

Program EVALPLOT  
(Version 2018-1)

by

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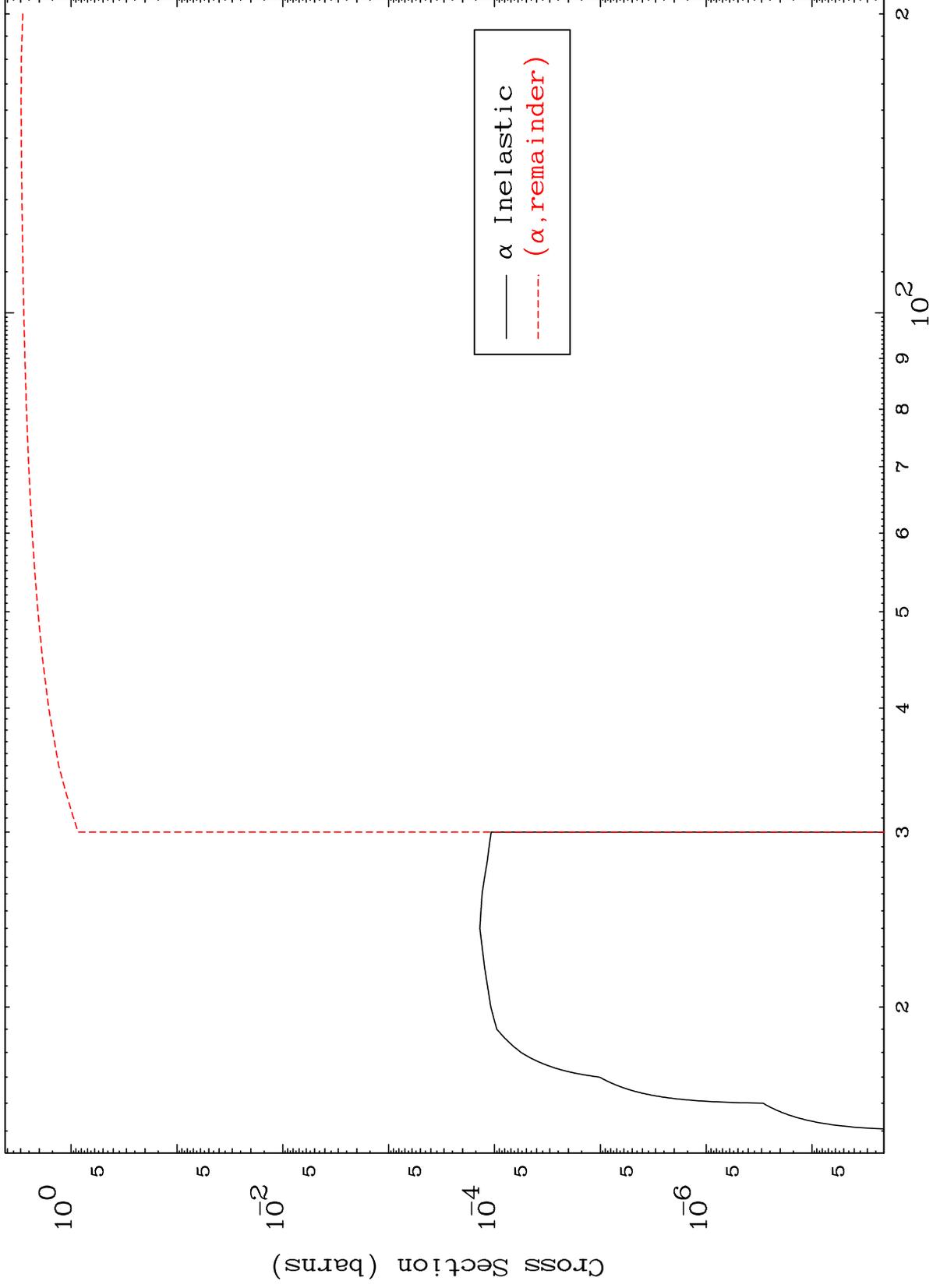
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 9422

