

Program EVALPLOT  
(Version 2018-1)

by

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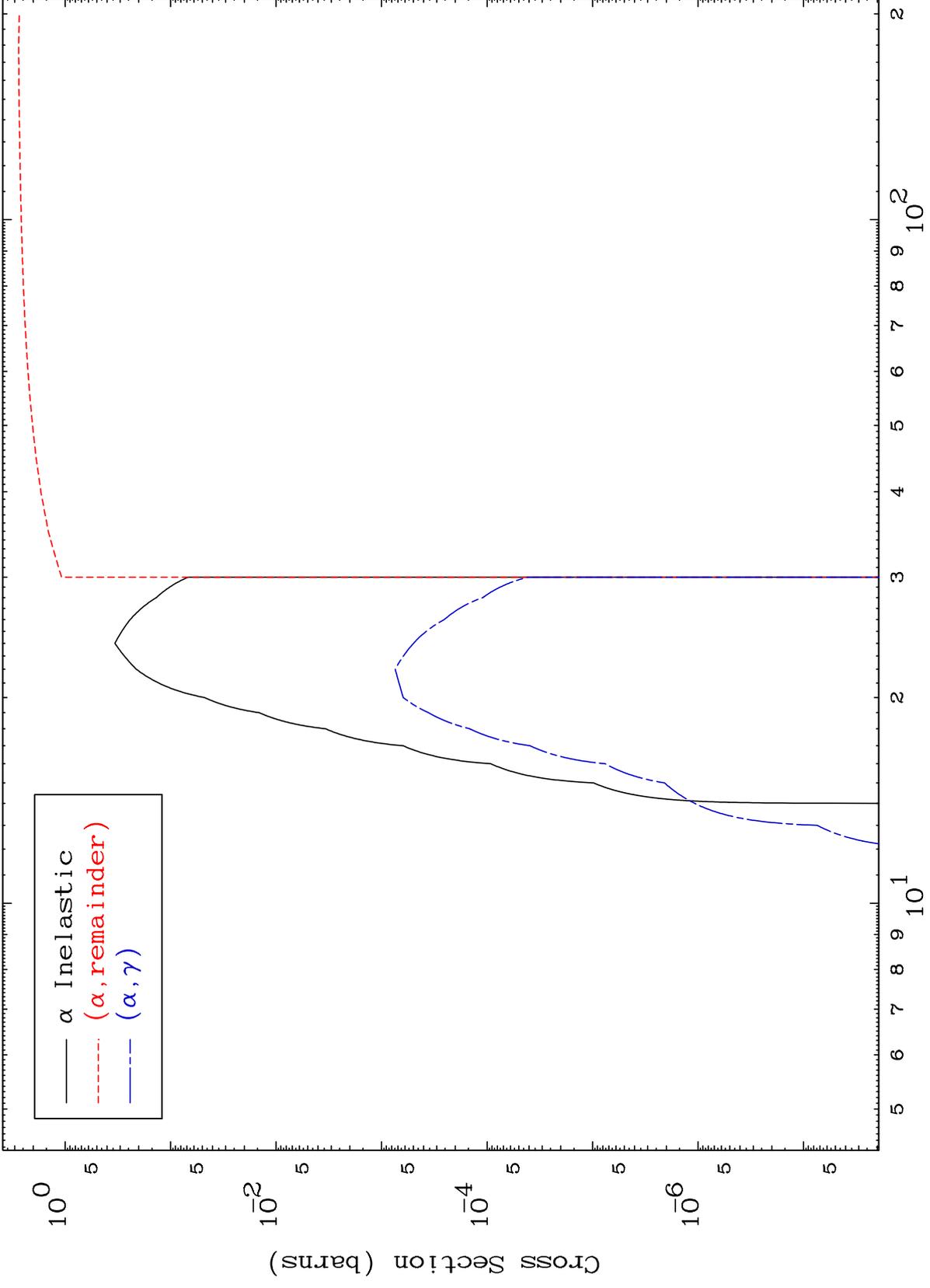
Press Mouse Button to Start

