

Program EVALPLOT  
(Version 2018-1)

by

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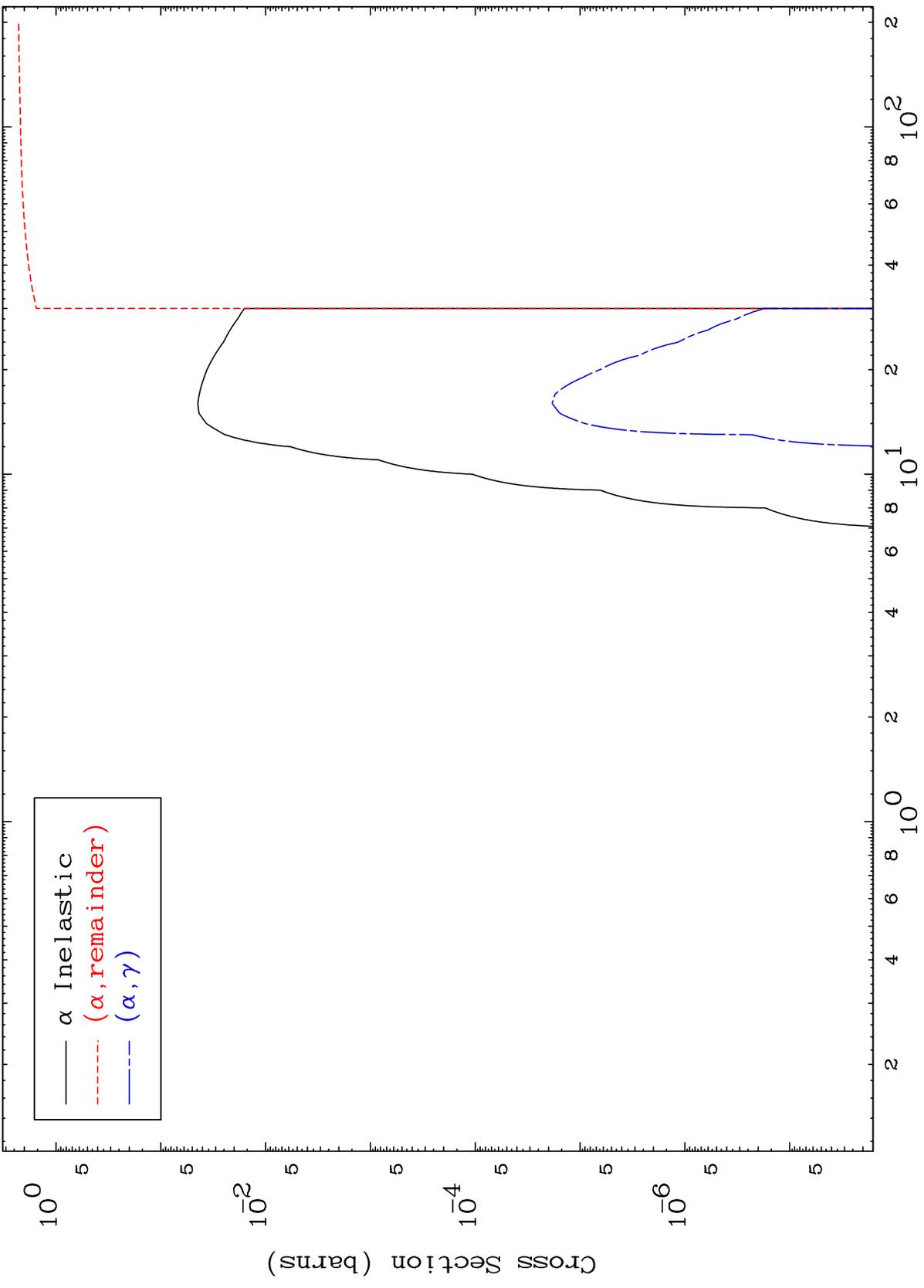
Press Mouse Button to Start

MAT 5070

$\alpha$  Major

50-Sn-127

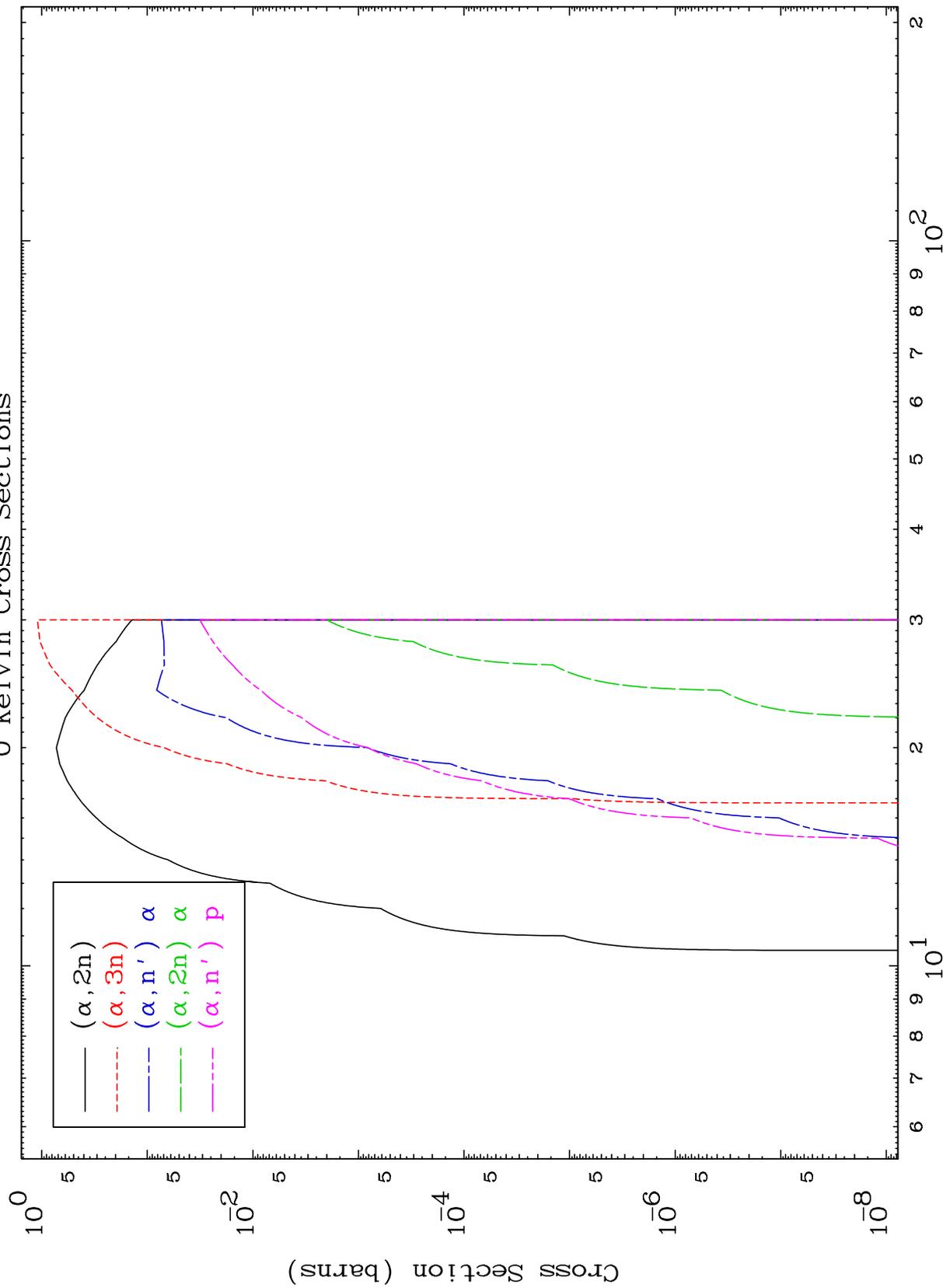
0 Kelvin Cross Sections



MAT 5070

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

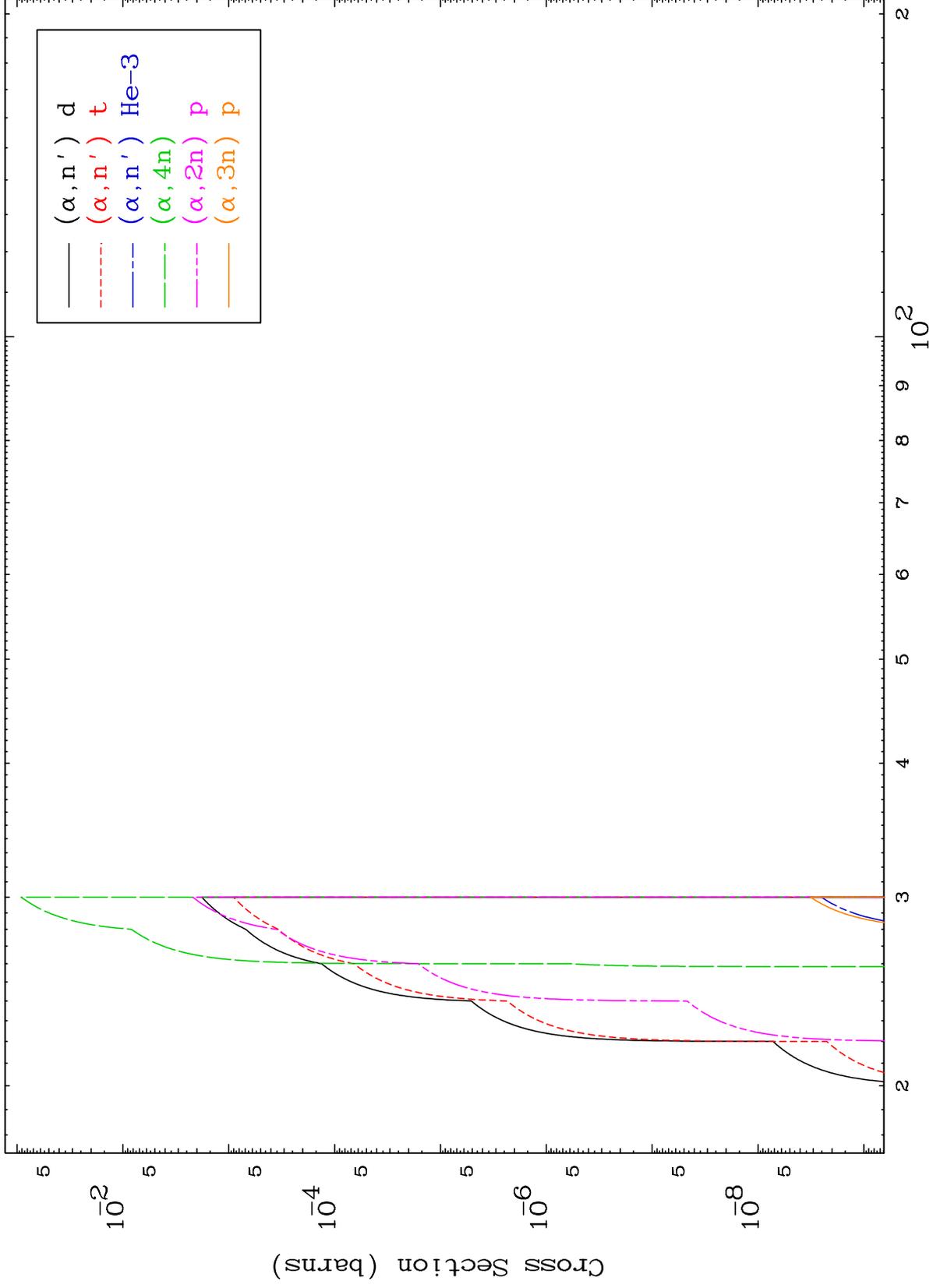
50-Sn-127



Incident Energy (MeV)