$\alpha$  Major  
0 Kelvin Cross Sections

94-Pu-234



1

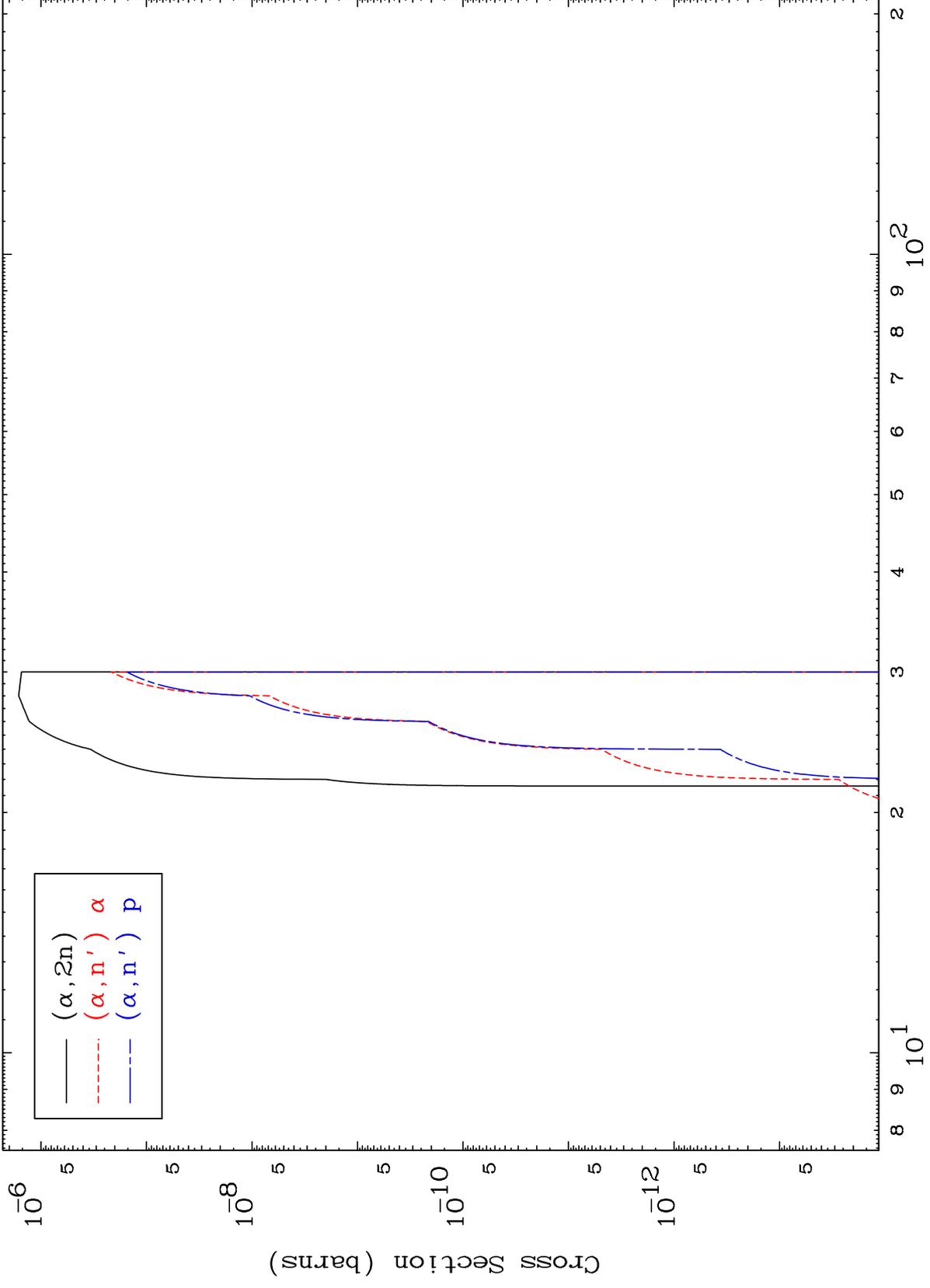
Incident Energy (MeV)