MAT 8107

$\alpha$  Major

81-T1-197

0 Kelvin Cross Sections



$\alpha$  Inelastic  
 $(\alpha, \text{remainder})$   
 $(\alpha, \gamma)$

1

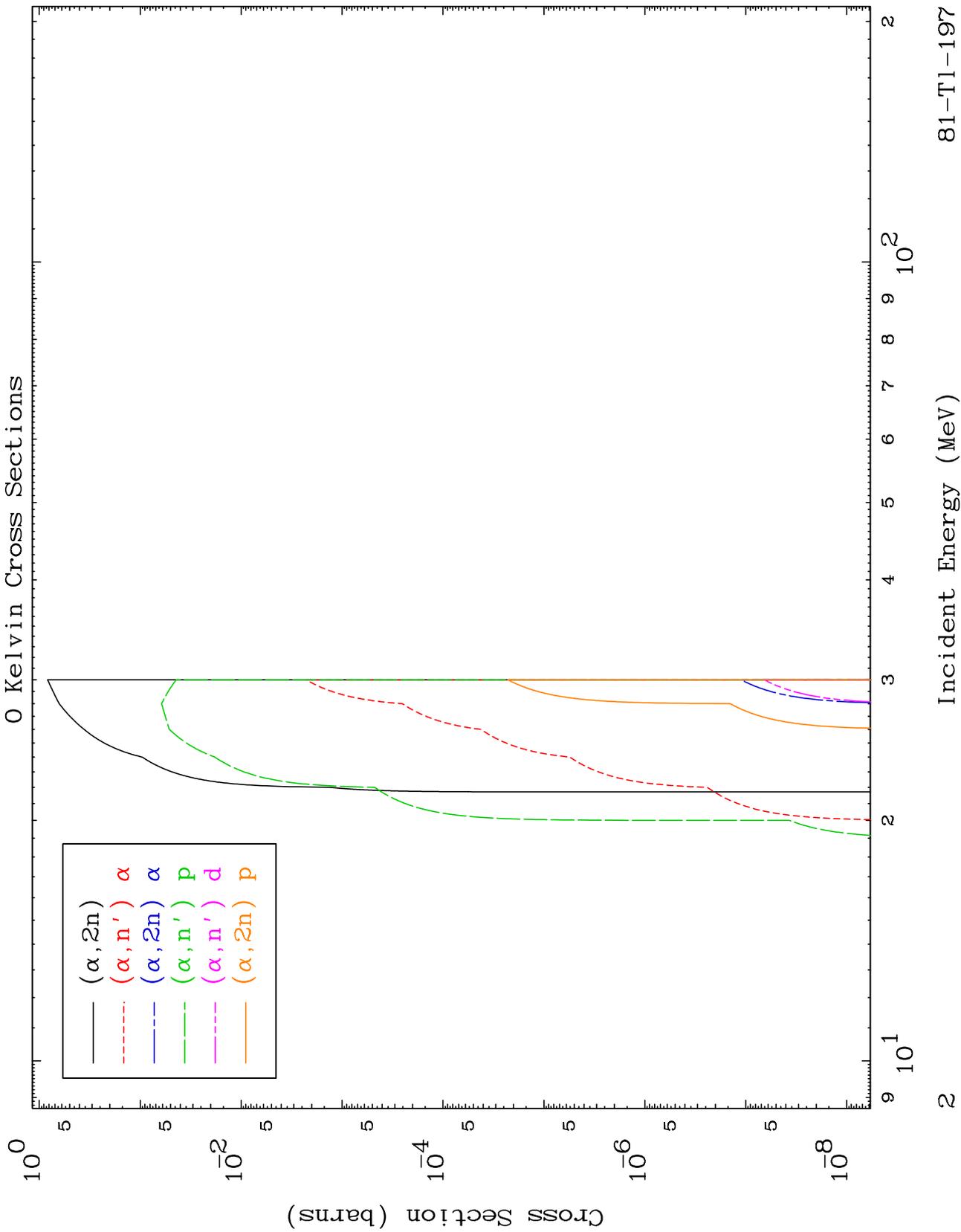
Incident Energy (MeV)

