

Program EVALPLOT  
(Version 2018-1)

by

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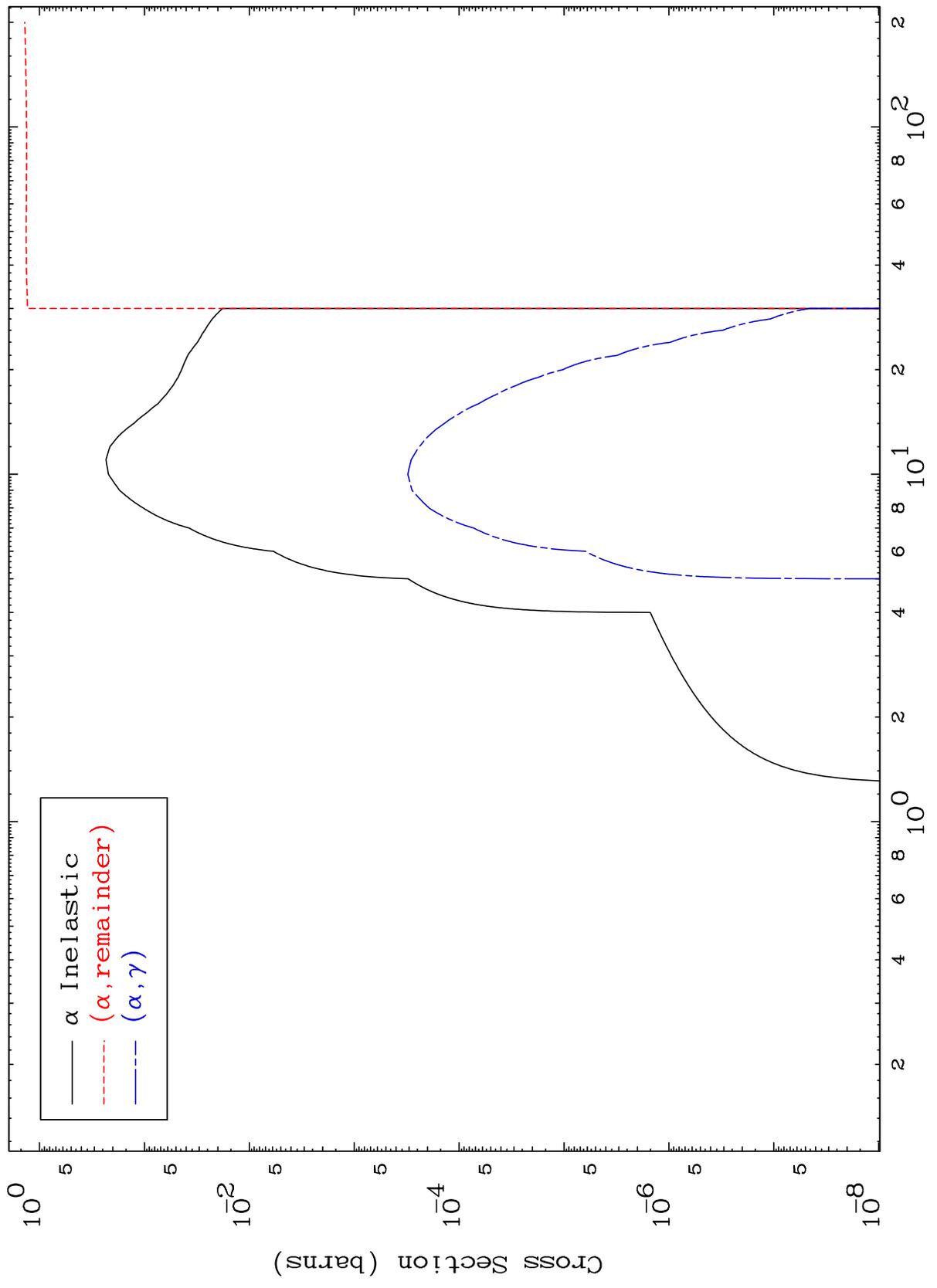
Press Mouse Button to Start