94-Pu-234

MAT 9422

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

94-Pu-234

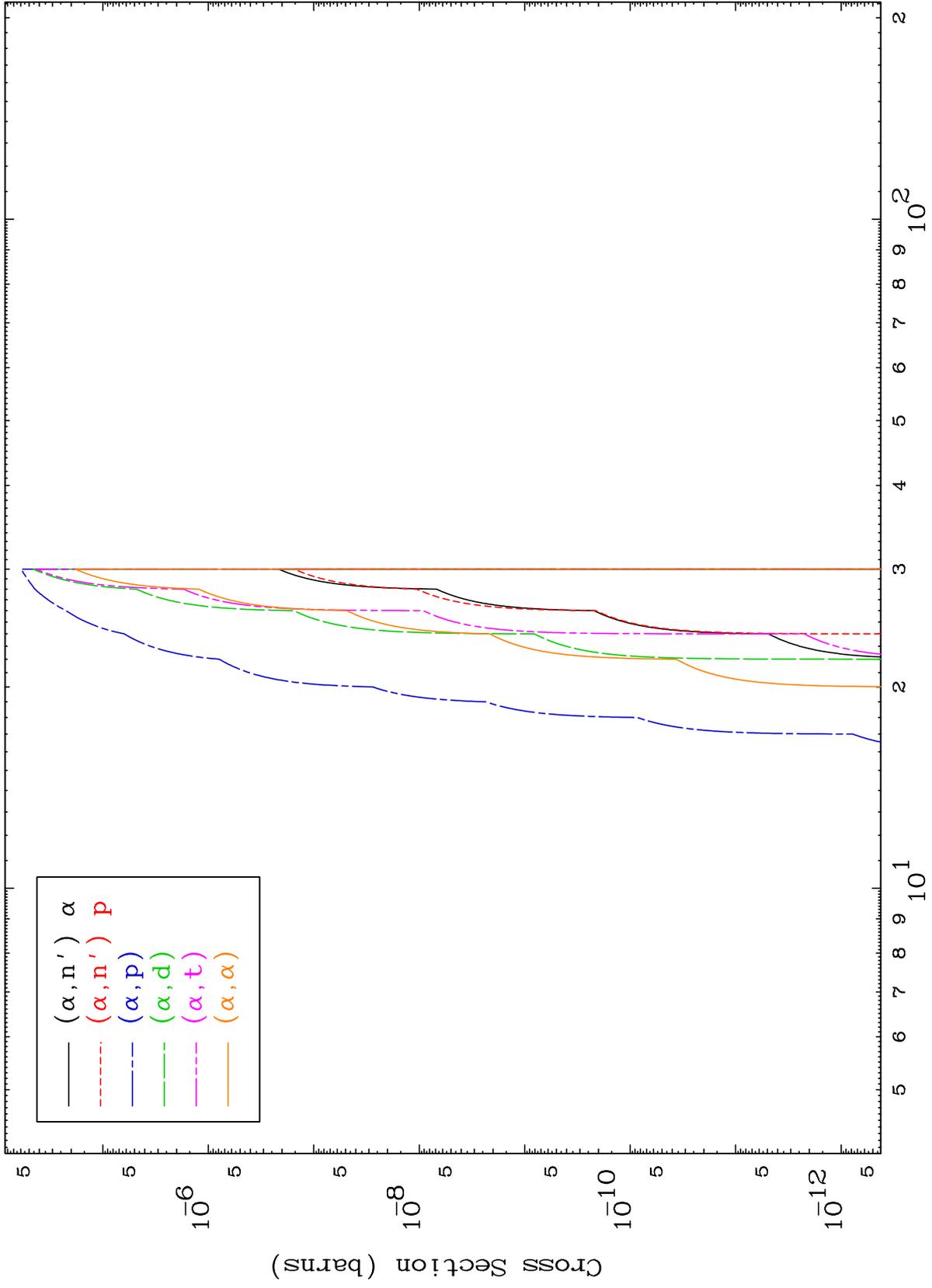


( $\alpha, 2n$ )  
( $\alpha, n'$ )  $\alpha$   
( $\alpha, n'$ )  $p$

94-Pu-234

Incident Energy (MeV)