50-Sn-127

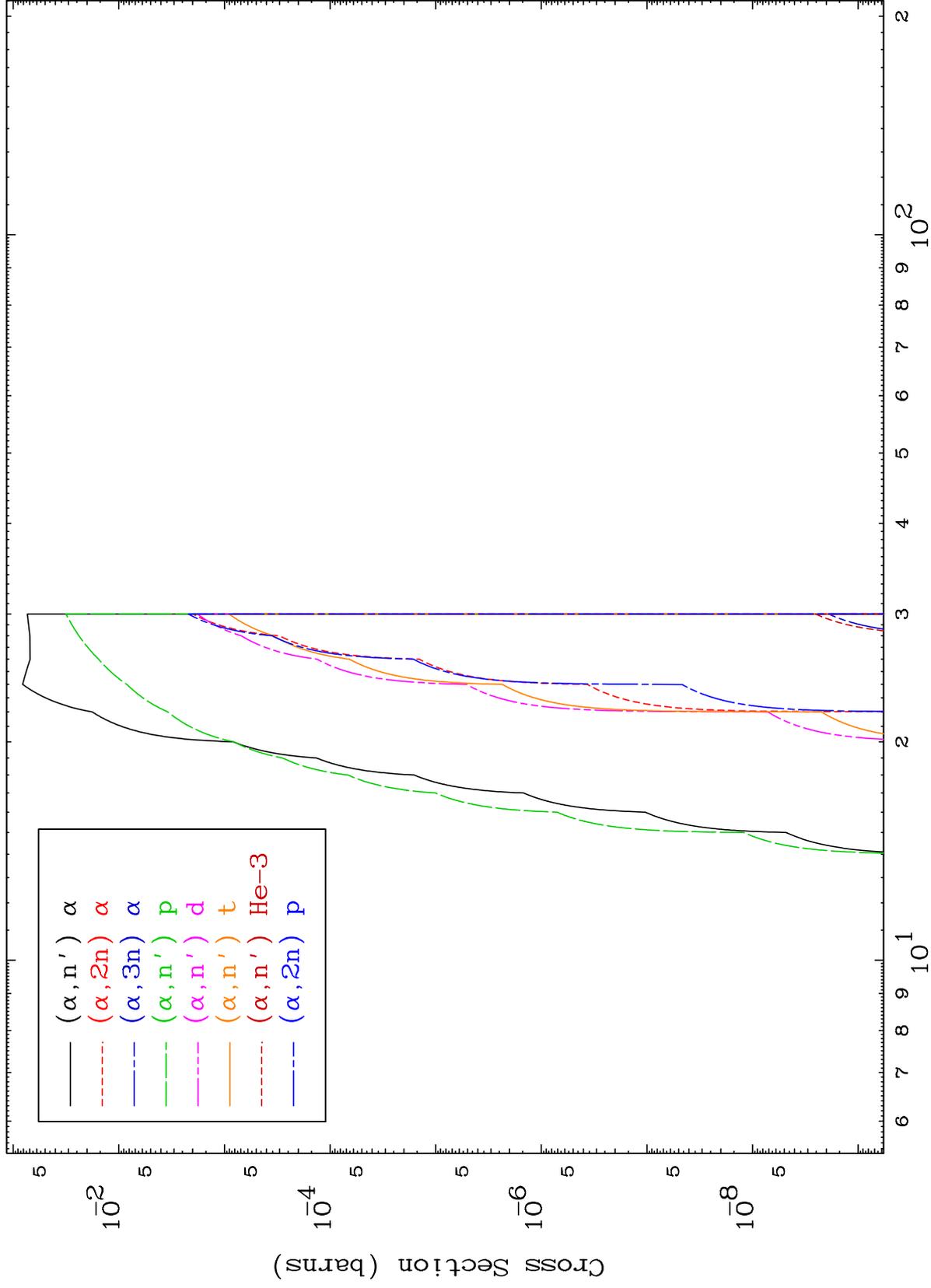
2



MAT 5070

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

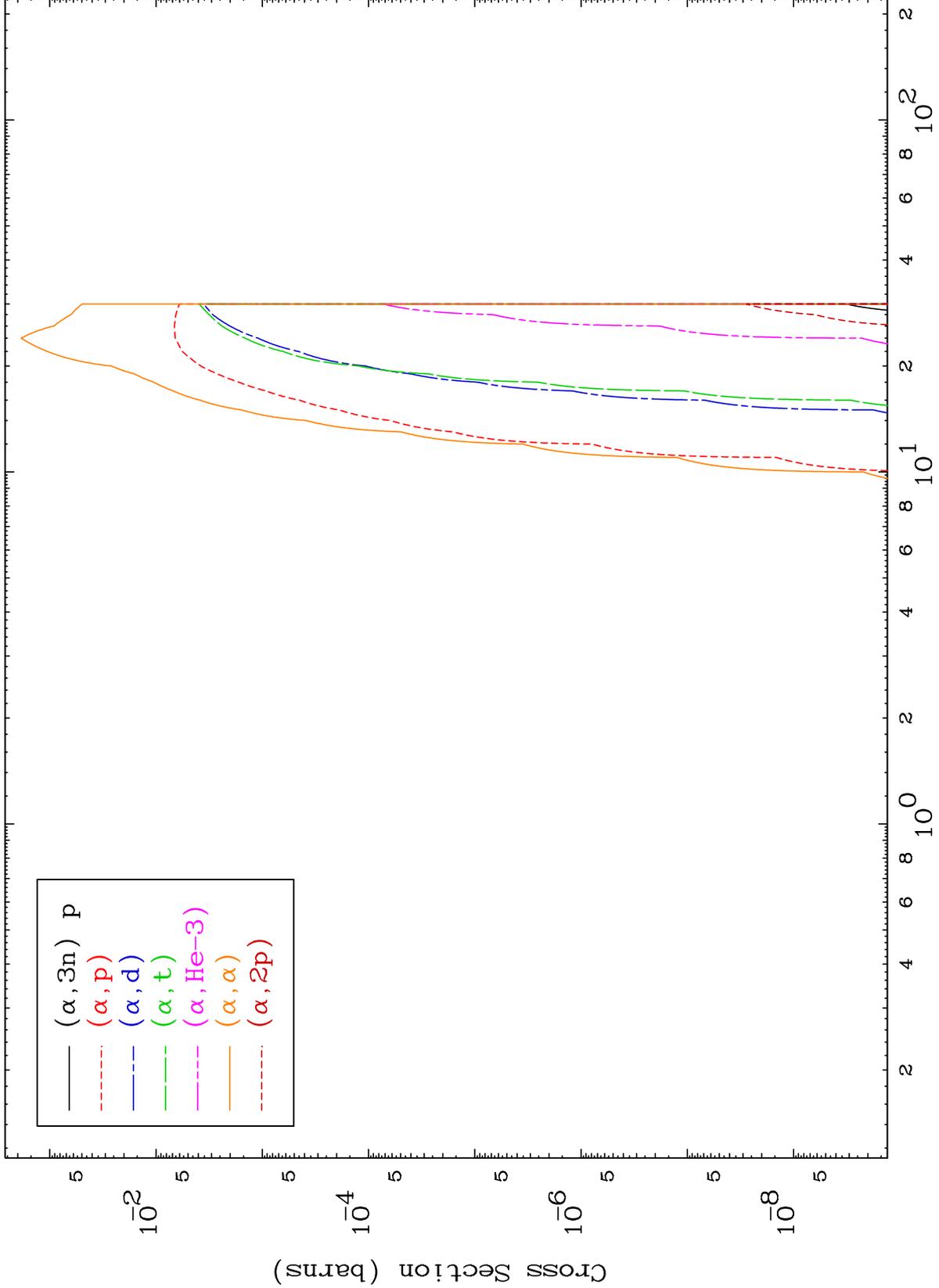
50-Sn-127



4

Incident Energy (MeV)