MAT 2123

$\alpha$  Major

21-Sc-44

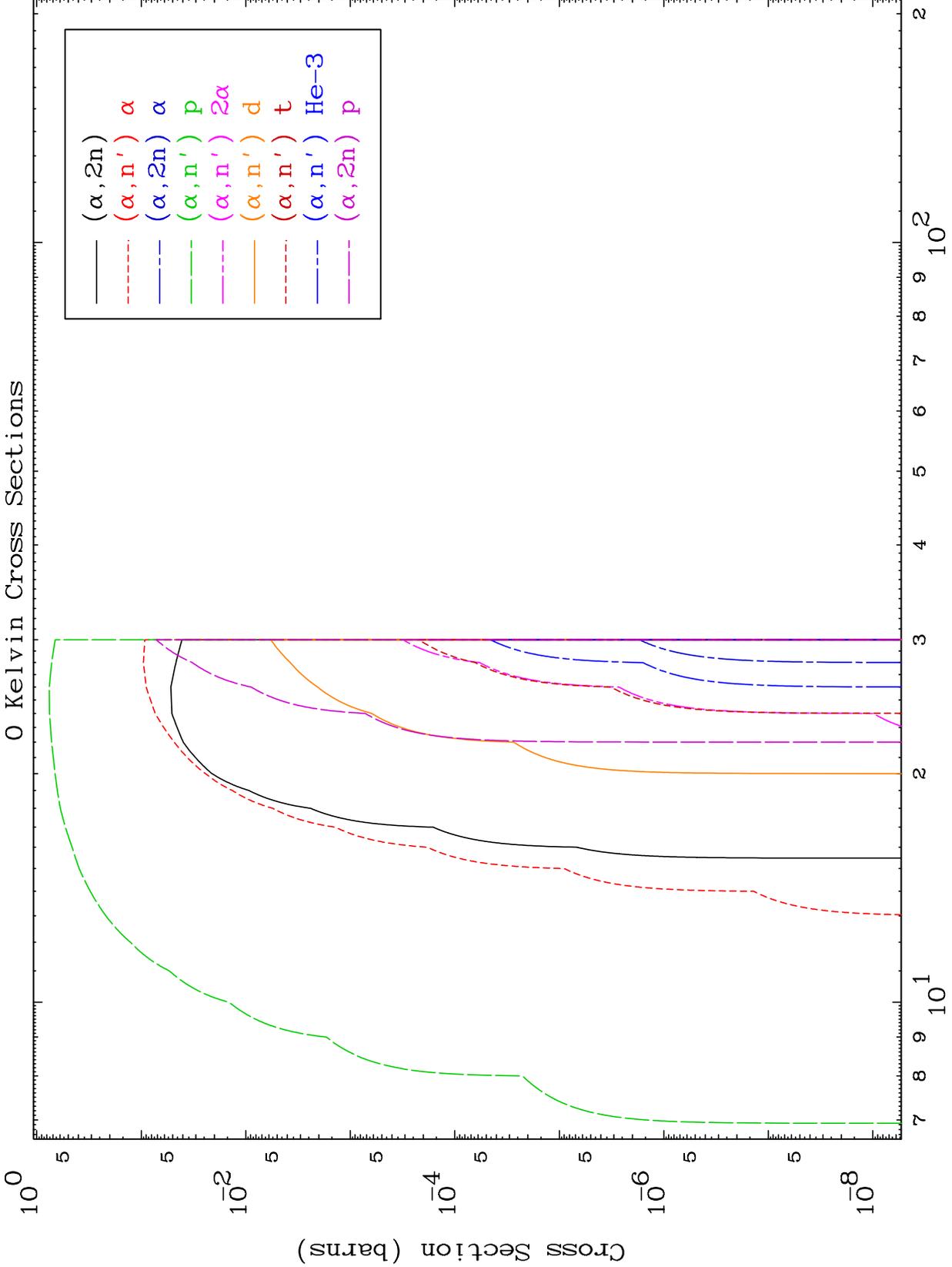
0 Kelvin Cross Sections



MAT 2123

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

21-Sc-44



2

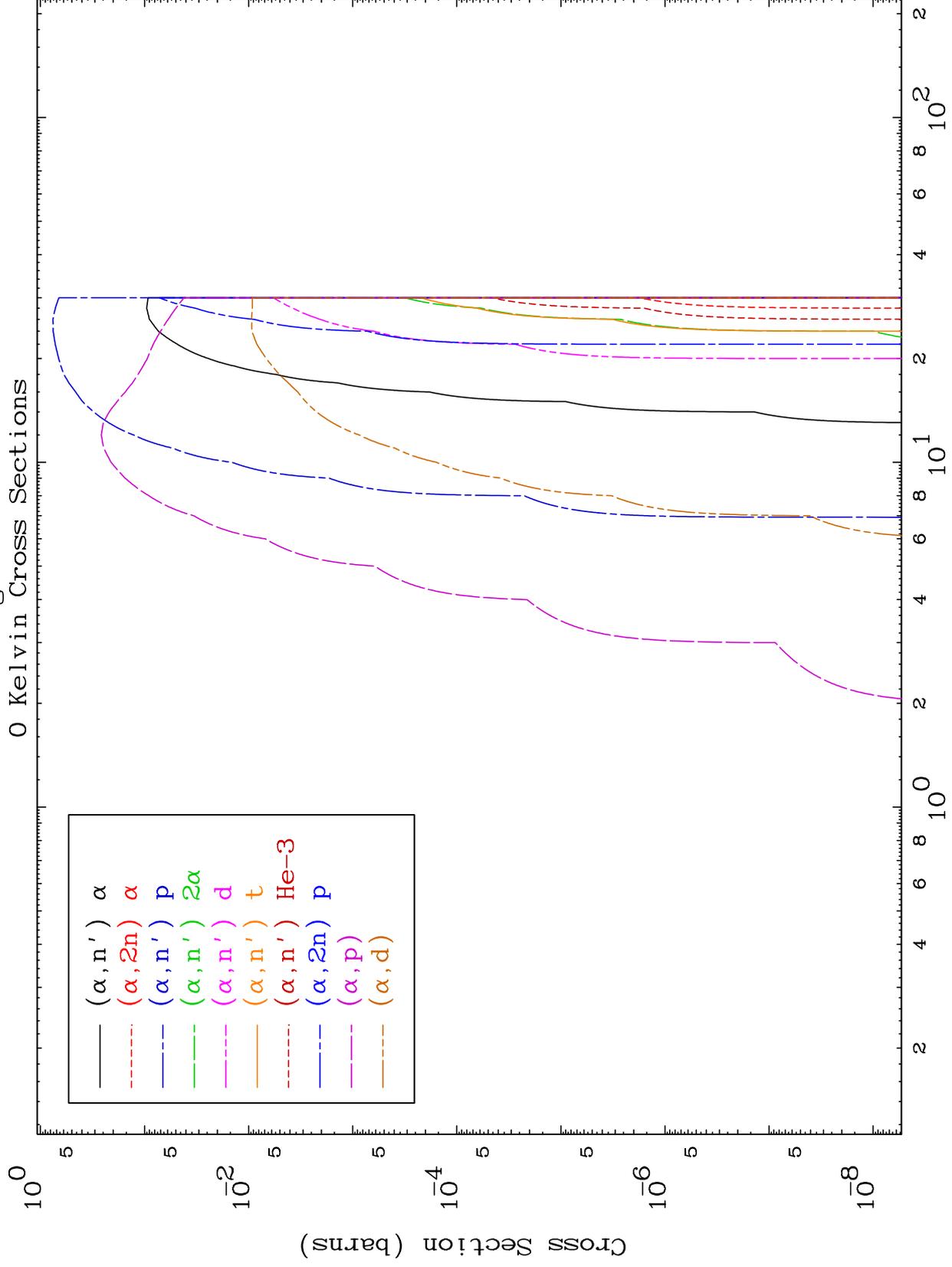
Incident Energy (MeV)