2

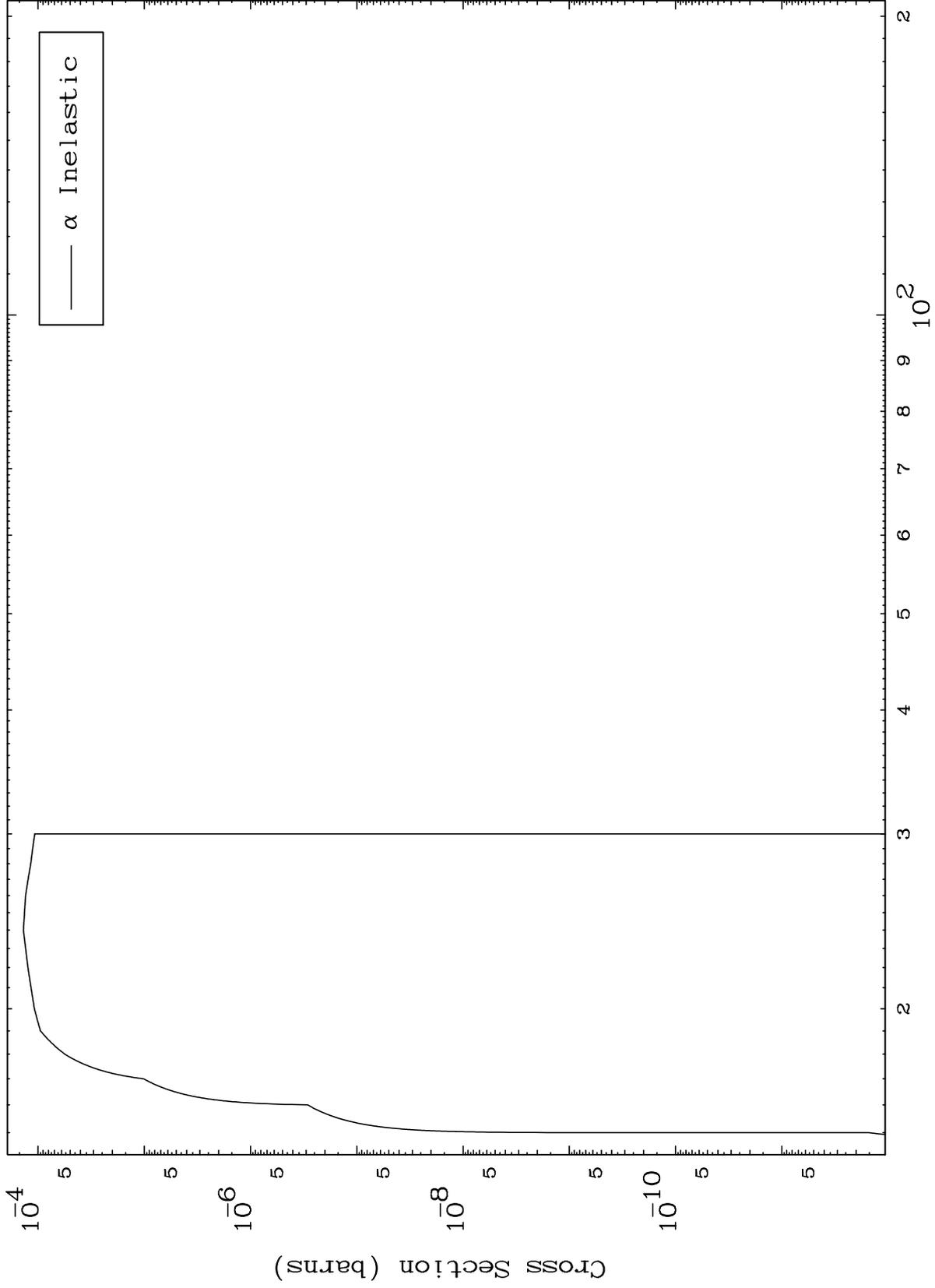


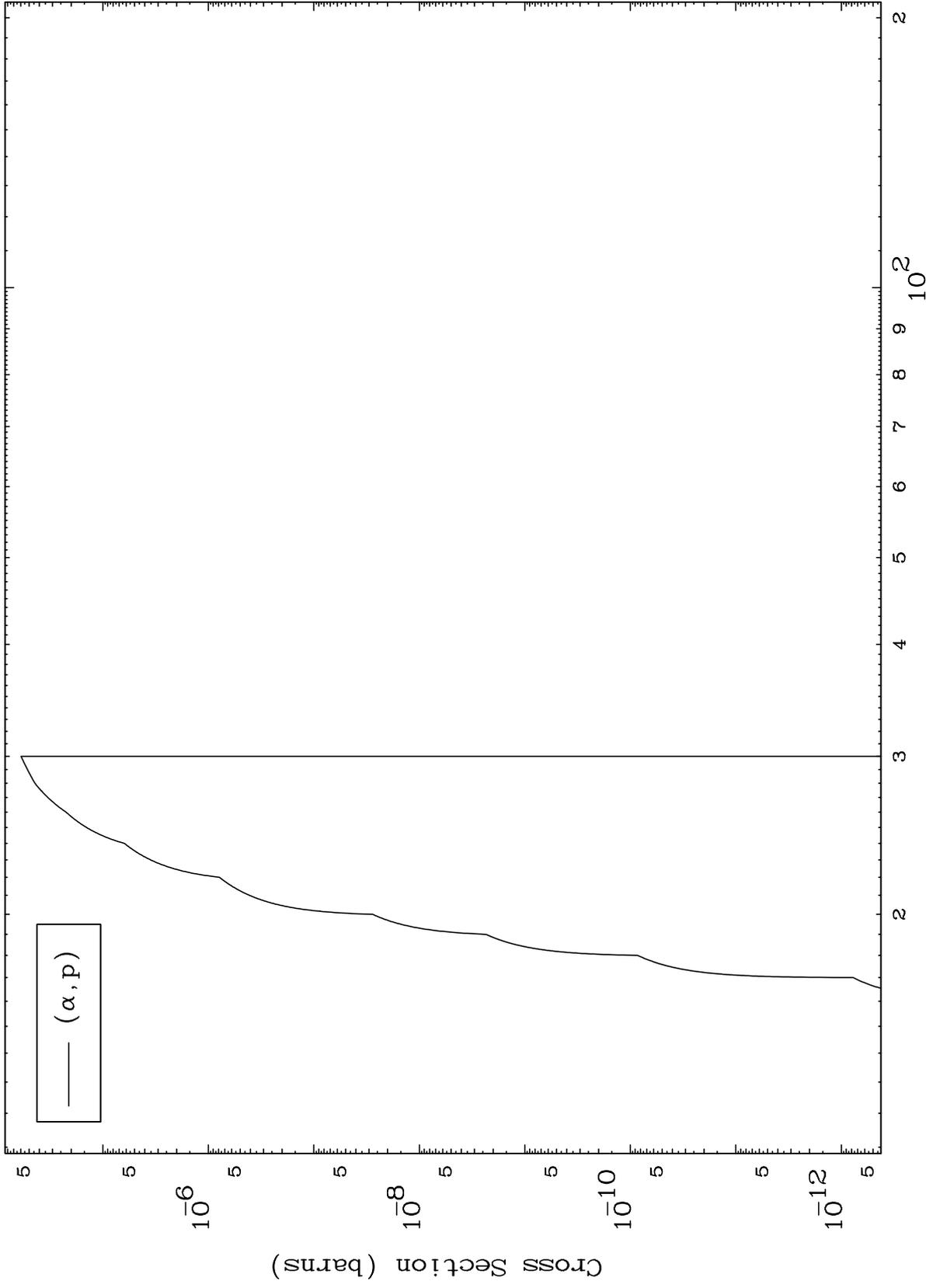
MAT 9422

( $\alpha, n'$ ) Level

94-Pu-234

0 Kelvin Cross Sections

