

Program EVALPLOT
(Version 2018-1)

by

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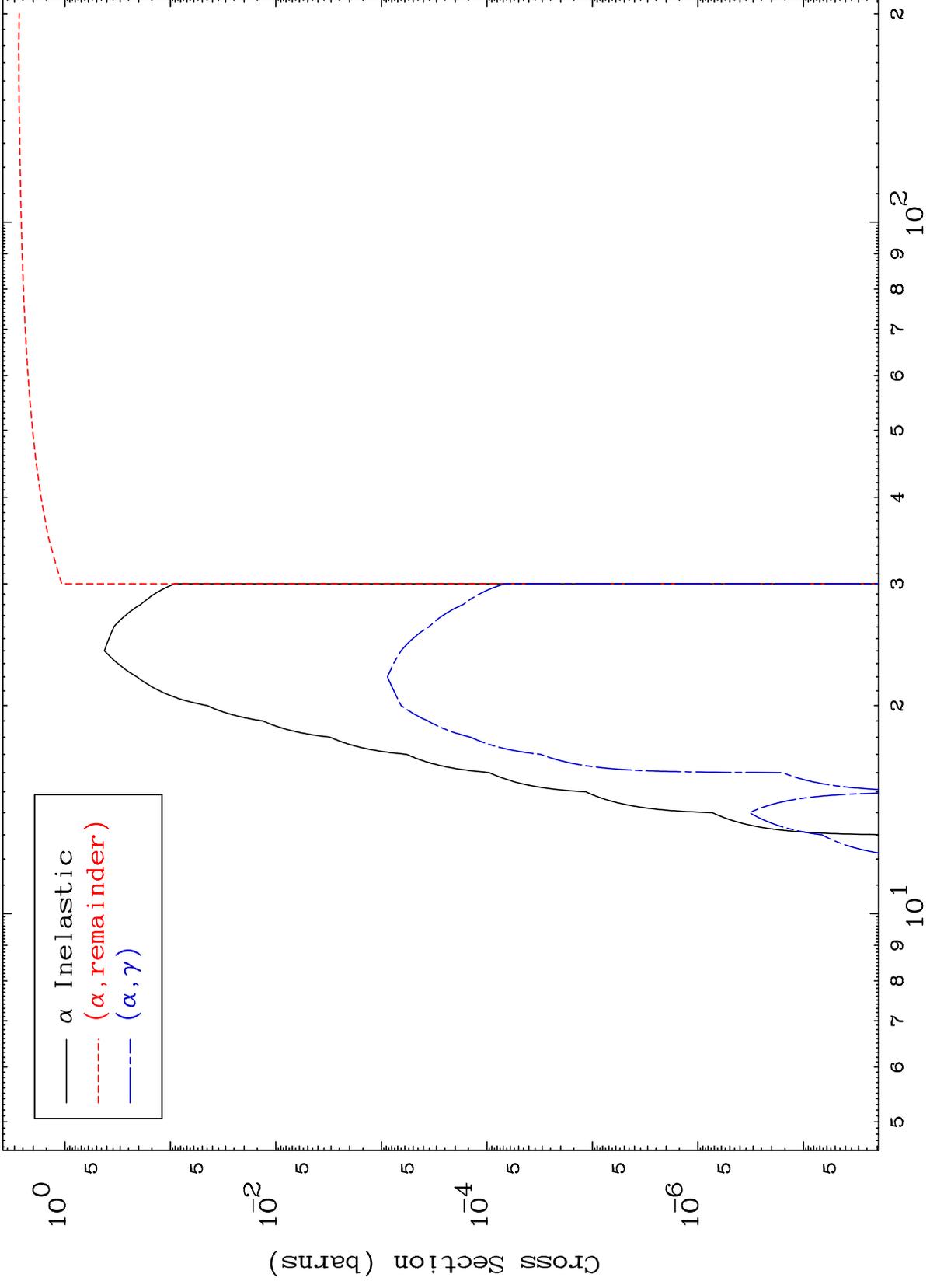
Press Mouse Button to Start

MAT 8104

α Major

81-T1-196

0 Kelvin Cross Sections



1

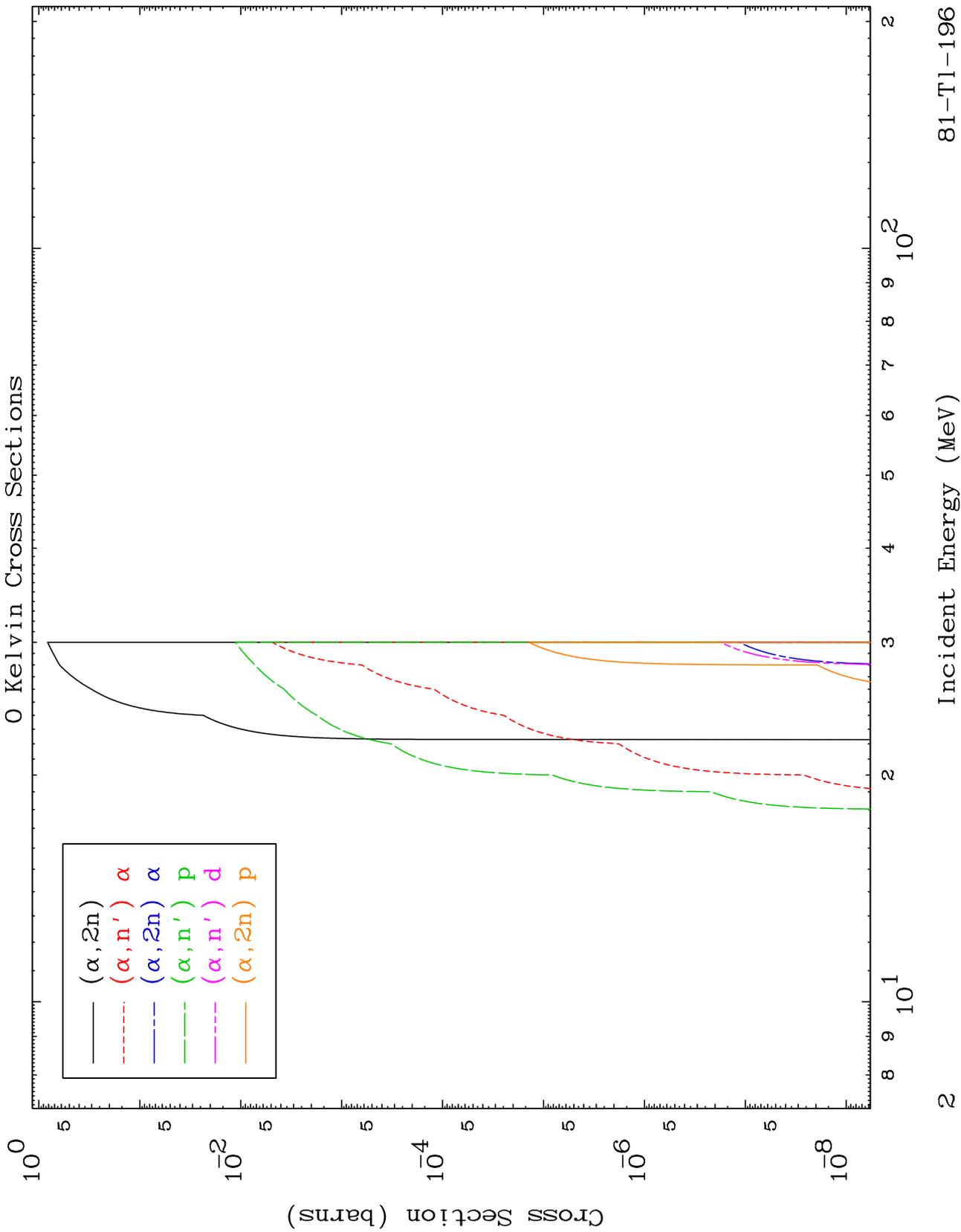
Incident Energy (MeV)