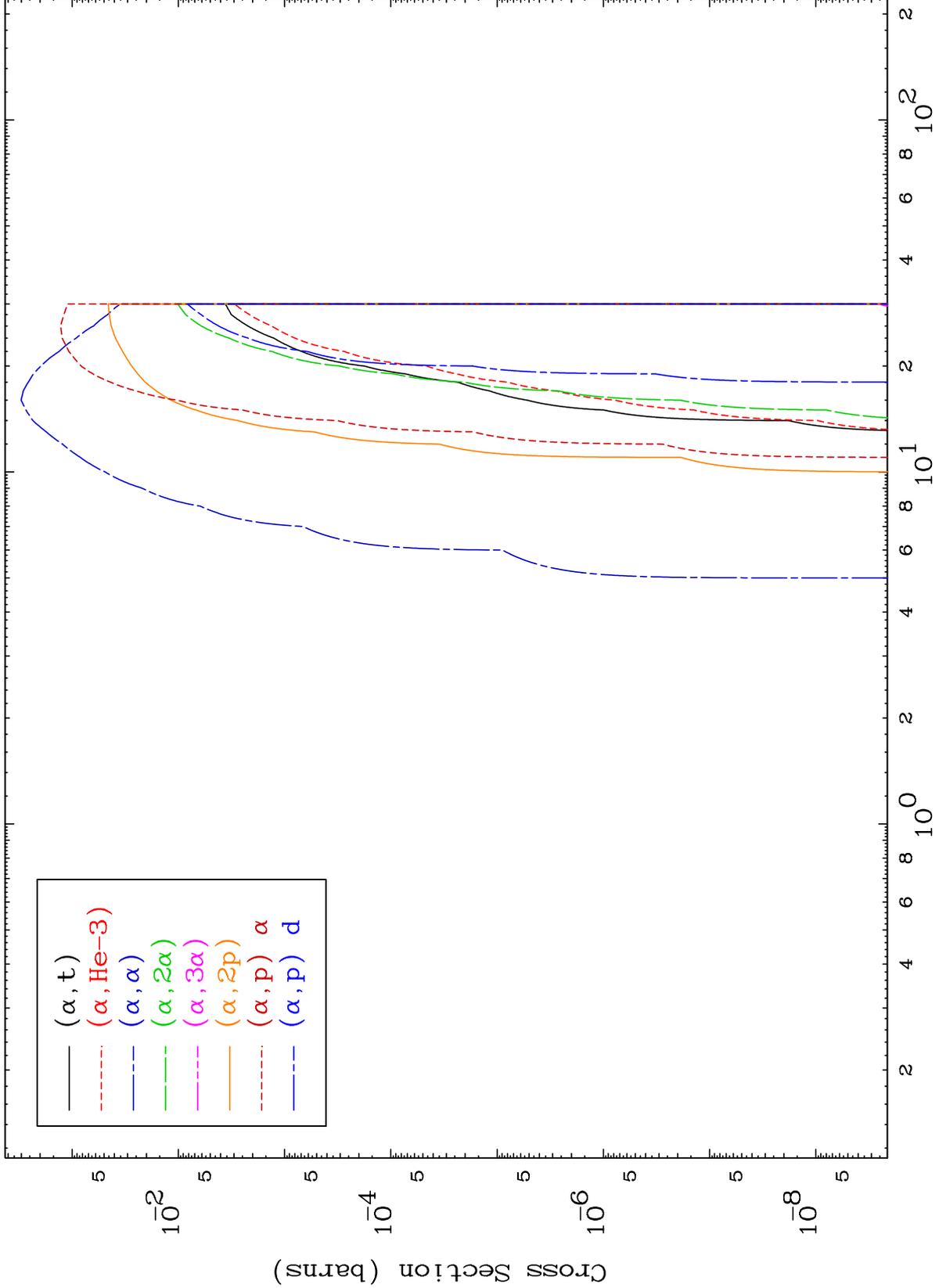
21-Sc-44

MAT 2123

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

21-Sc-44

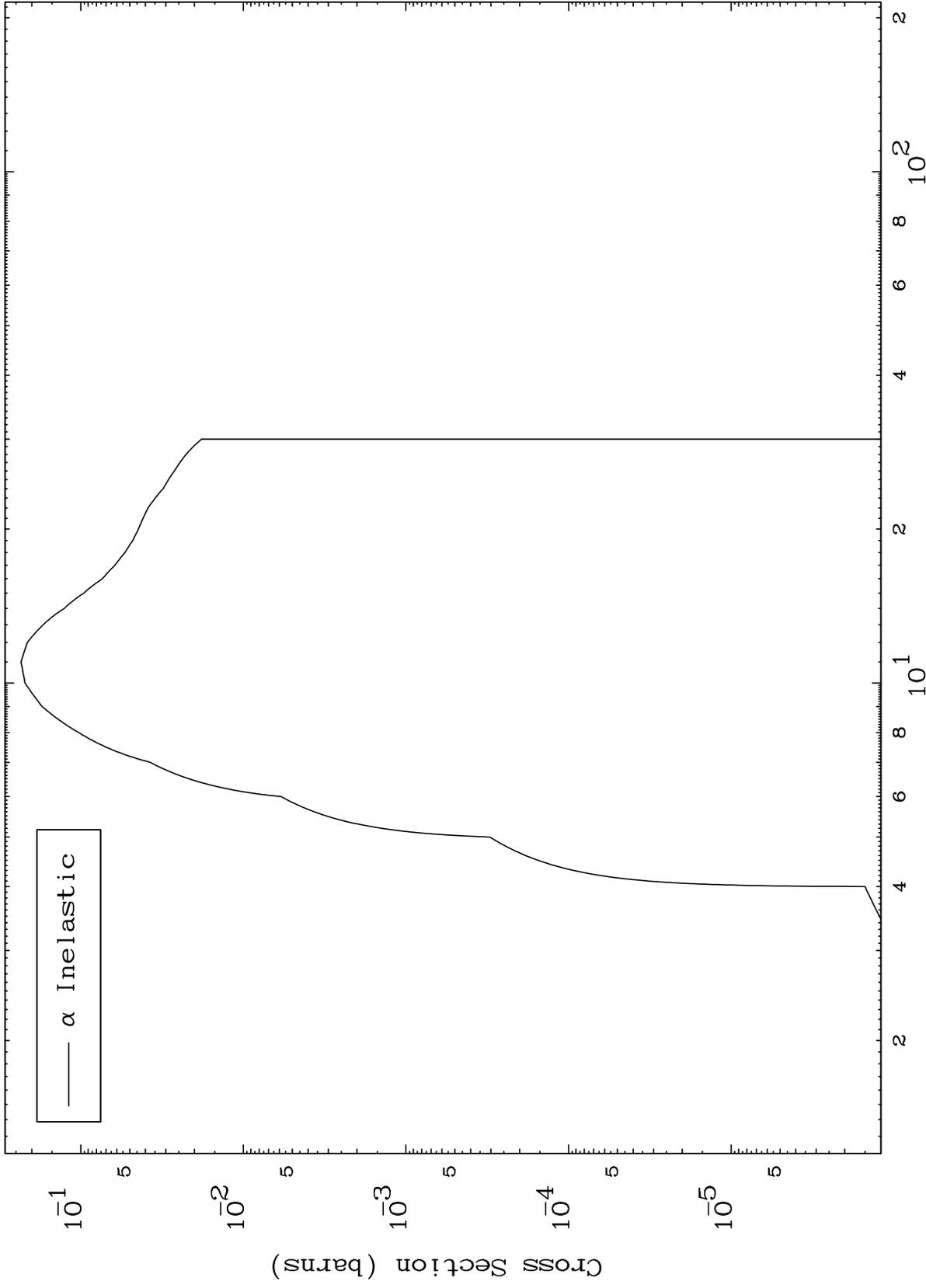




MAT 2123

21-Sc-44

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections



5

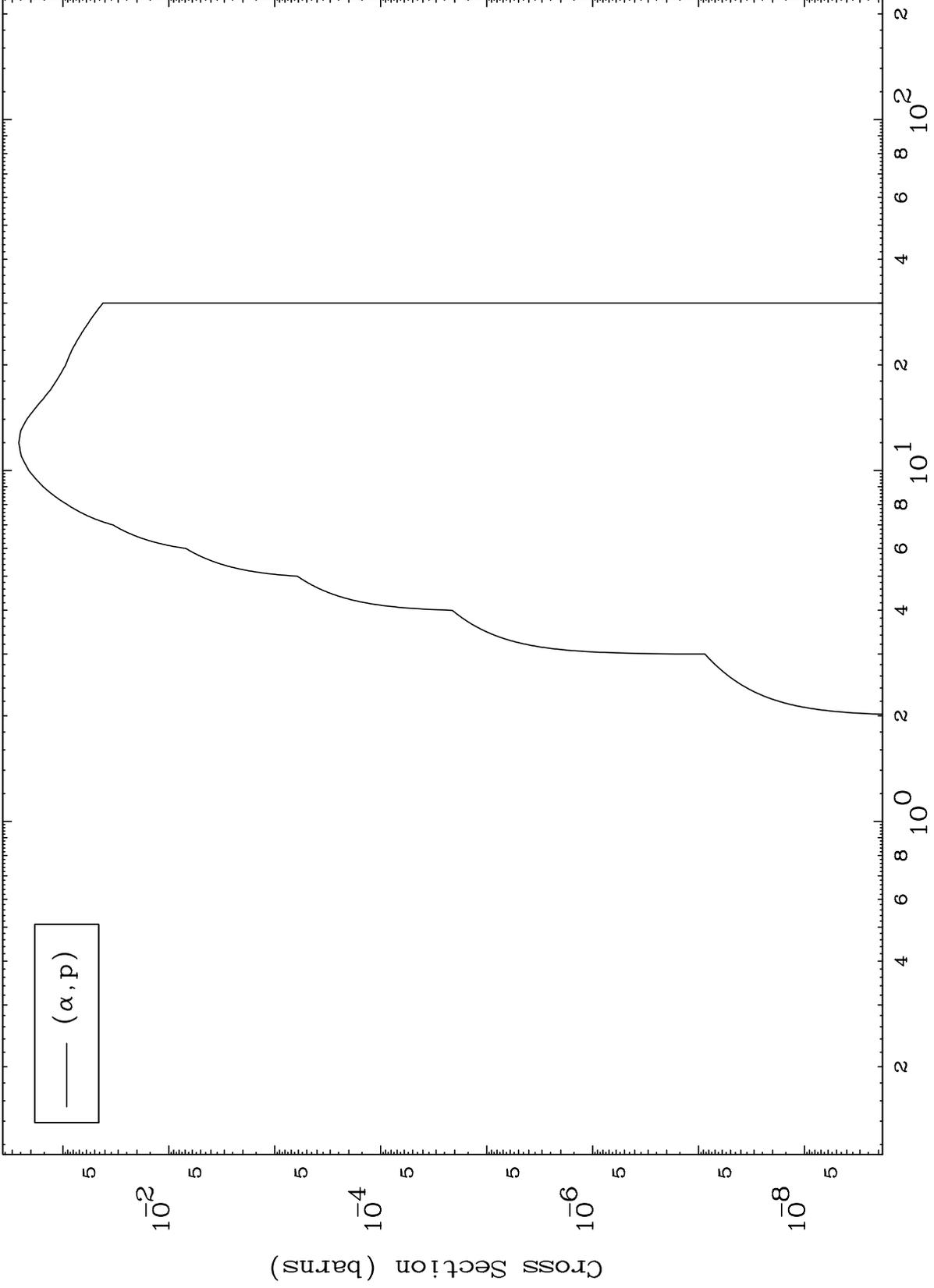
Incident Energy (MeV)