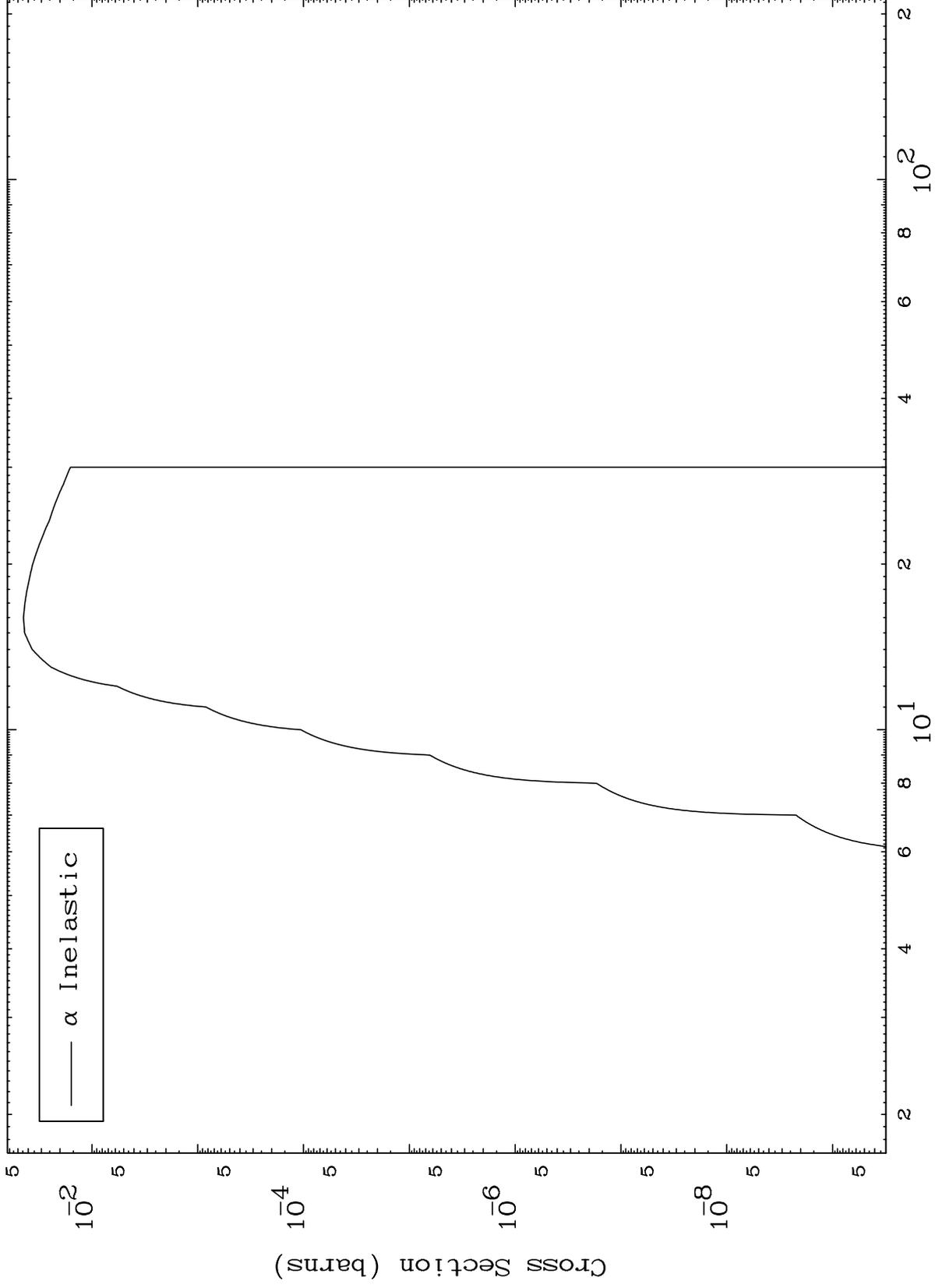
50-Sn-127



MAT 5070

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

50-Sn-127



6

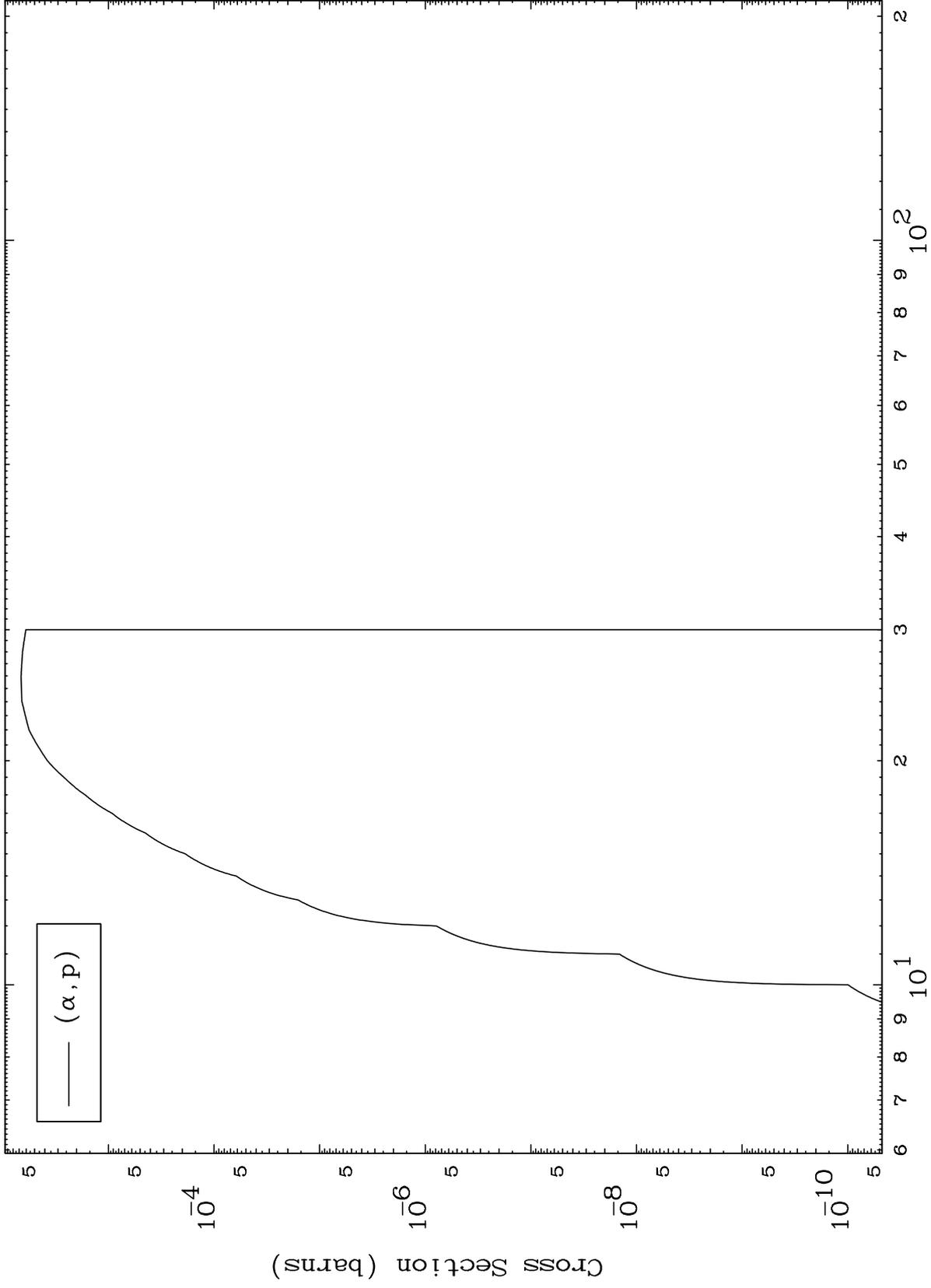
Incident Energy (MeV)

50-Sn-127

MAT 5070

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

50-Sn-127



7

Incident Energy (MeV)

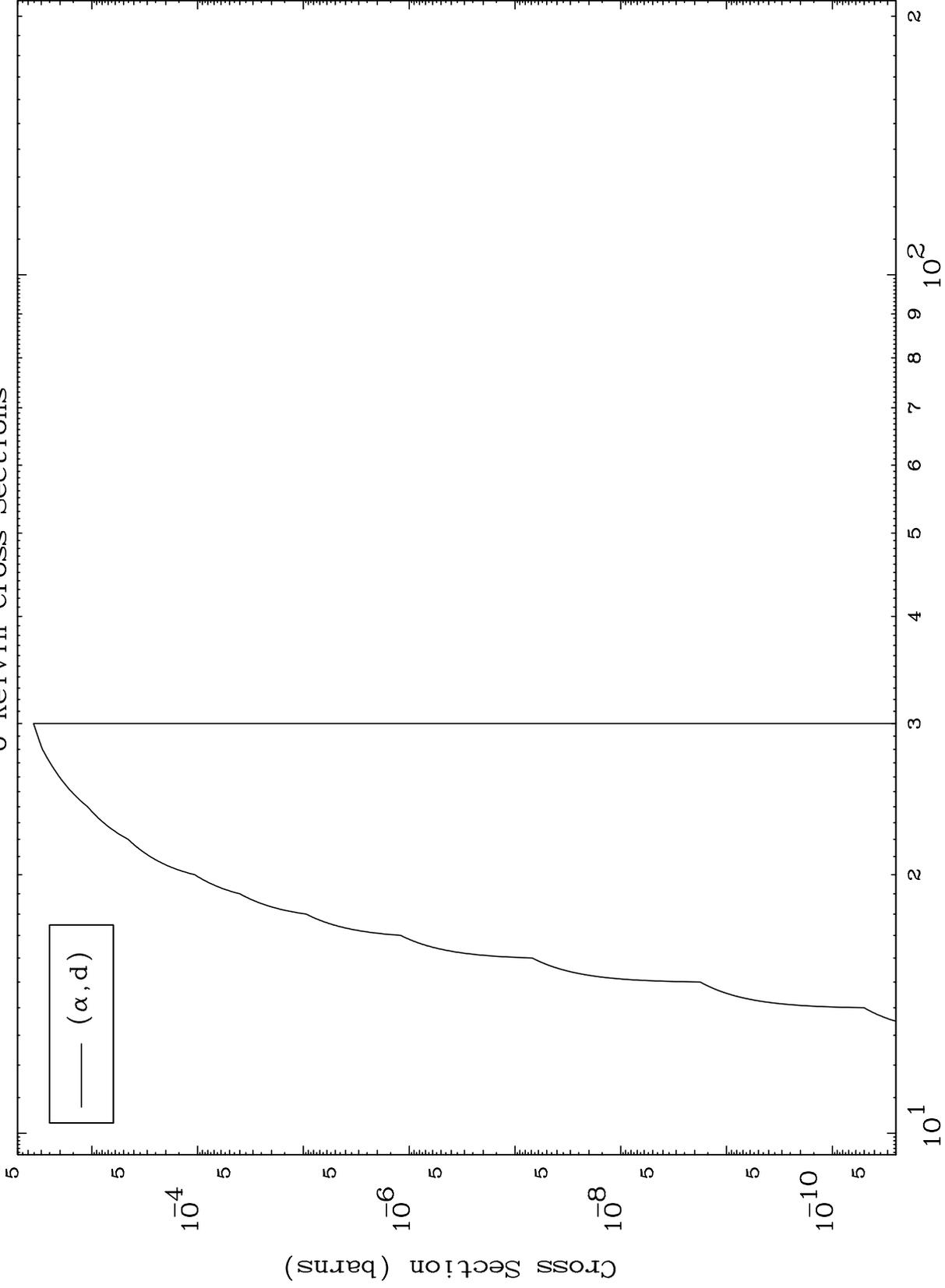
50-Sn-127

MAT 5070

( $\alpha, d$ ) Levels

50-Sn-127

0 Kelvin Cross Sections



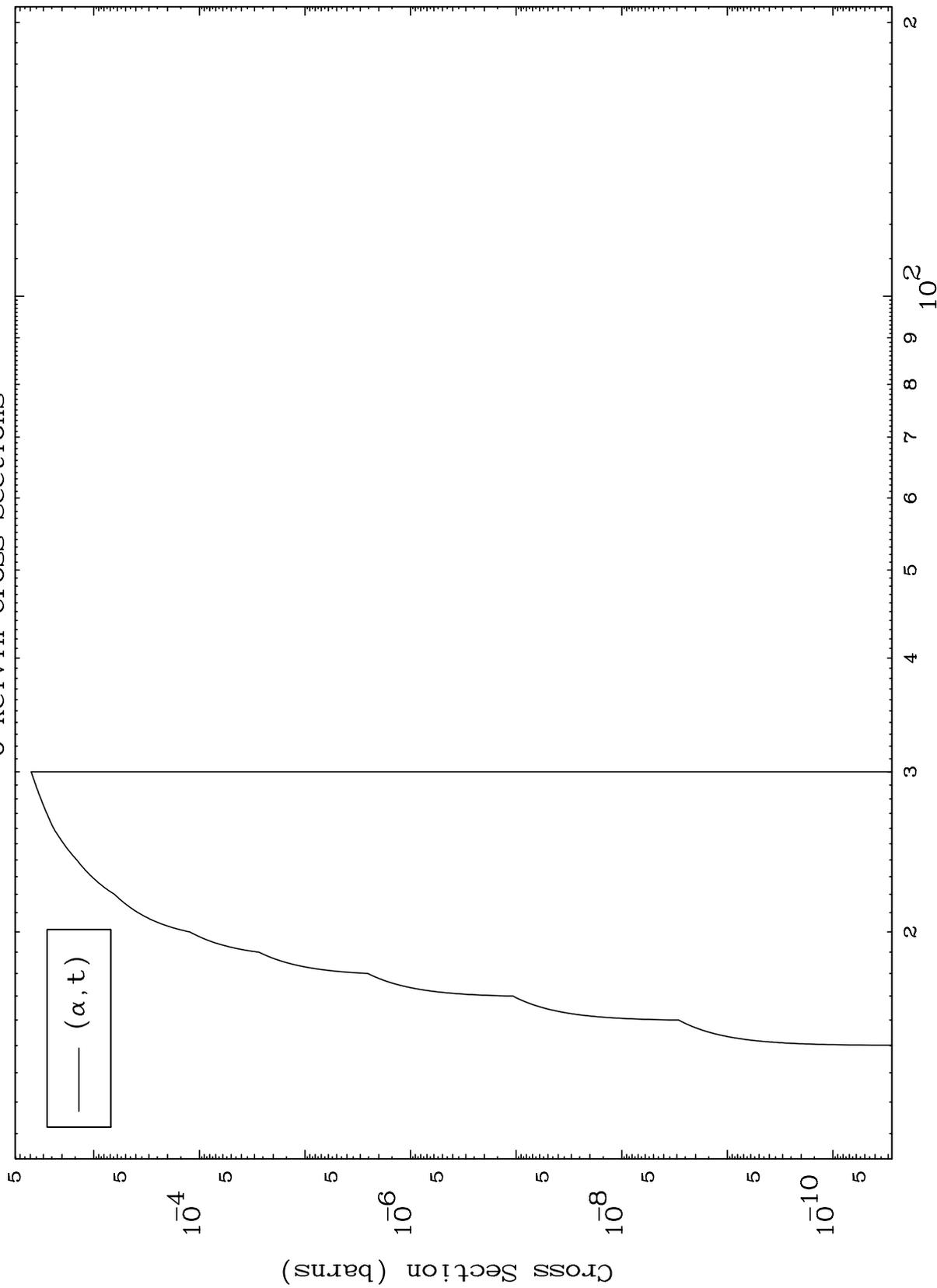
Incident Energy (MeV)

