

Program EVALPLOT  
(Version 2018-1)

by

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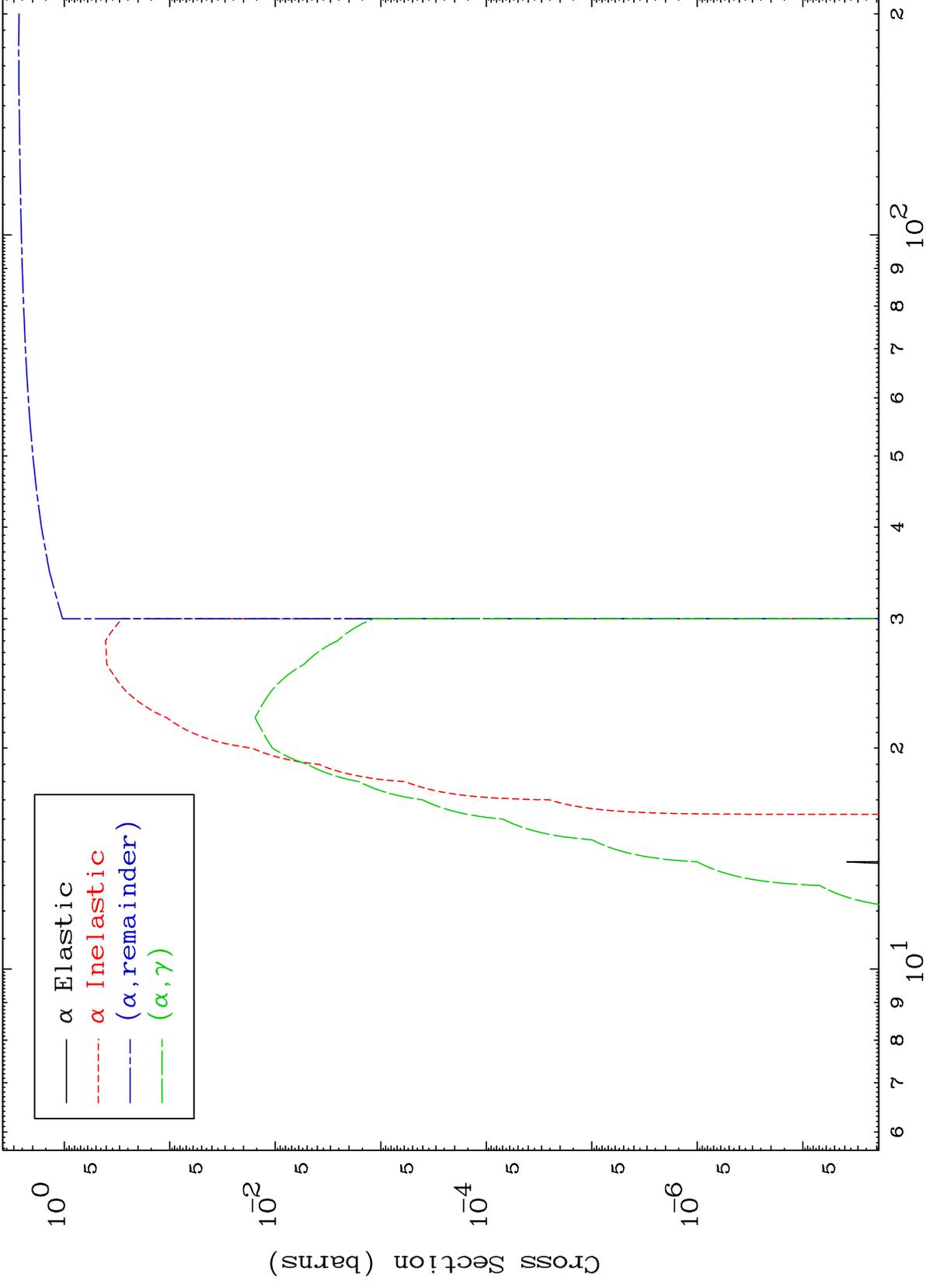
Press Mouse Button to Start