81-T1-196

MAT 8104

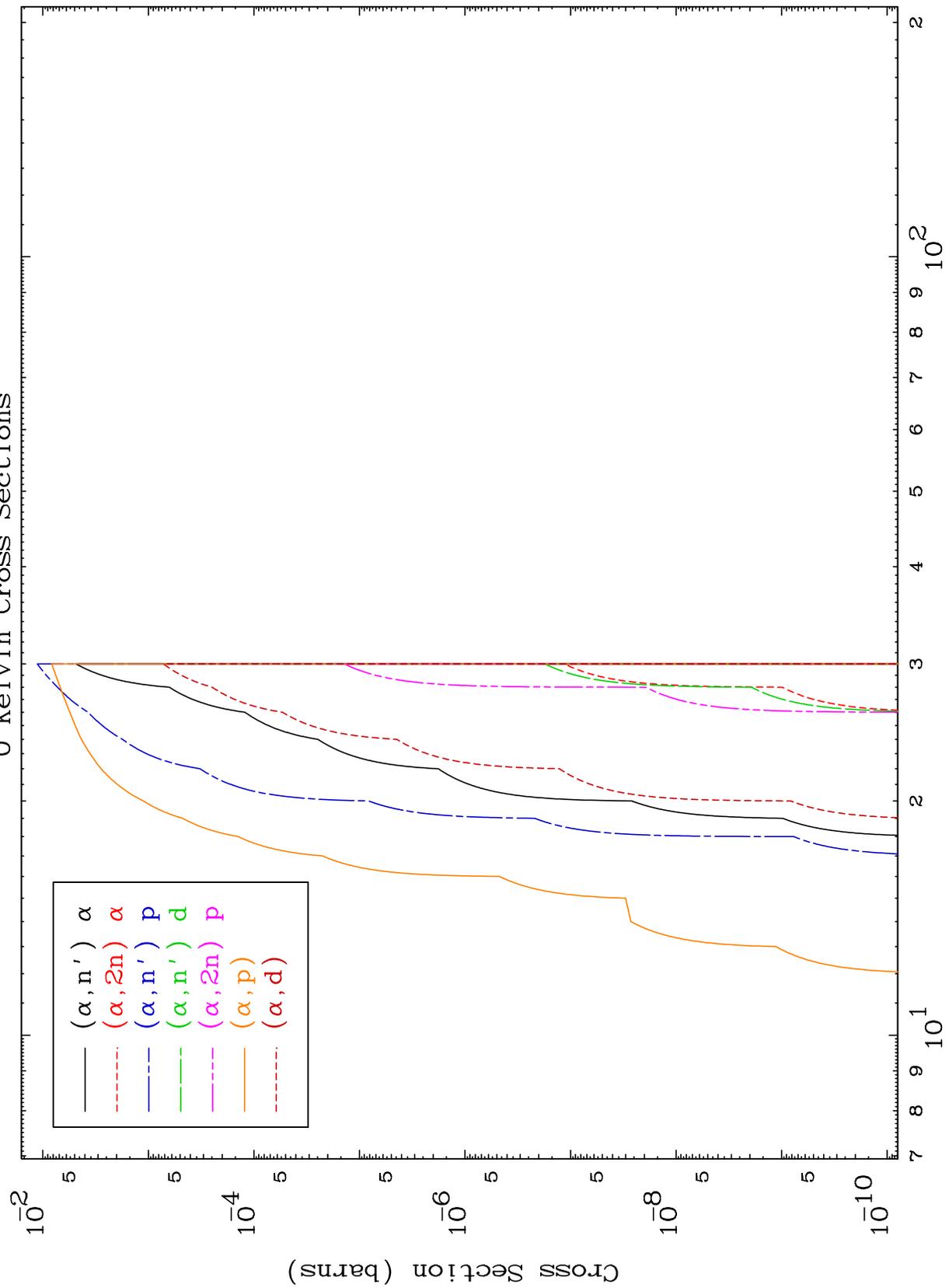
α Neutron Production

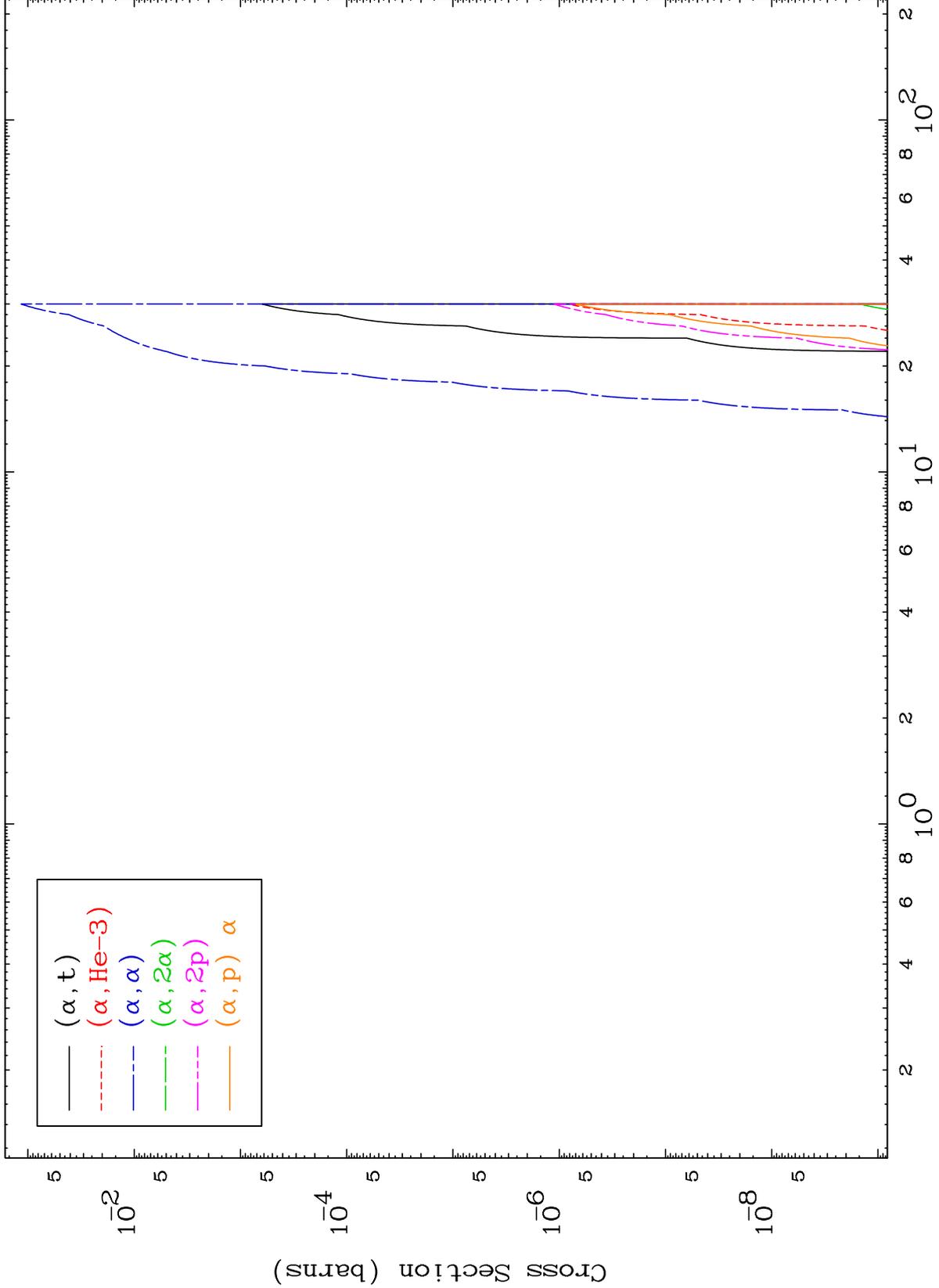
81-T1-196

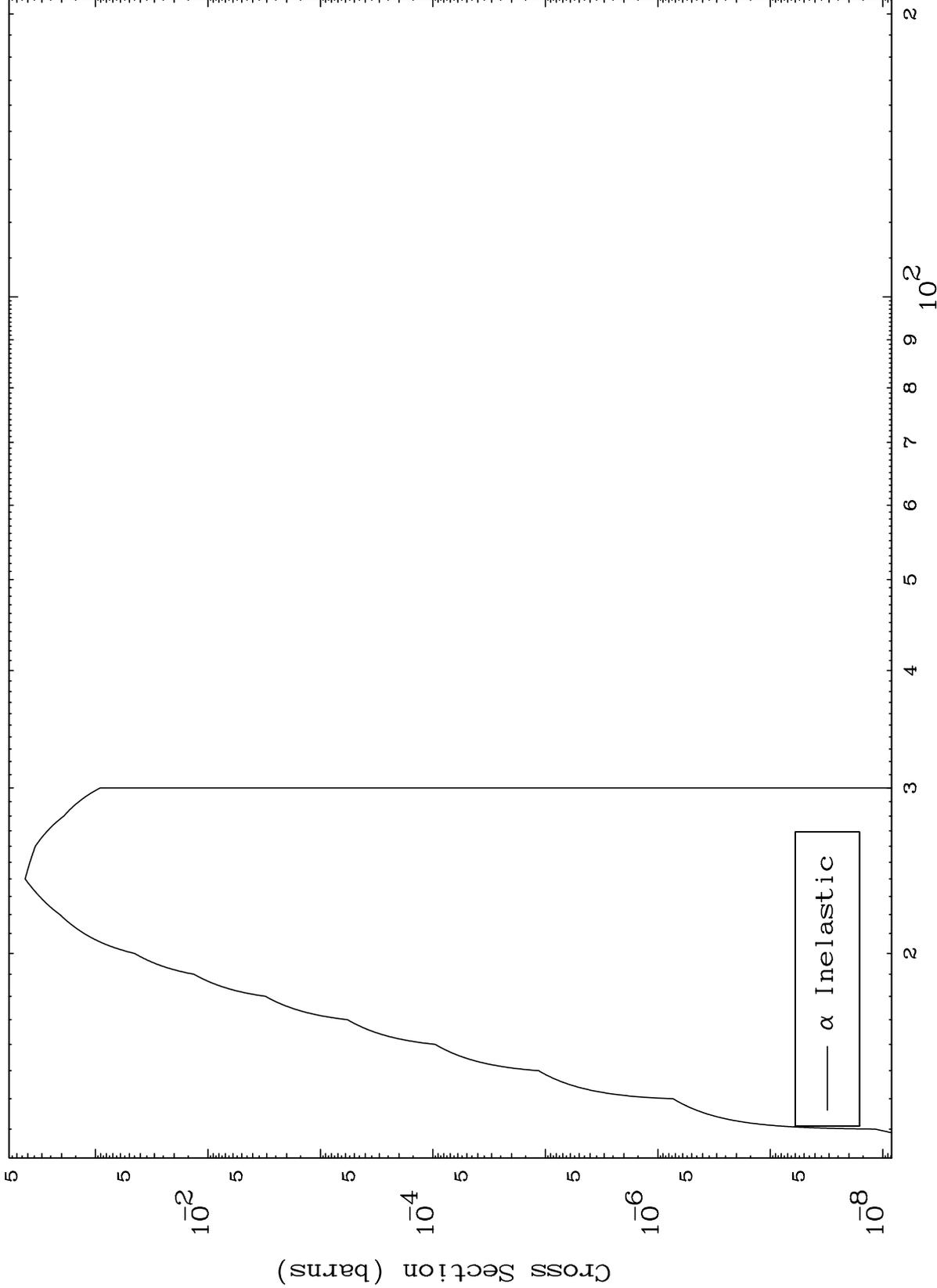


2

81-T1-196



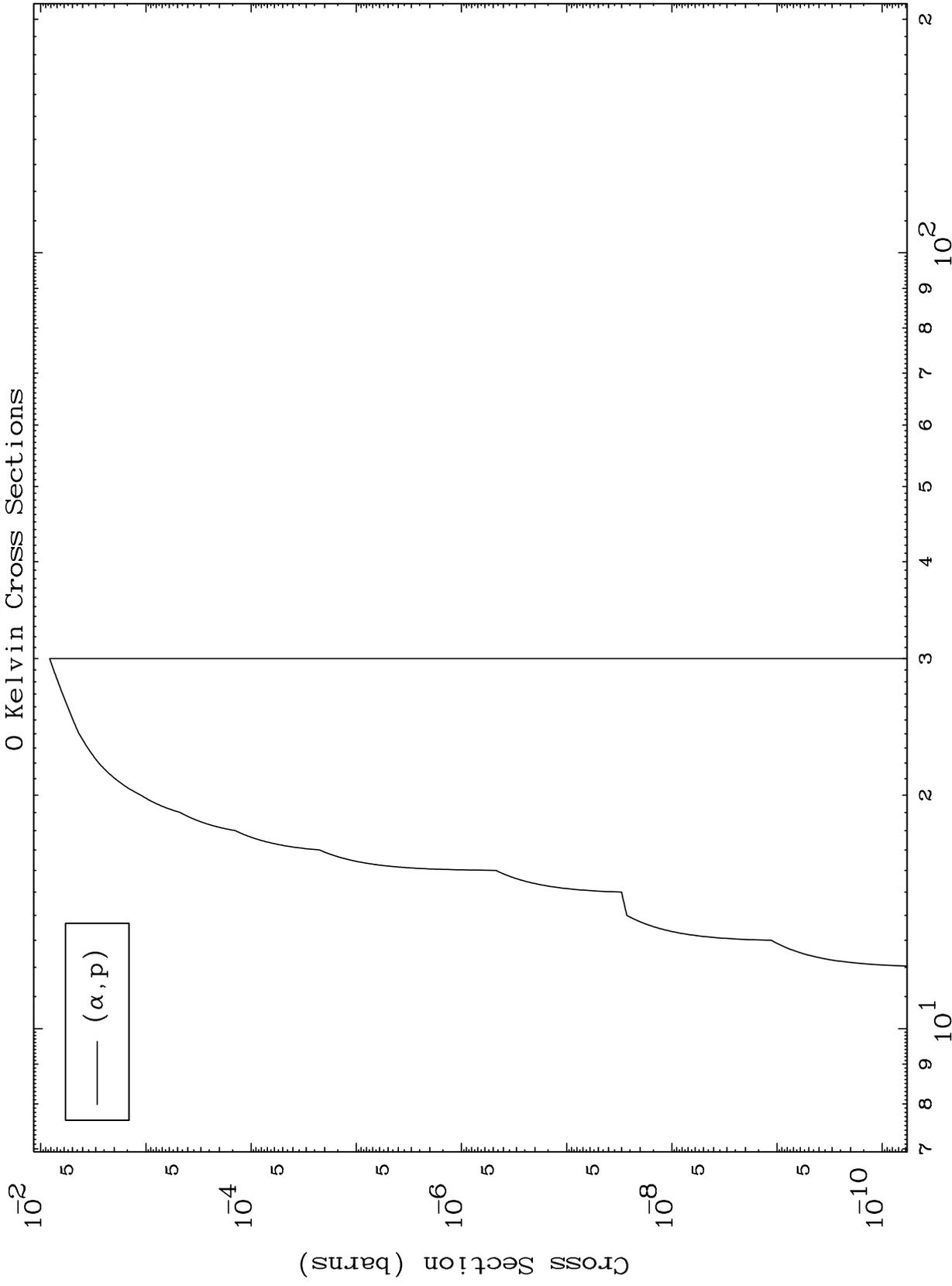




MAT 8104

(α, p) Levels