21-Sc-44

MAT 2123

21-Sc-44

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections



6

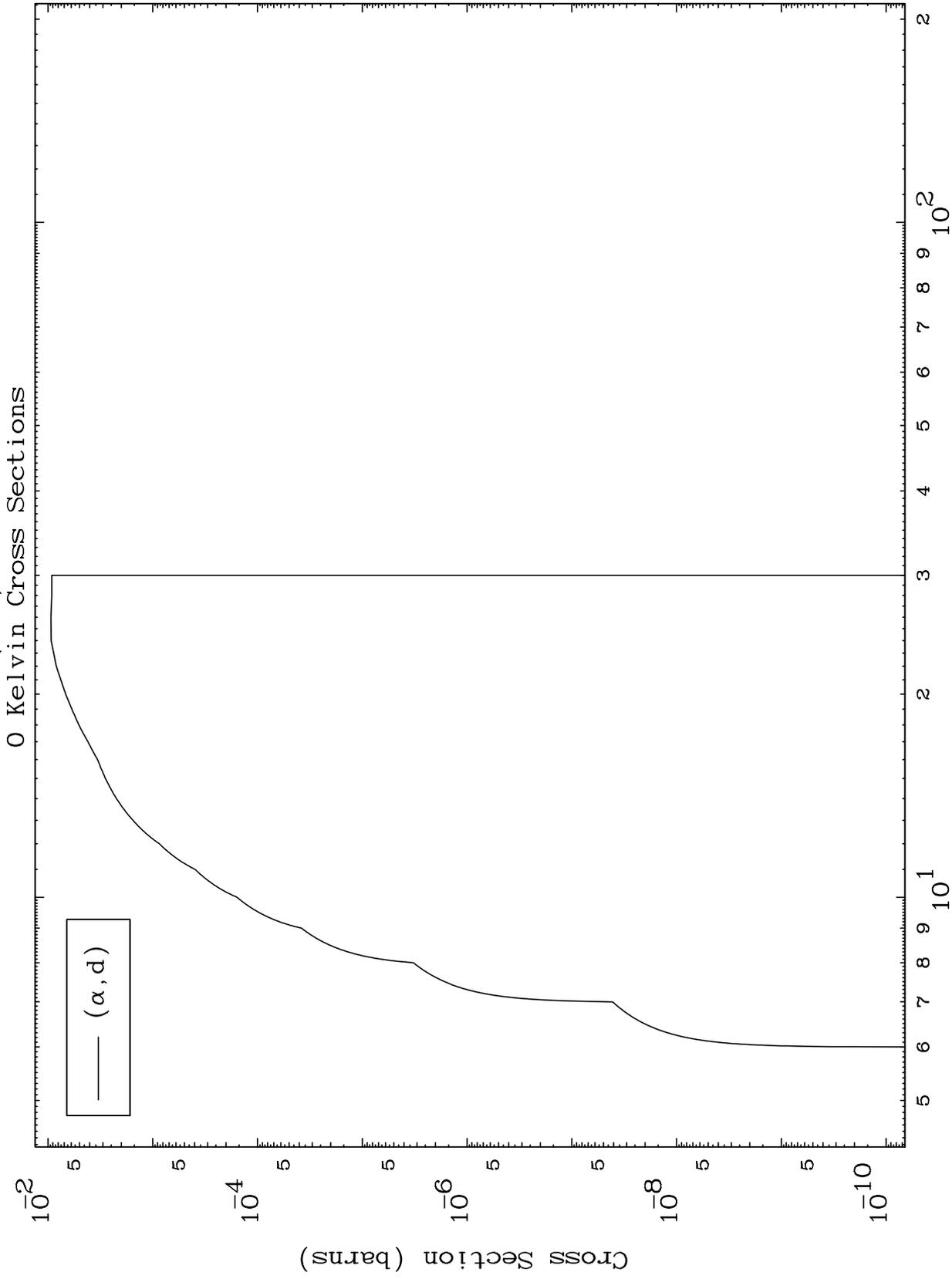
Incident Energy (MeV)

21-Sc-44

MAT 2123

( $\alpha, d$ ) Levels

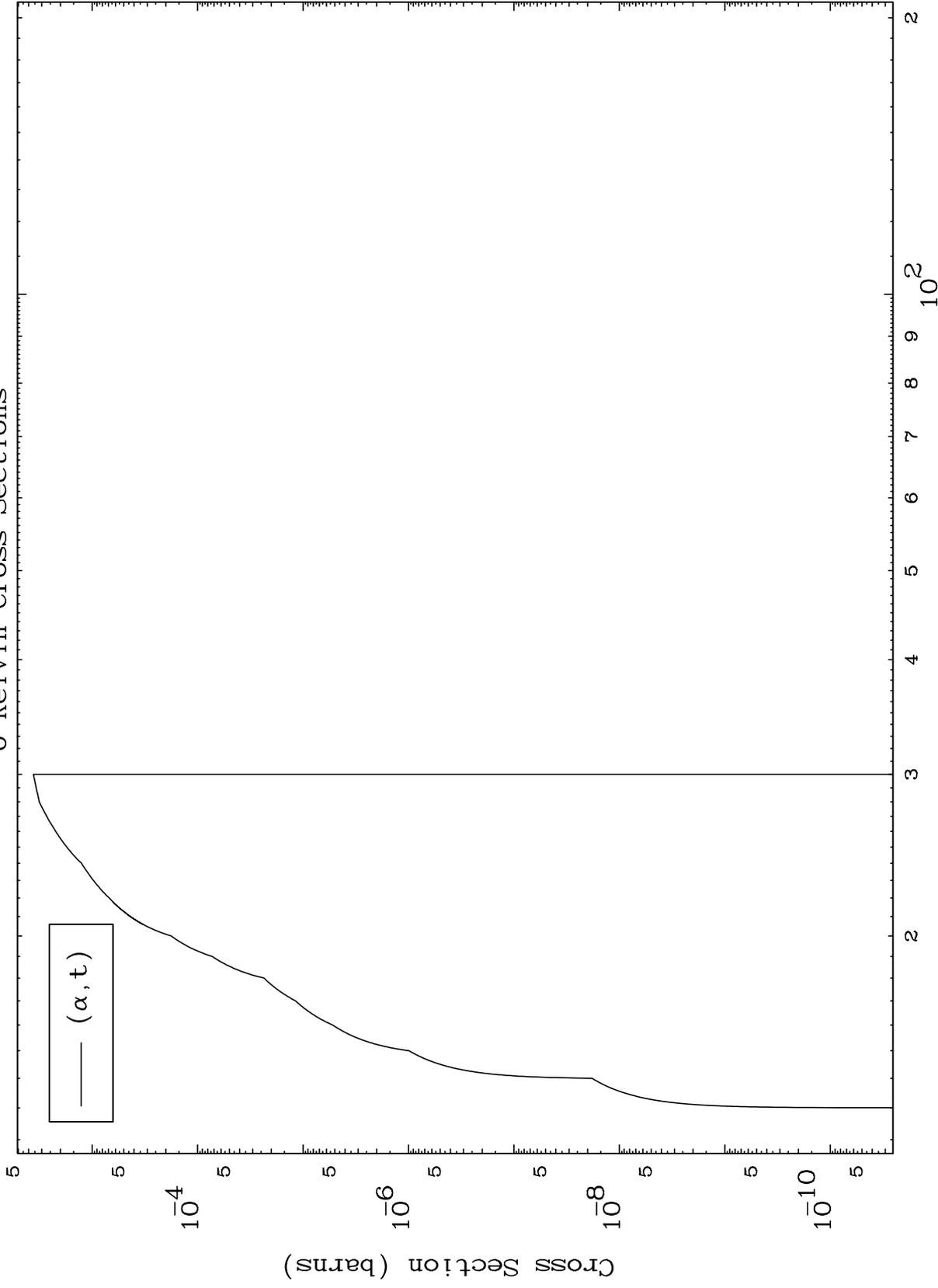
21-Sc-44



MAT 2123

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

21-Sc-44



8

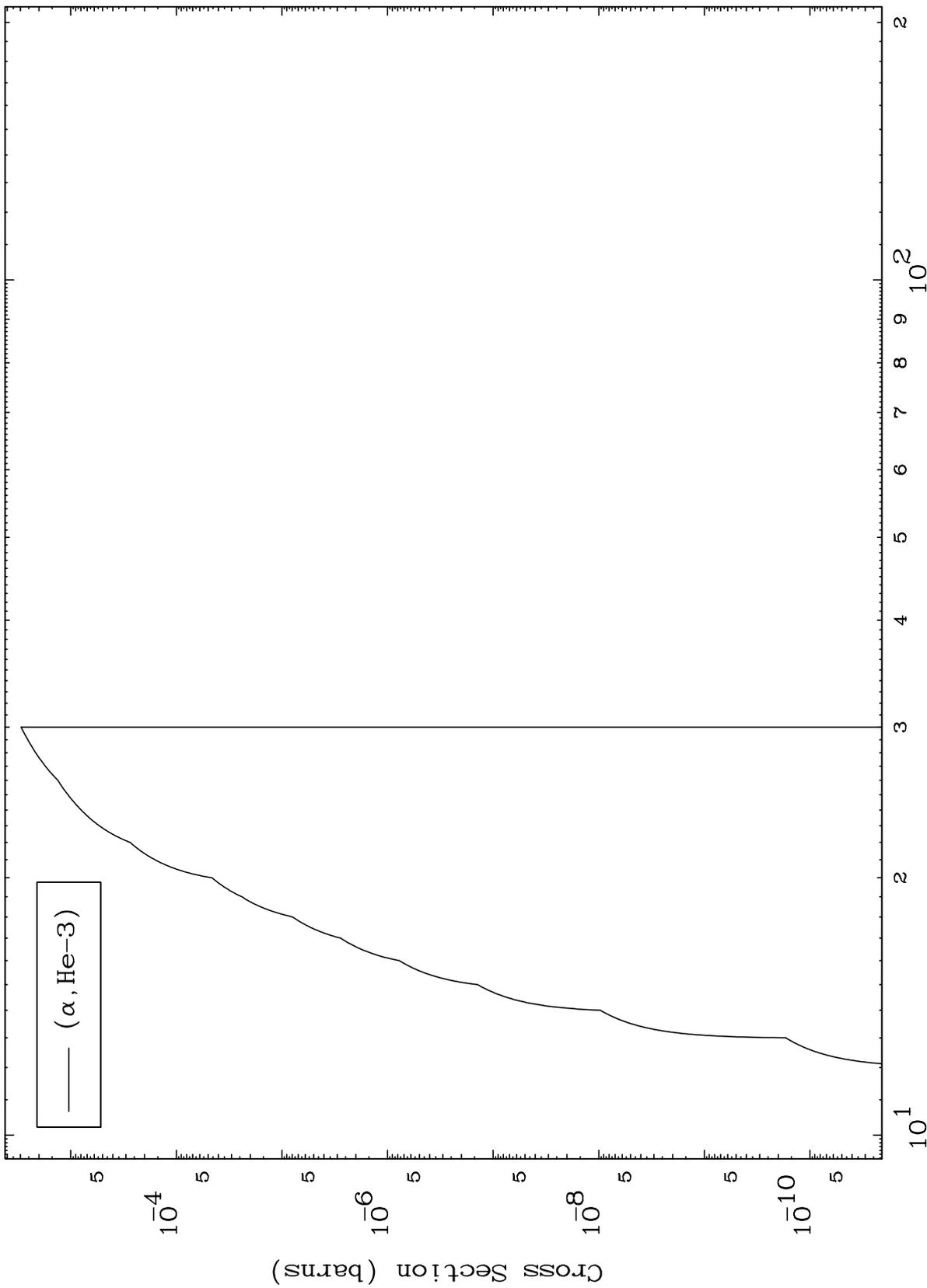
Incident Energy (MeV)