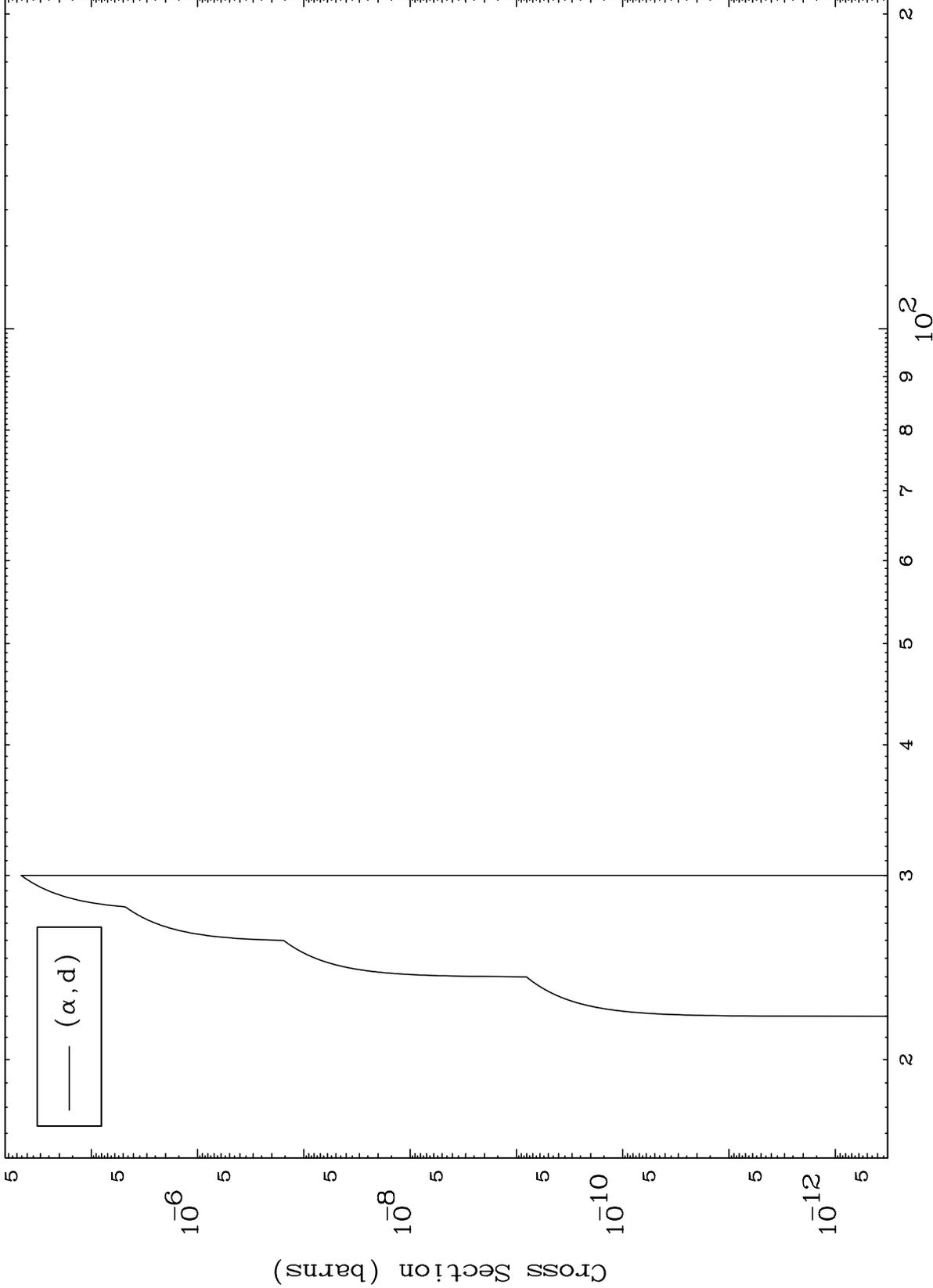




MAT 9422

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

94-Pu-234



6

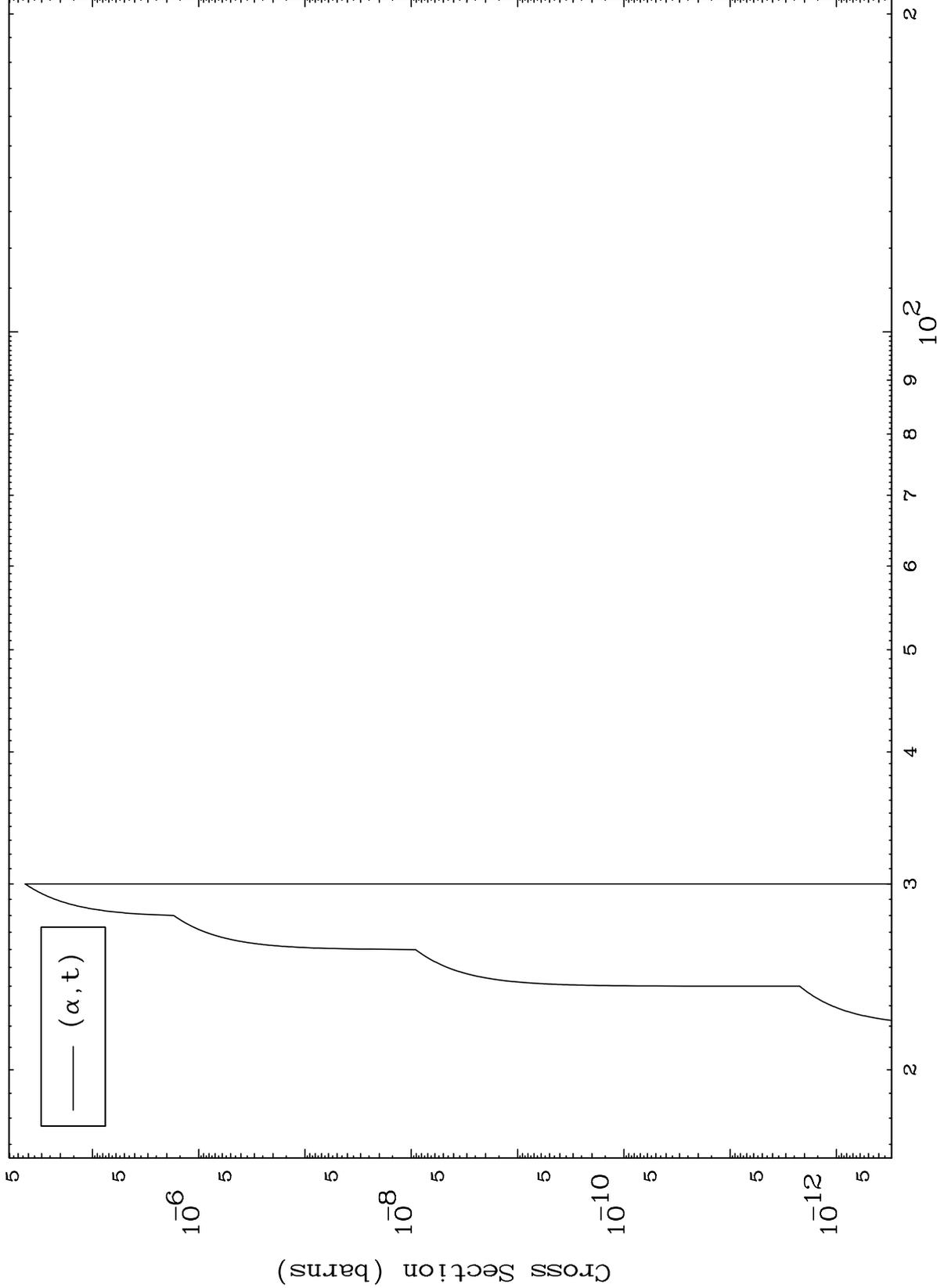
Incident Energy (MeV)

