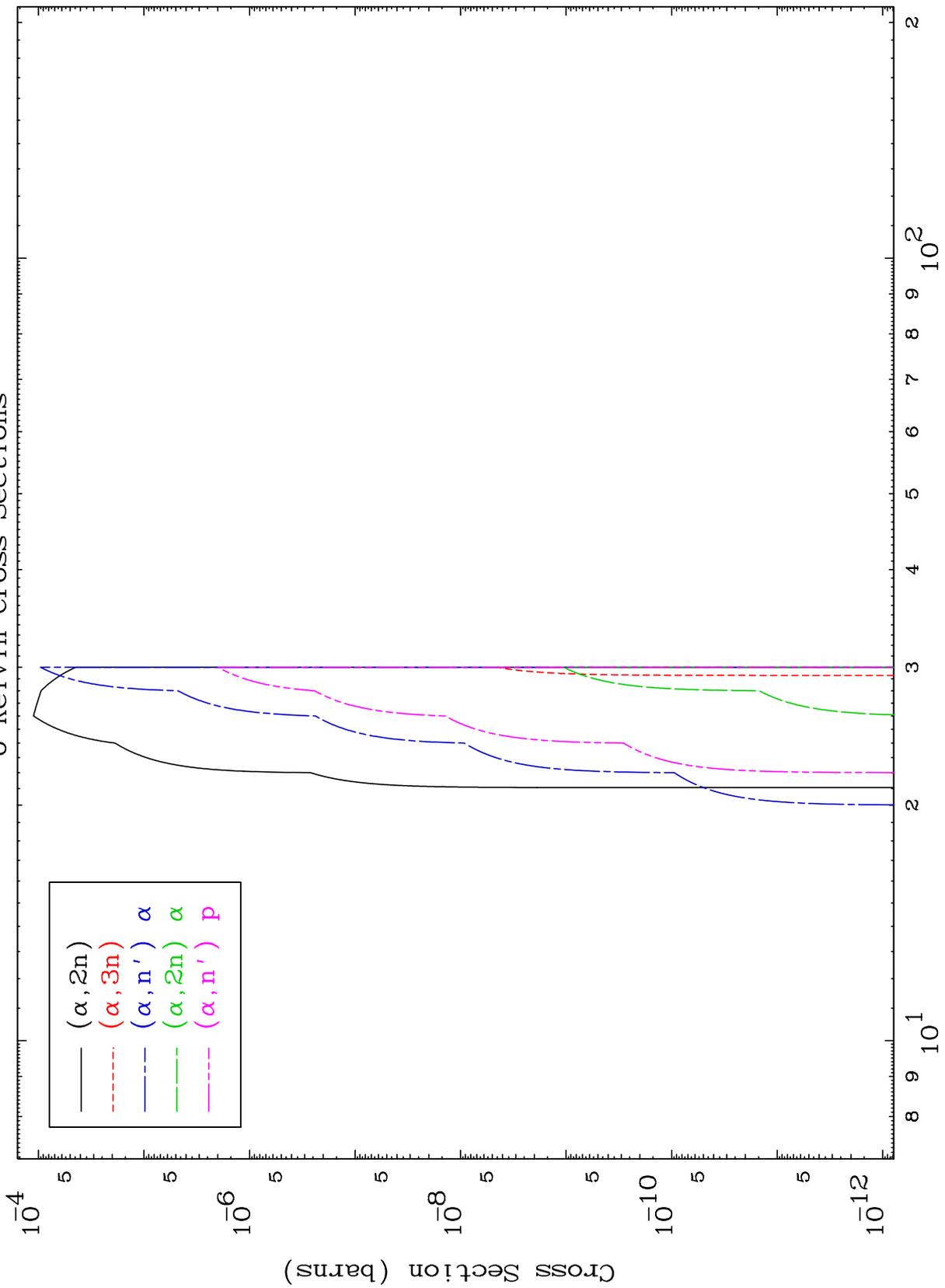
Incident Energy (MeV)

91-Pa-227

MAT 9119

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

91-Pa-227



2

Incident Energy (MeV)