21-Sc-44

MAT 2123

21-Sc-44

( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections



21-Sc-44

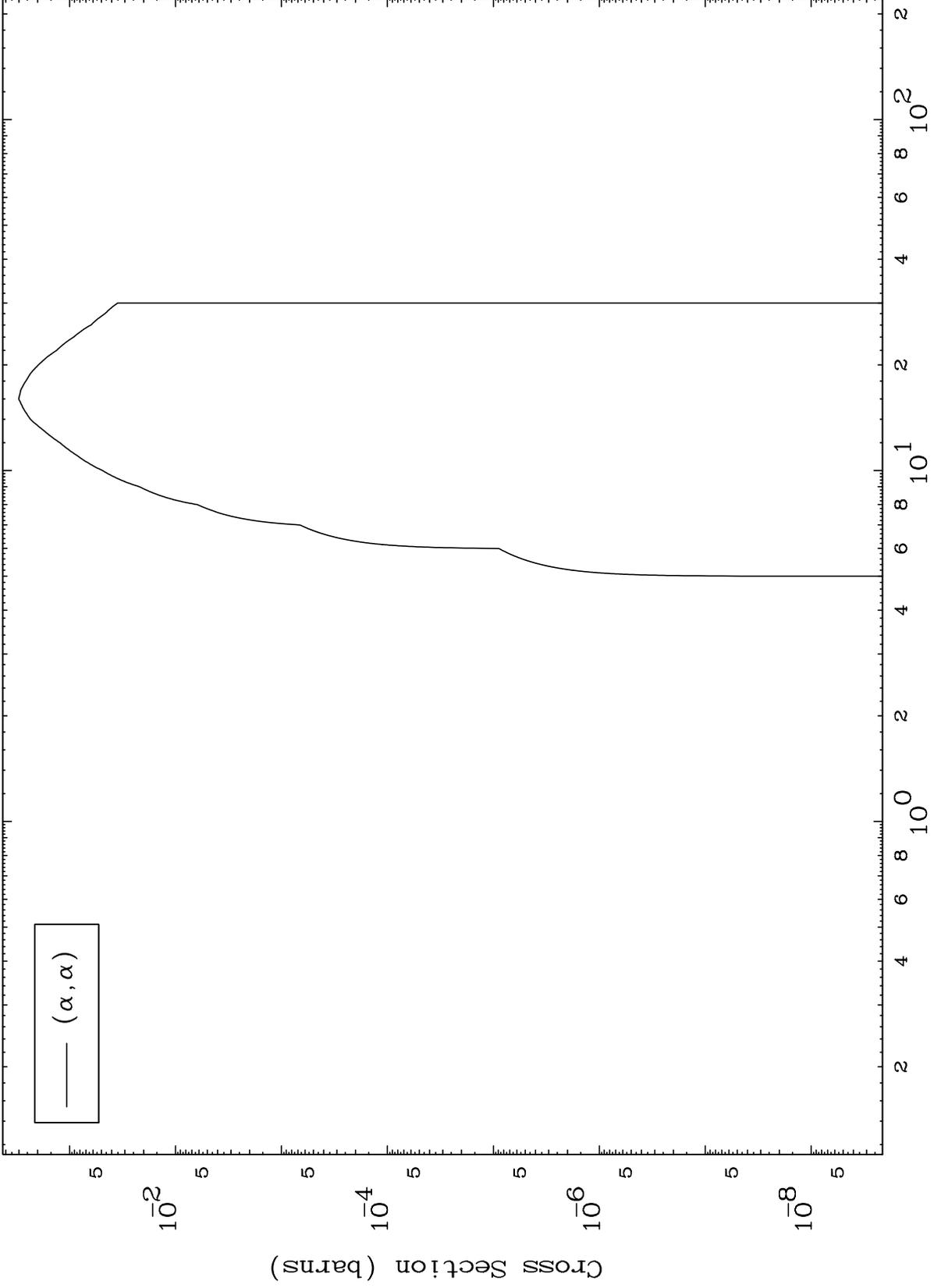
Incident Energy (MeV)

MAT 2123

( $\alpha, \alpha$ ) Levels

21-Sc-44

0 Kelvin Cross Sections



10

Incident Energy (MeV)