50-Sn-127

MAT 5070

50-Sn-127

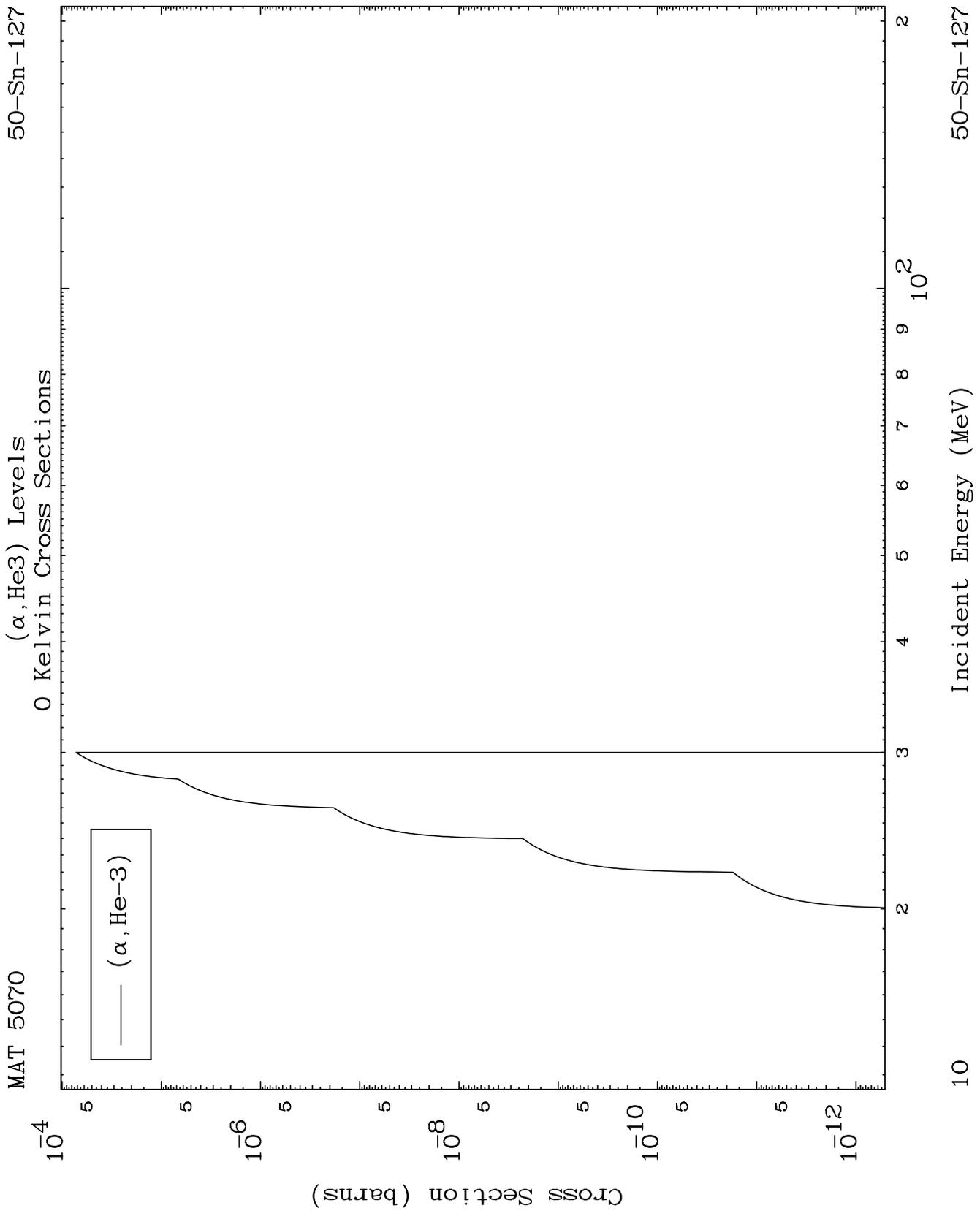
( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections



50-Sn-127

Incident Energy (MeV)

9

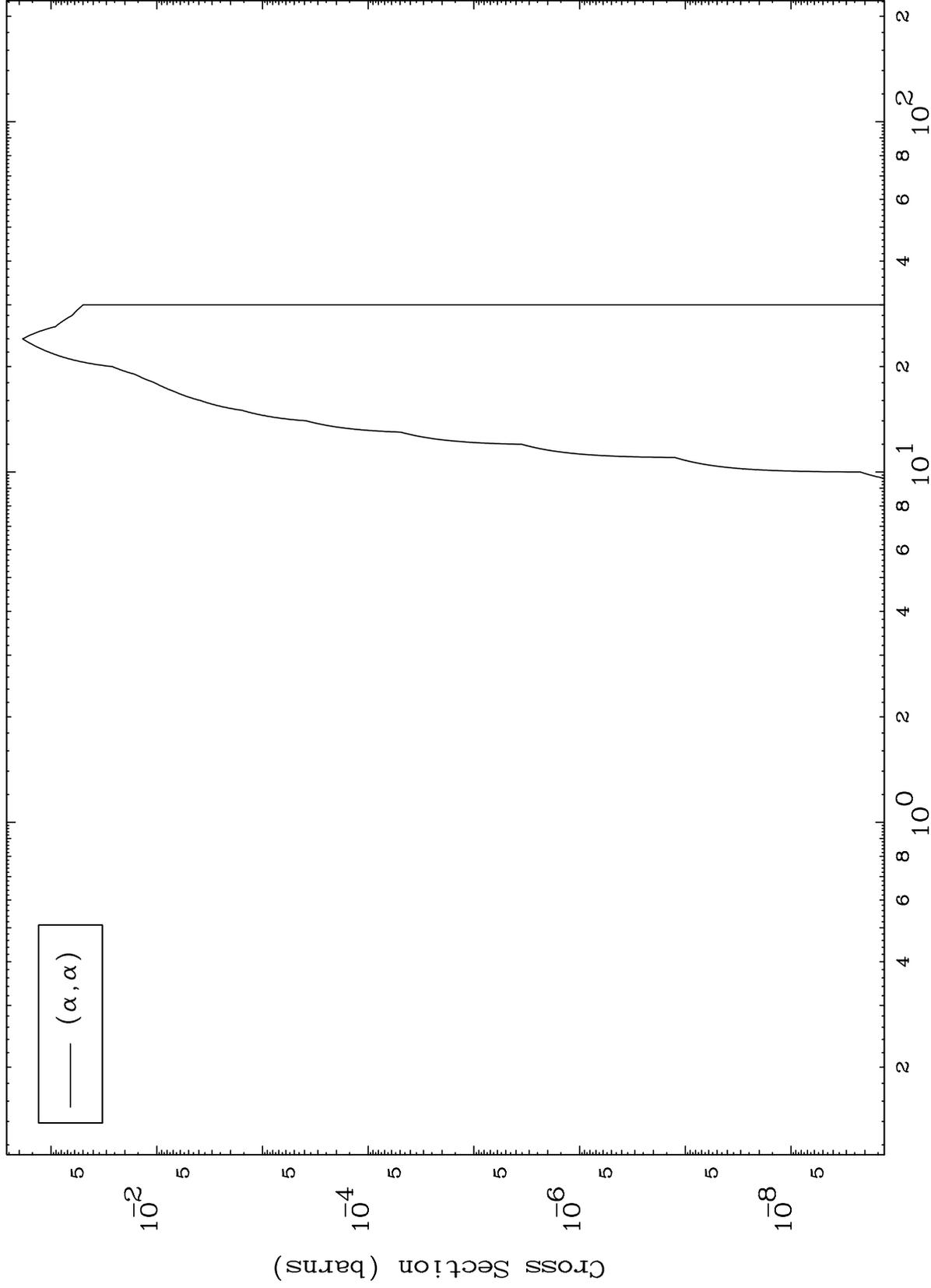


MAT 5070

( $\alpha, \alpha$ ) Levels

50-Sn-127

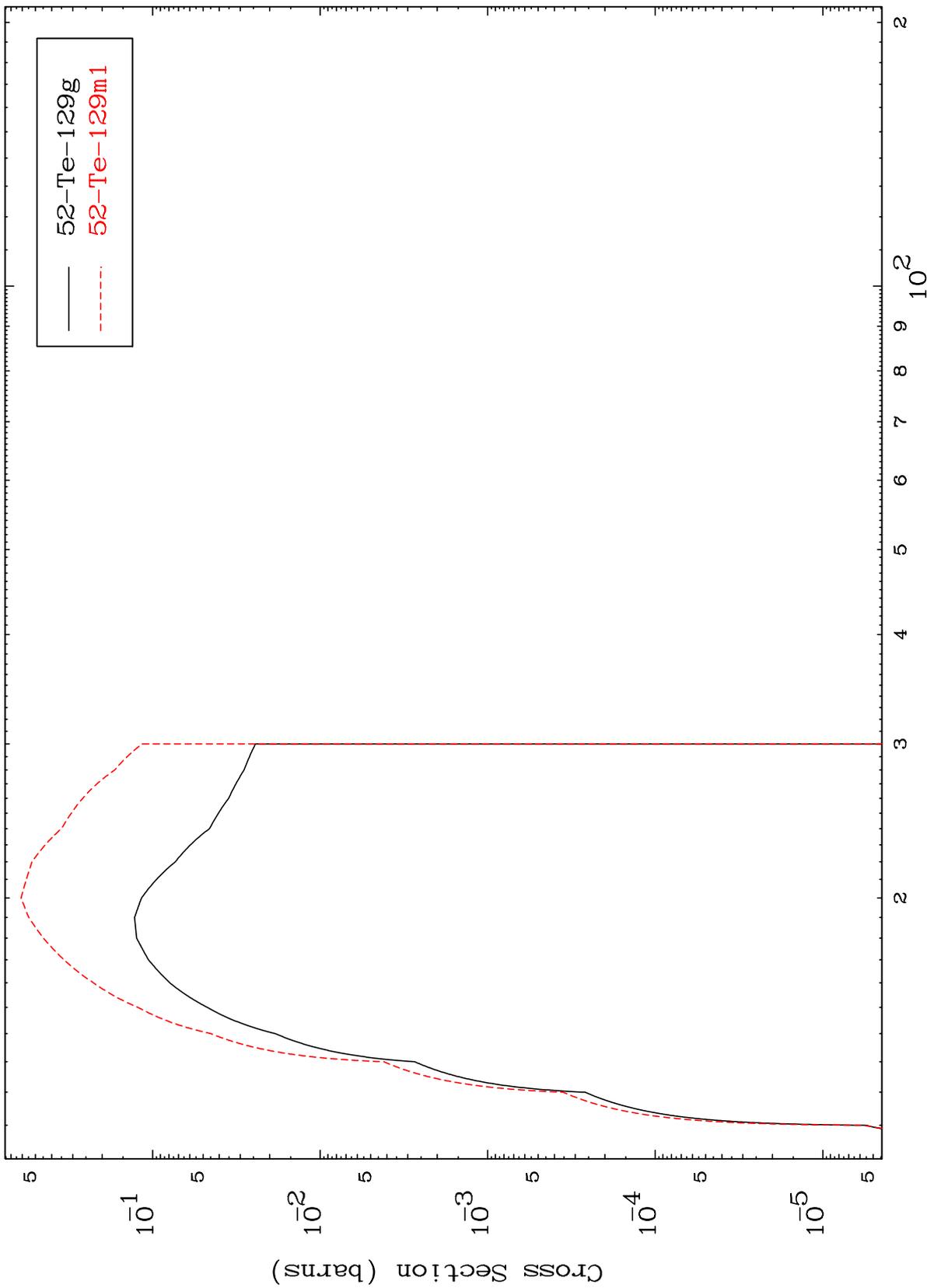
0 Kelvin Cross Sections



MAT 5070

50-Sn-127

$(\alpha, 2n)$   
Radionuclide Production Cross Section



12

Incident Energy (MeV)

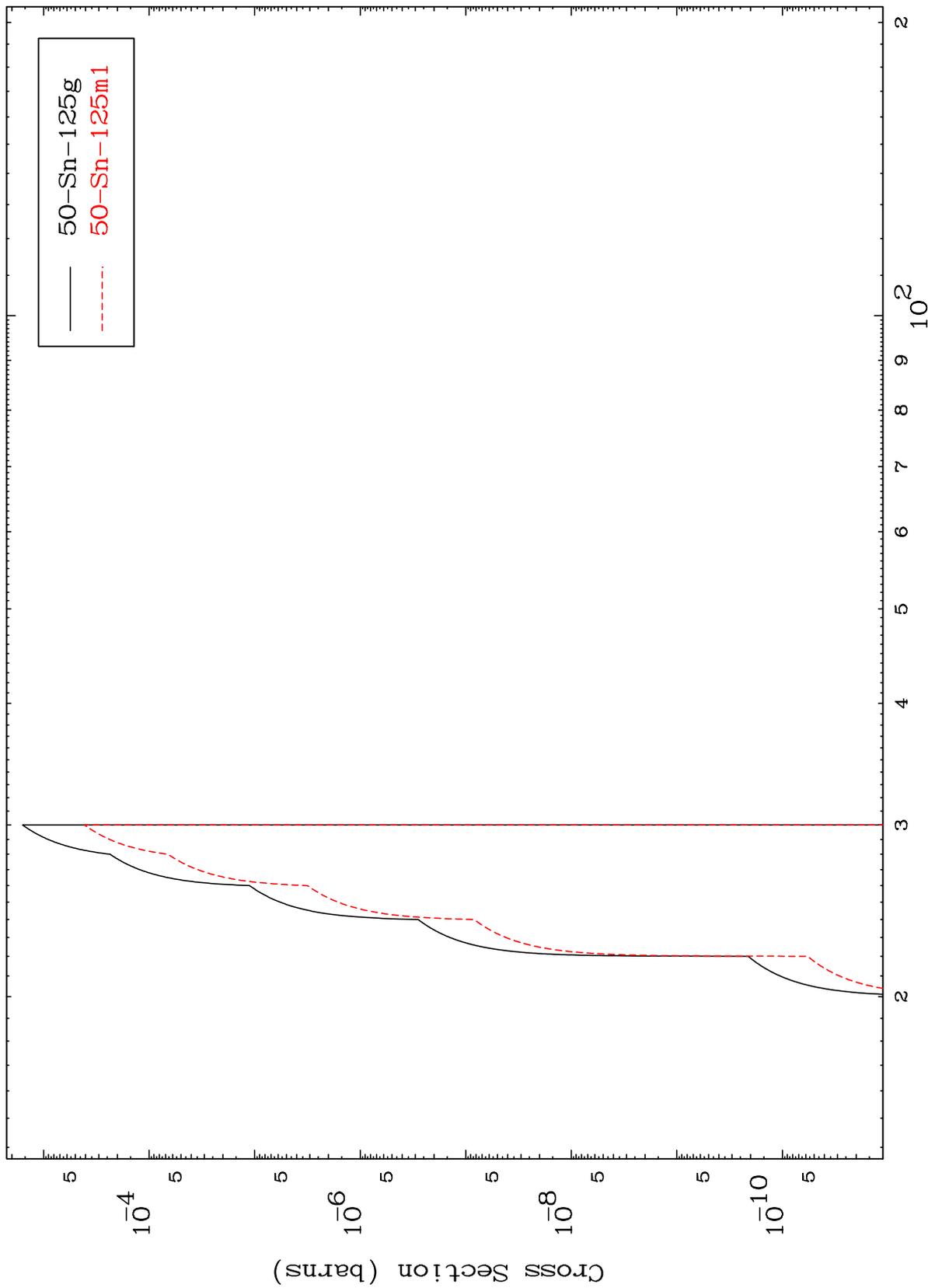
50-Sn-127

MAT 5070

50-Sn-127

$(\alpha, 2n) \alpha$

Radionuclide Production Cross Section



13

Incident Energy (MeV)