0 Kelvin Cross Sections

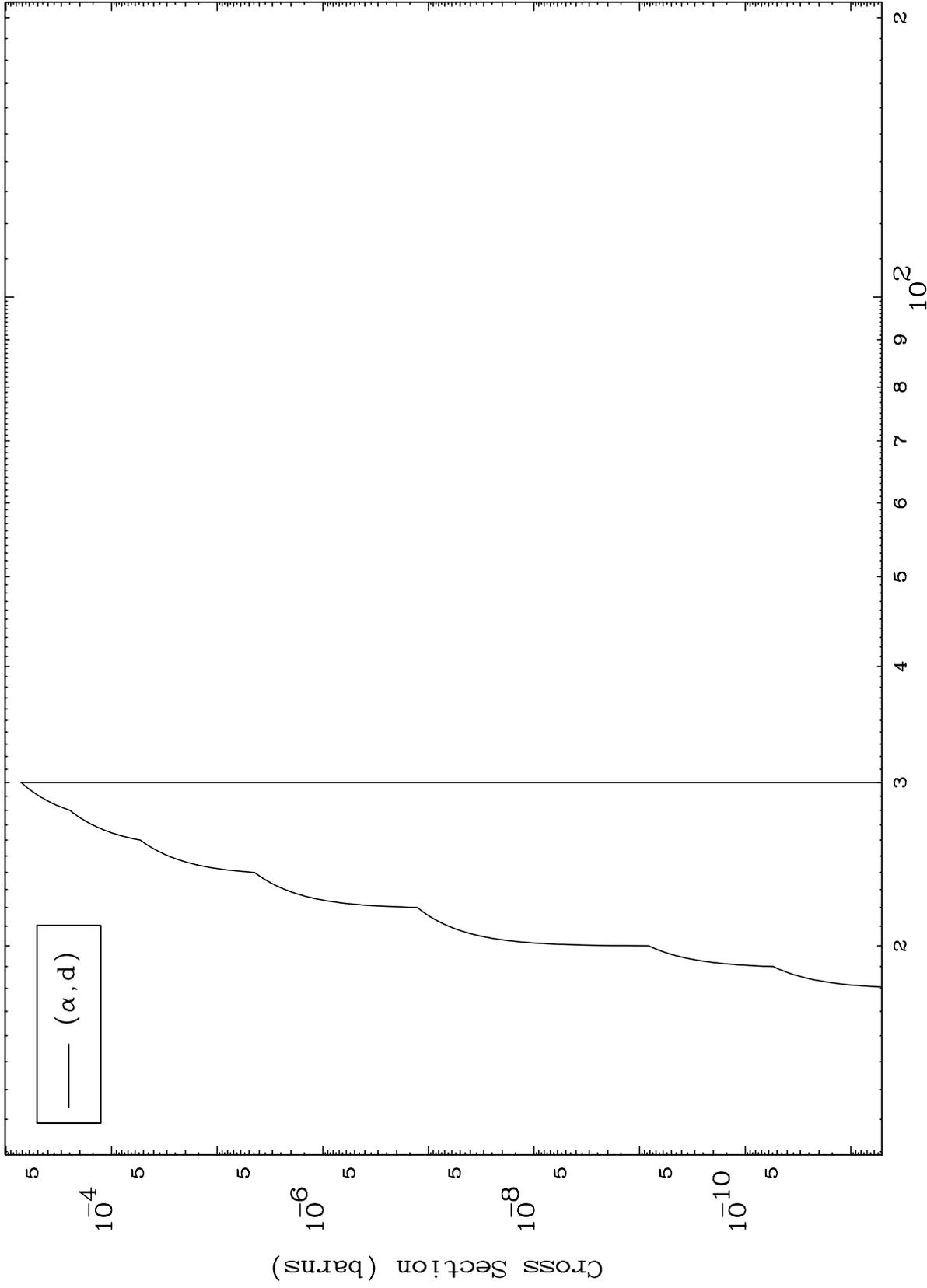
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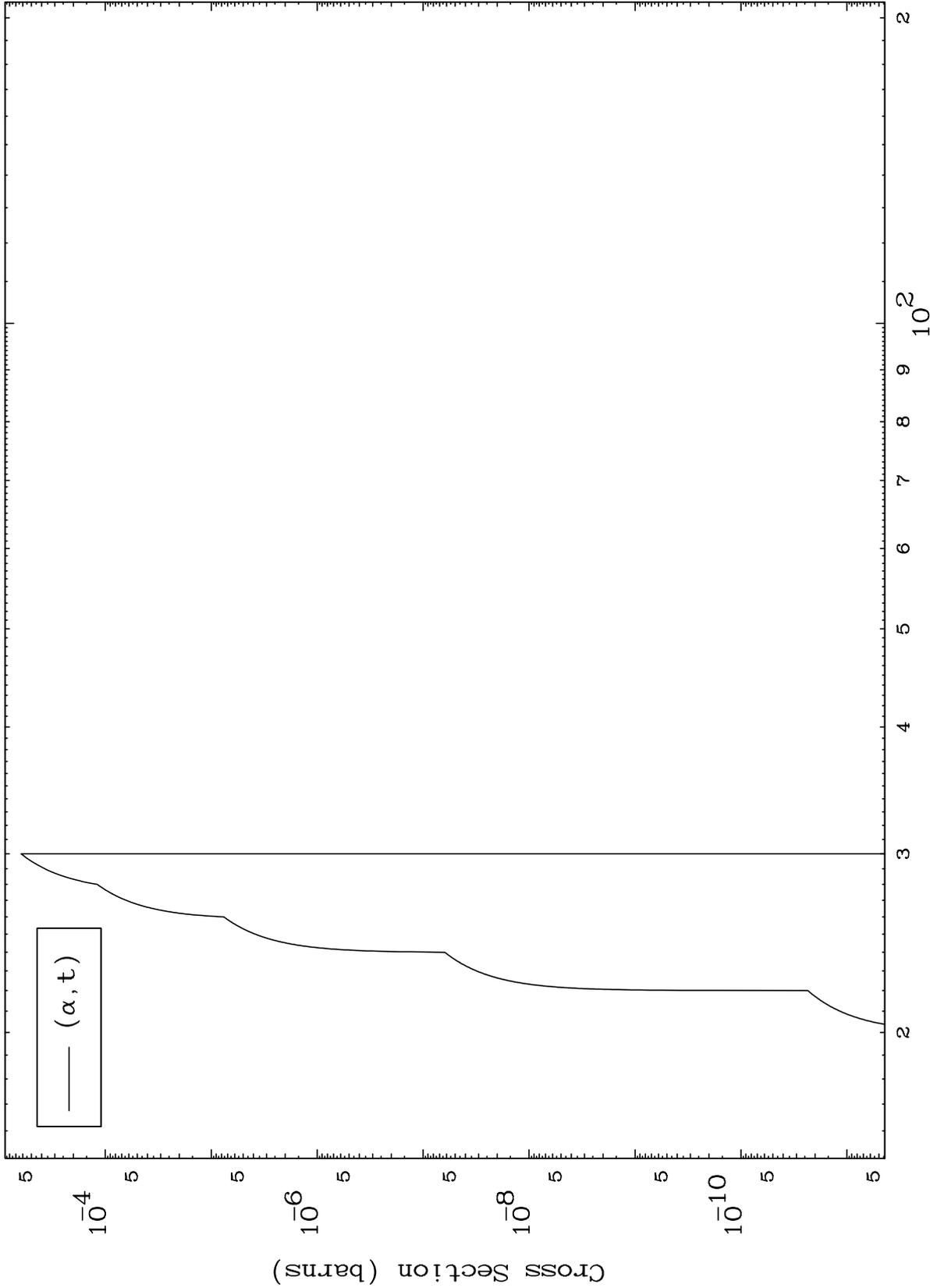


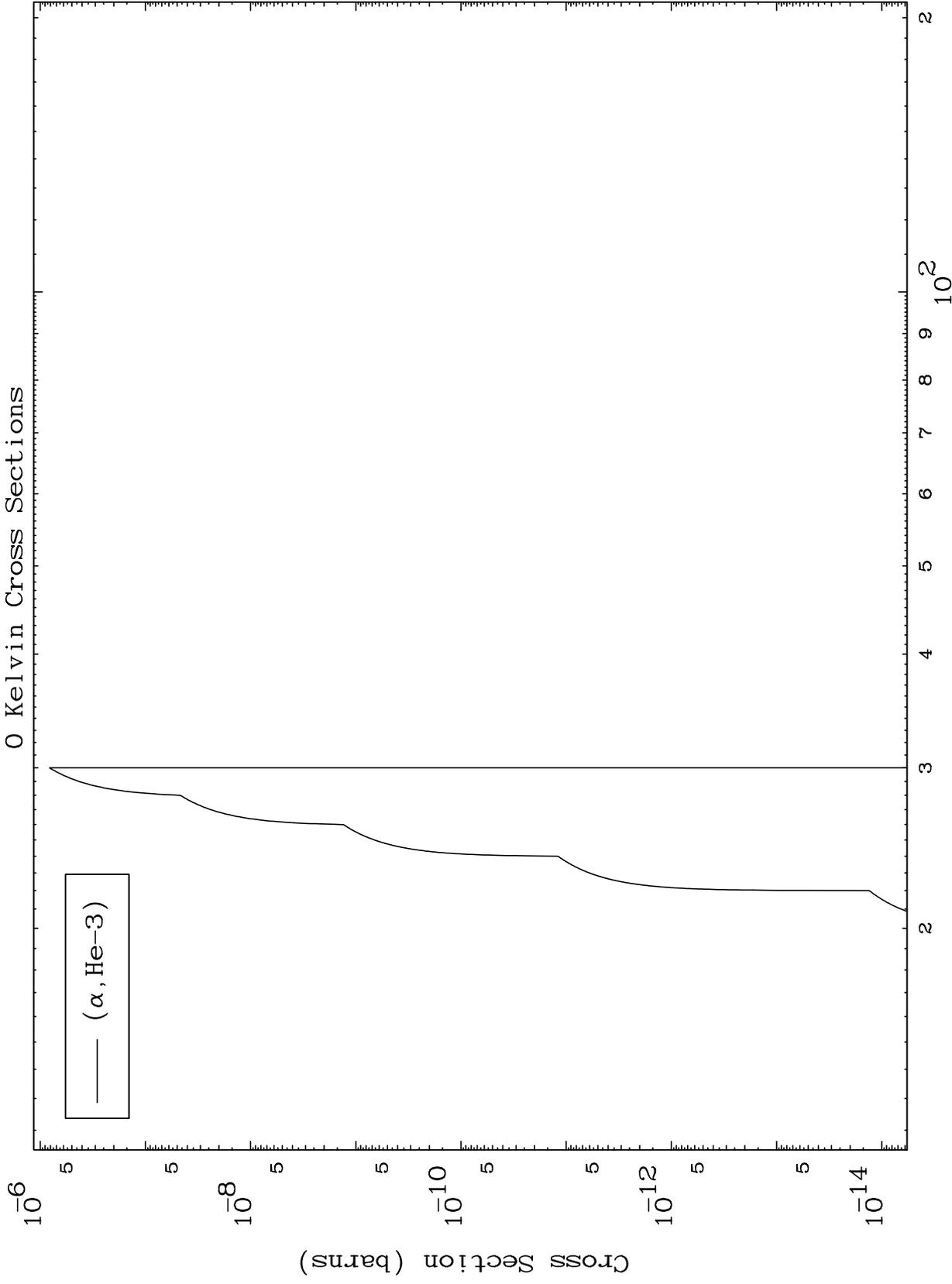
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Incident Energy (MeV)