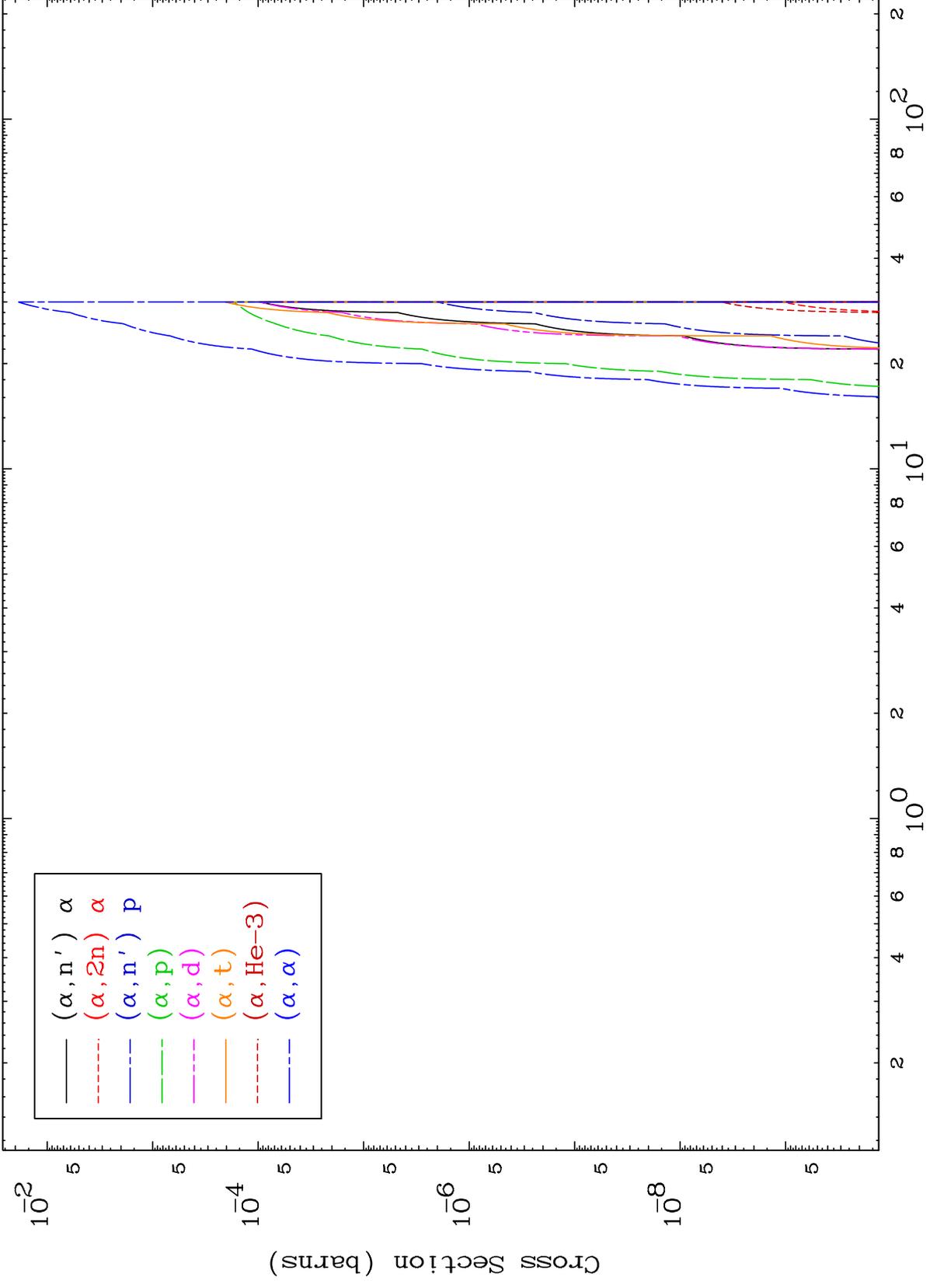
91-Pa-227

MAT 9119

$\alpha$  Charged Particle

91-Pa-227