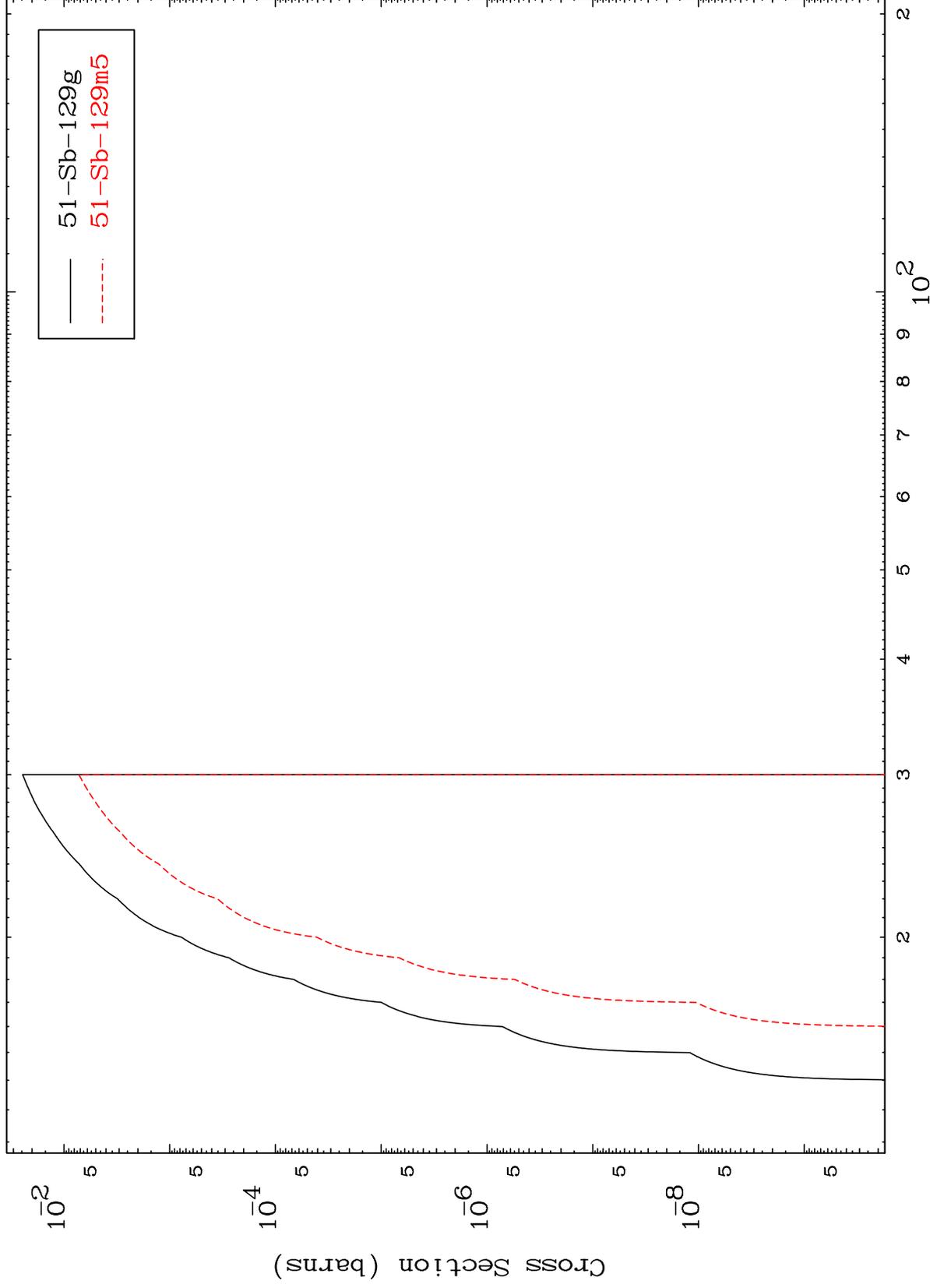
50-Sn-127

MAT 5070

$(\alpha, n')$  p

50-Sn-127

Radionuclide Production Cross Section



14

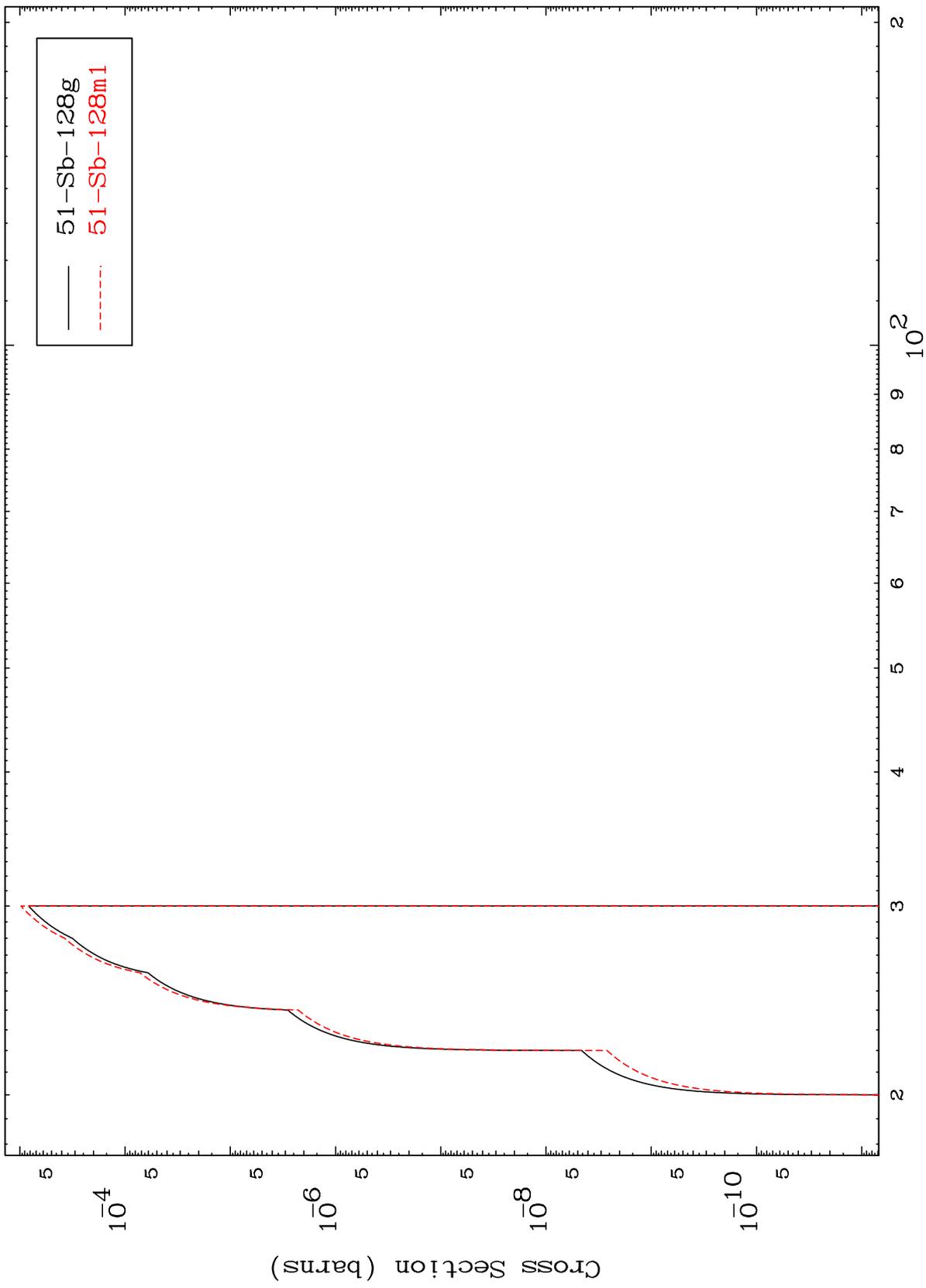
Incident Energy (MeV)

50-Sn-127

MAT 5070

50-Sn-127

$(\alpha, n')$  d  
Radionuclide Production Cross Section



15

Incident Energy (MeV)