81-T1-196



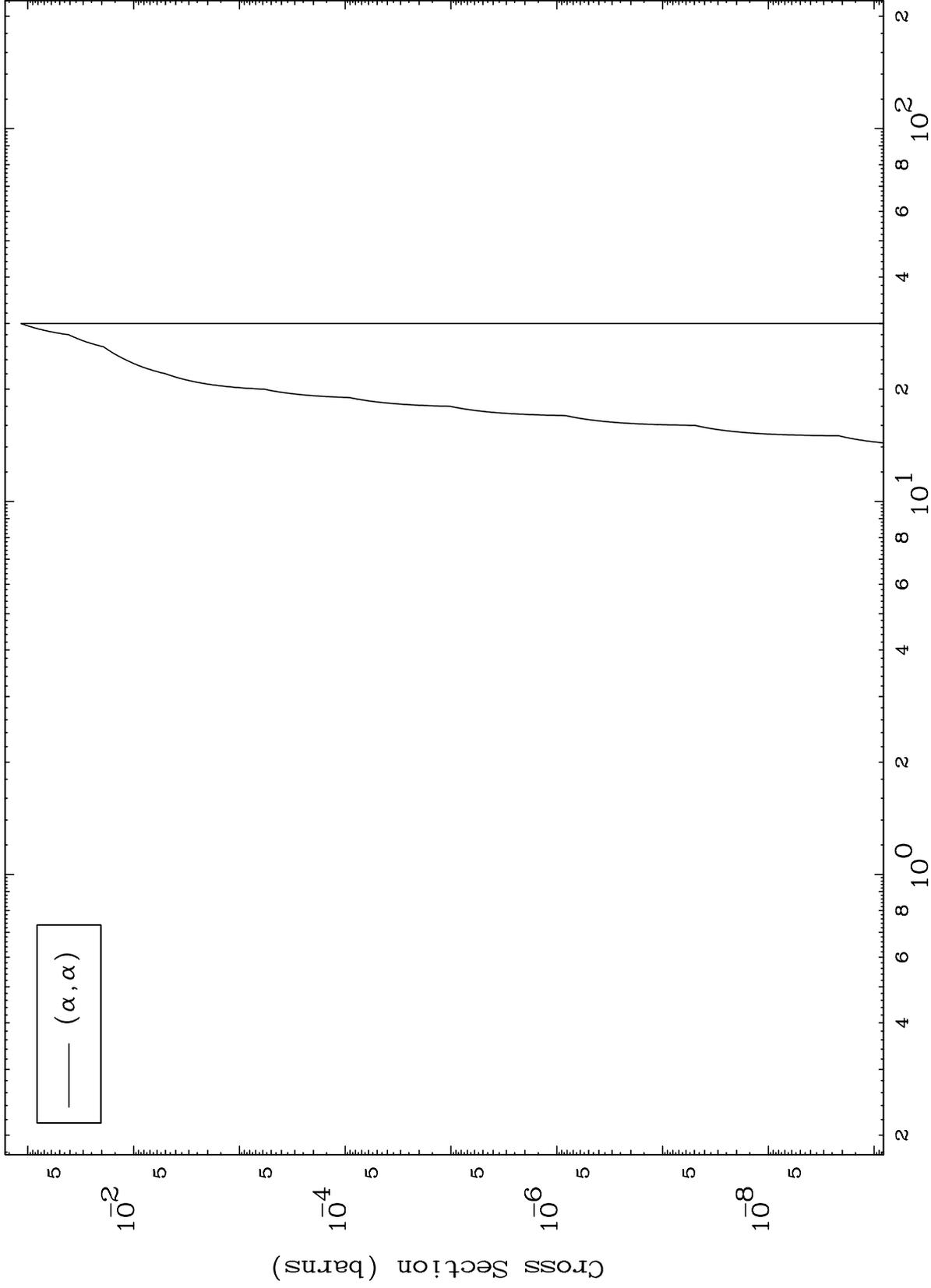




MAT 8104

(α, α) Levels
0 Kelvin Cross Sections

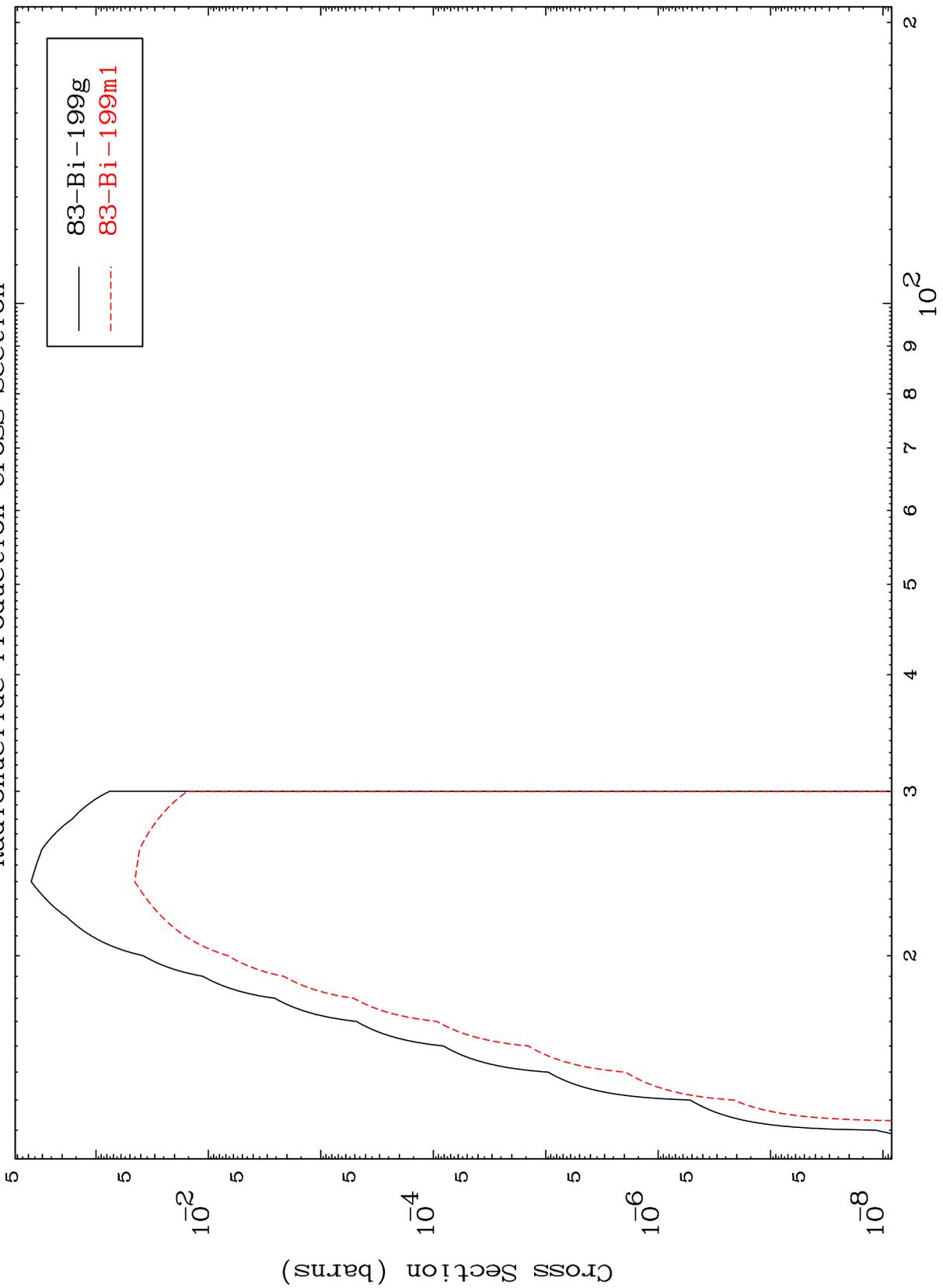
81-Tl-196



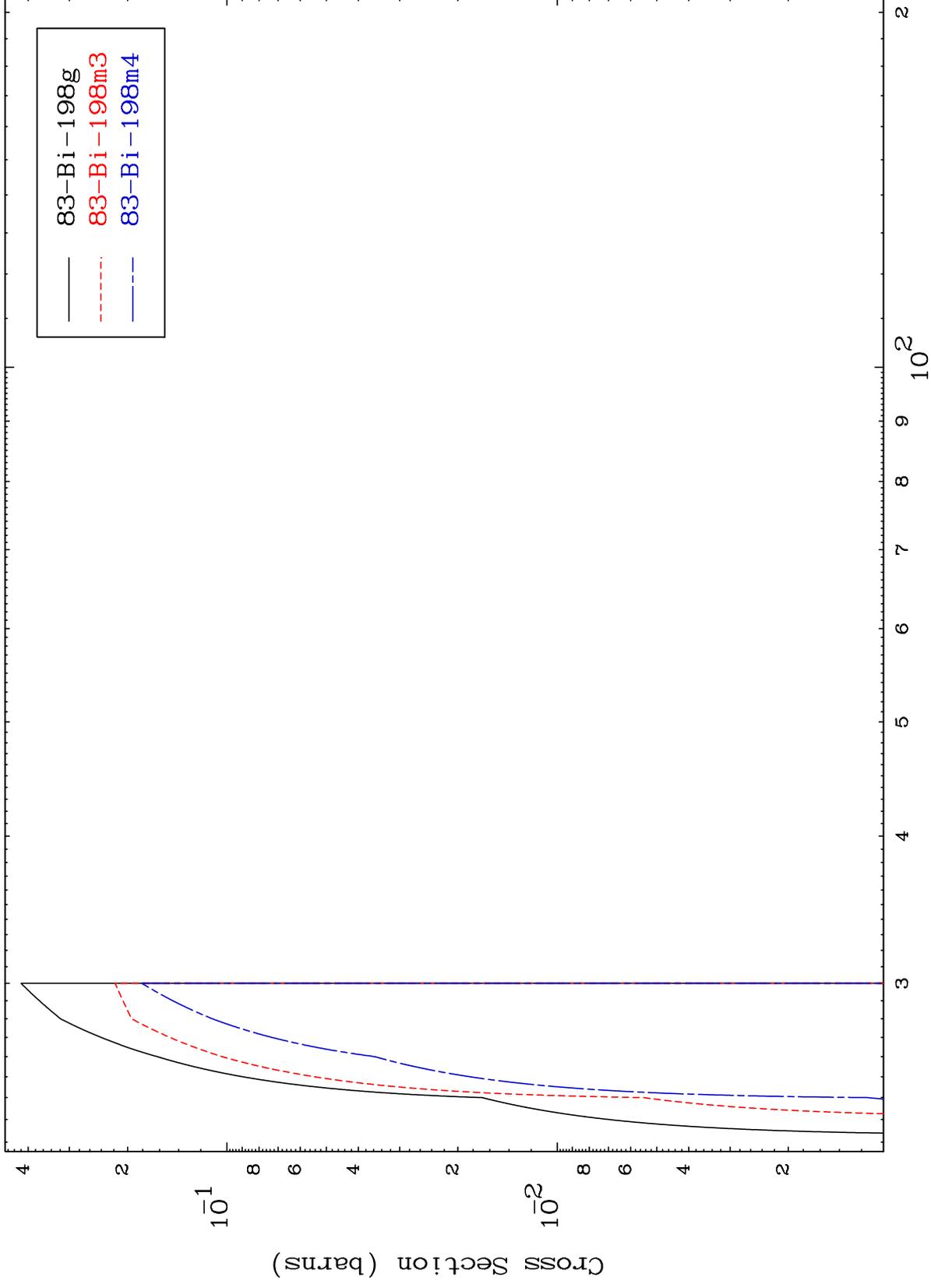
10

Incident Energy (MeV)