MAT 8089

$\alpha$  Major

81-Tl-191

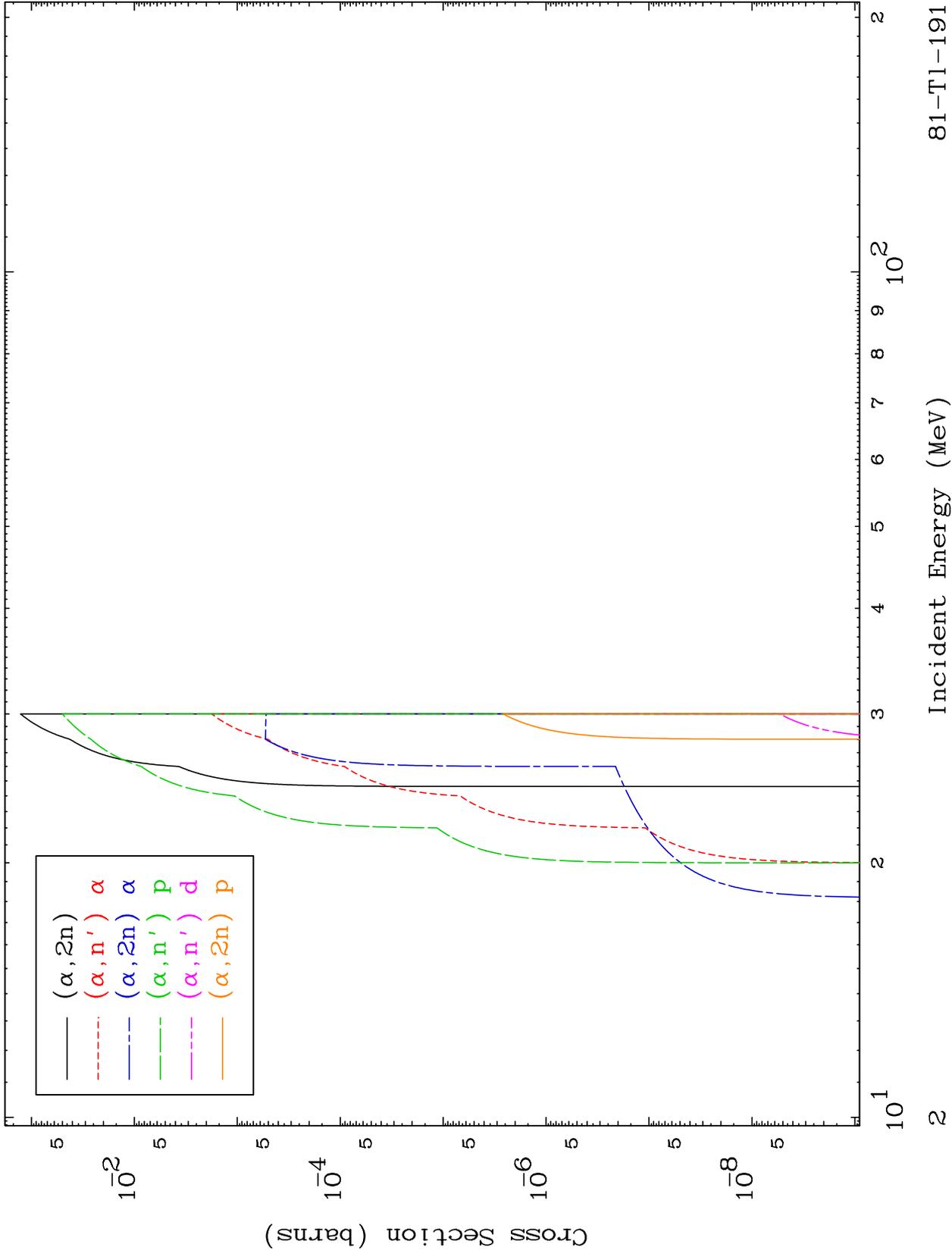
0 Kelvin Cross Sections

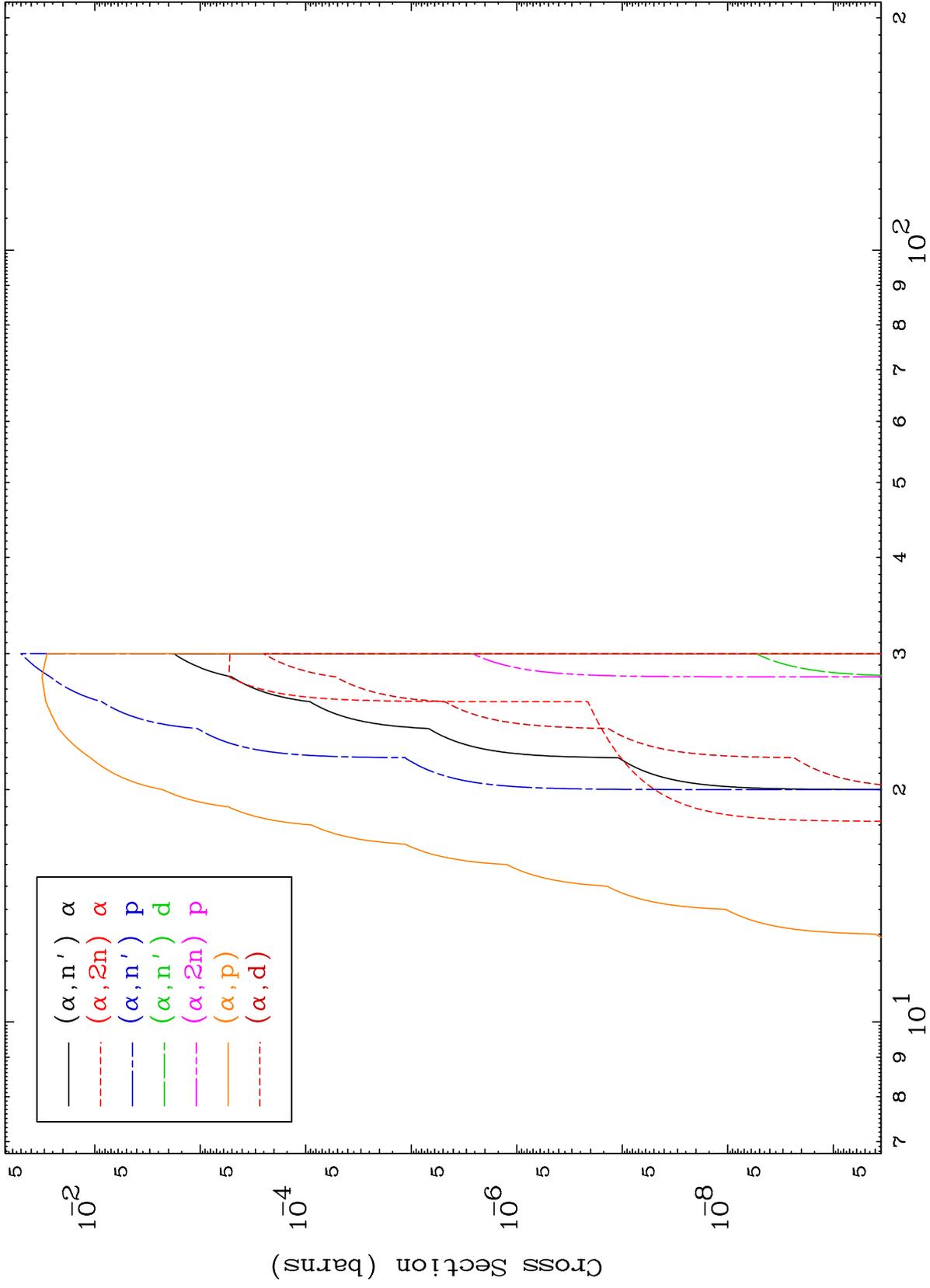


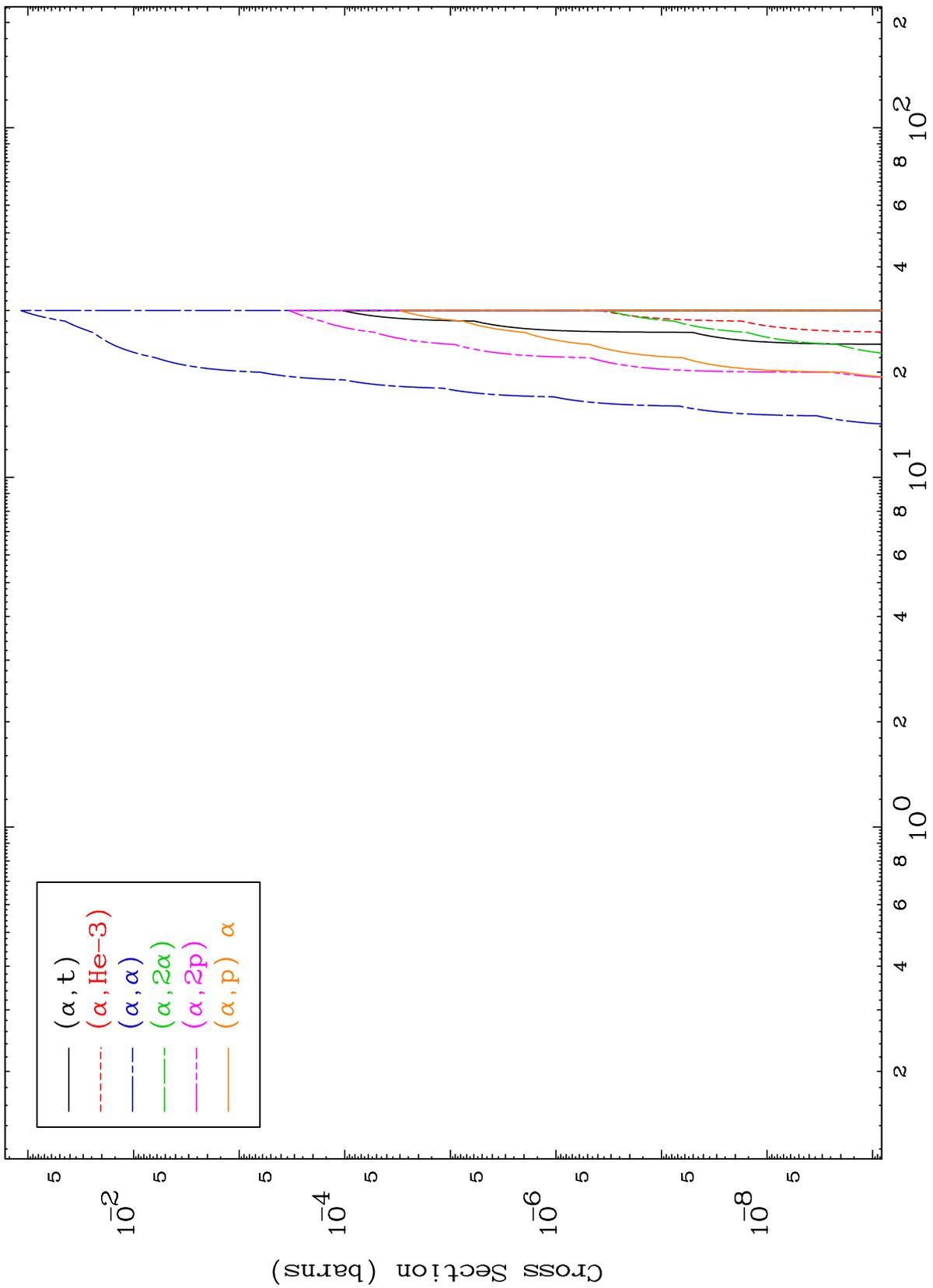
1

Incident Energy (MeV)