94-Pu-234

MAT 9422

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

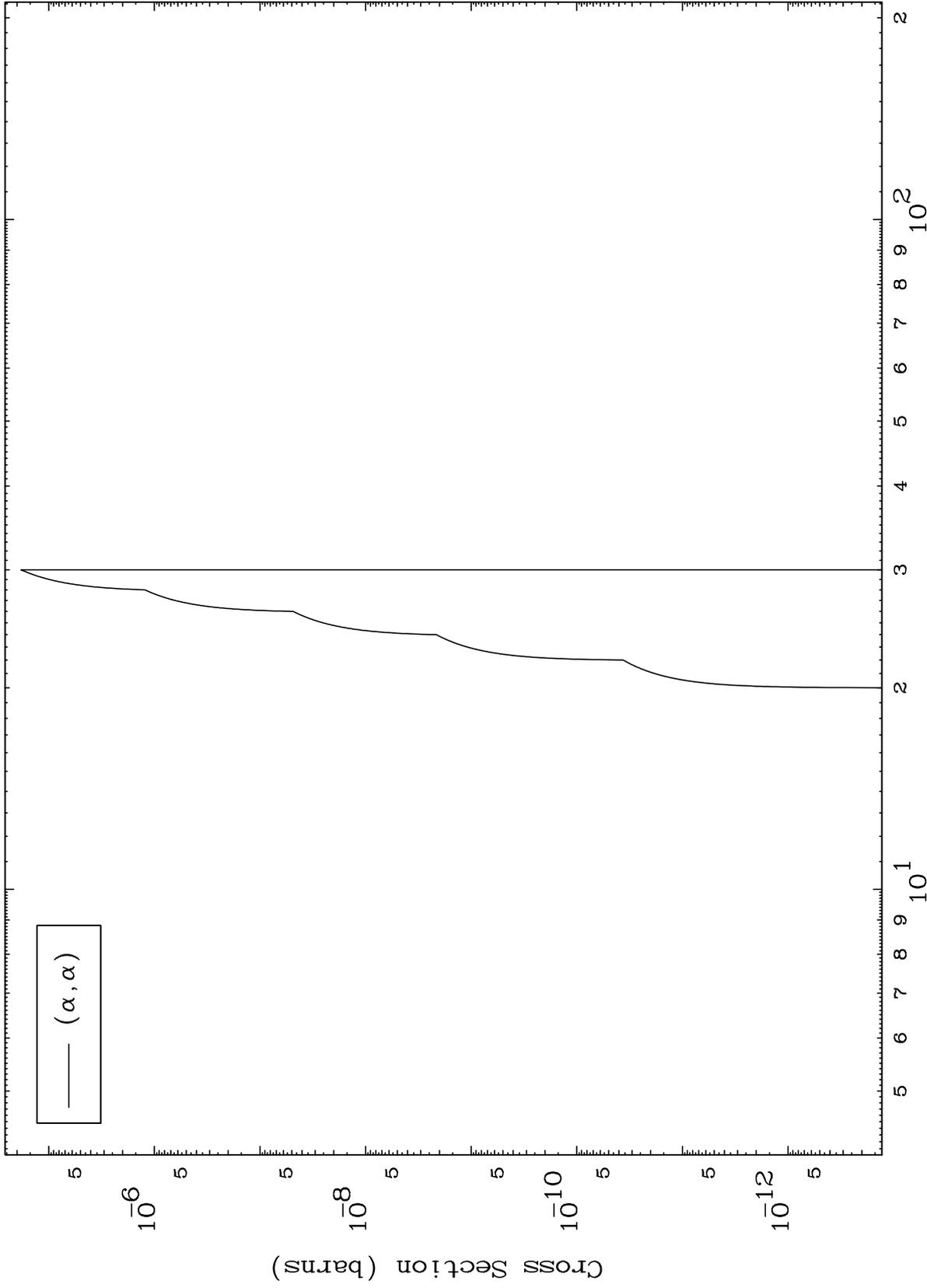
94-Pu-234



7

Incident Energy (MeV)