81-Tl-196



Radionuclide Production Cross Section

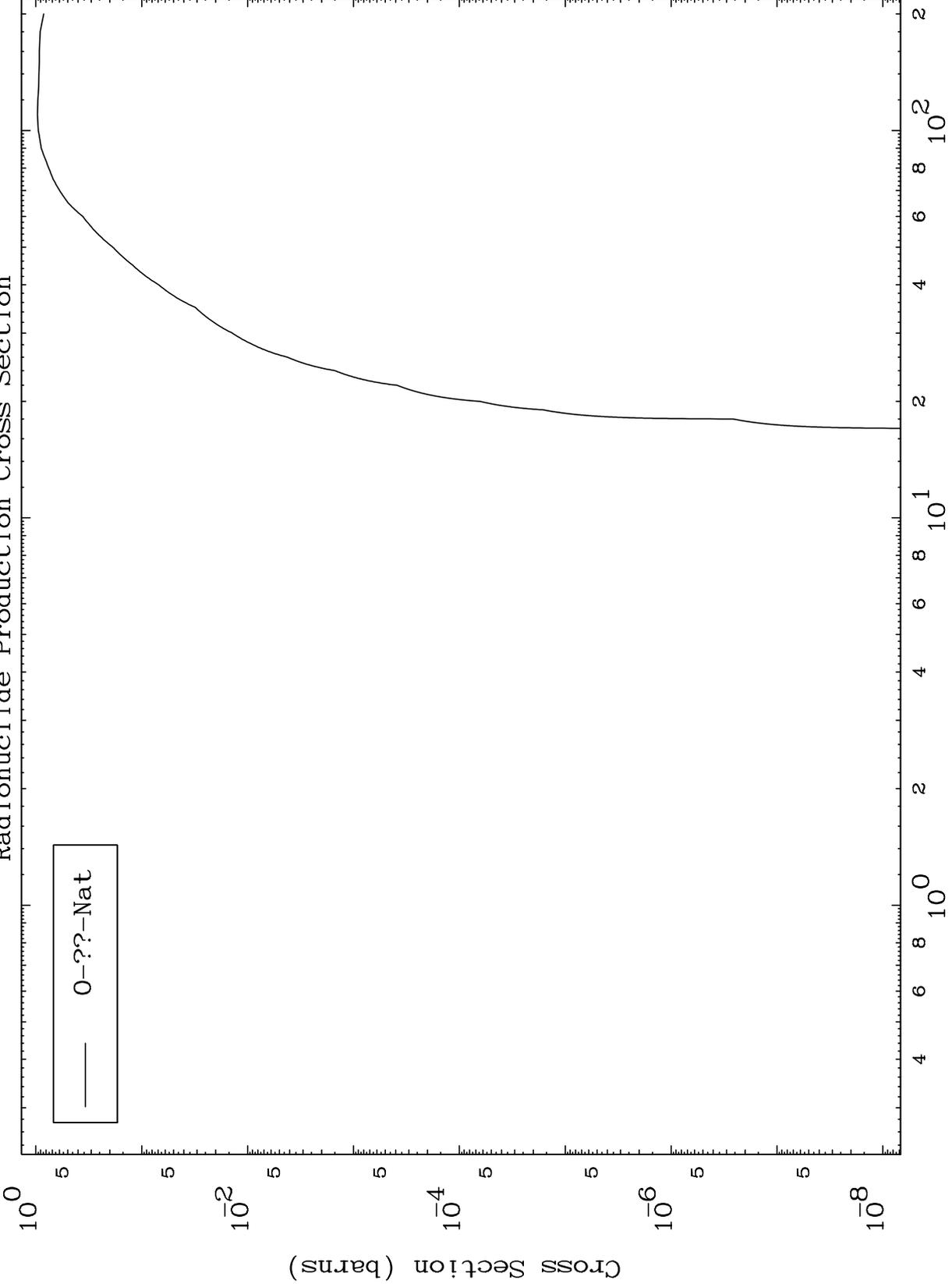


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α Fission

81-Tl-196

Radionuclide Production Cross Section



13

Incident Energy (MeV)

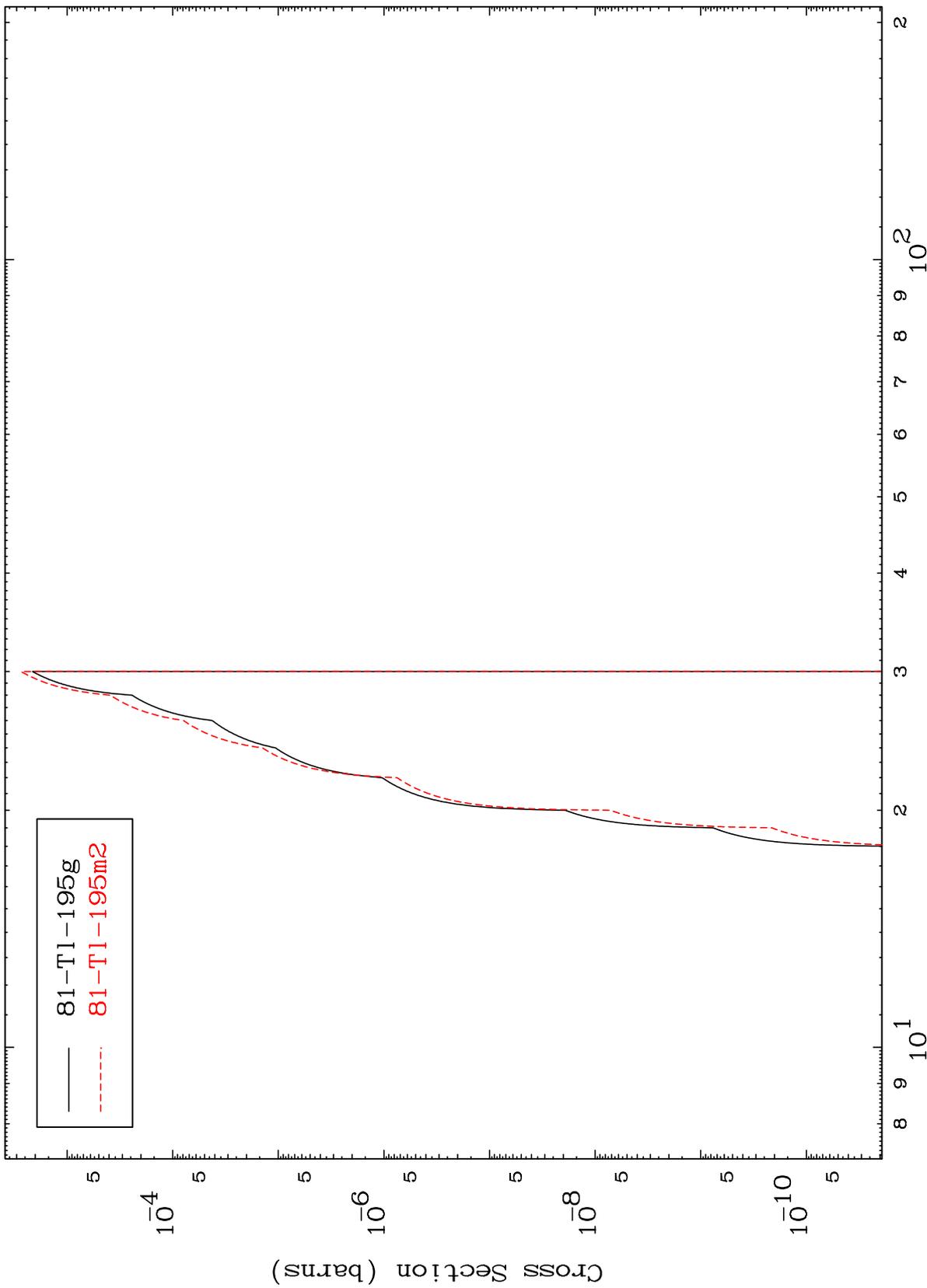
81-Tl-196

MAT 8104

81-Tl-196

(α, n') α

Radionuclide Production Cross Section



81-Tl-196

Incident Energy (MeV)