81-Tl-191





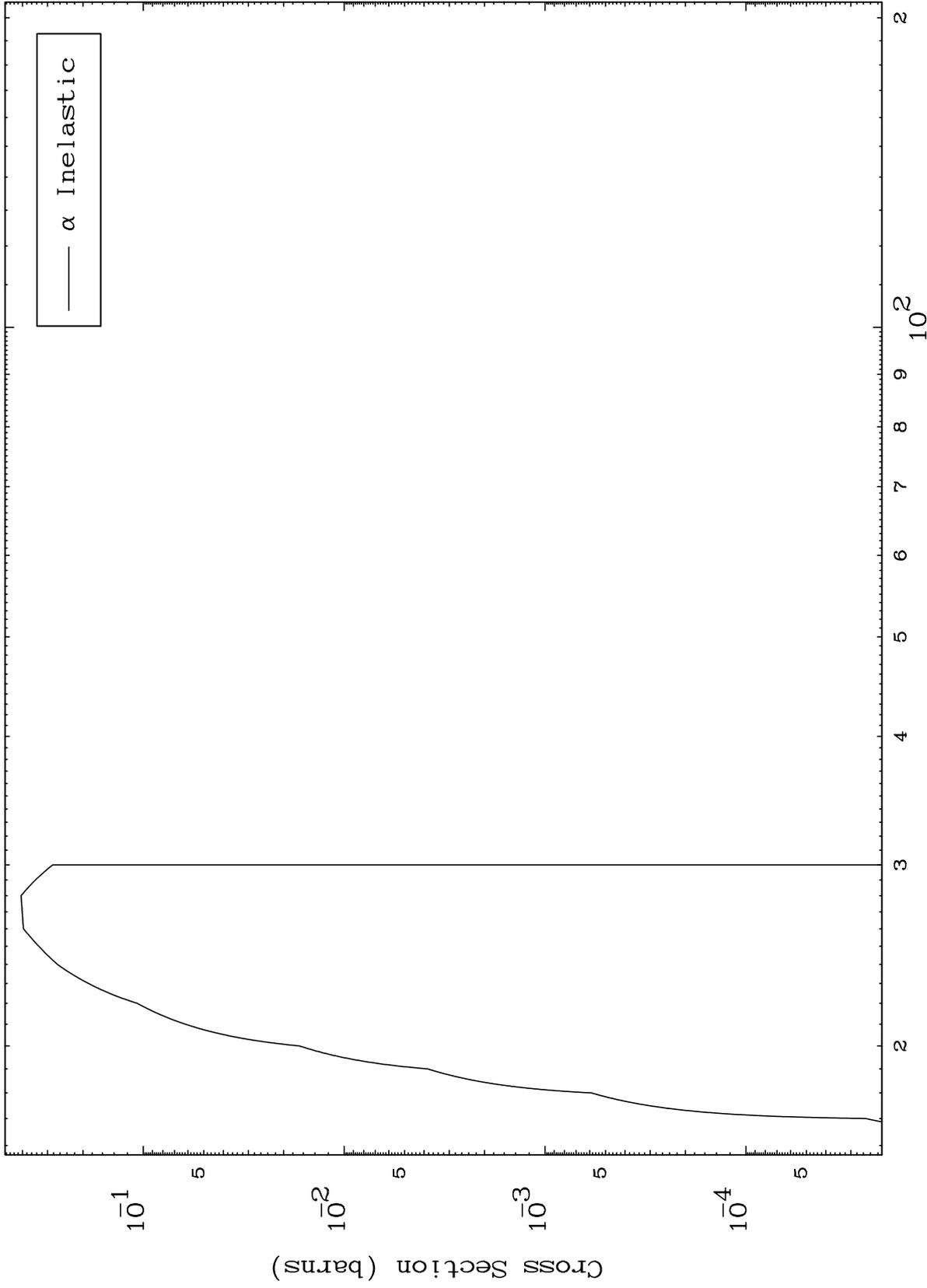


MAT 8089

( $\alpha, n'$ ) Level

81-Tl-191

0 Kelvin Cross Sections



5

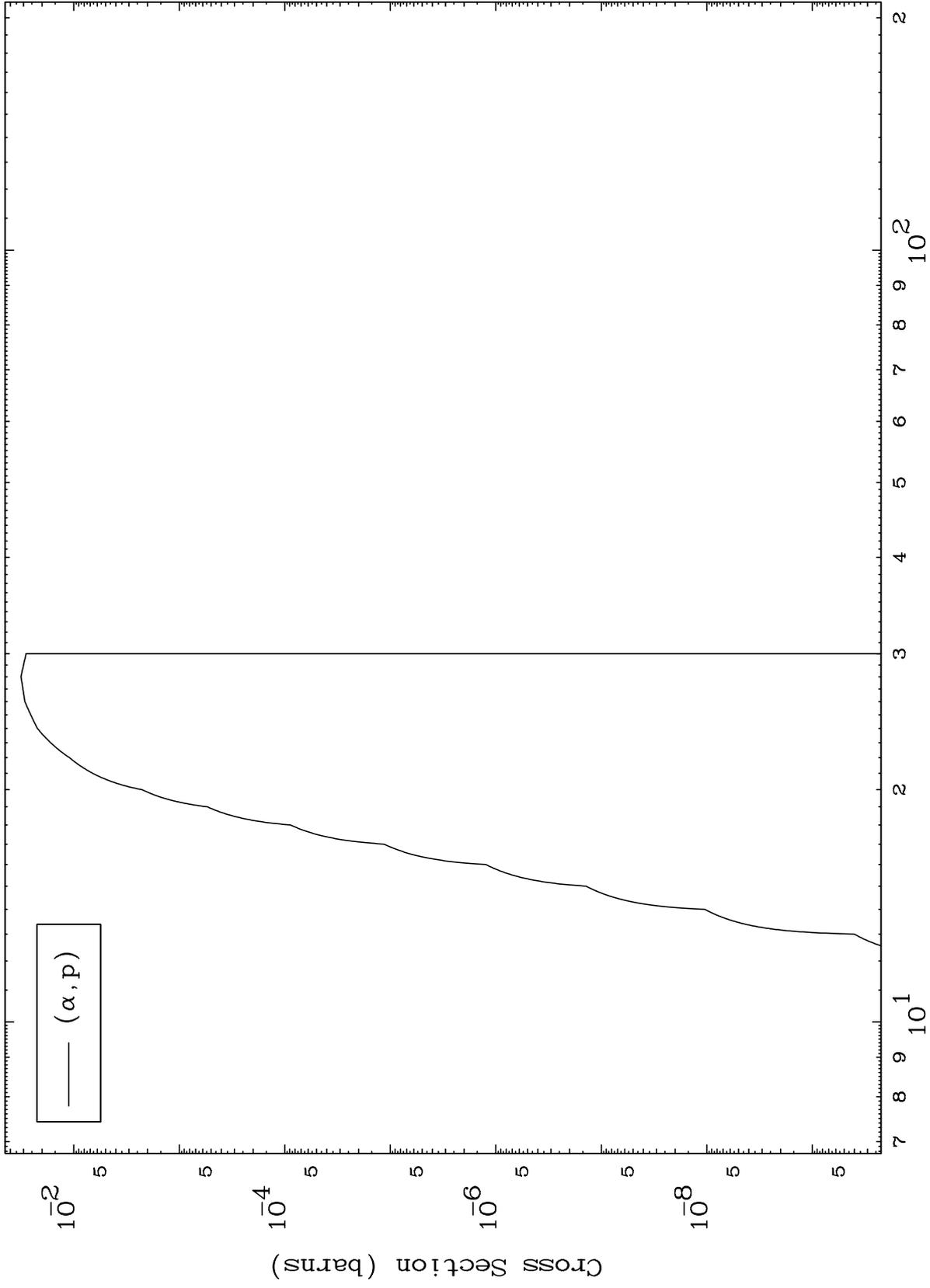
Incident Energy (MeV)