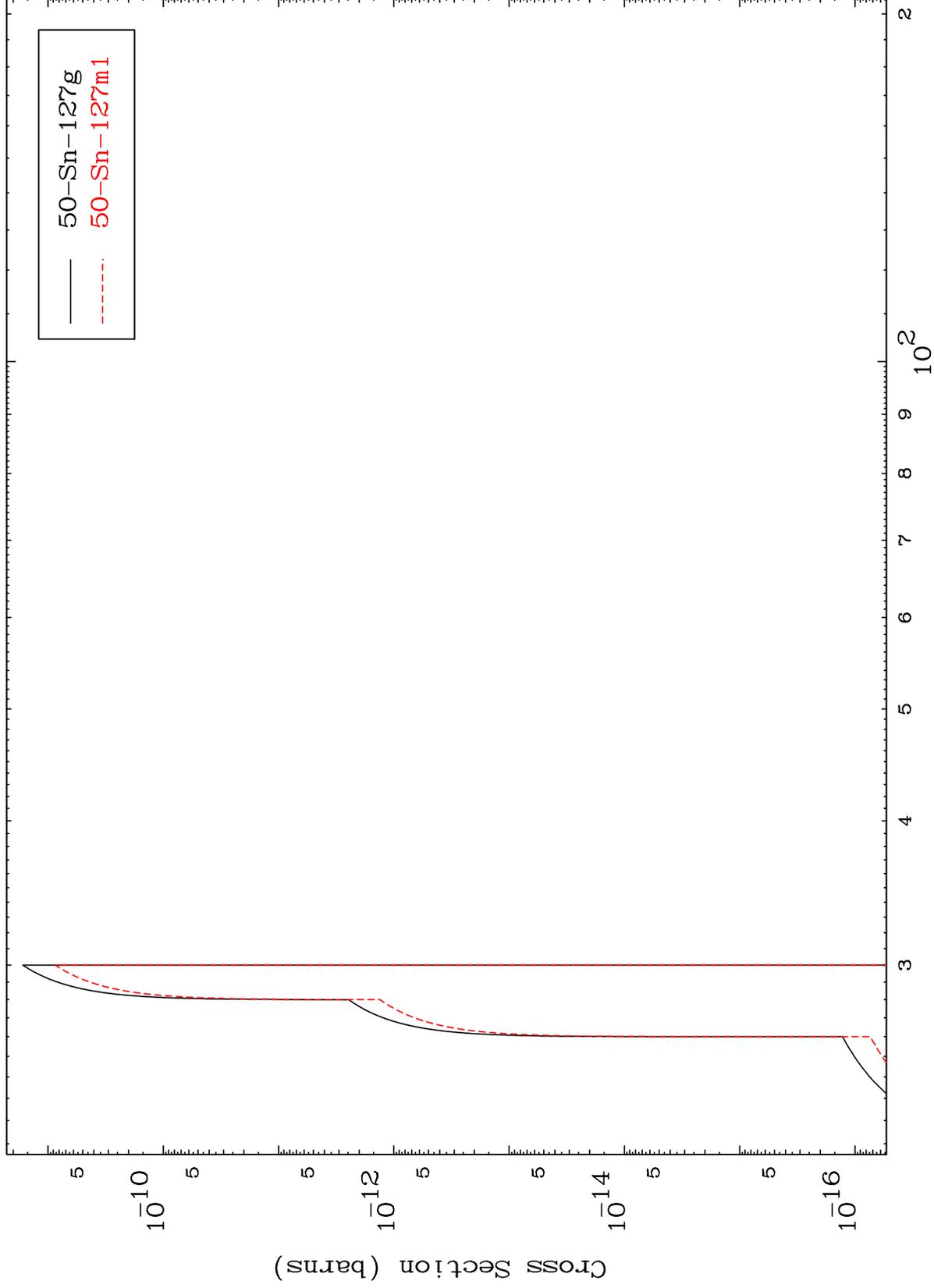
50-Sn-127

MAT 5070

( $\alpha, n'$ ) He-3

50-Sn-127

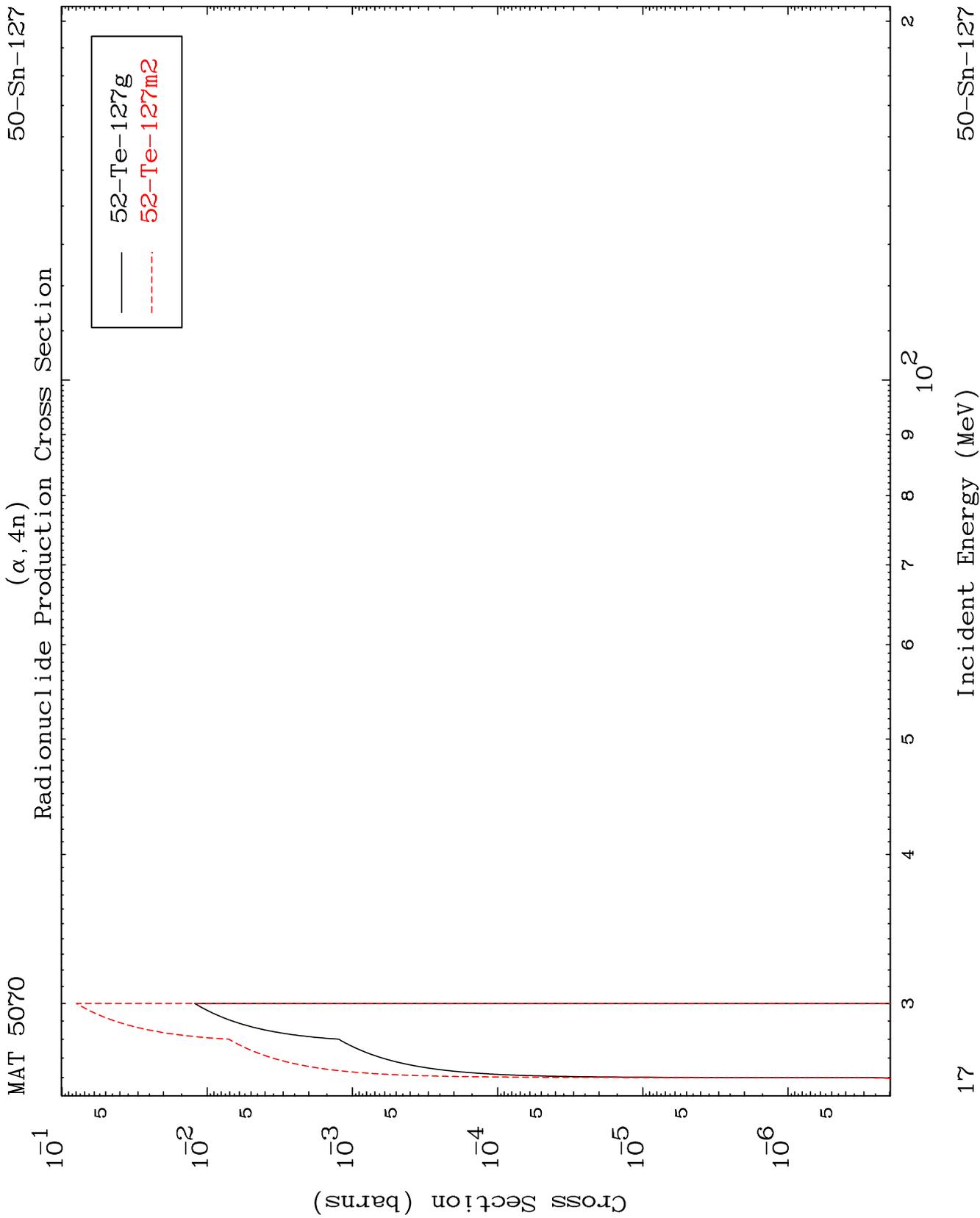
Radionuclide Production Cross Section



16

Incident Energy (MeV)

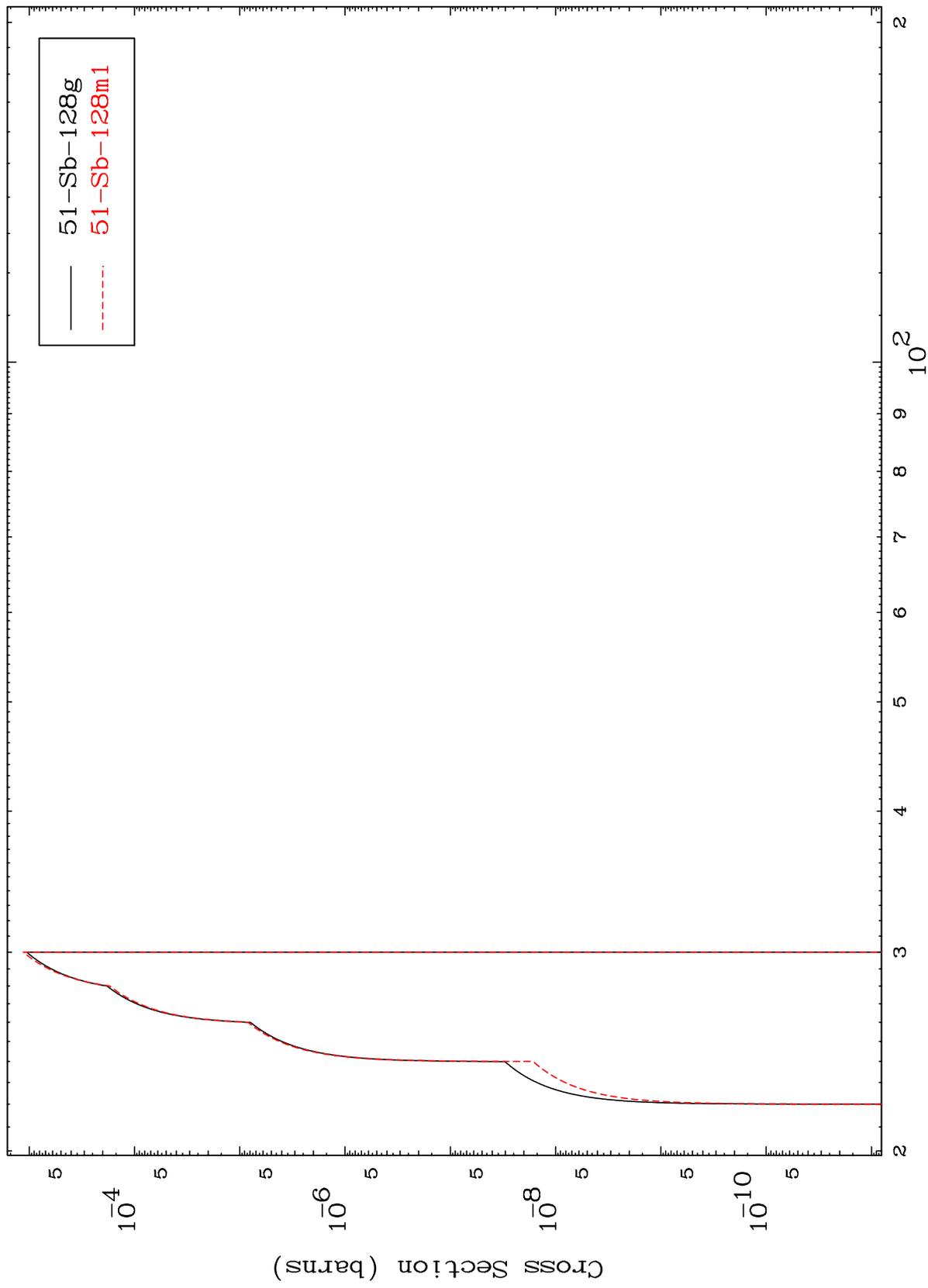
50-Sn-127



MAT 5070

50-Sn-127

$(\alpha, 2n)$  p  
Radionuclide Production Cross Section



18

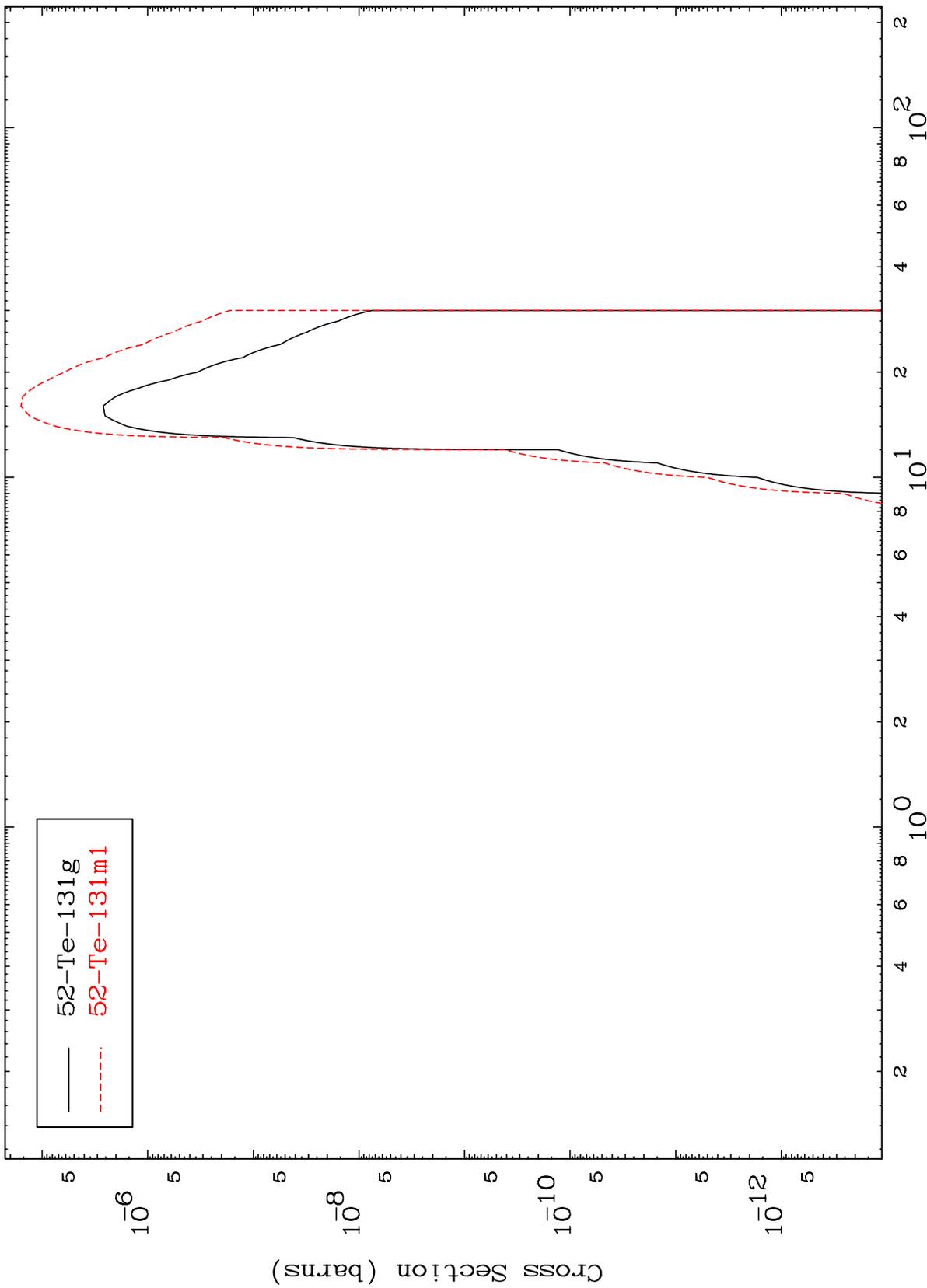
Incident Energy (MeV)