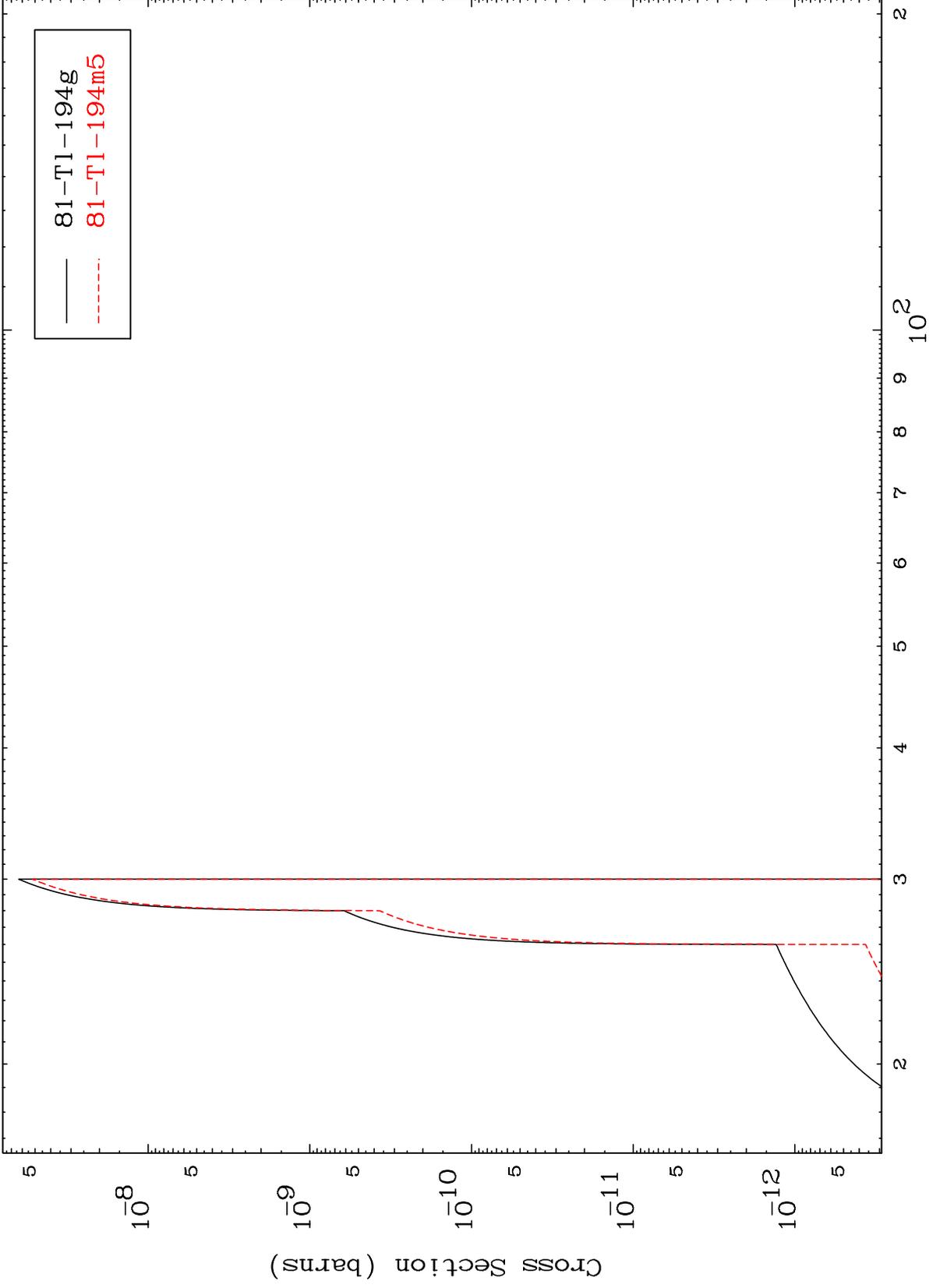
14

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$(\alpha, 2n)$ α

81-Tl-196

Radionuclide Production Cross Section

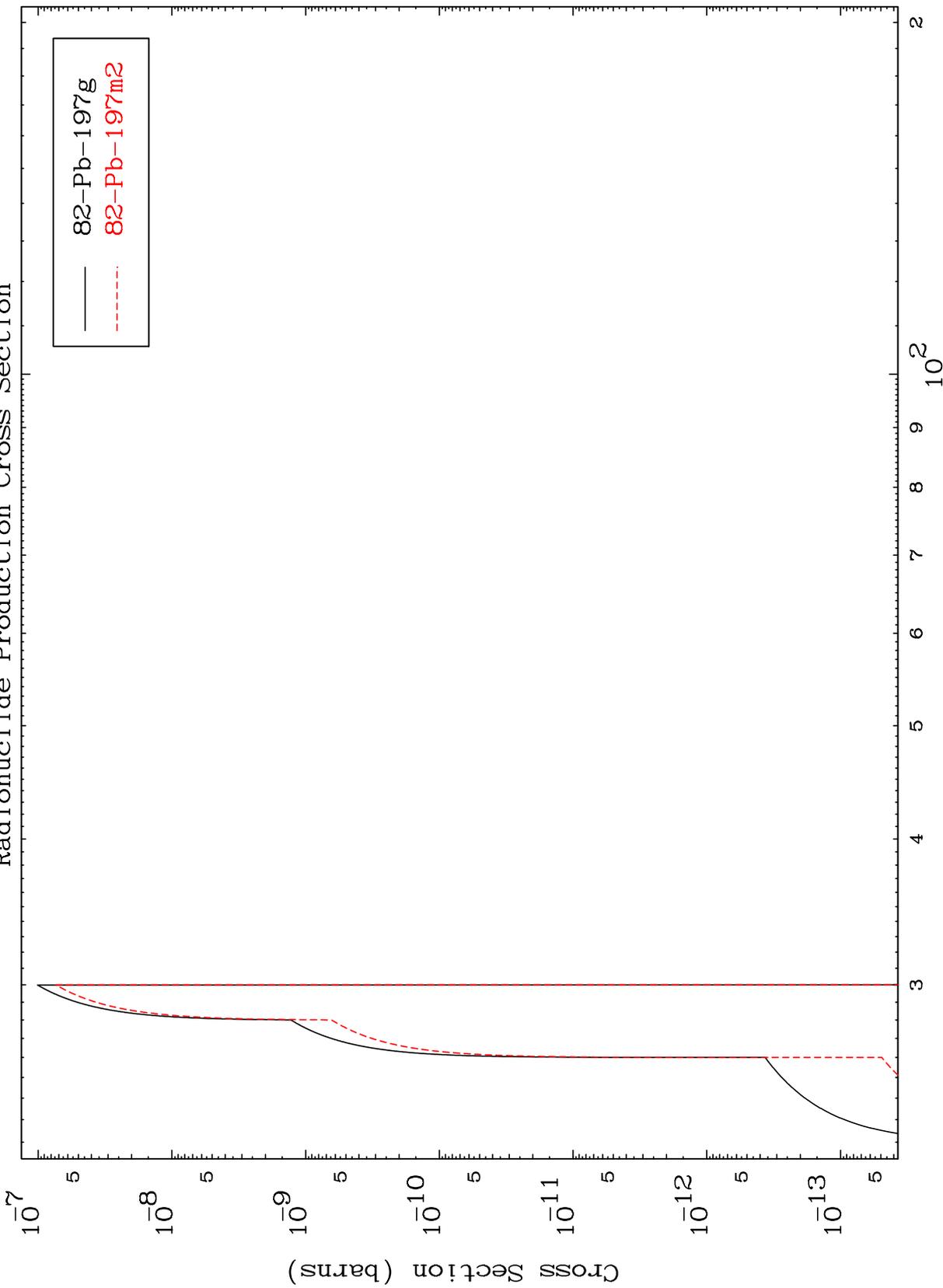


15

Incident Energy (MeV)

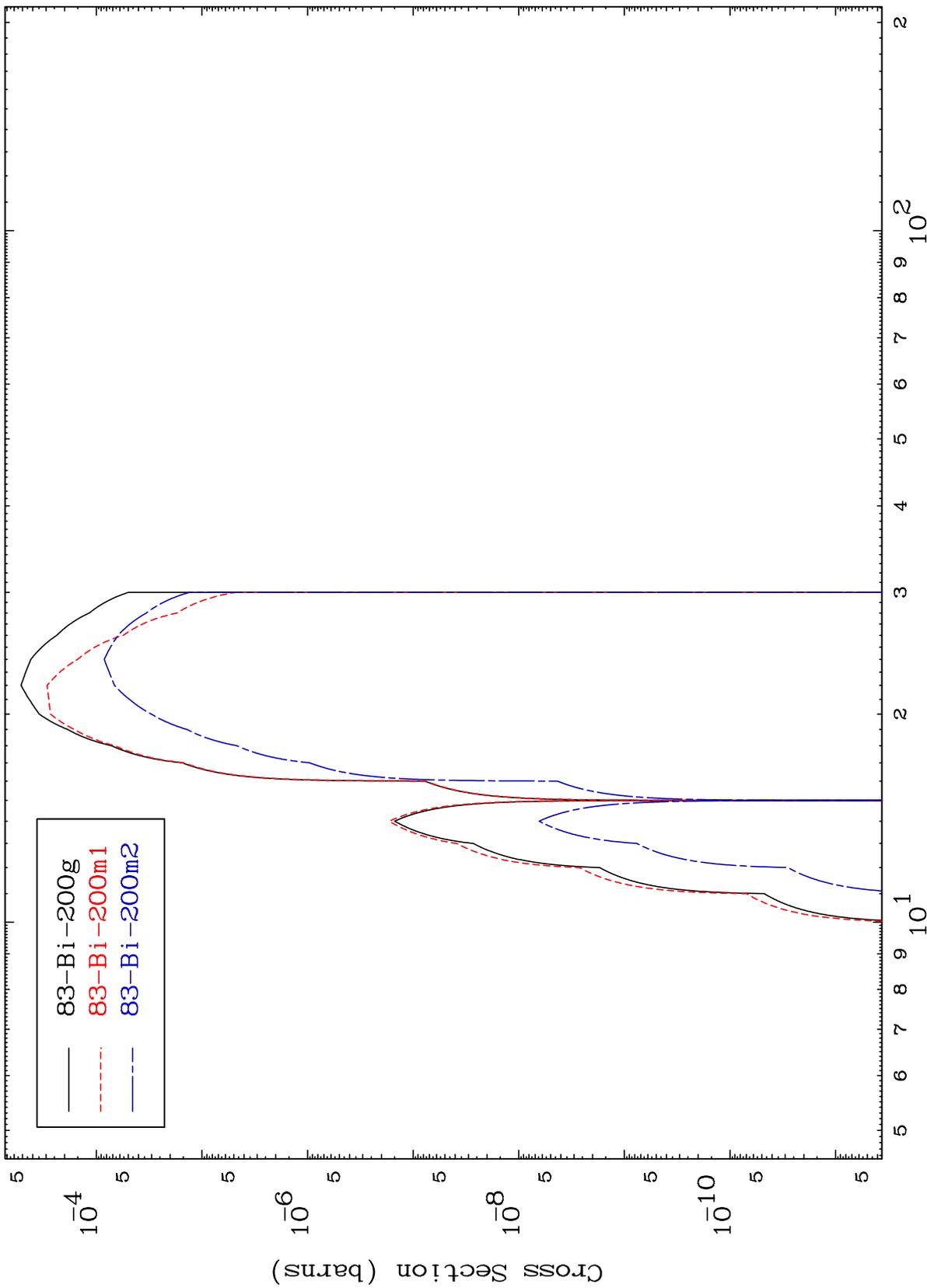
81-Tl-196

Radionuclide Production Cross Section



82-Pb-197g
82-Pb-197m2

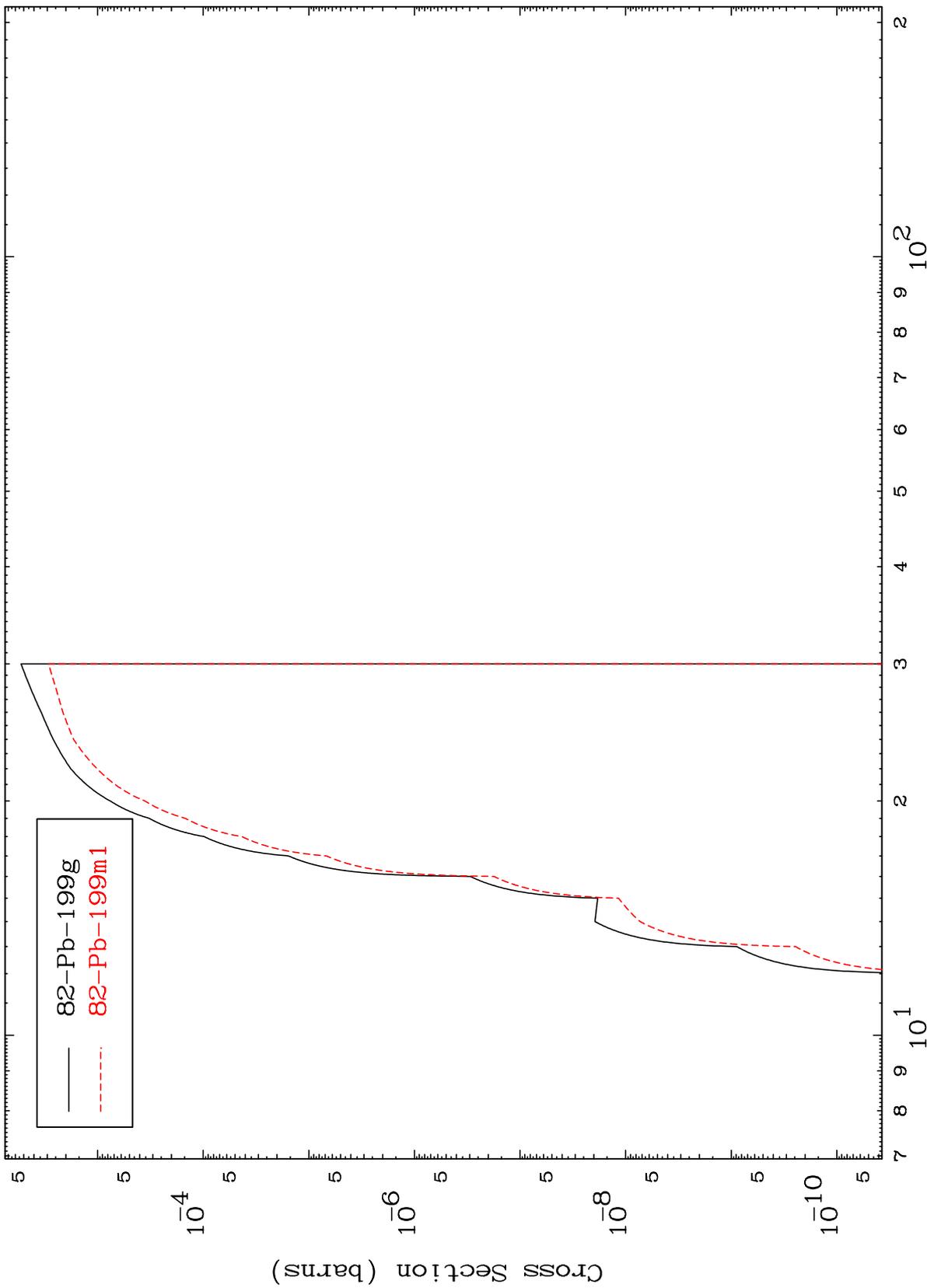
Radionuclide Production Cross Section (α, γ)