81-Tl-191

MAT 8089

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

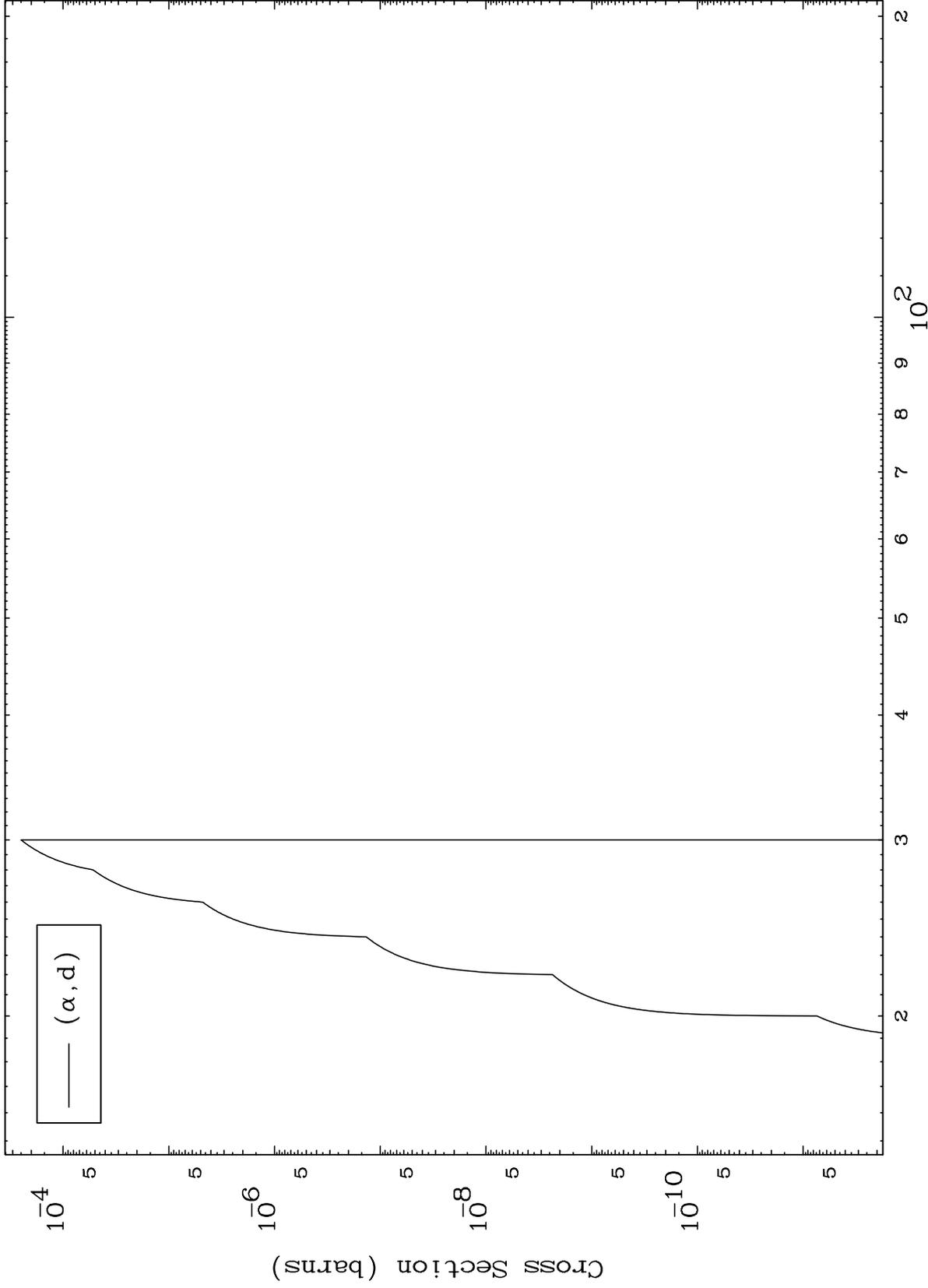
81-Tl-191

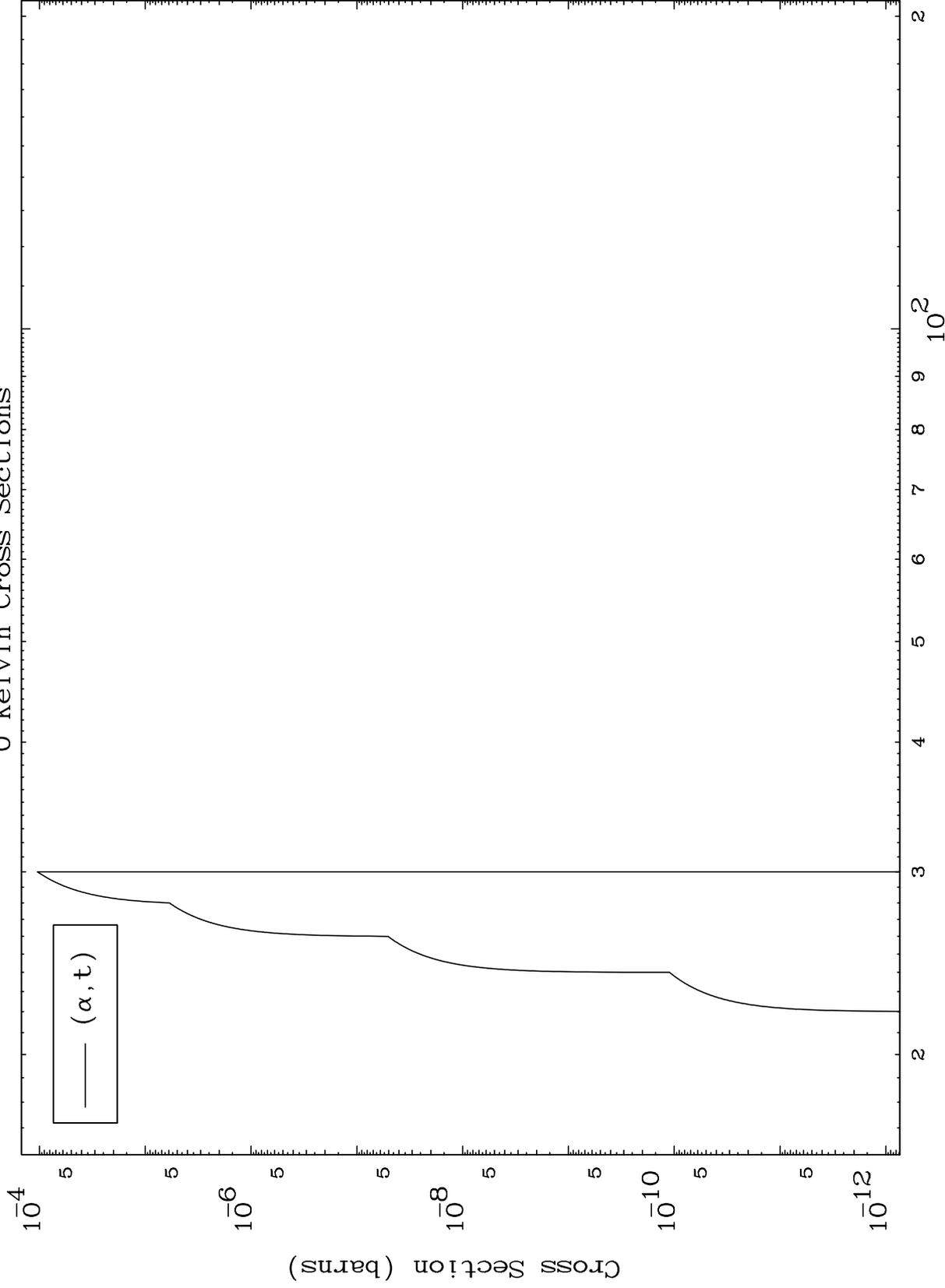


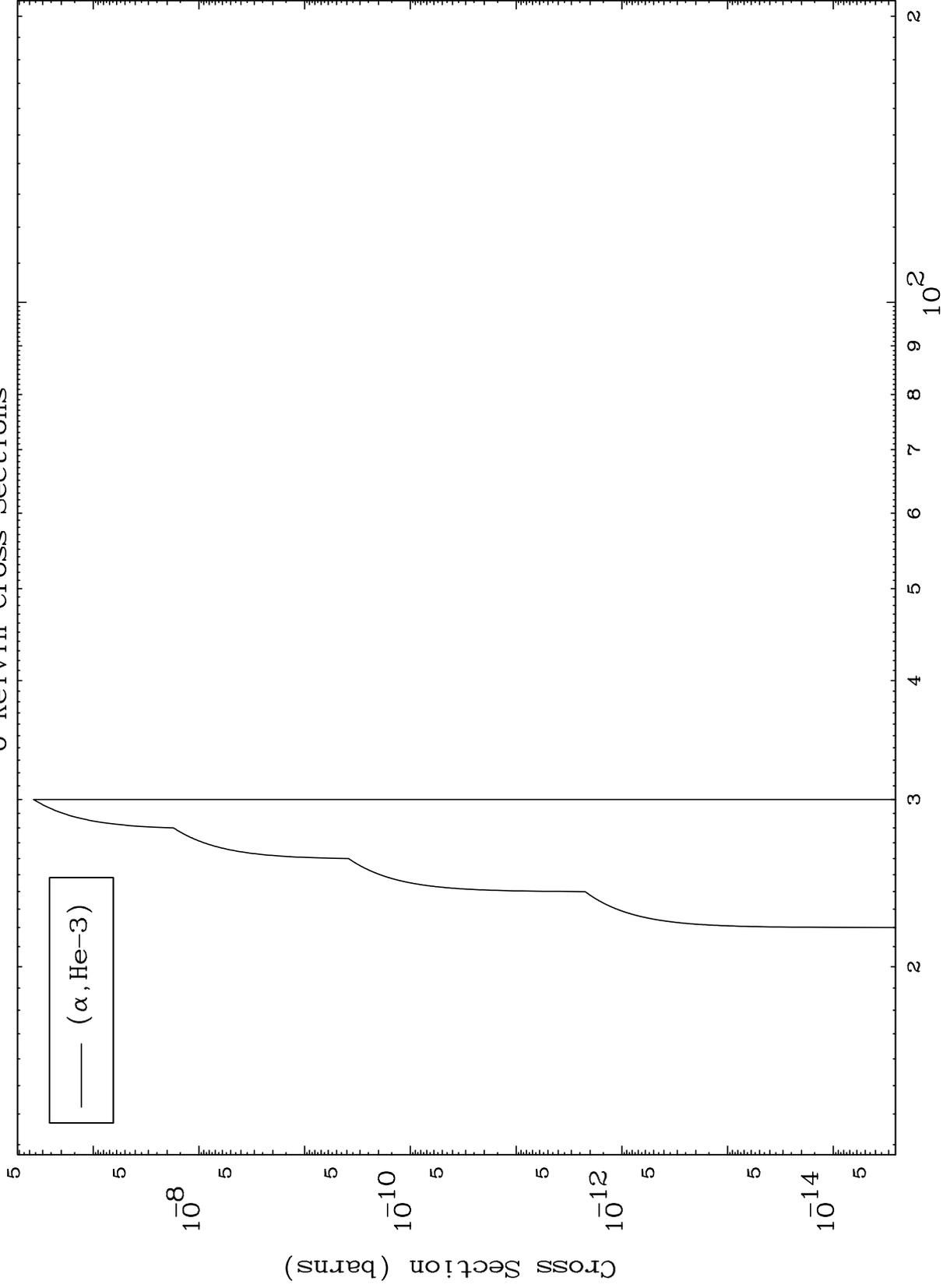
6

Incident Energy (MeV)