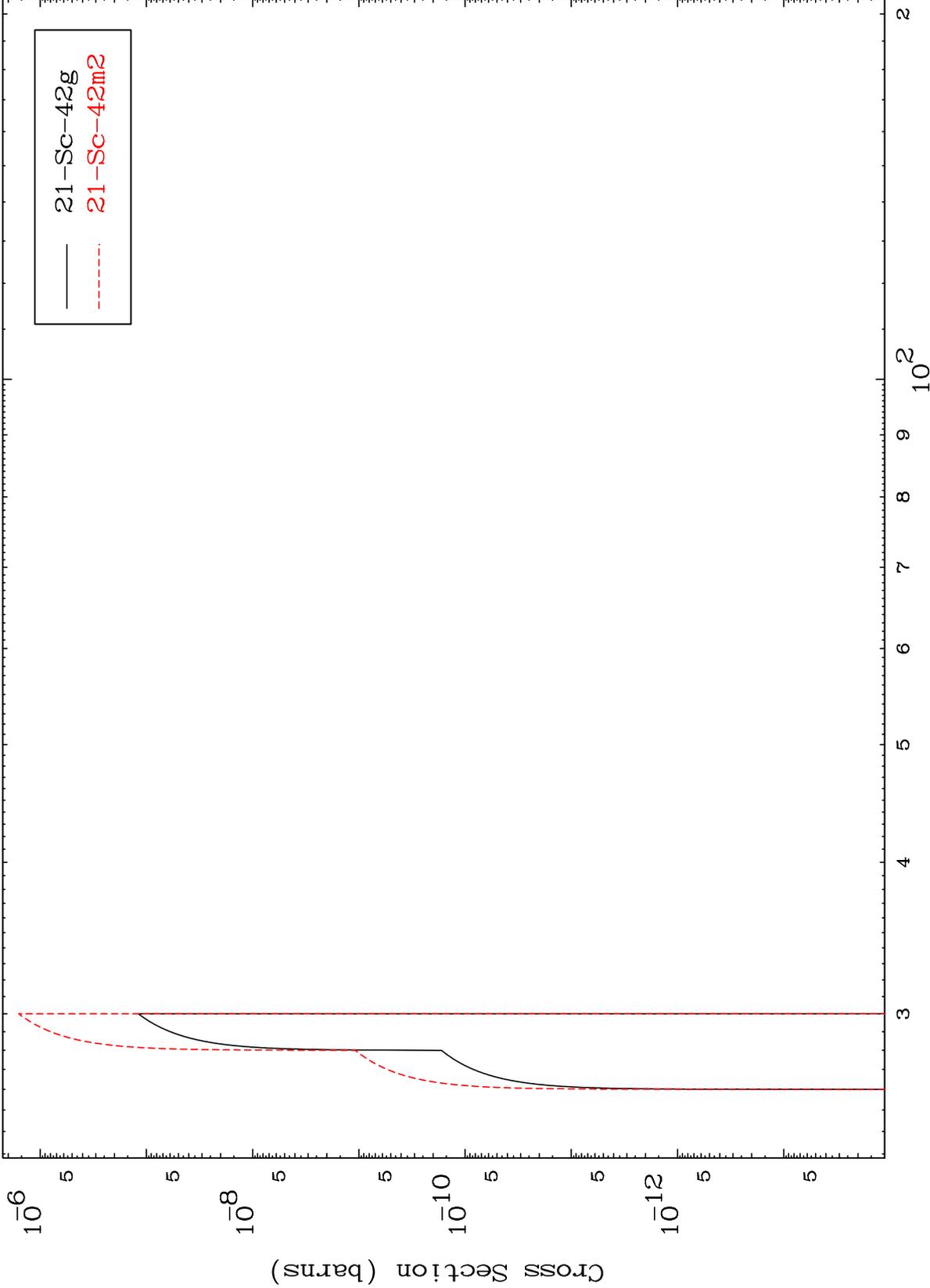
21-Sc-44

MAT 2123

$(\alpha, 2n)$   $\alpha$

21-Sc-44

Radionuclide Production Cross Section



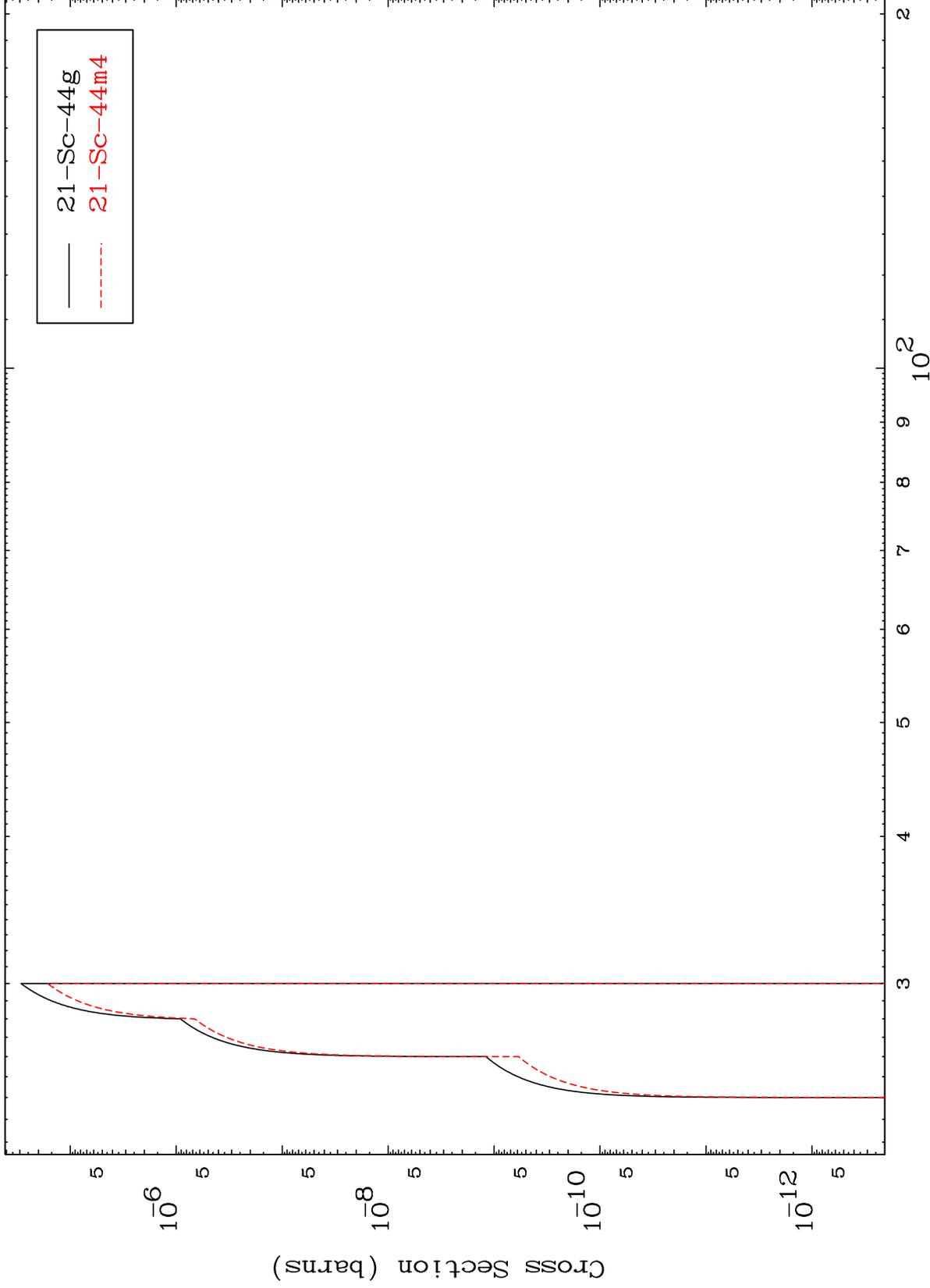
21-Sc-42g  
21-Sc-42m2

MAT 2123

( $\alpha, n'$ ) He-3

21-Sc-44

Radionuclide Production Cross Section



12

Incident Energy (MeV)

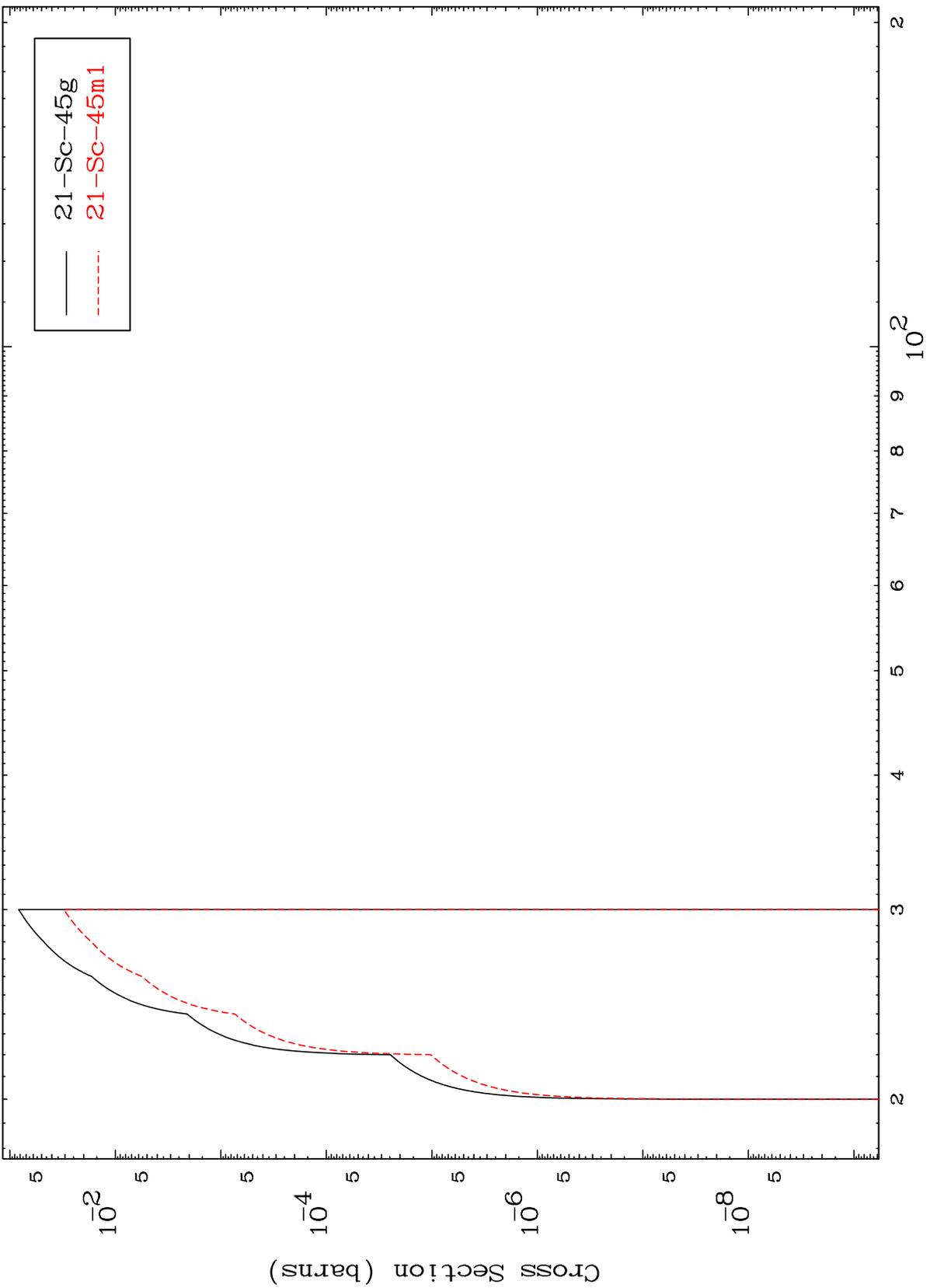
21-Sc-44

MAT 2123

( $\alpha, 2n$ ) p

21-Sc-44

Radionuclide Production Cross Section



13

Incident Energy (MeV)