0 Kelvin Cross Sections



91-Pa-227

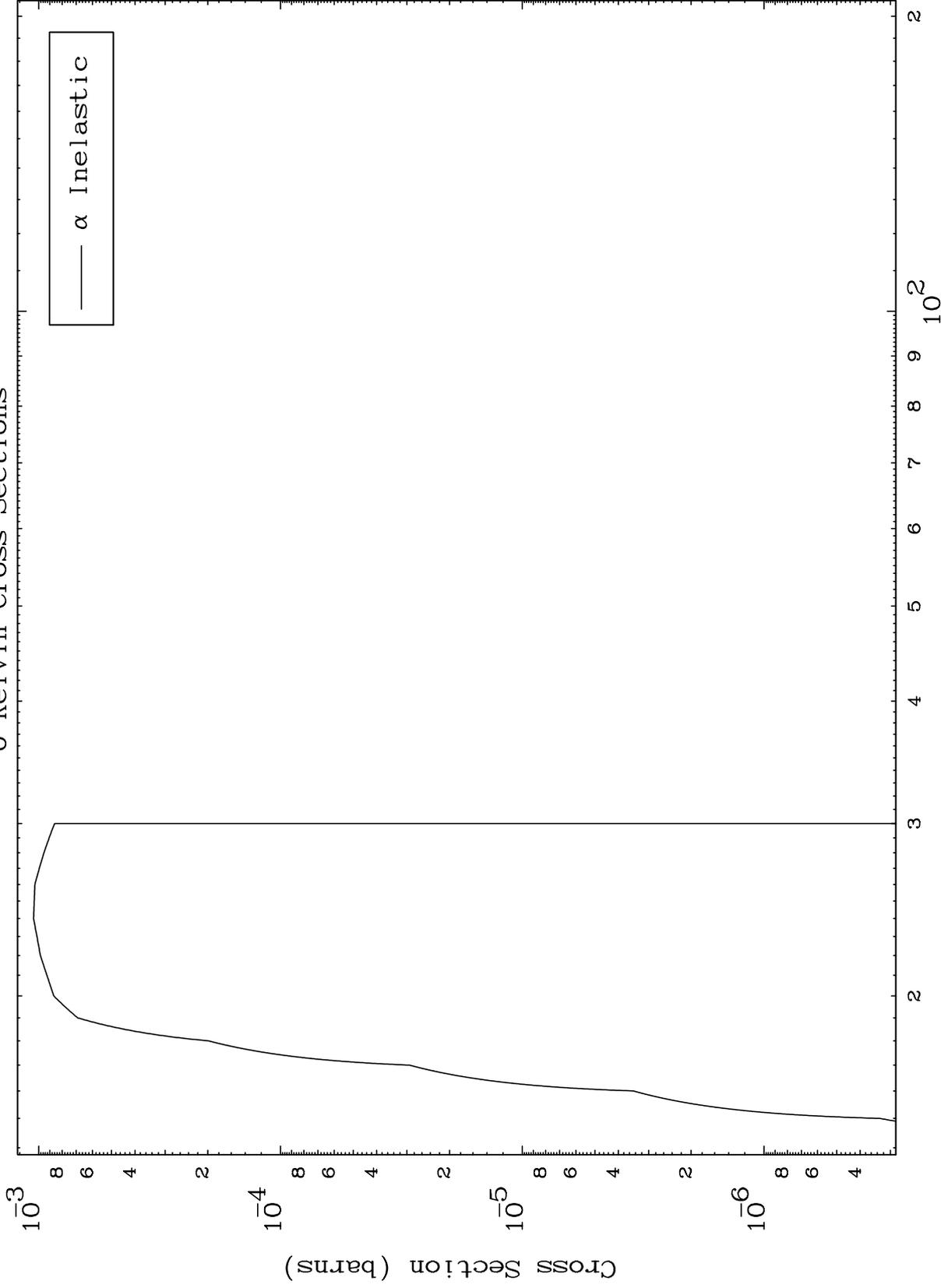
Incident Energy (MeV)