81-Tl-191





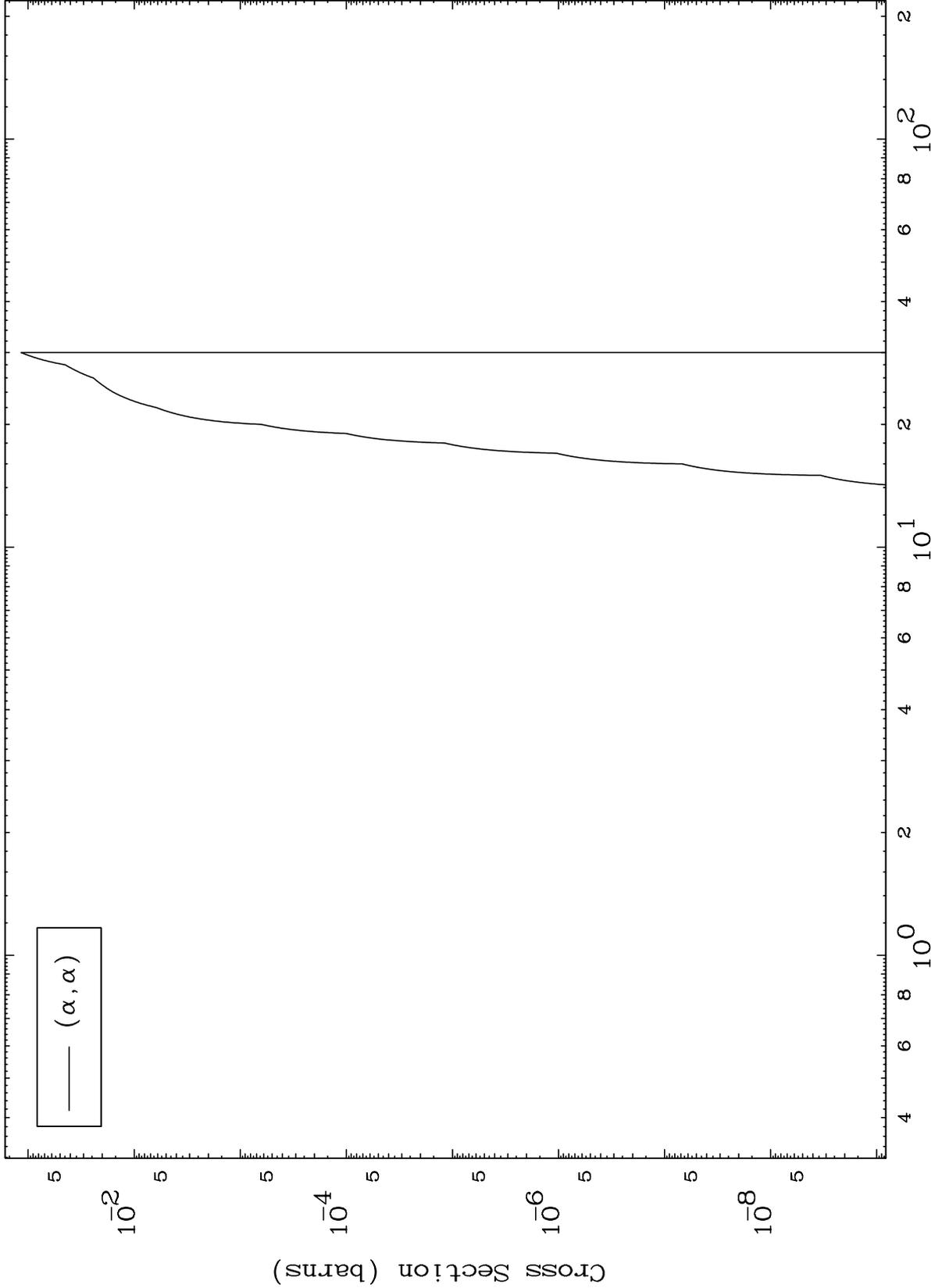


MAT 8089

( $\alpha, \alpha$ ) Levels

81-Tl-191

0 Kelvin Cross Sections

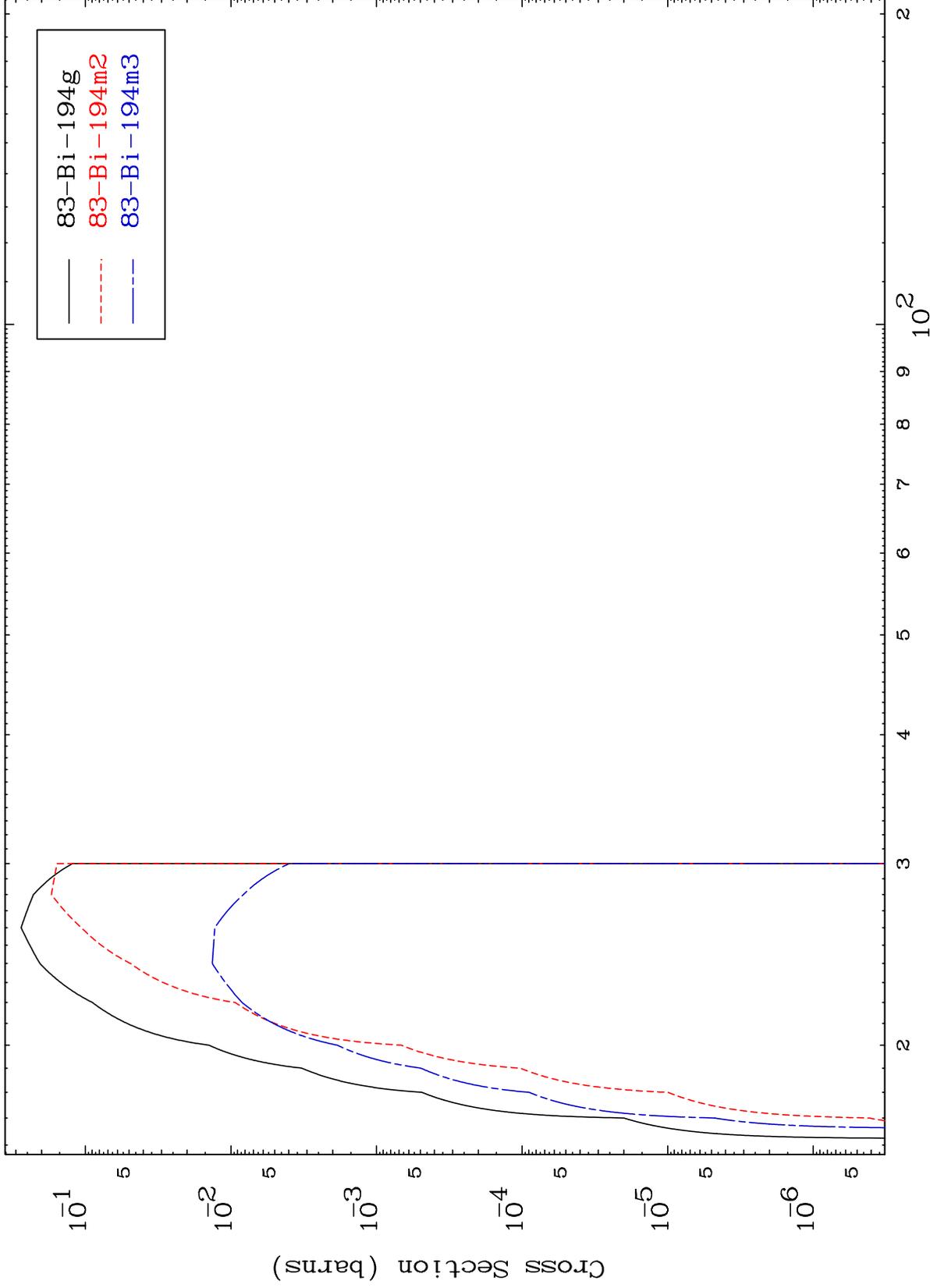


10

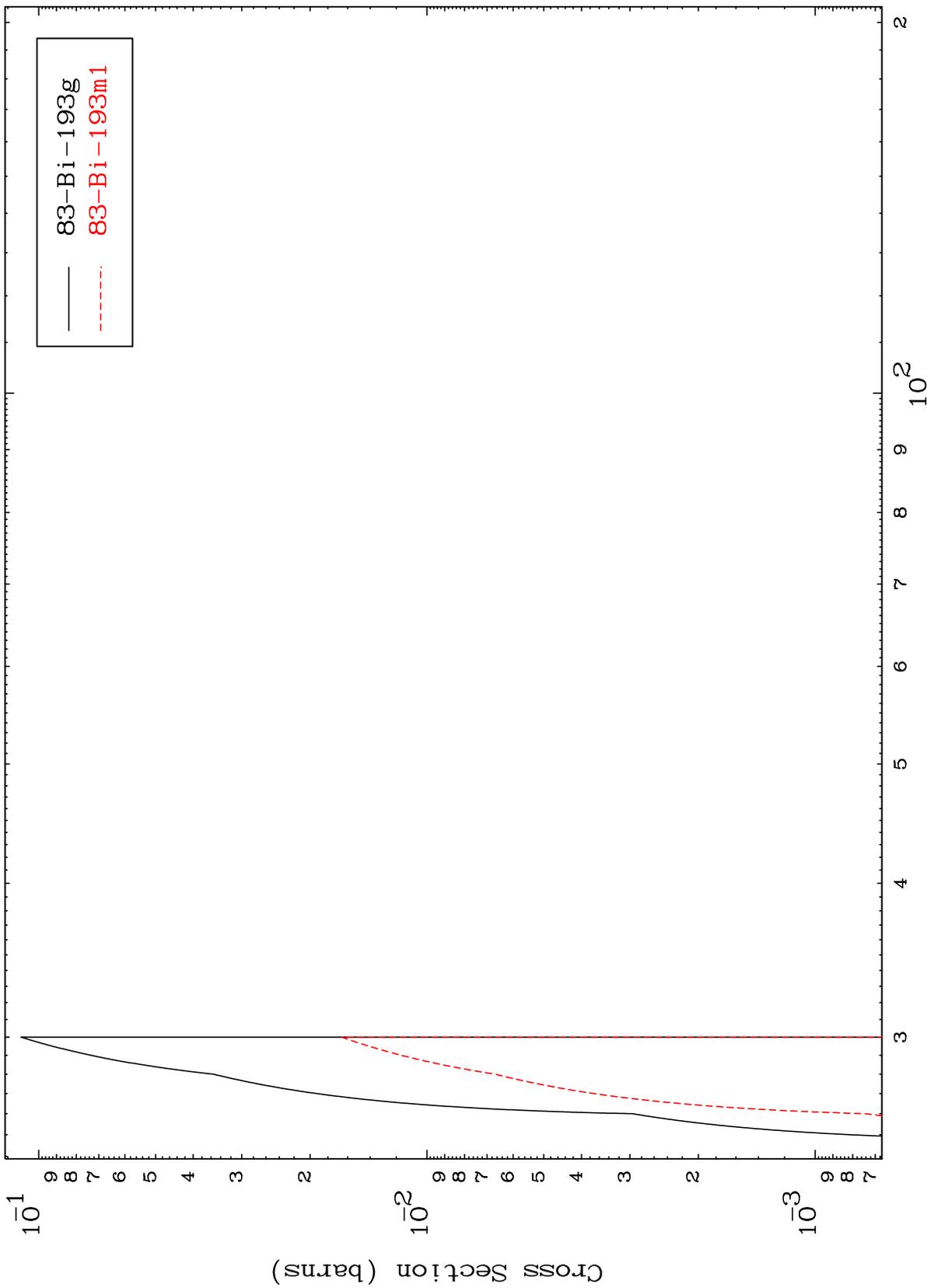
Incident Energy (MeV)

81-Tl-191

Radionuclide Production Cross Section



Radionuclide Production Cross Section  
( $\alpha, 2n$ )