50-Sn-127

MAT 5070

50-Sn-127

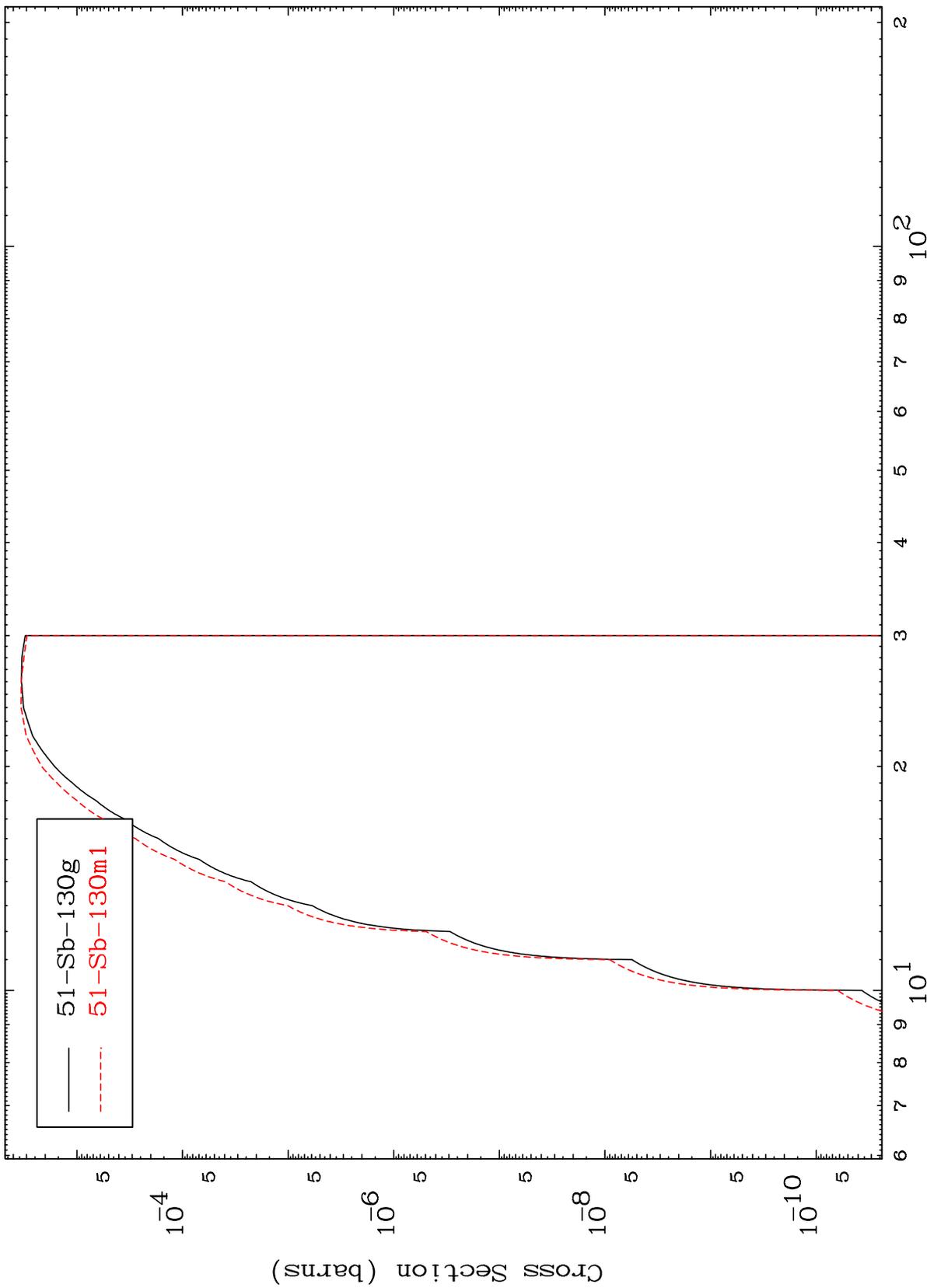
Radionuclide Production Cross Section  
( $\alpha, \gamma$ )



MAT 5070

50-Sn-127

( $\alpha, p$ )  
Radionuclide Production Cross Section



Incident Energy (MeV)

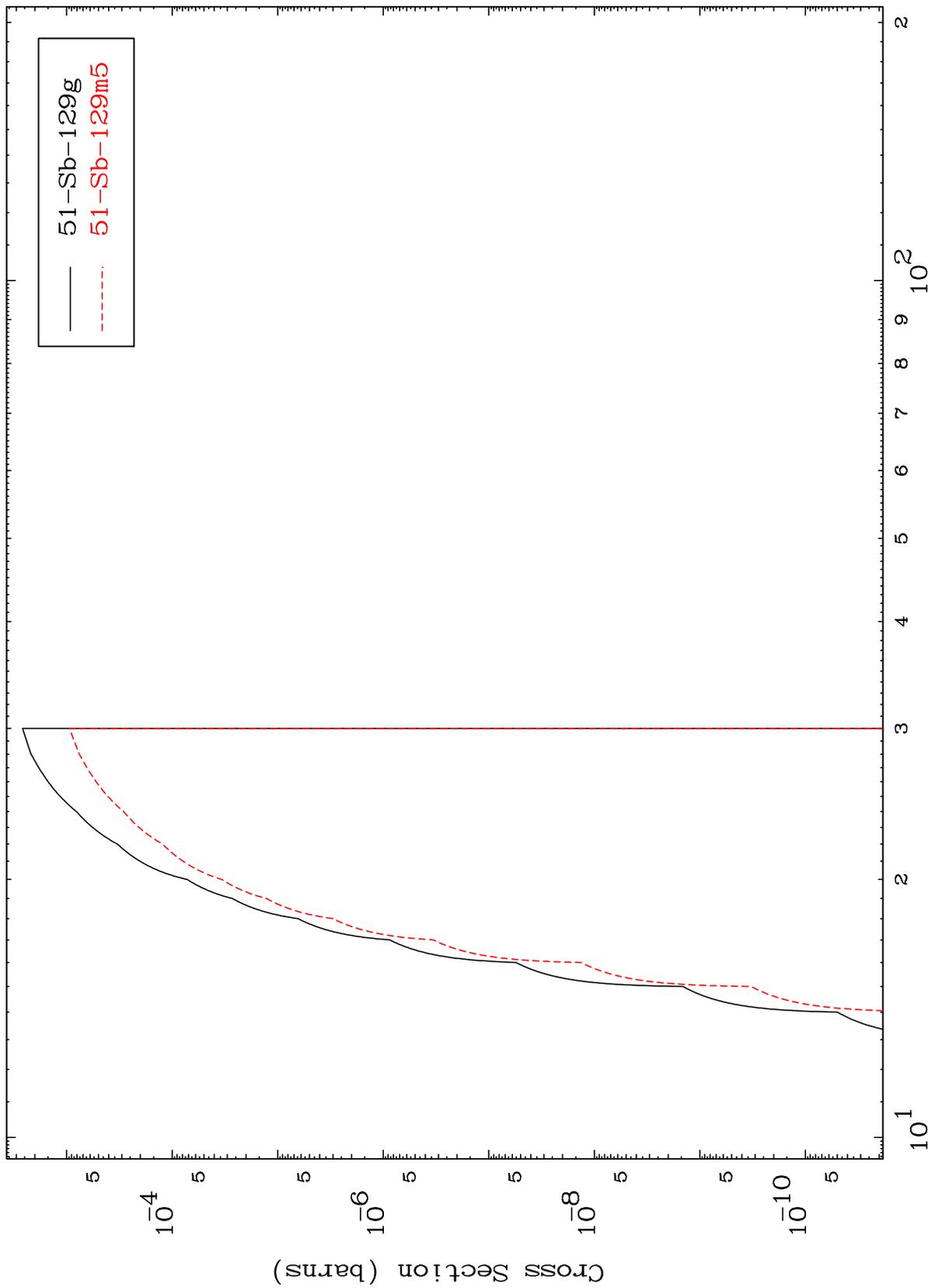
50-Sn-127

20

MAT 5070

50-Sn-127

( $\alpha, d$ )  
Radionuclide Production Cross Section

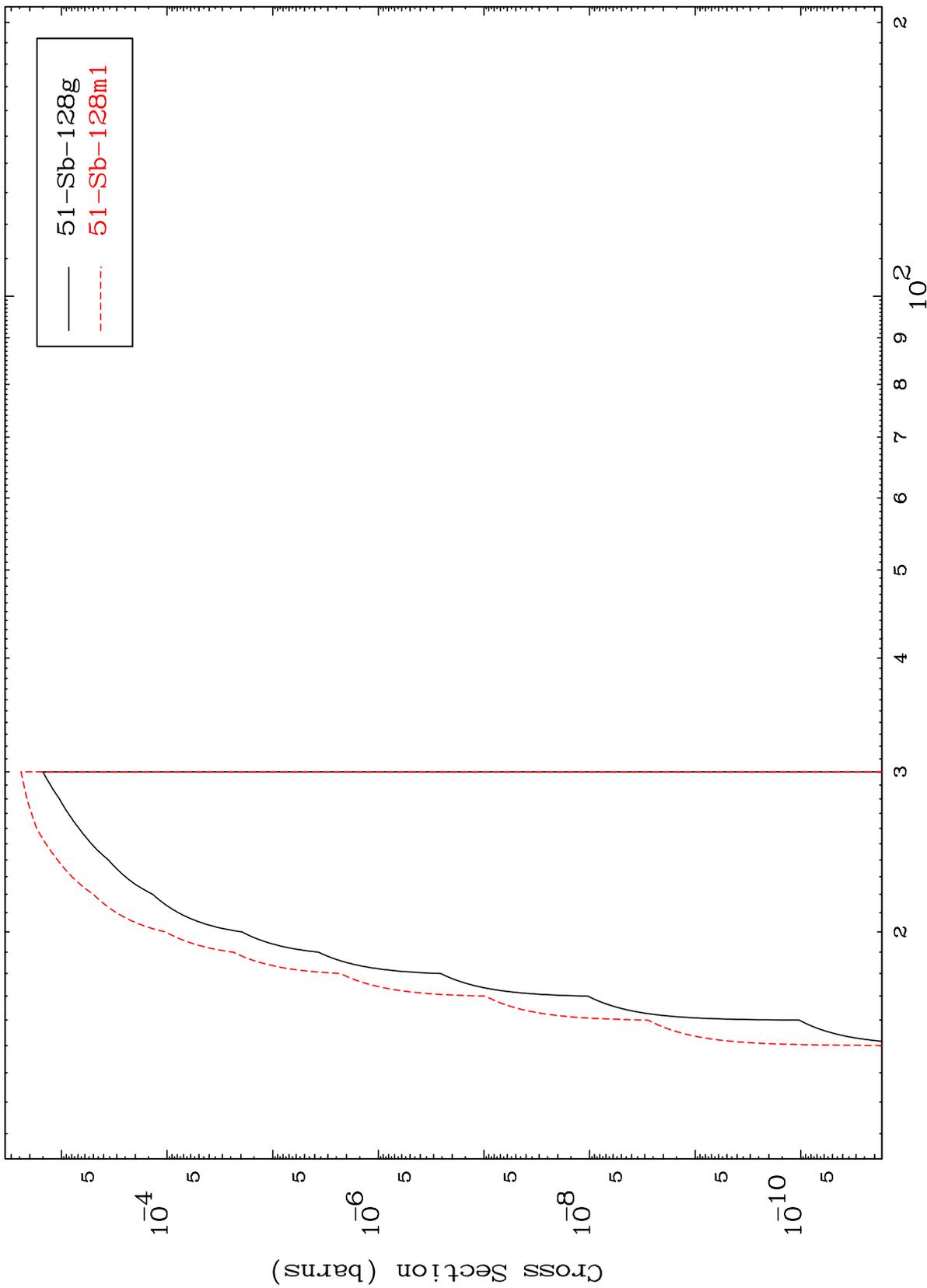


50-Sn-127

Incident Energy (MeV)