MAT 91119

( $\alpha, n'$ ) Level

91-Pa-227

0 Kelvin Cross Sections



4

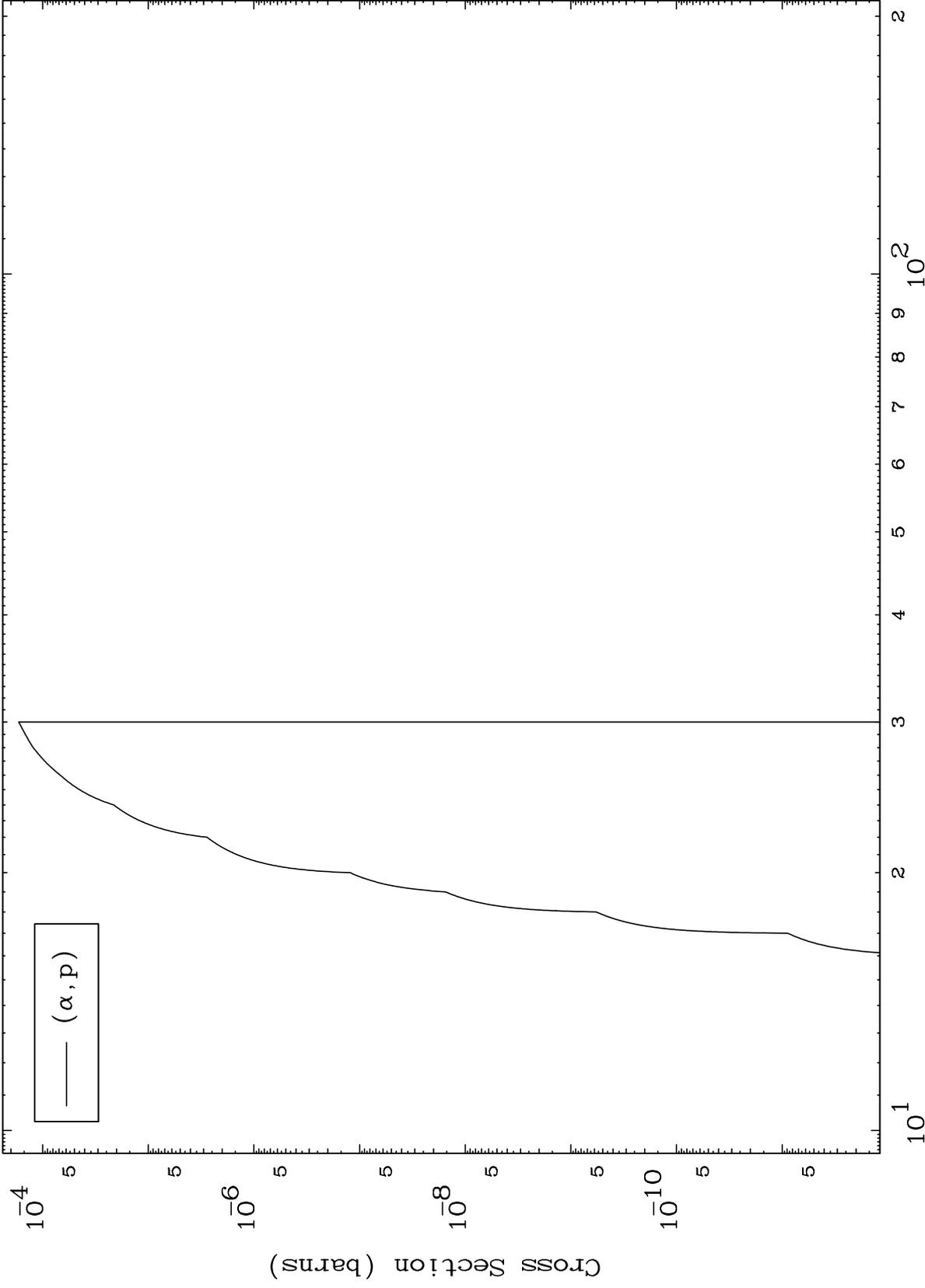
Incident Energy (MeV)