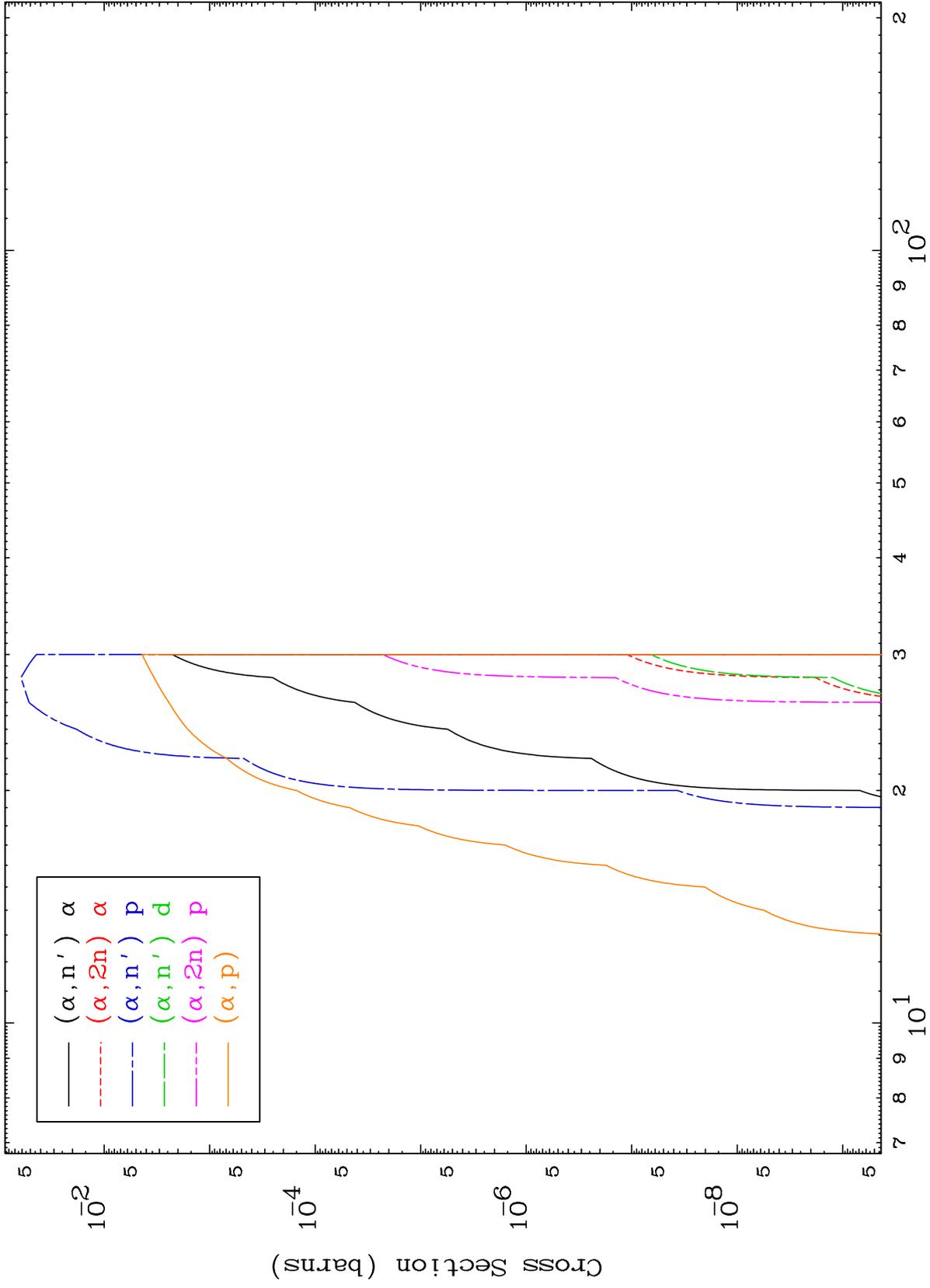
81-T1-197

MAT 8107

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