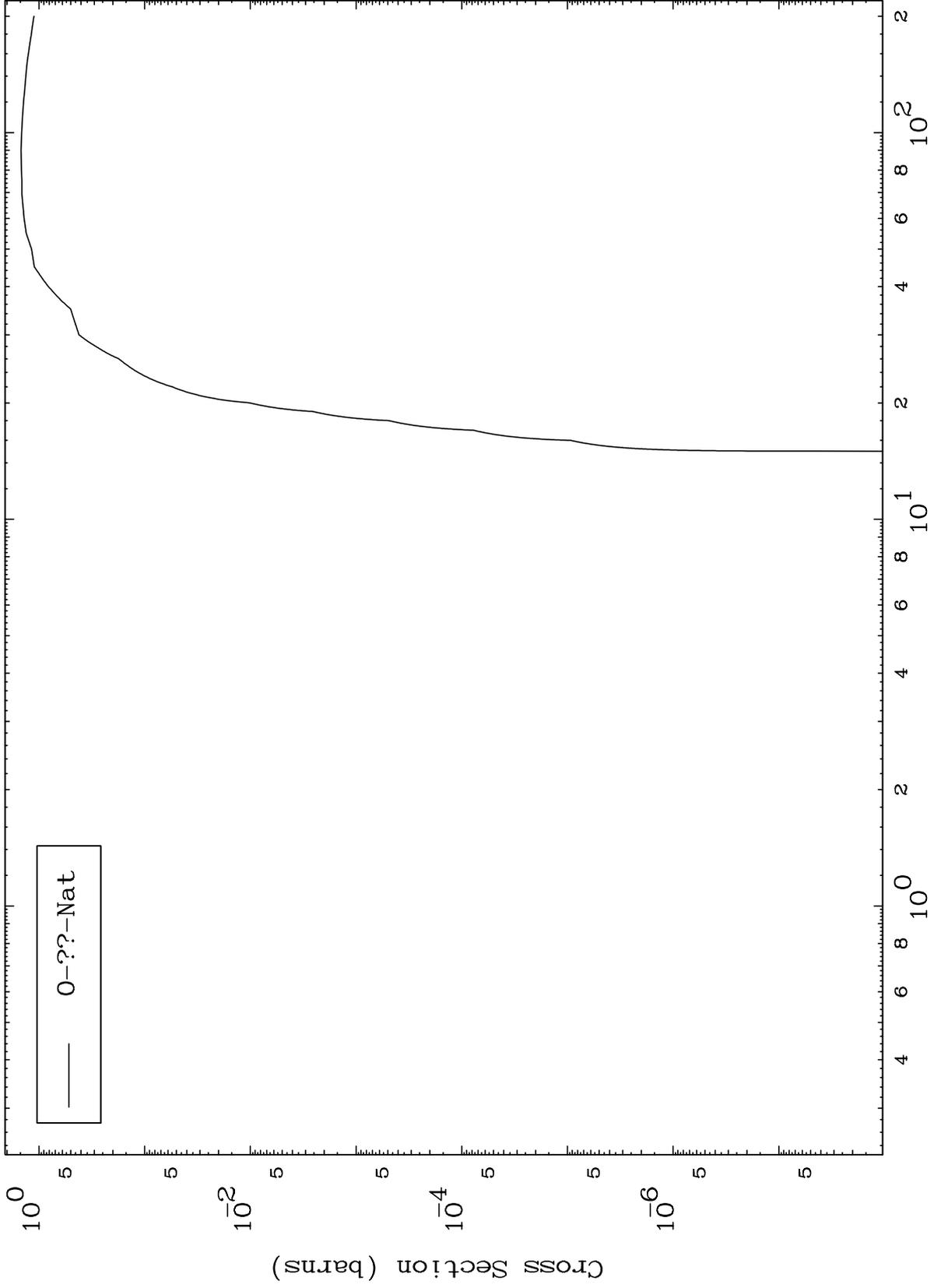


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$\alpha$  Fission

81-Tl-191

Radionuclide Production Cross Section

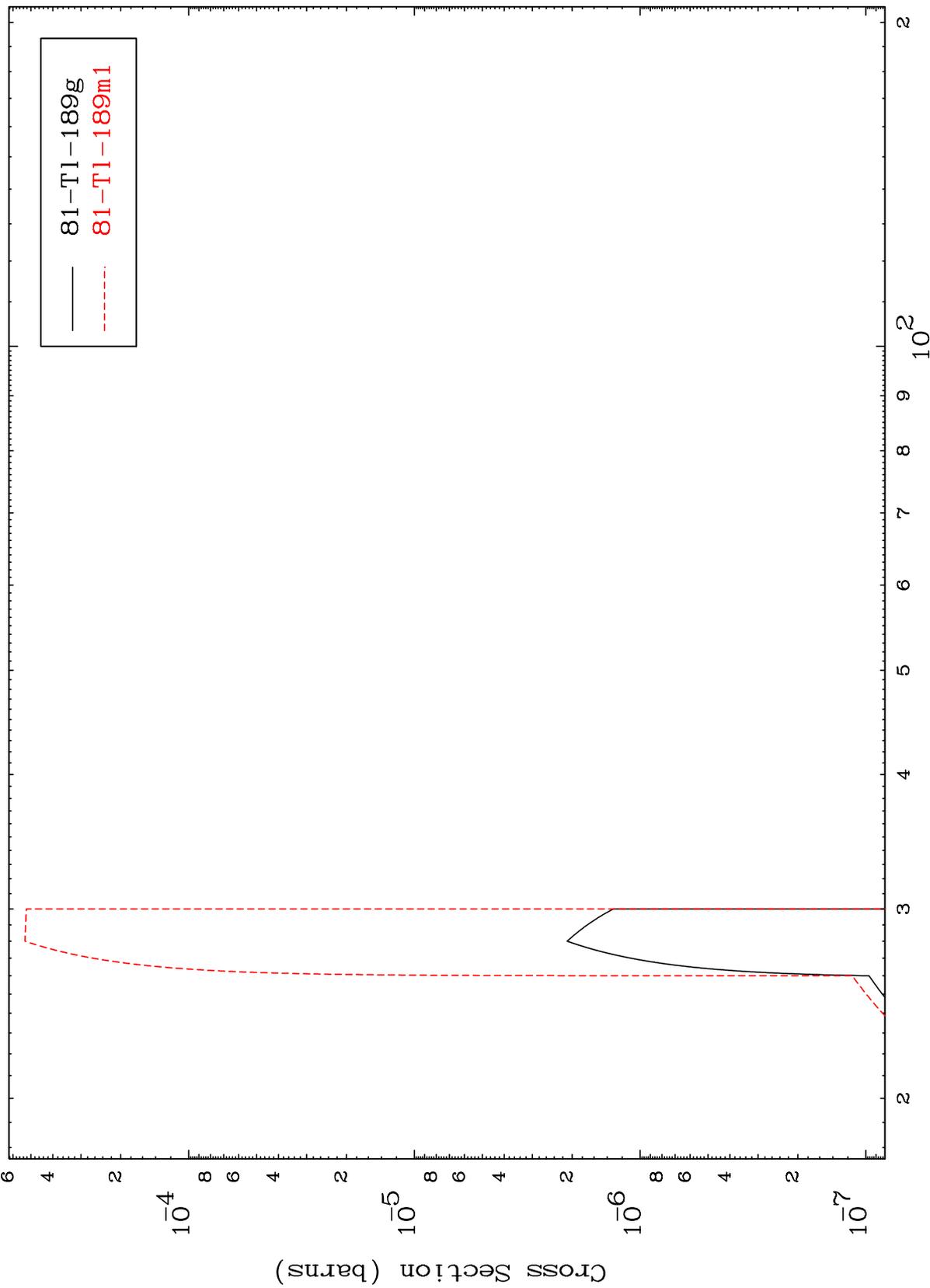


MAT 8089

$(\alpha, 2n)$   $\alpha$

81-Tl-191

Radionuclide Production Cross Section

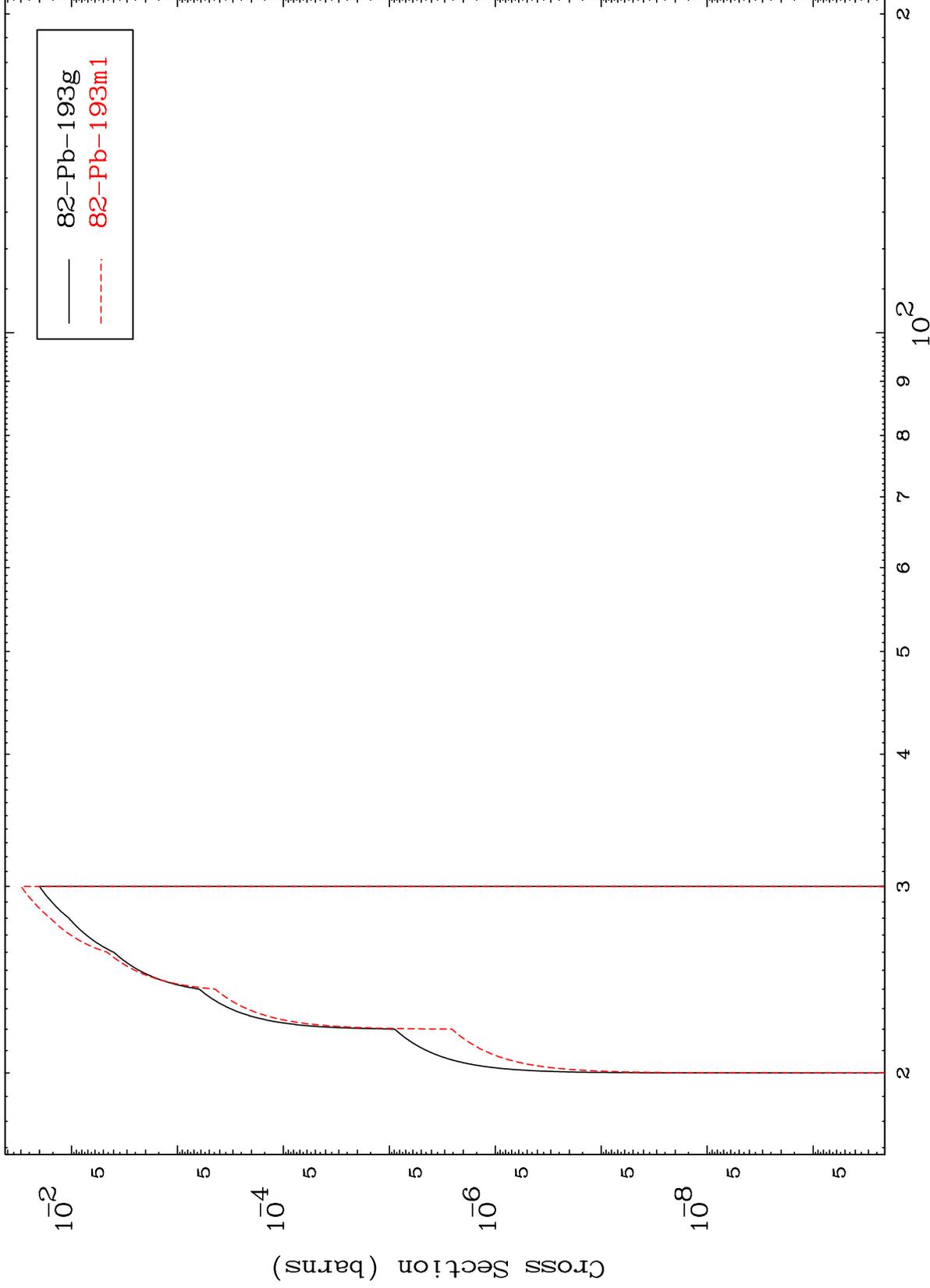


14

Incident Energy (MeV)

81-Tl-191

Radionuclide Production Cross Section

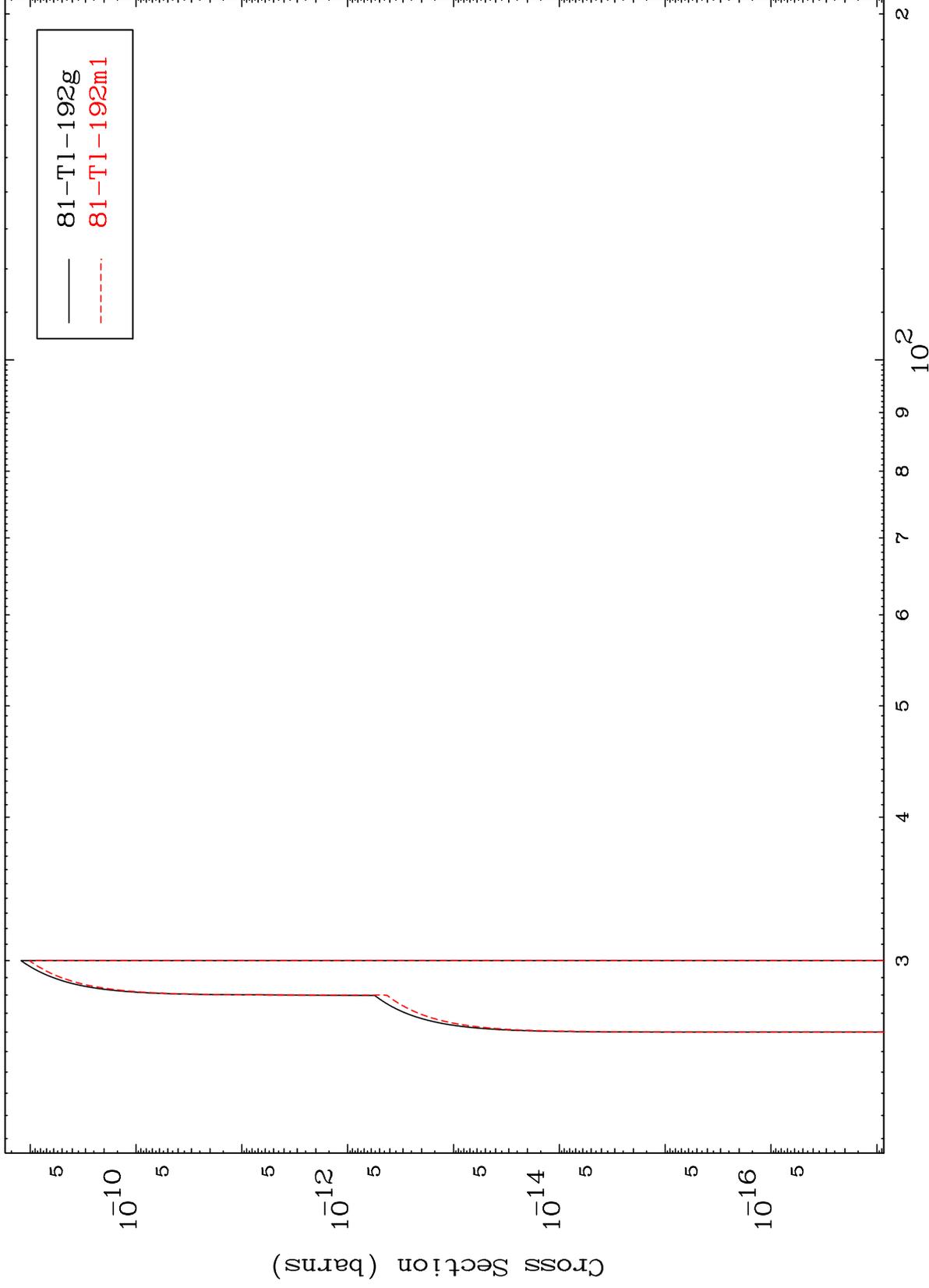


MAT 8089

$(\alpha, 2n)$  p

81-Tl-191

Radionuclide Production Cross Section



16

Incident Energy (MeV)