94-Pu-234

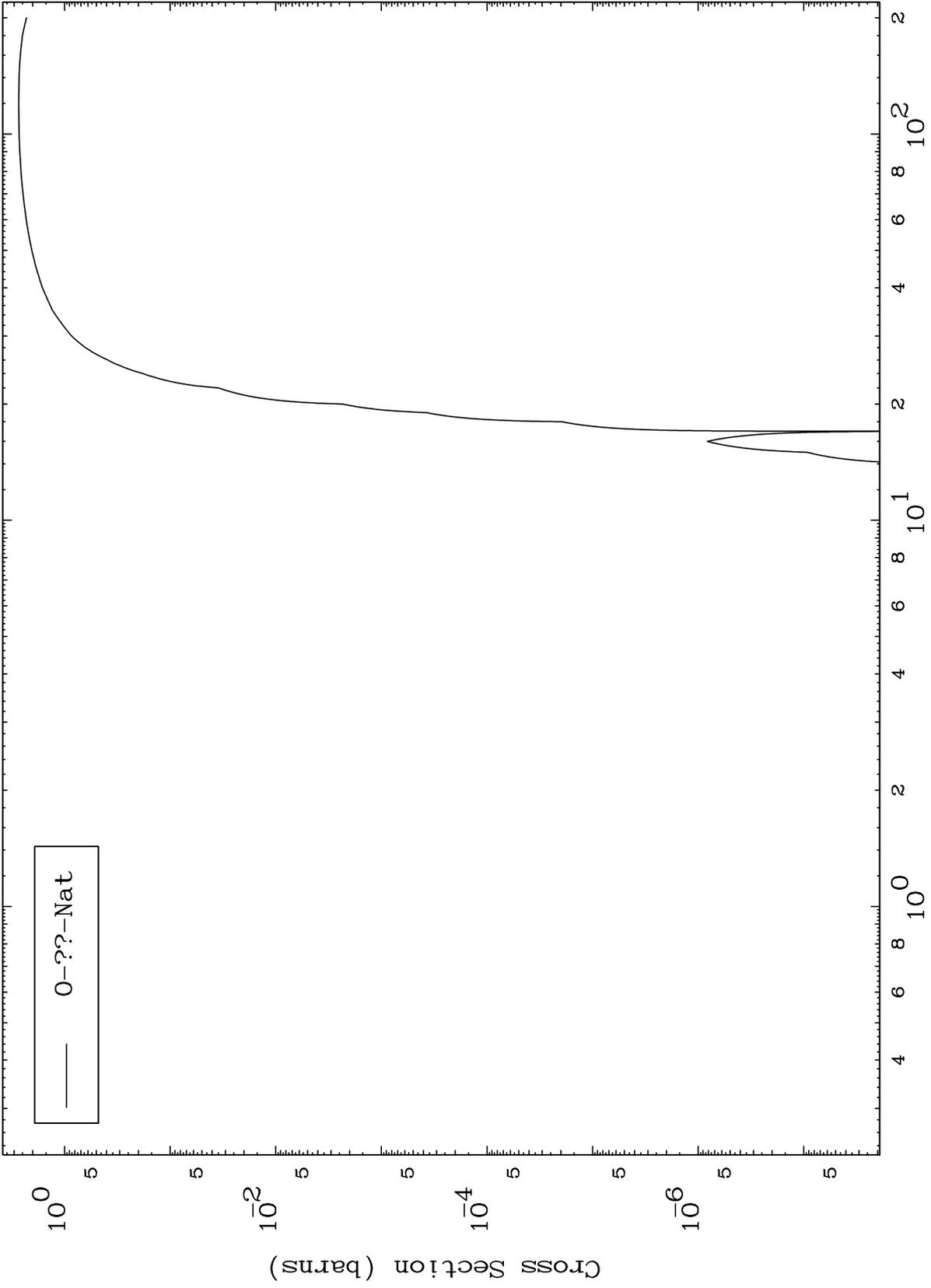


MAT 9422

94-Pu-234

$\alpha$  Fission

Radionuclide Production Cross Section

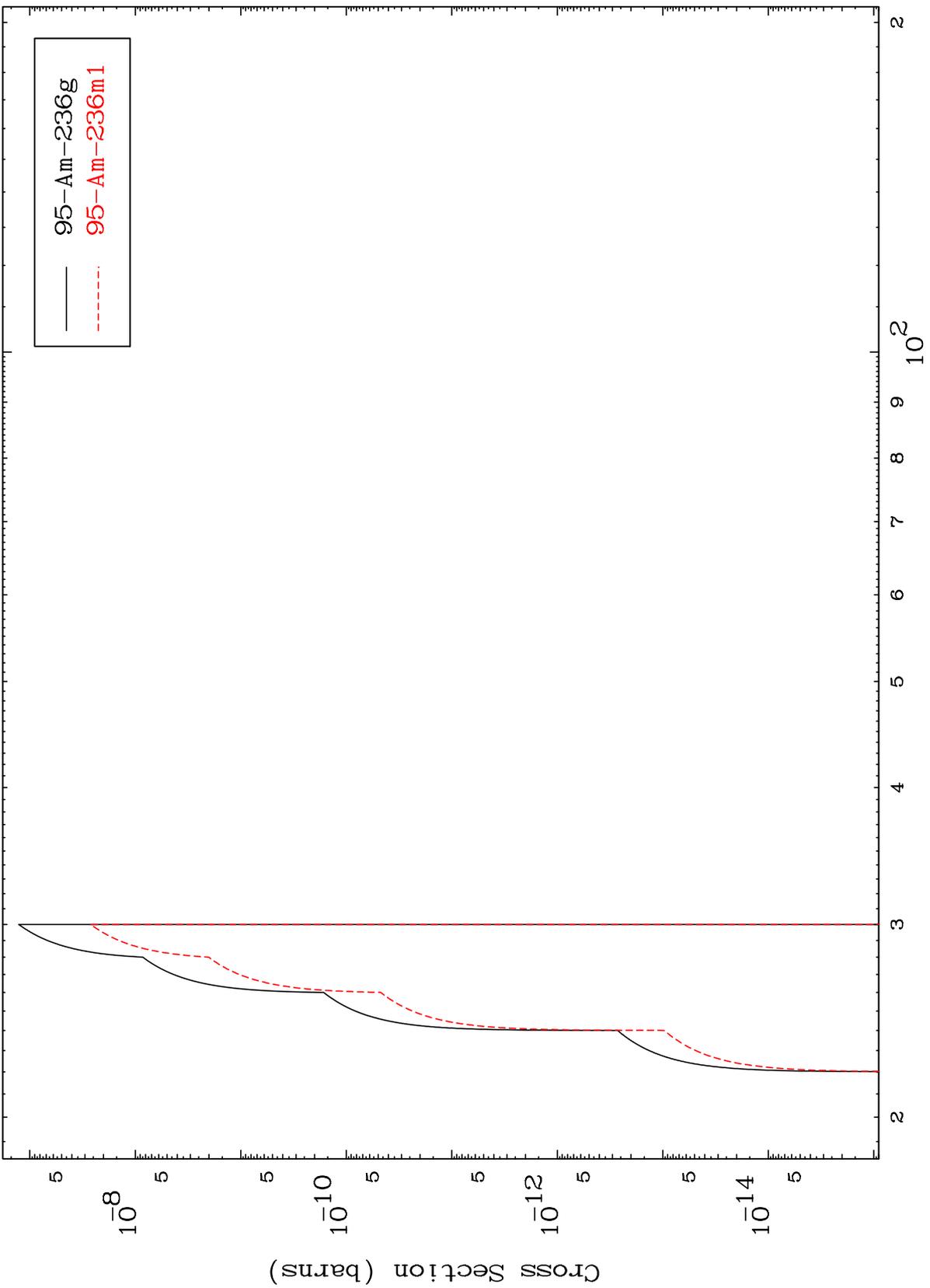


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$(\alpha, n')$  p