81-T1-197

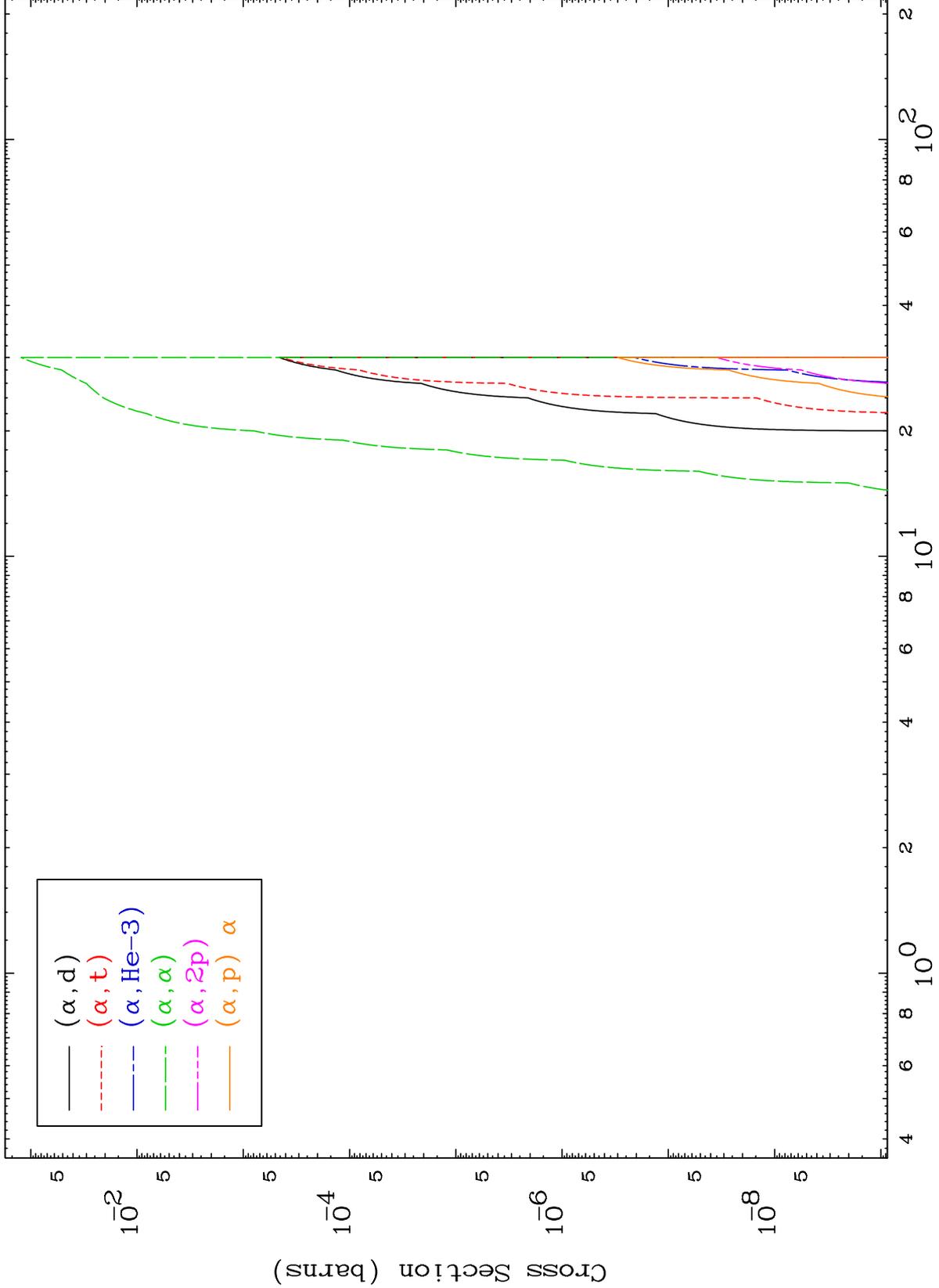




MAT 8107

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