91-Pa-227

MAT 91119

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

91-Pa-227



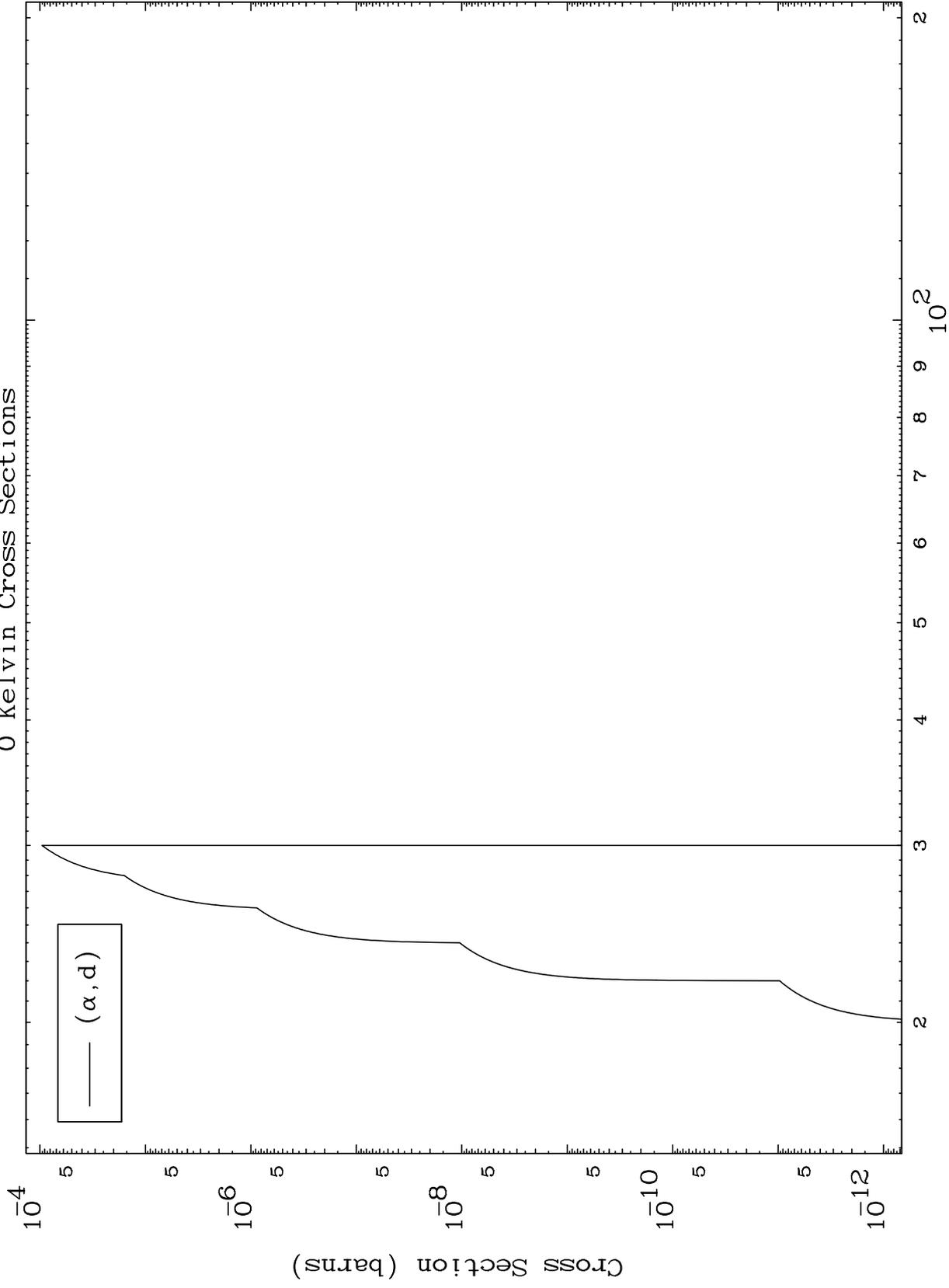
Incident Energy (MeV)