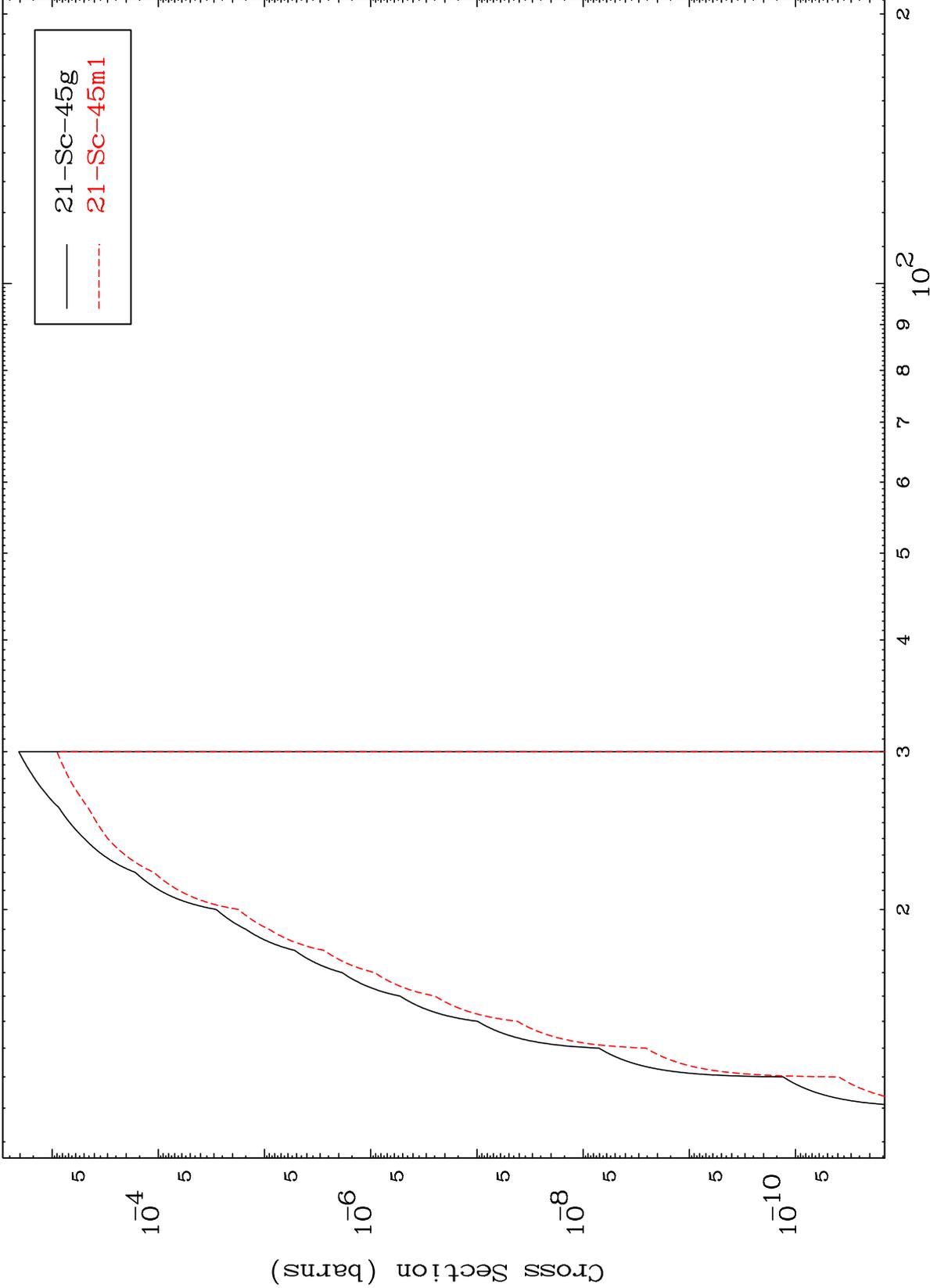
21-Sc-44

MAT 2123

( $\alpha, \text{He-3}$ )

21-Sc-44

Radionuclide Production Cross Section



14

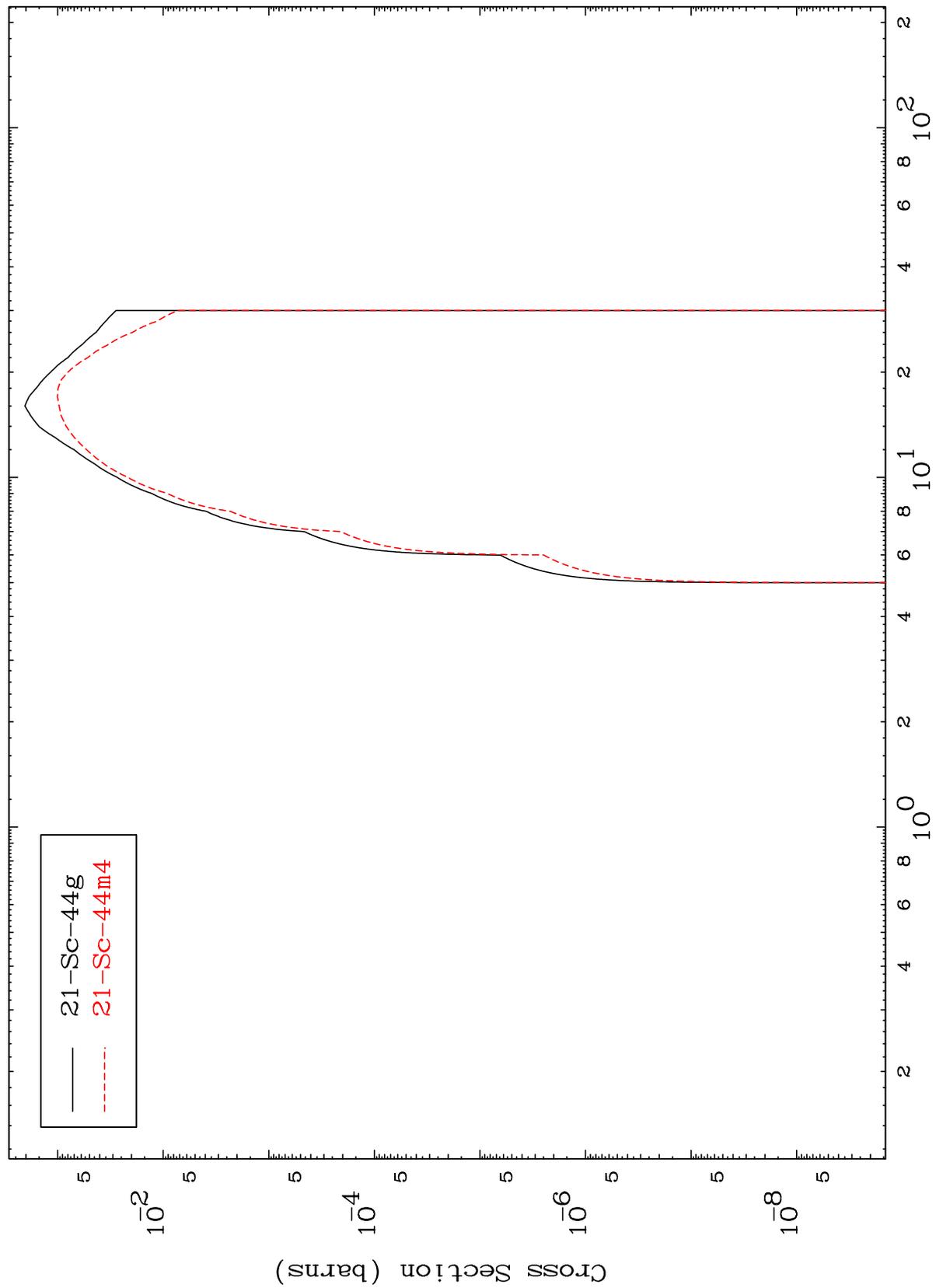
Incident Energy (MeV)