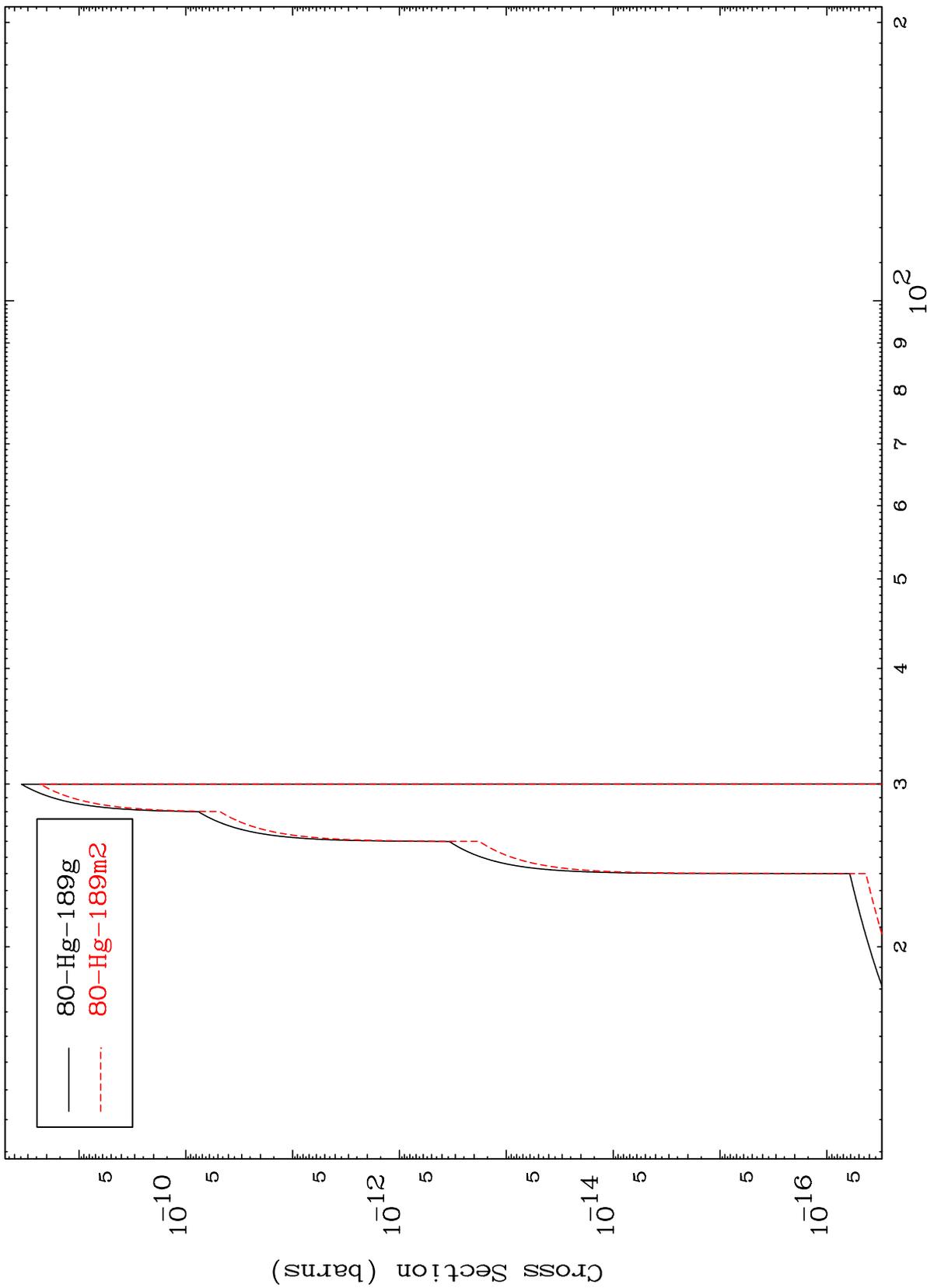
81-Tl-191

MAT 8089

( $\alpha, n'$ ) p  $\alpha$

81-Tl-191

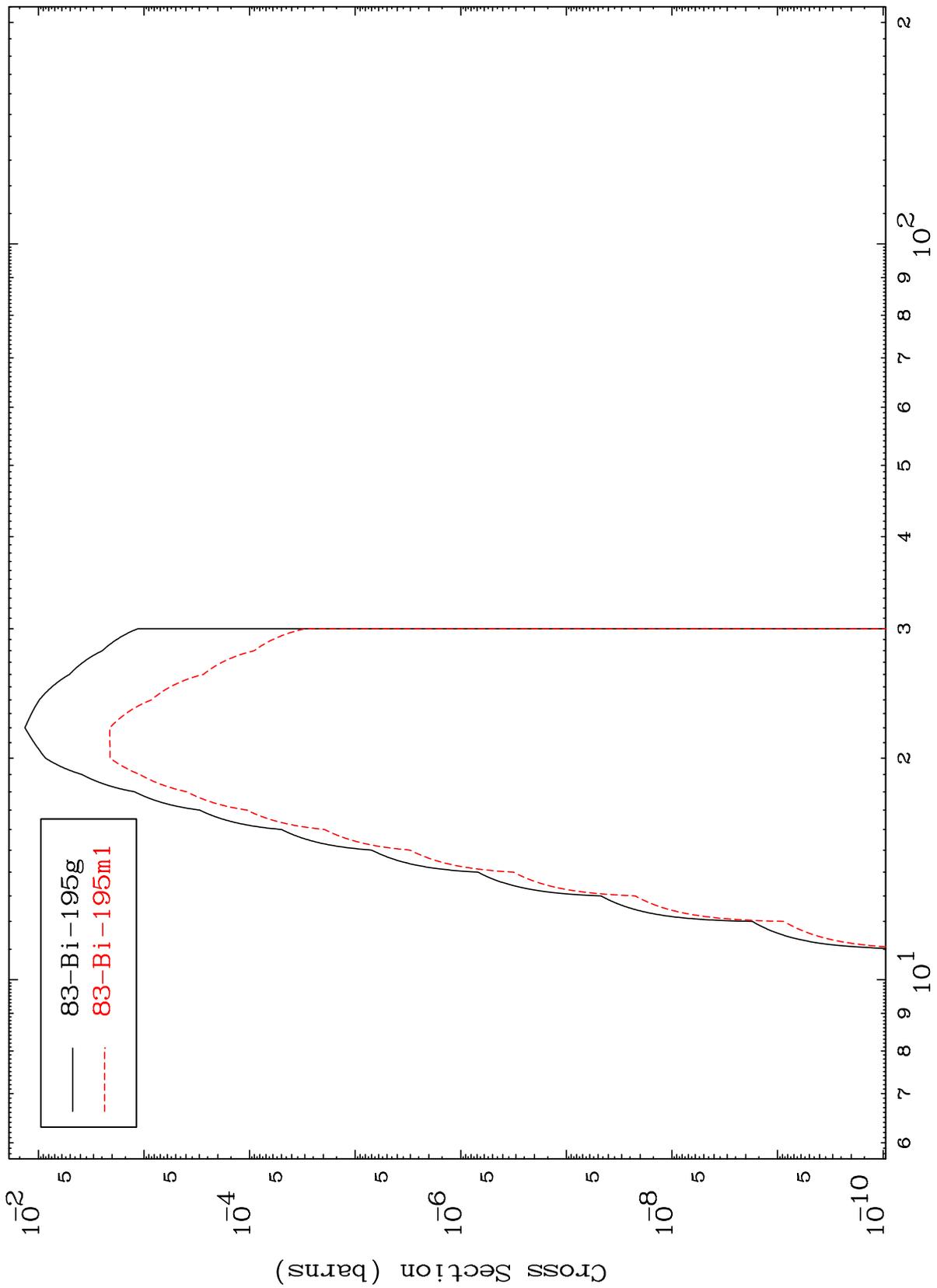
Radionuclide Production Cross Section



MAT 8089

81-Tl-191

Radionuclide Production Cross Section  
( $\alpha, \gamma$ )