91-Pa-227

MAT 9119

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

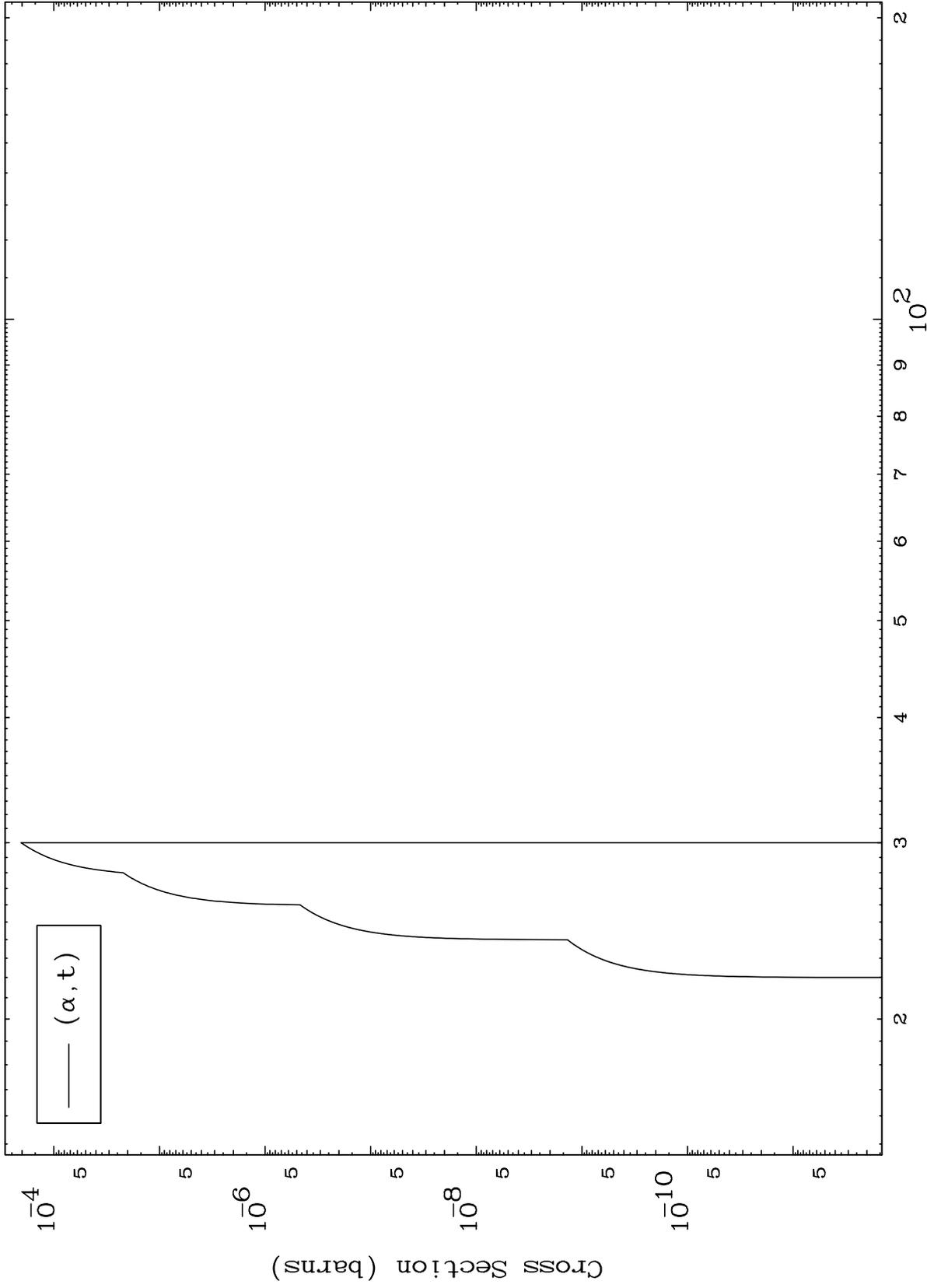
91-Pa-227

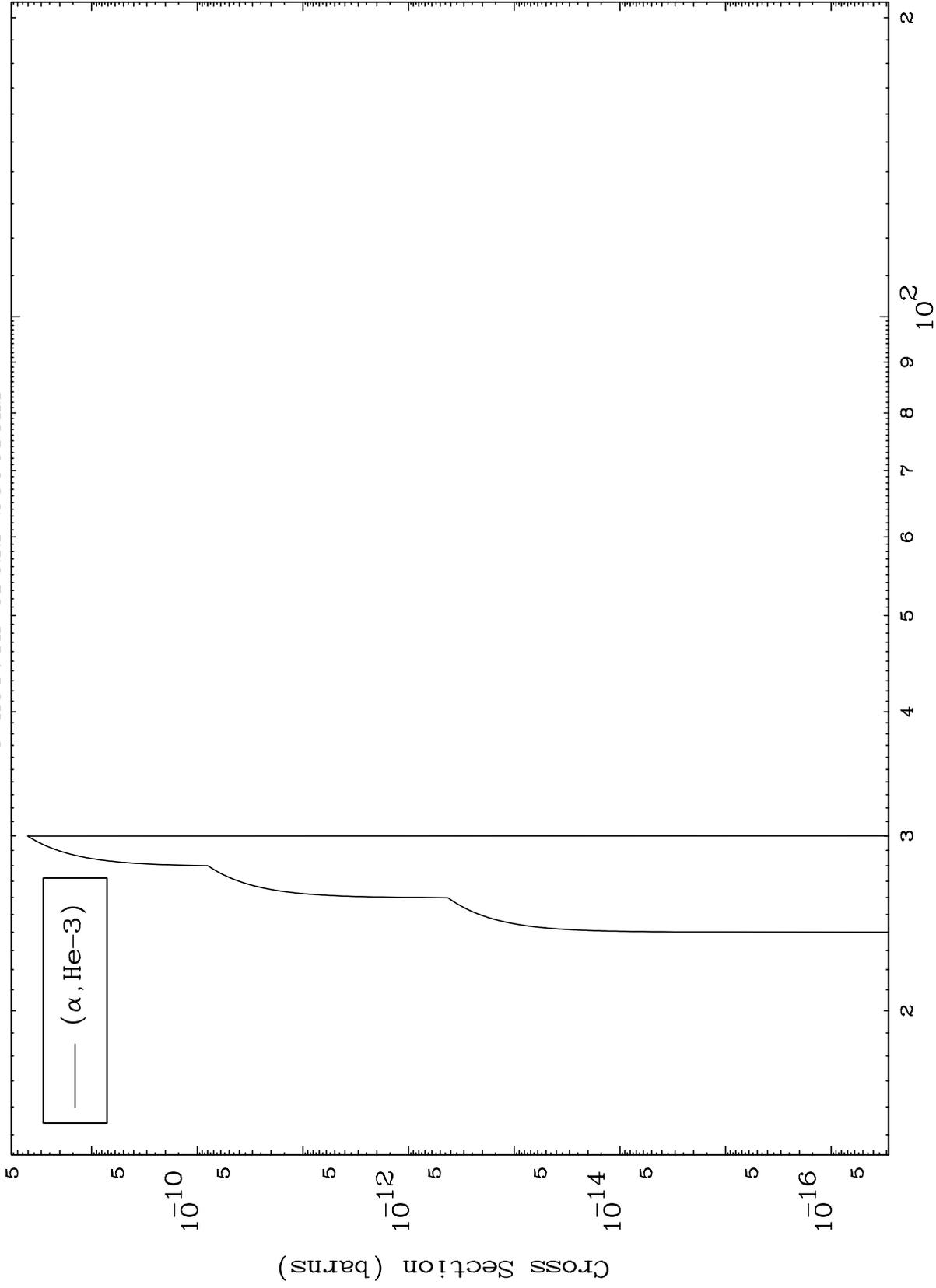


6

Incident Energy (MeV)