21-Sc-44

MAT 2123

21-Sc-44

( $\alpha, \alpha$ )  
Radionuclide Production Cross Section

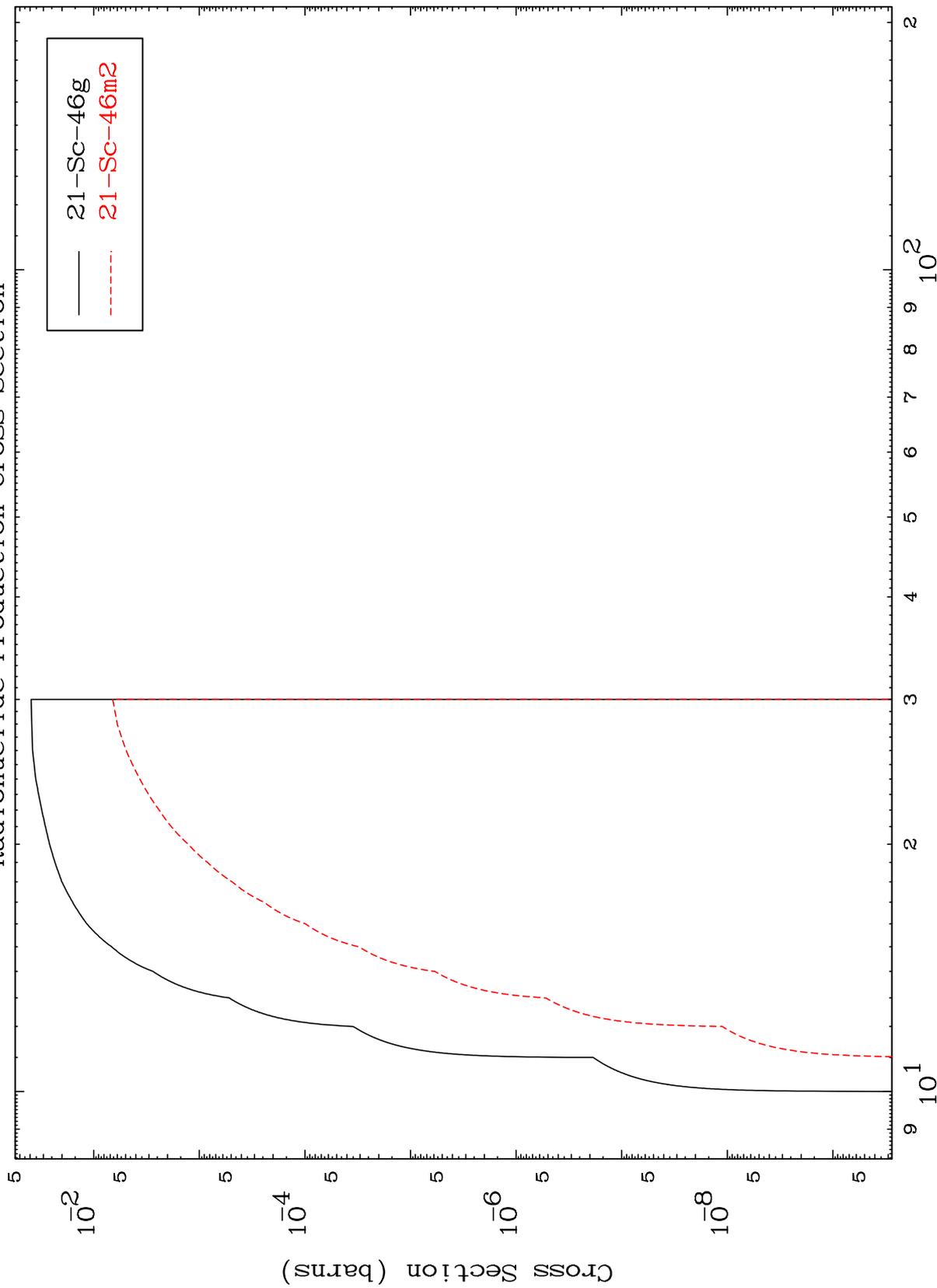


— 21-Sc-44g  
- - - 21-Sc-44m4

MAT 2123

21-Sc-44

( $\alpha, 2p$ )  
Radionuclide Production Cross Section

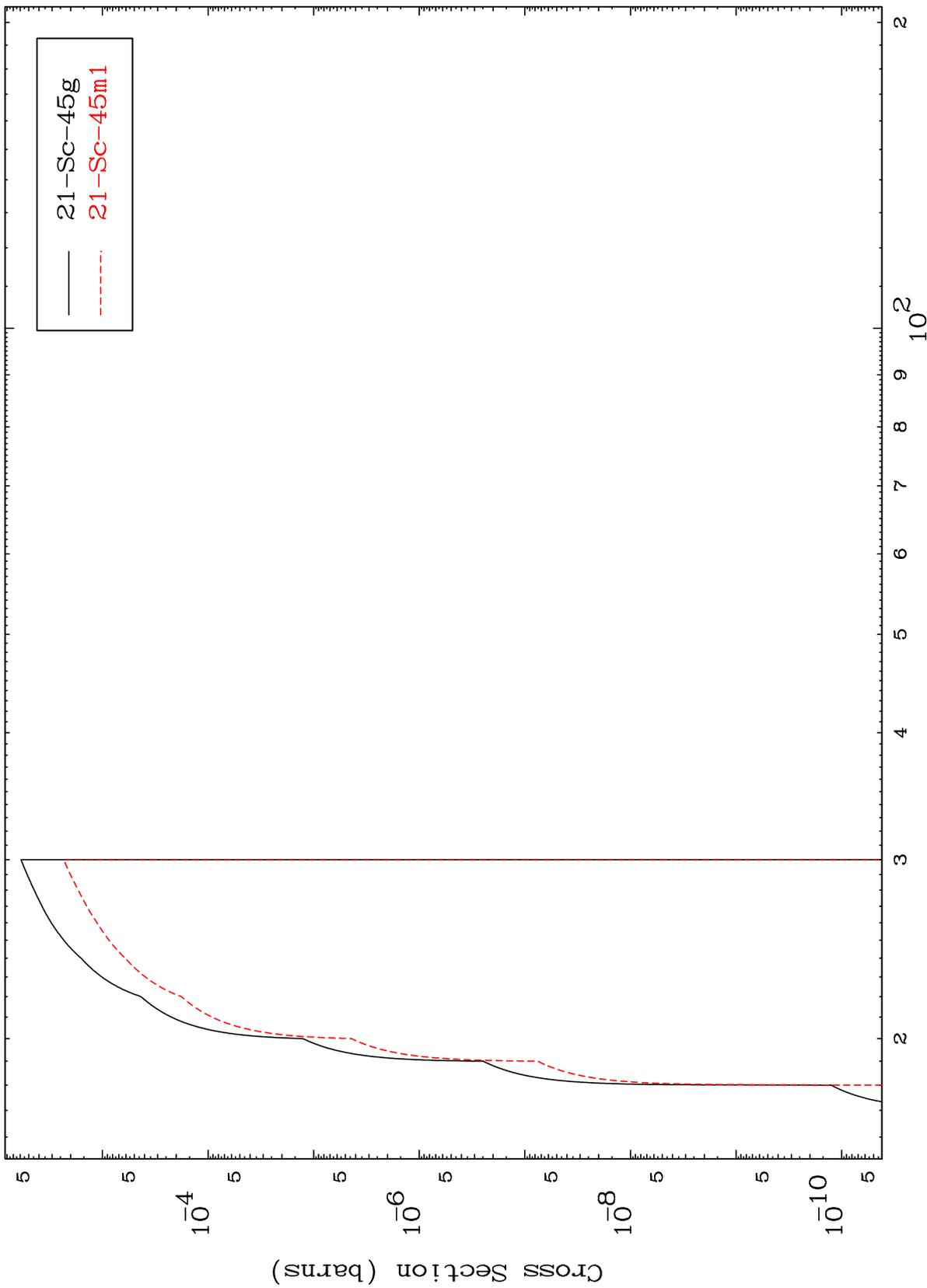


21-Sc-44

Incident Energy (MeV)

16

Radionuclide Production Cross Section



Radionuclide Production Cross Section