94-Pu-234

Radionuclide Production Cross Section



10

Incident Energy (MeV)

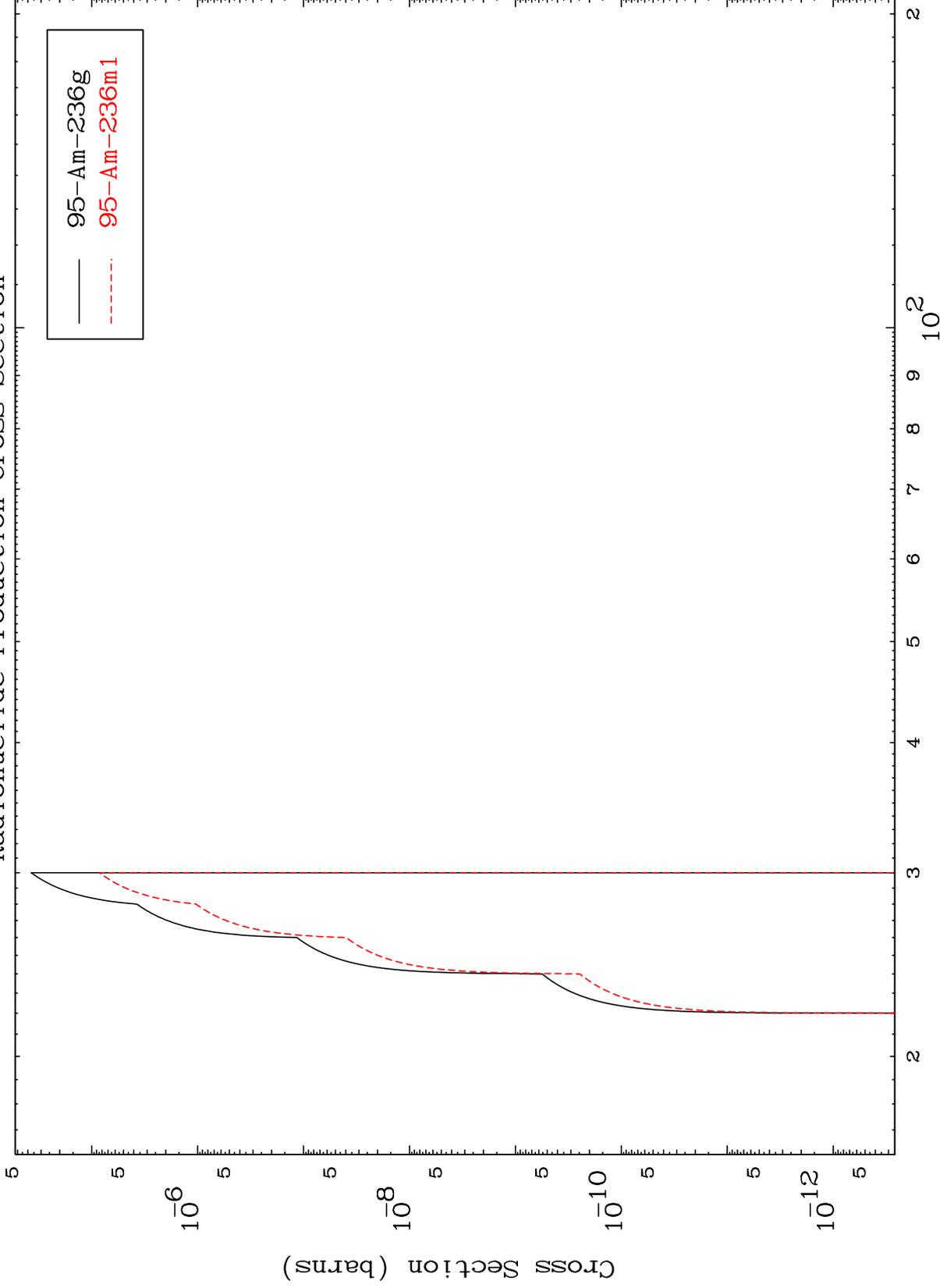
94-Pu-234

MAT 9422

( $\alpha, d$ )

94-Pu-234

Radionuclide Production Cross Section



11

Incident Energy (MeV)

94-Pu-234