91-Pa-227



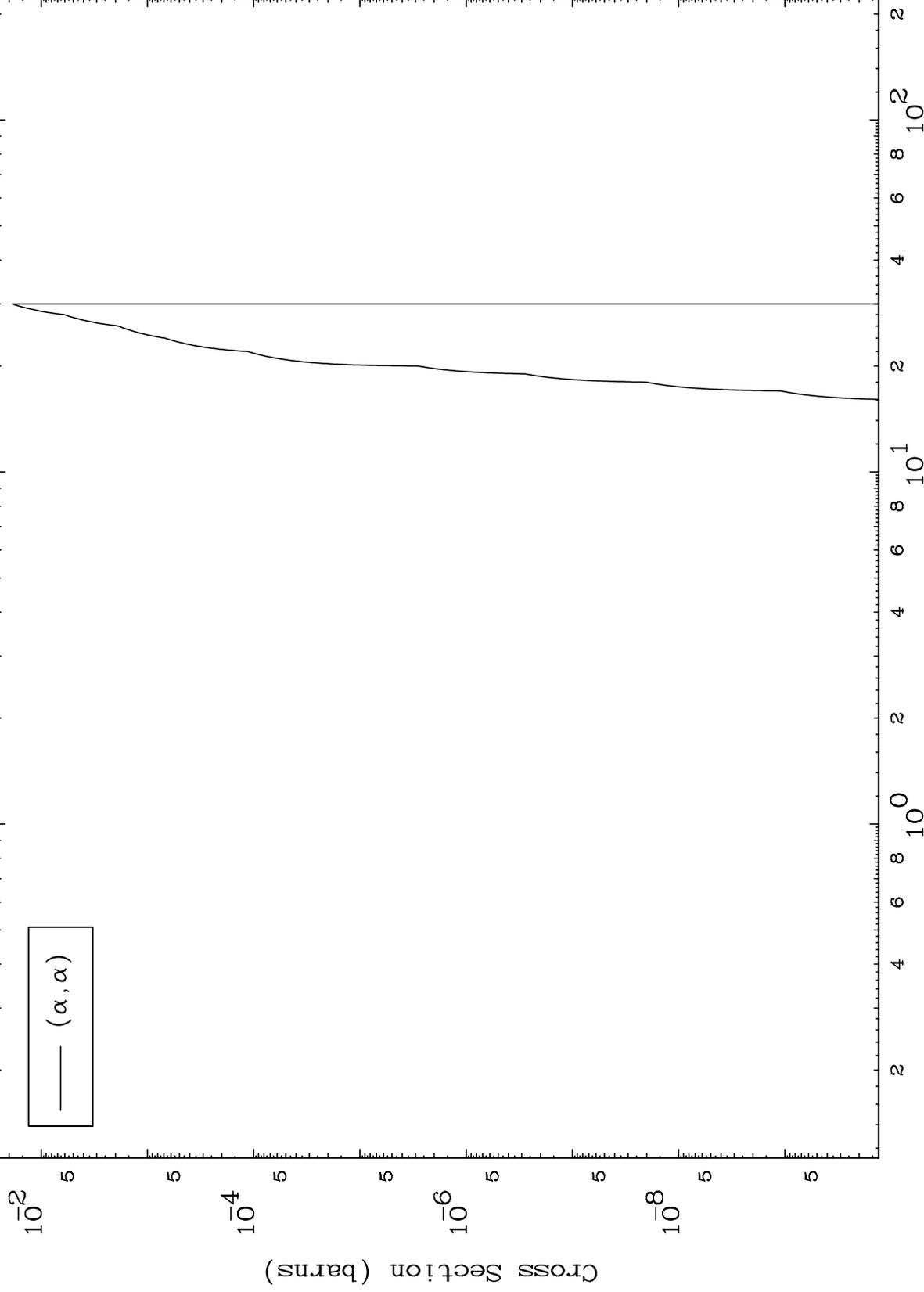


MAT 91119

( $\alpha, \alpha$ ) Levels

91-Pa-227

0 Kelvin Cross Sections