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81-T1-196

(α, p)
Radionuclide Production Cross Section

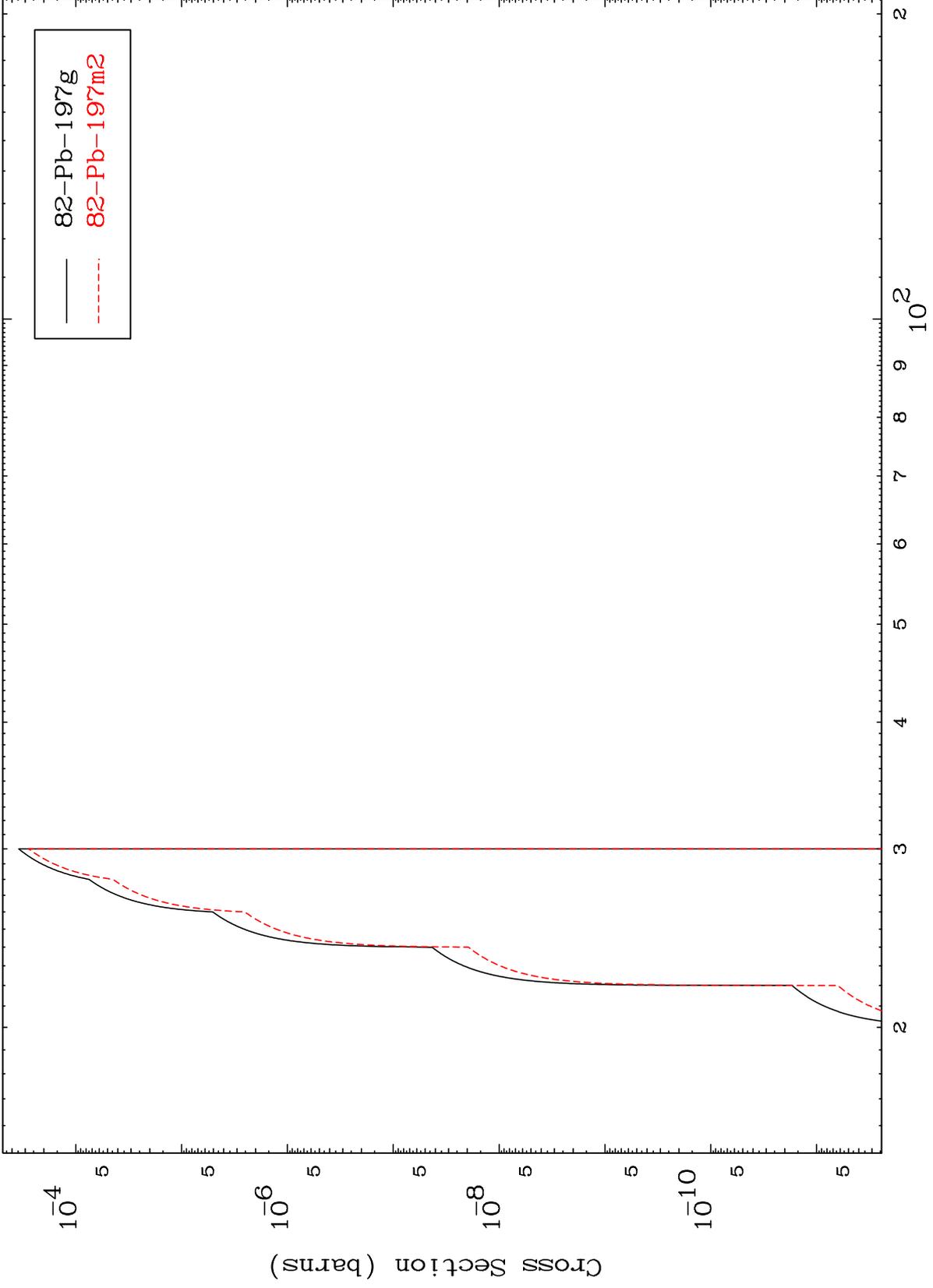


19

Incident Energy (MeV)

81-T1-196

(α, t)
Radionuclide Production Cross Section

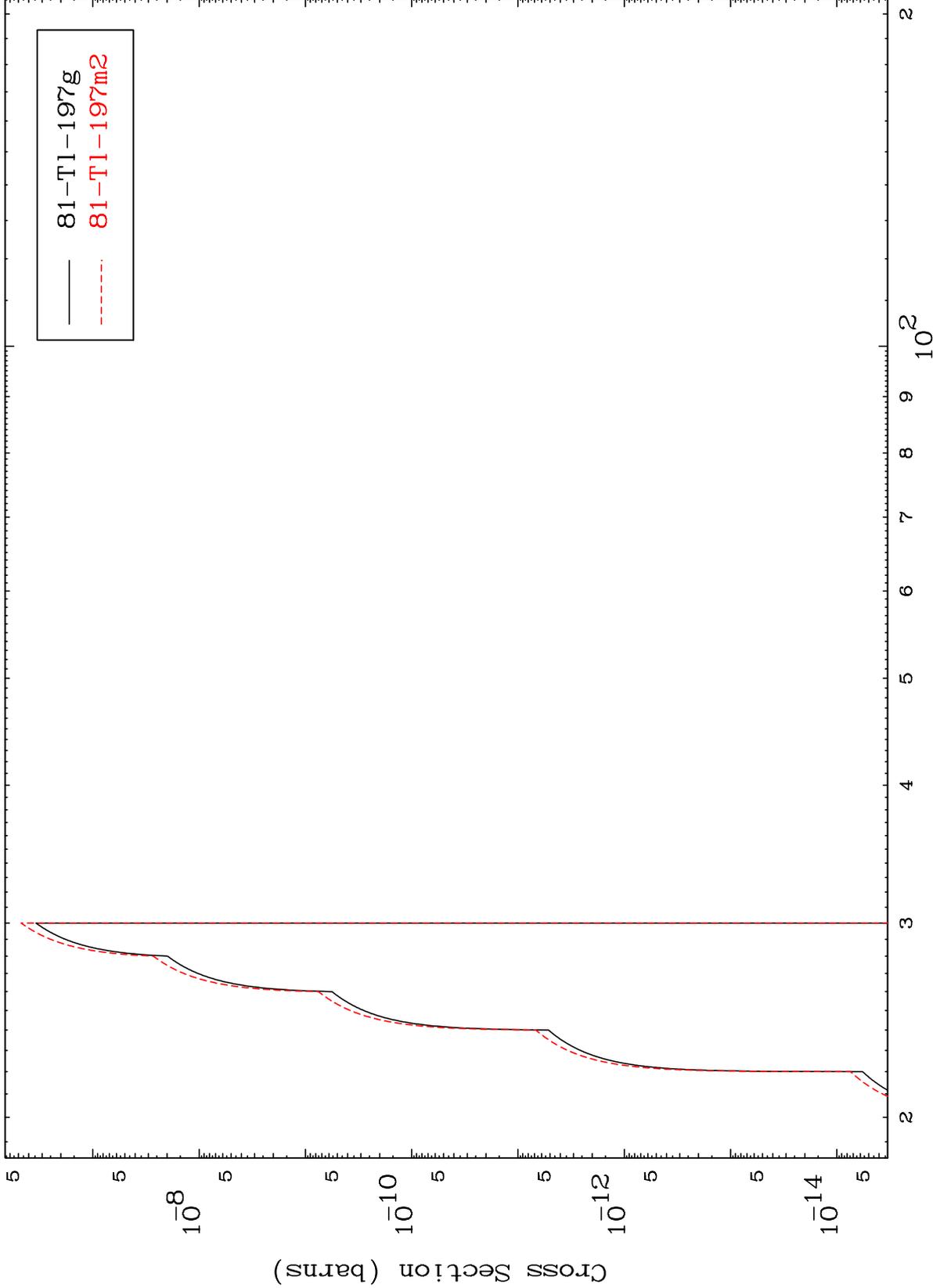


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($\alpha, \text{He-3}$)

81-Tl-196

Radionuclide Production Cross Section



21

Incident Energy (MeV)

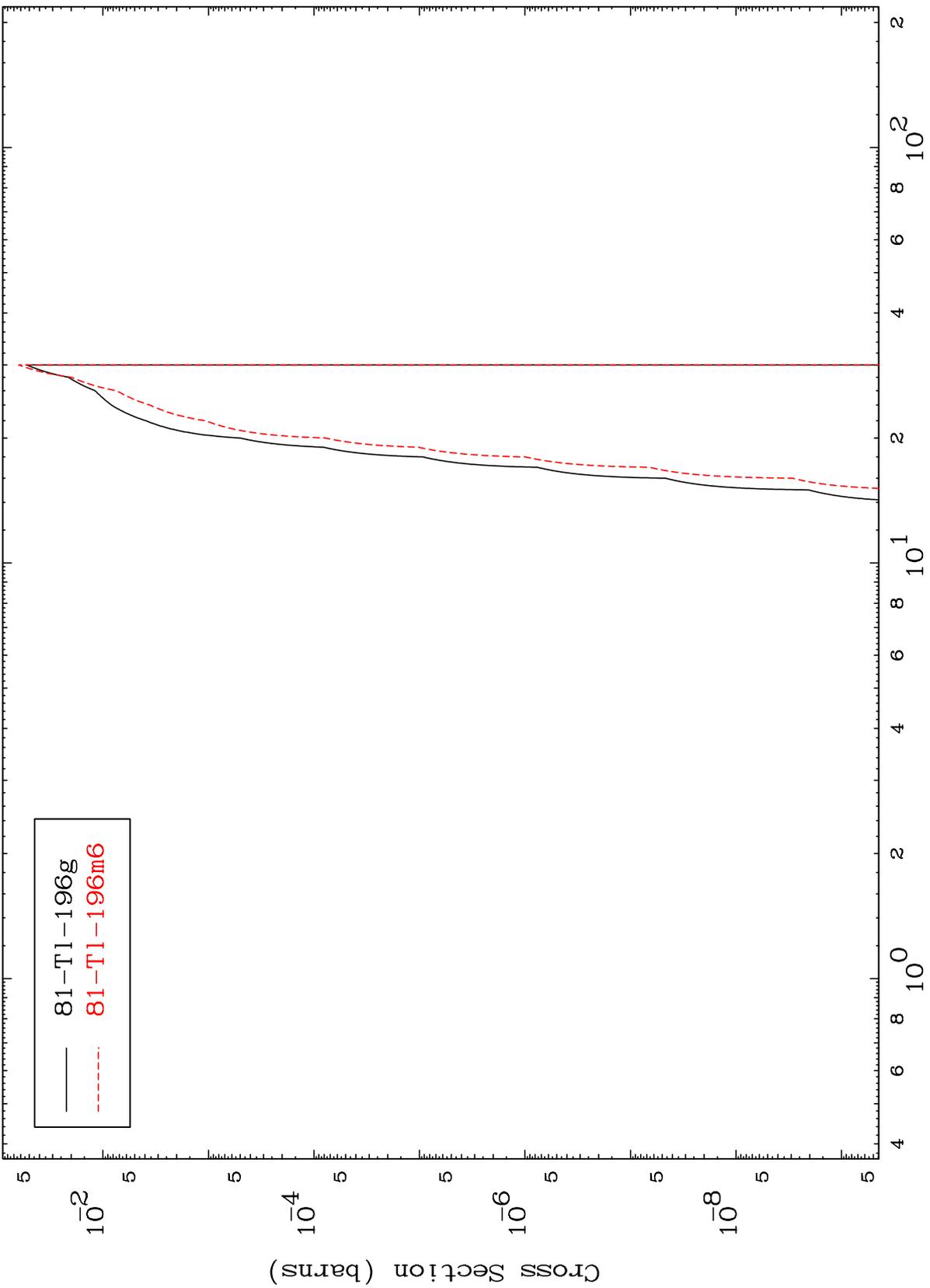
81-Tl-196

MAT 8104

(α, α)