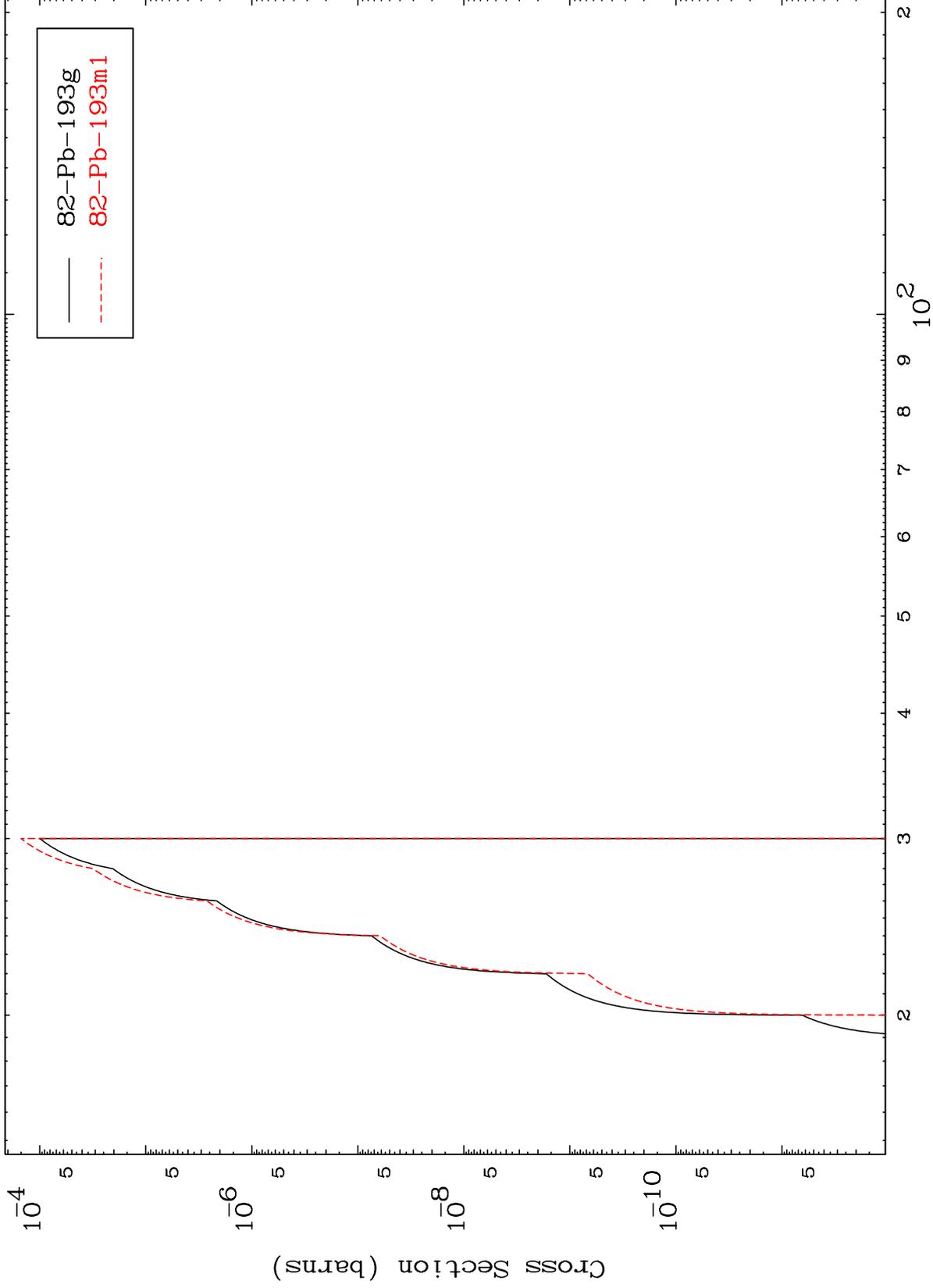
83-Bi-195g  
83-Bi-195m1

81-Tl-191

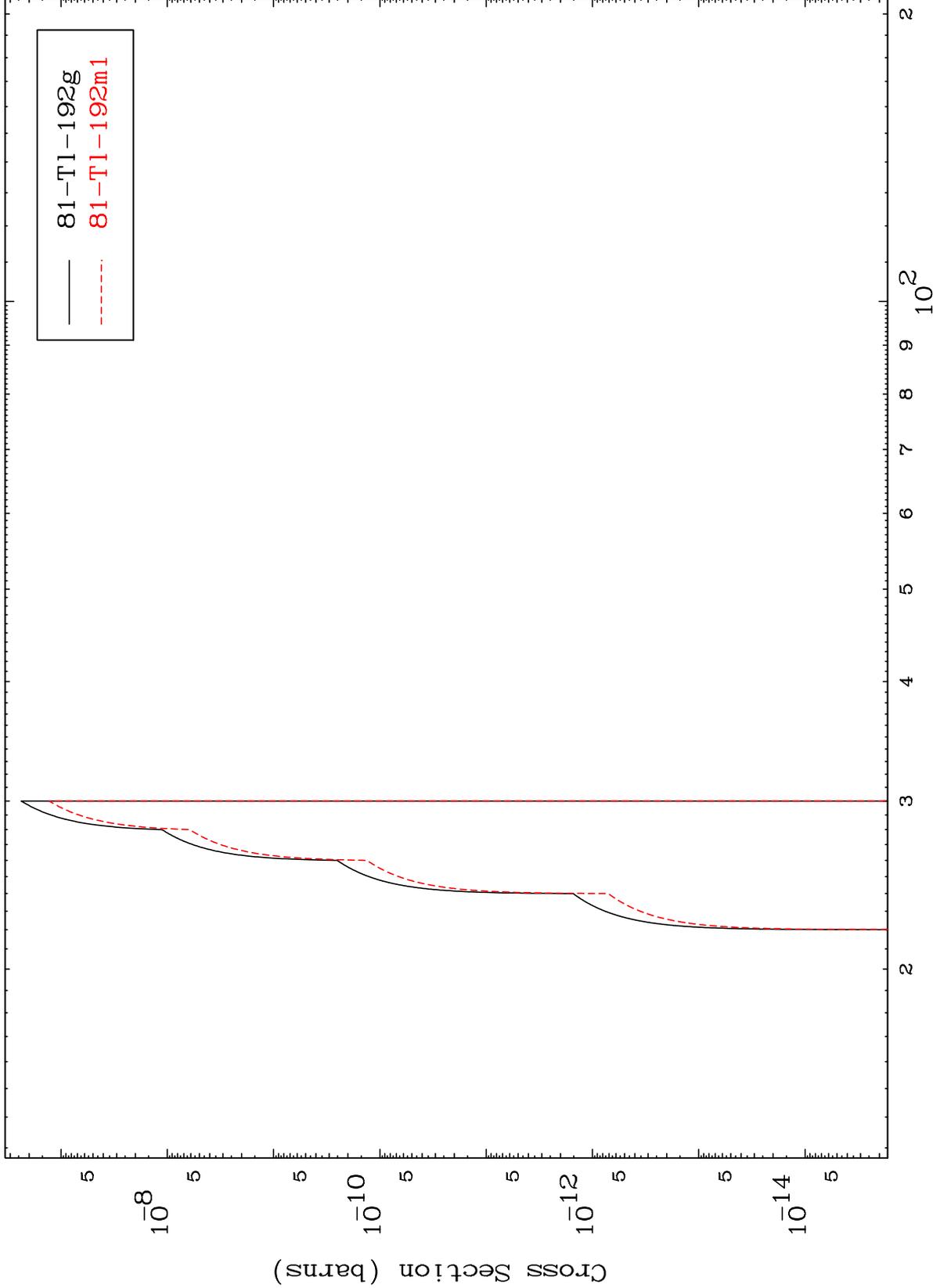
Incident Energy (MeV)

18

Radionuclide Production Cross Section



Radionuclide Production Cross Section

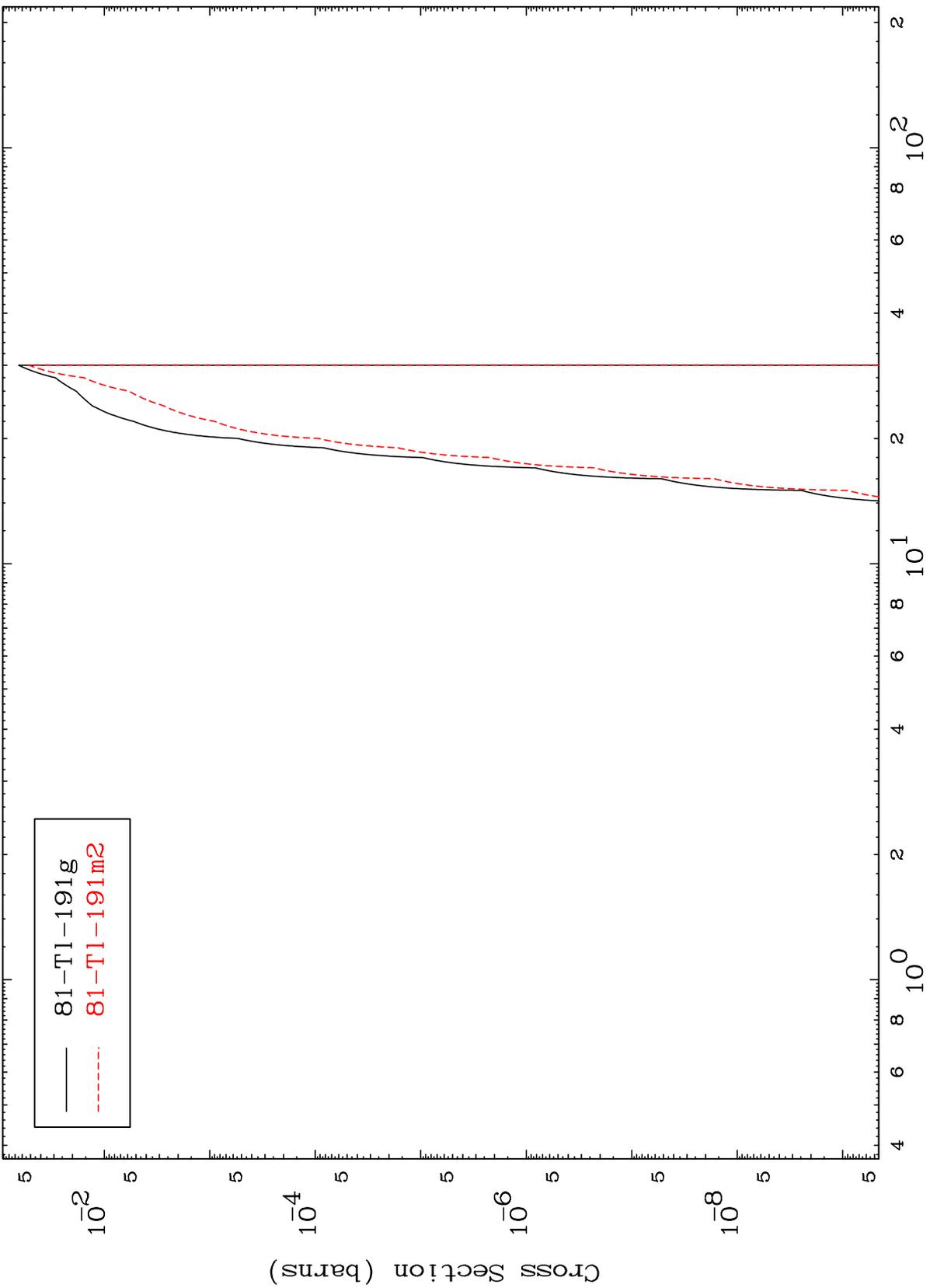


MAT 8089

( $\alpha, \alpha$ )

81-Tl-191

Radionuclide Production Cross Section

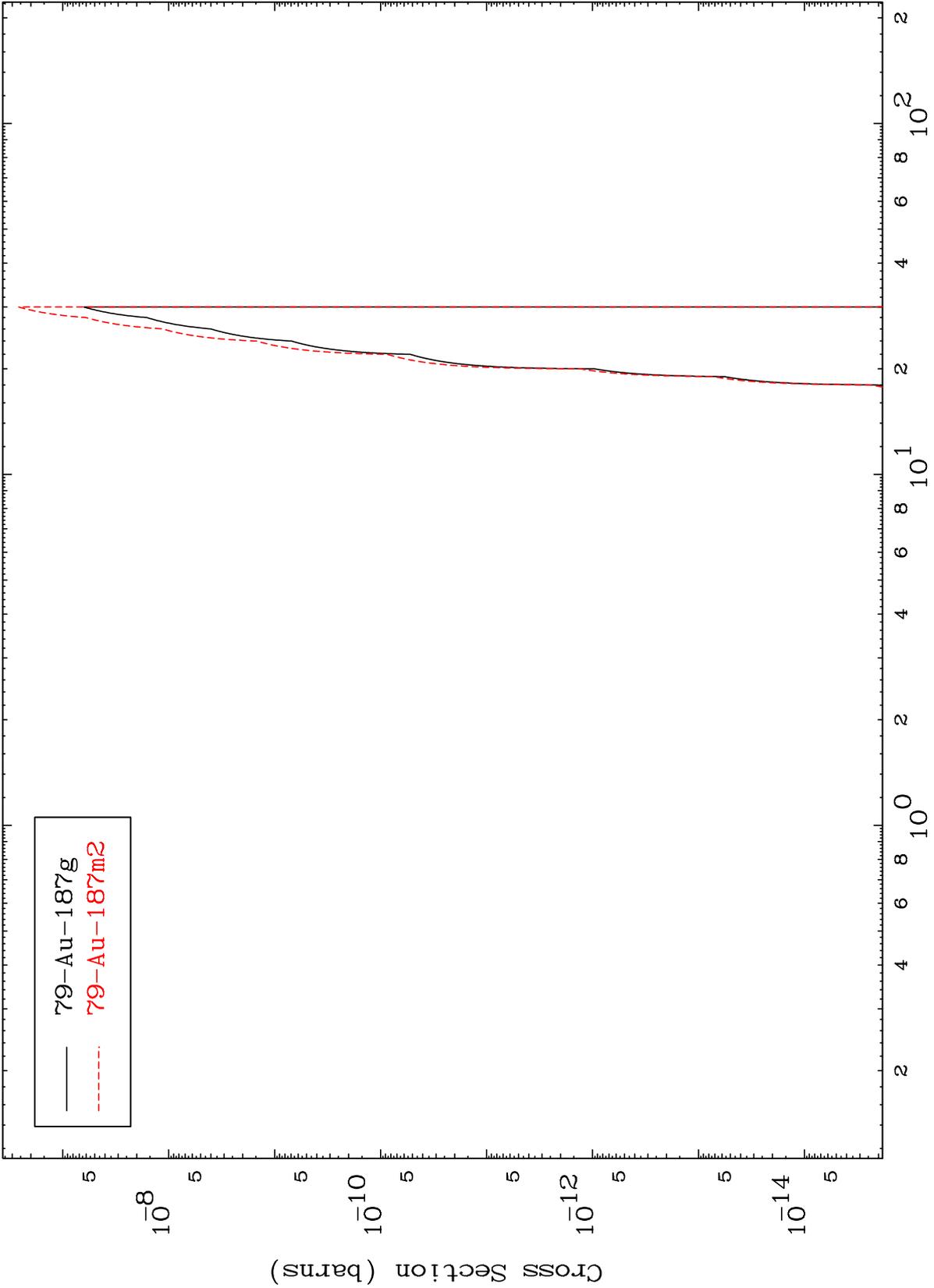


MAT 8089

( $\alpha, 2\alpha$ )

81-Tl-191

Radionuclide Production Cross Section



MAT 8089

81-Tl-191

( $\alpha, 2p$ )  
Radionuclide Production Cross Section