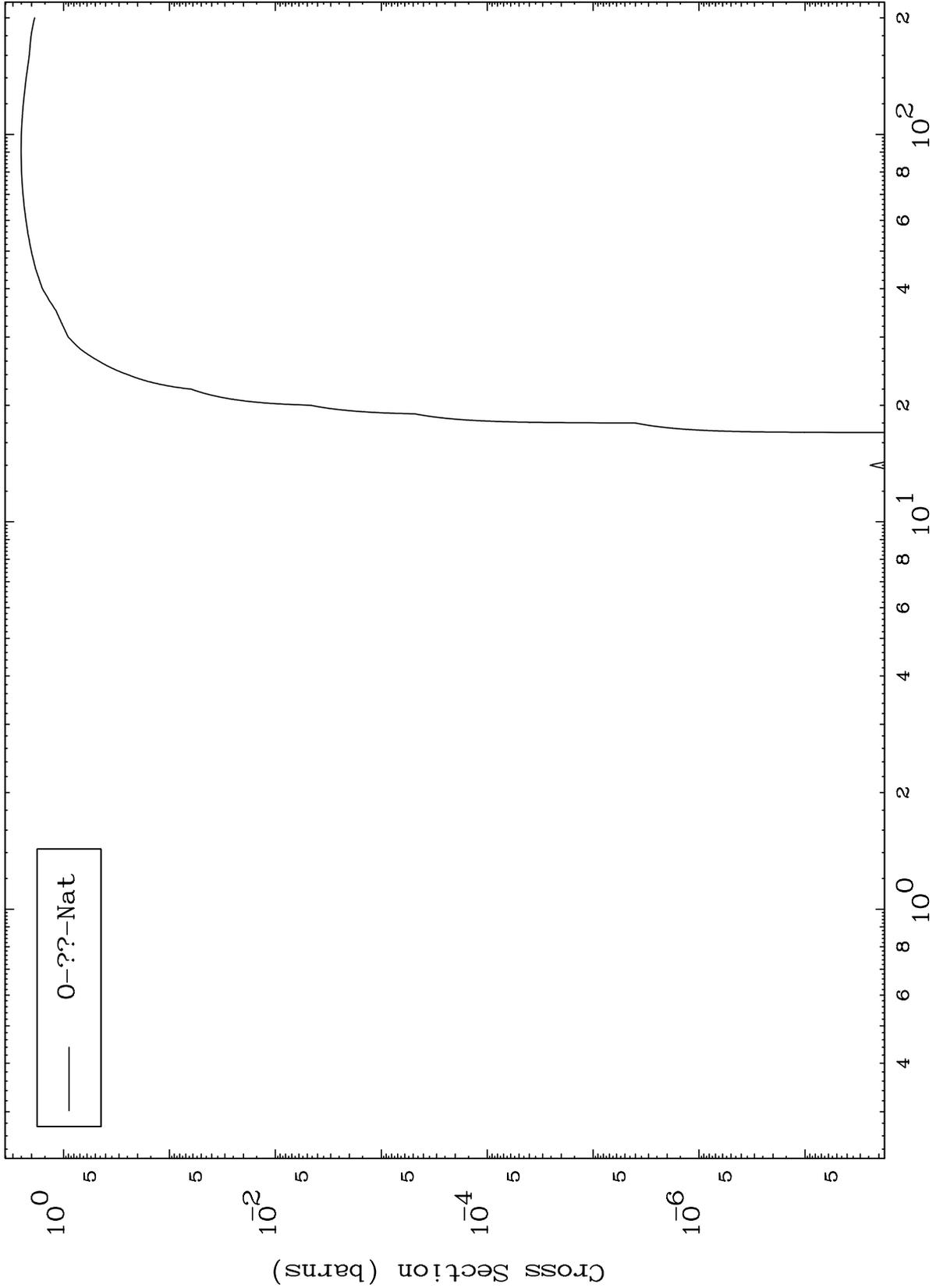


MAT 9119

91-Pa-227

$\alpha$  Fission

Radionuclide Production Cross Section



0-??-Nat

10

Incident Energy (MeV)

91-Pa-227