81-Tl-196

Radionuclide Production Cross Section



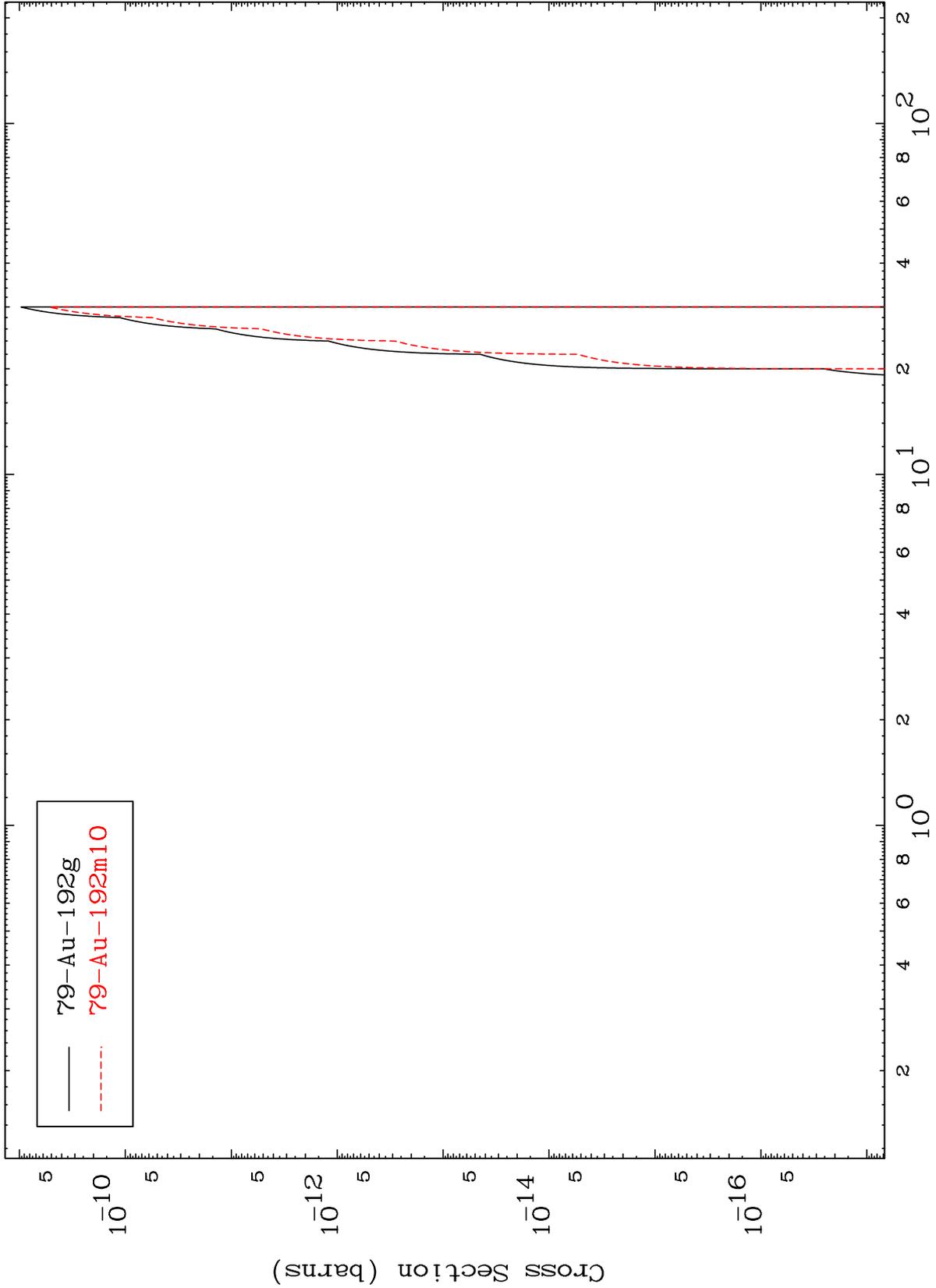
81-Tl-196g
81-Tl-196m6

MAT 8104

($\alpha, 2\alpha$)

81-Tl-196

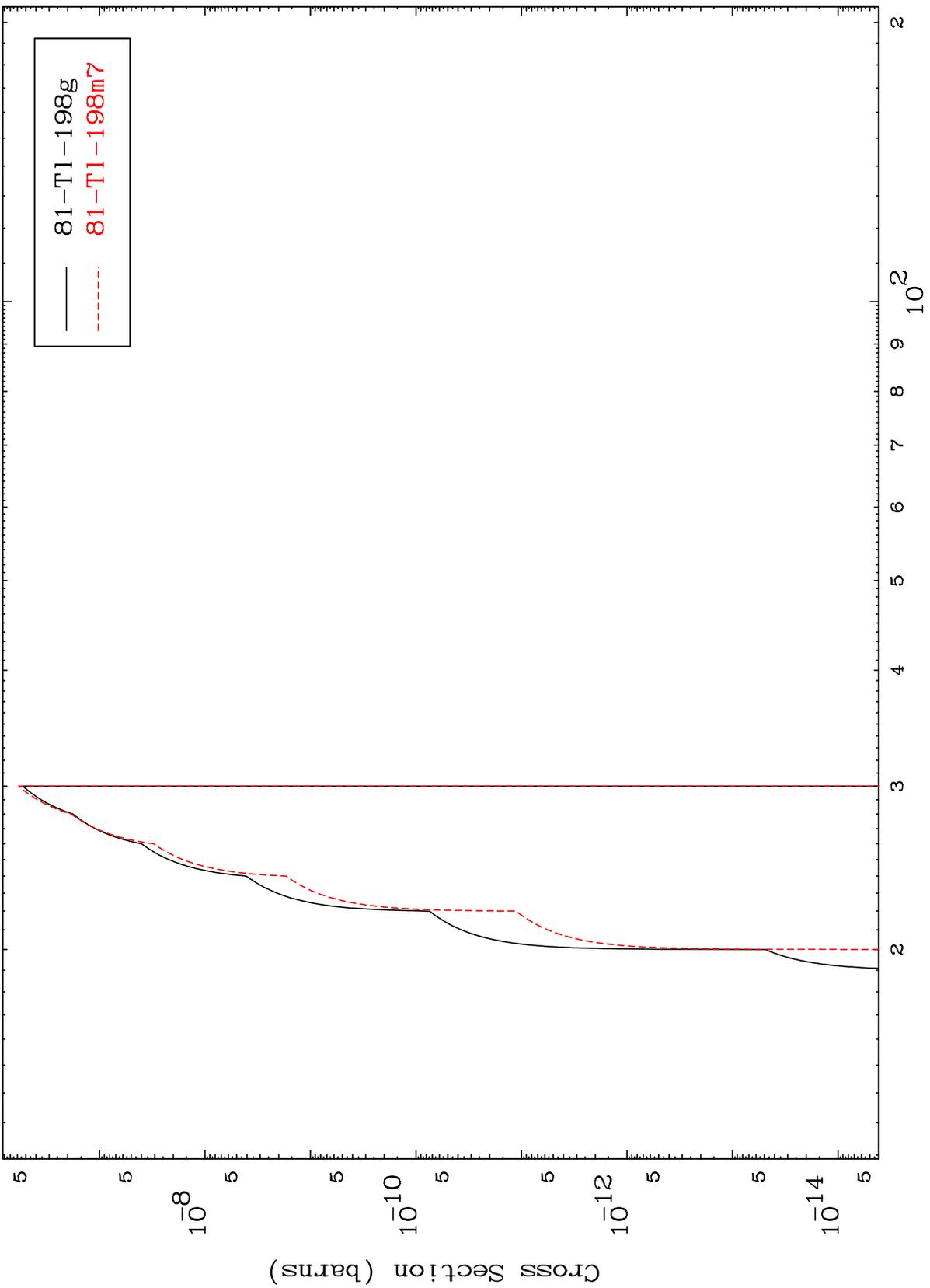
Radionuclide Production Cross Section



MAT 8104

81-Tl-196

($\alpha, 2p$)
Radionuclide Production Cross Section



24

Incident Energy (MeV)

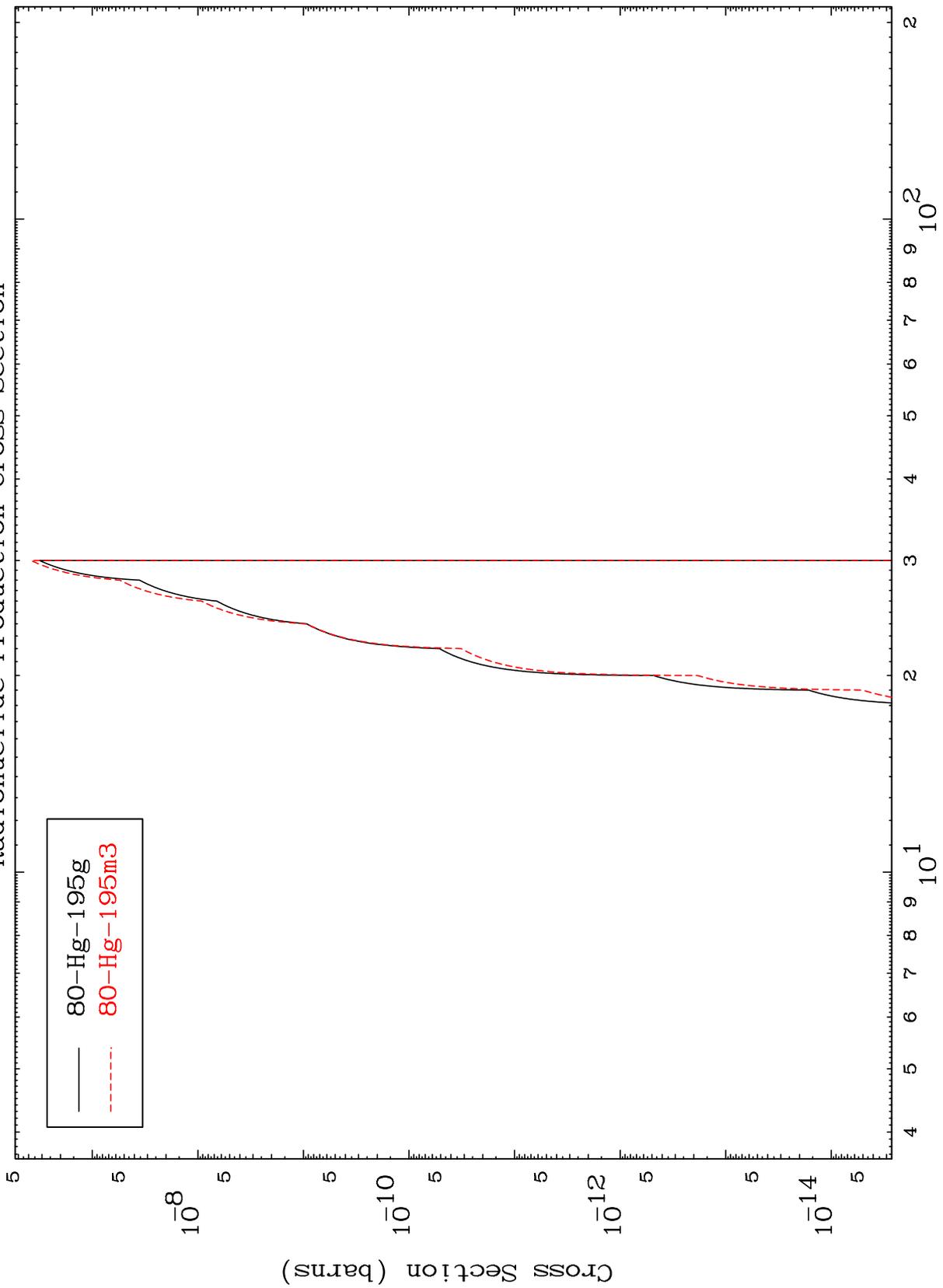
81-Tl-196

MAT 8104

(α, p) α

81-T1-196

Radionuclide Production Cross Section



25

Incident Energy (MeV)

81-T1-196