21

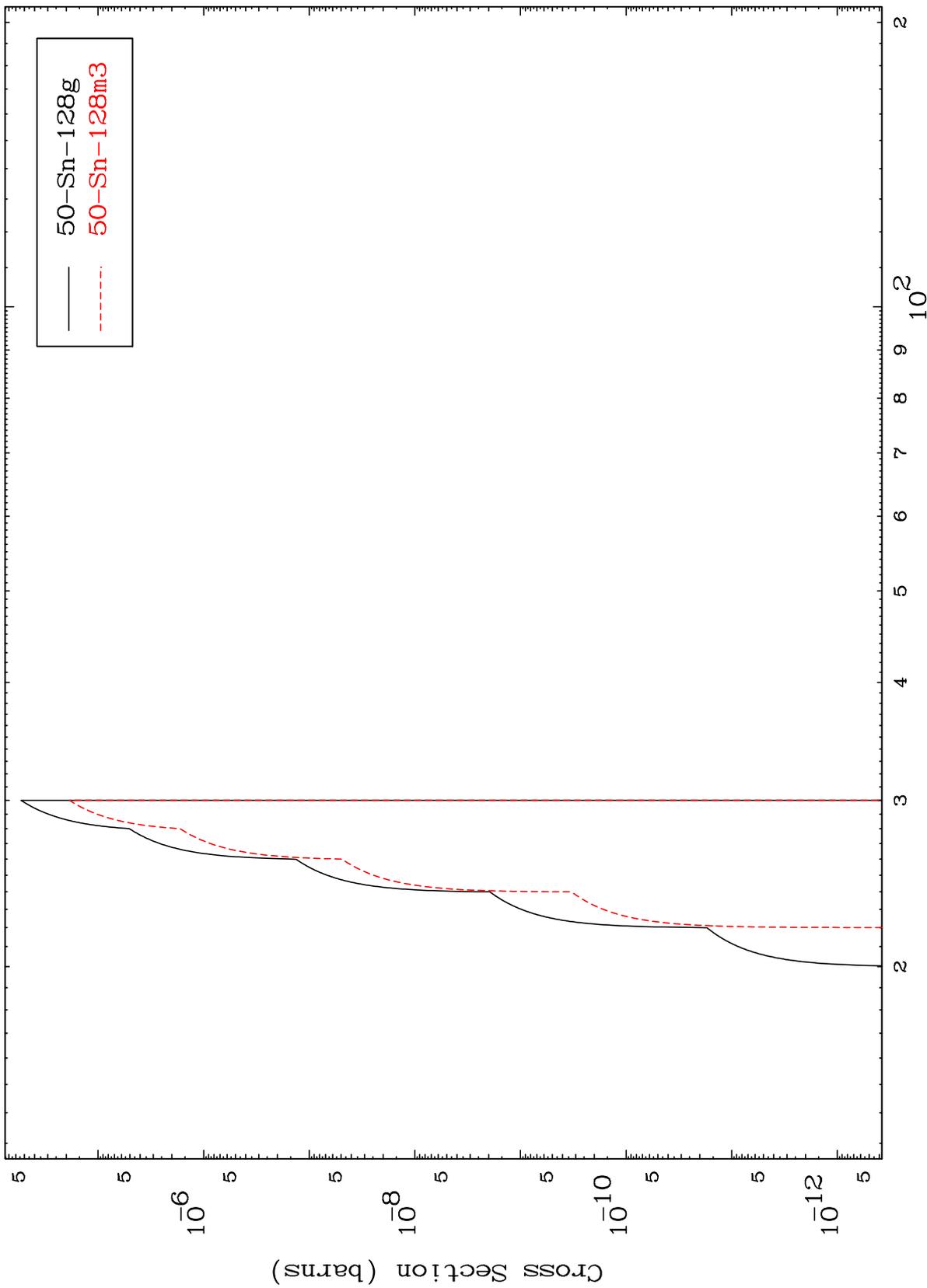
( $\alpha, t$ )  
Radionuclide Production Cross Section



MAT 5070

50-Sn-127

( $\alpha, \text{He-3}$ )  
Radionuclide Production Cross Section



50-Sn-127

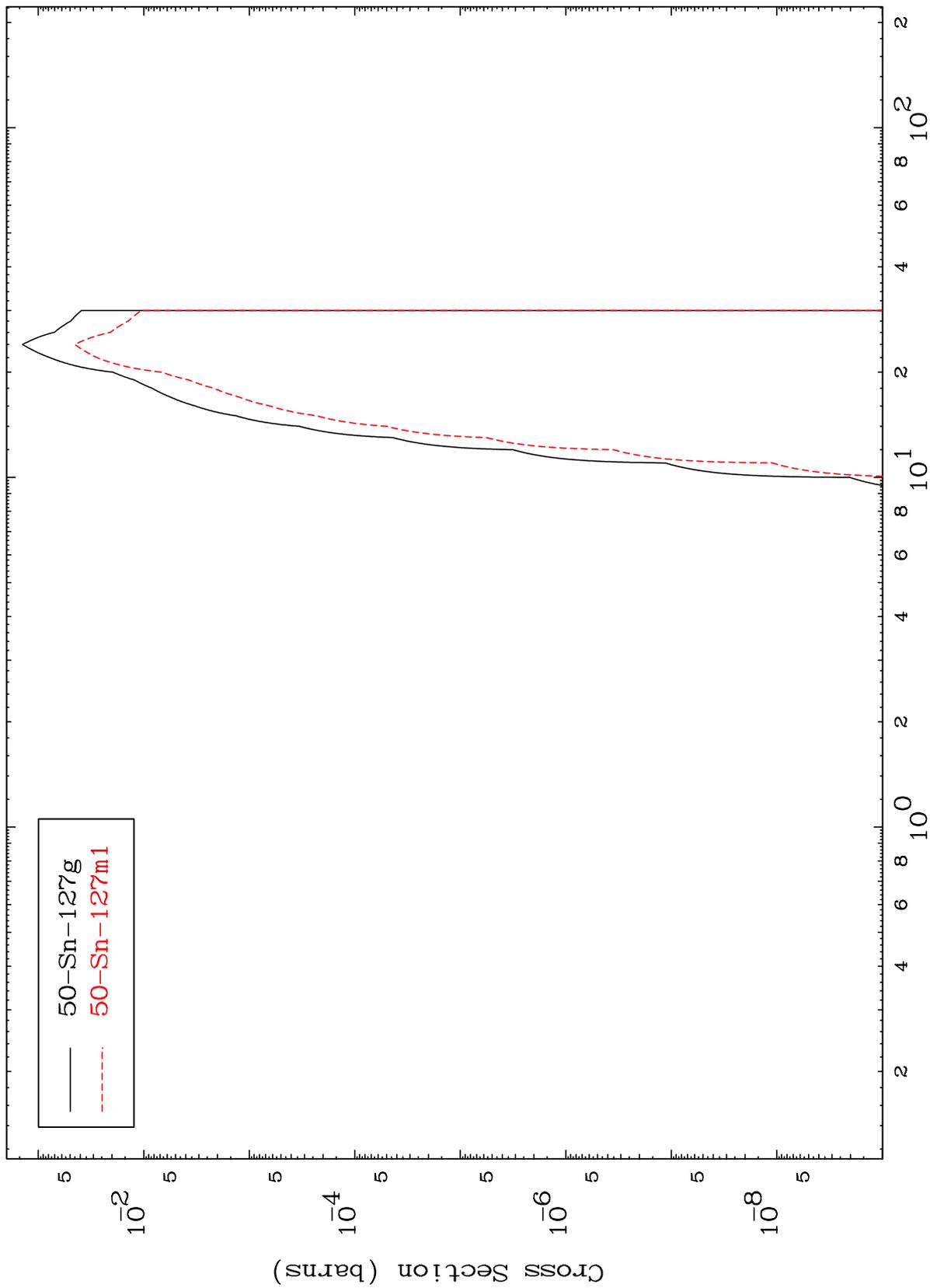
Incident Energy (MeV)

23

MAT 5070

50-Sn-127

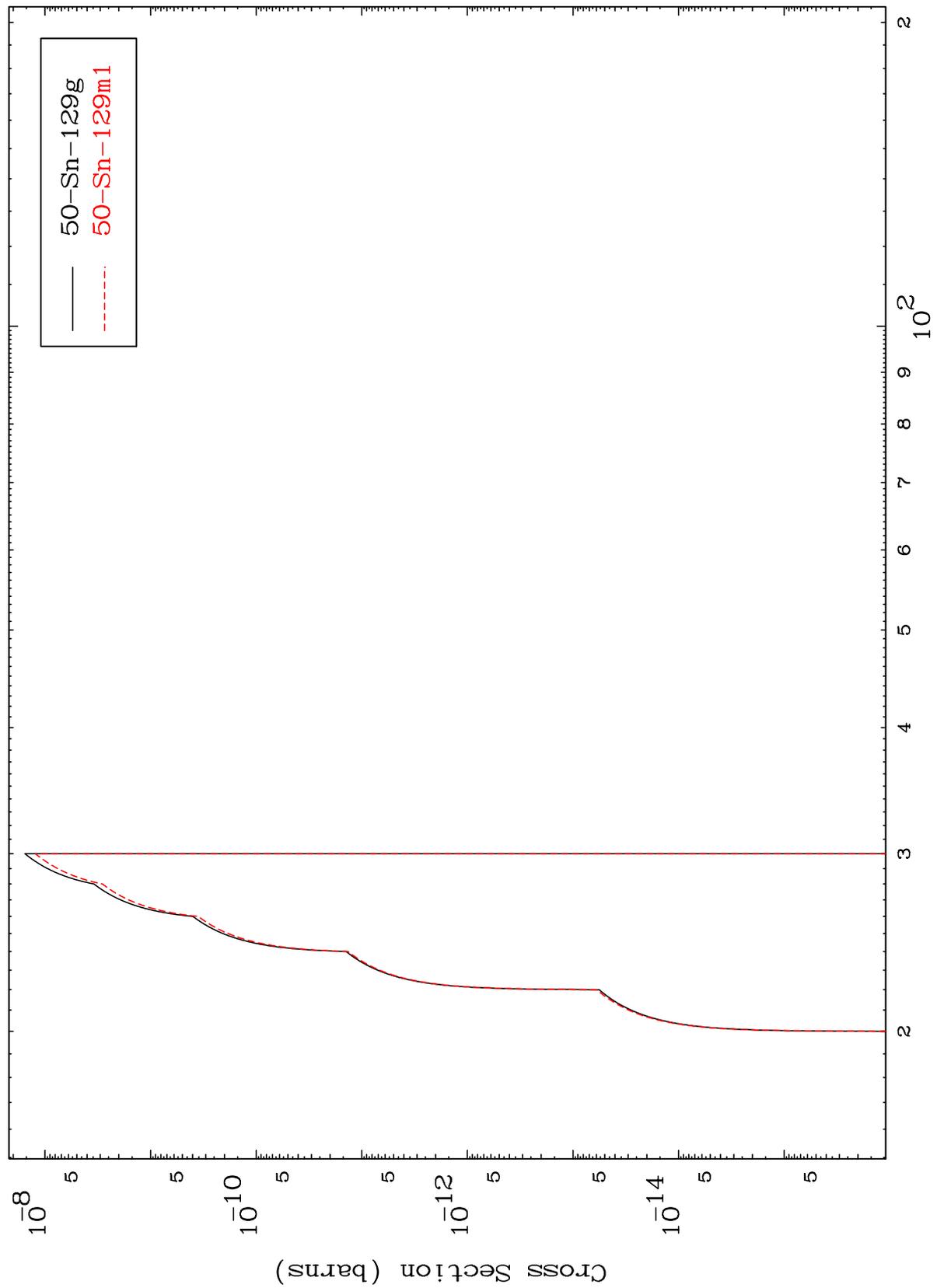
Radionuclide Production Cross Section  
( $\alpha, \alpha$ )



MAT 5070

50-Sn-127

Radionuclide Production Cross Section  
( $\alpha, 2p$ )



25

50-Sn-127

Incident Energy (MeV)