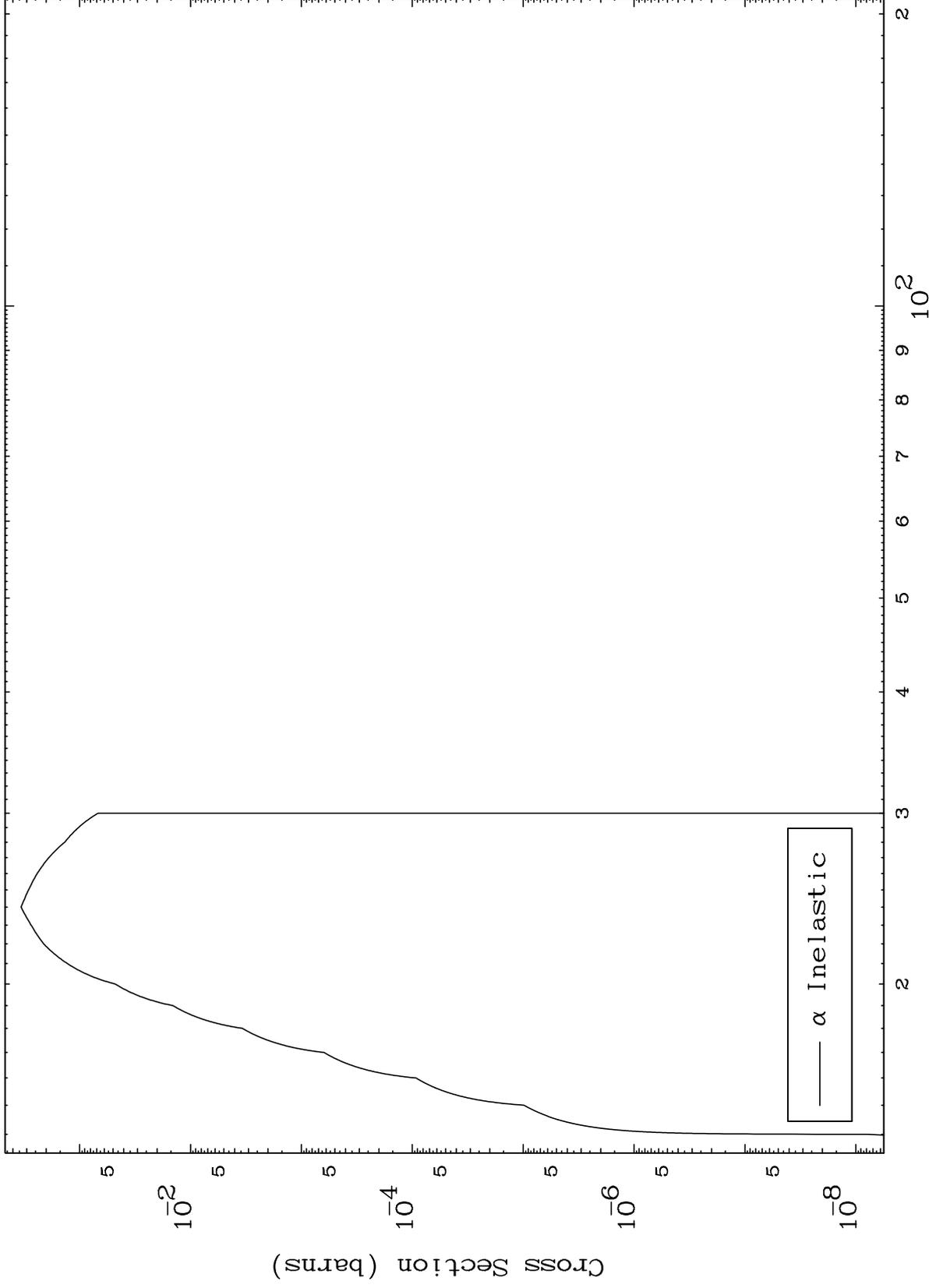
81-Tl-197



4

Incident Energy (MeV)

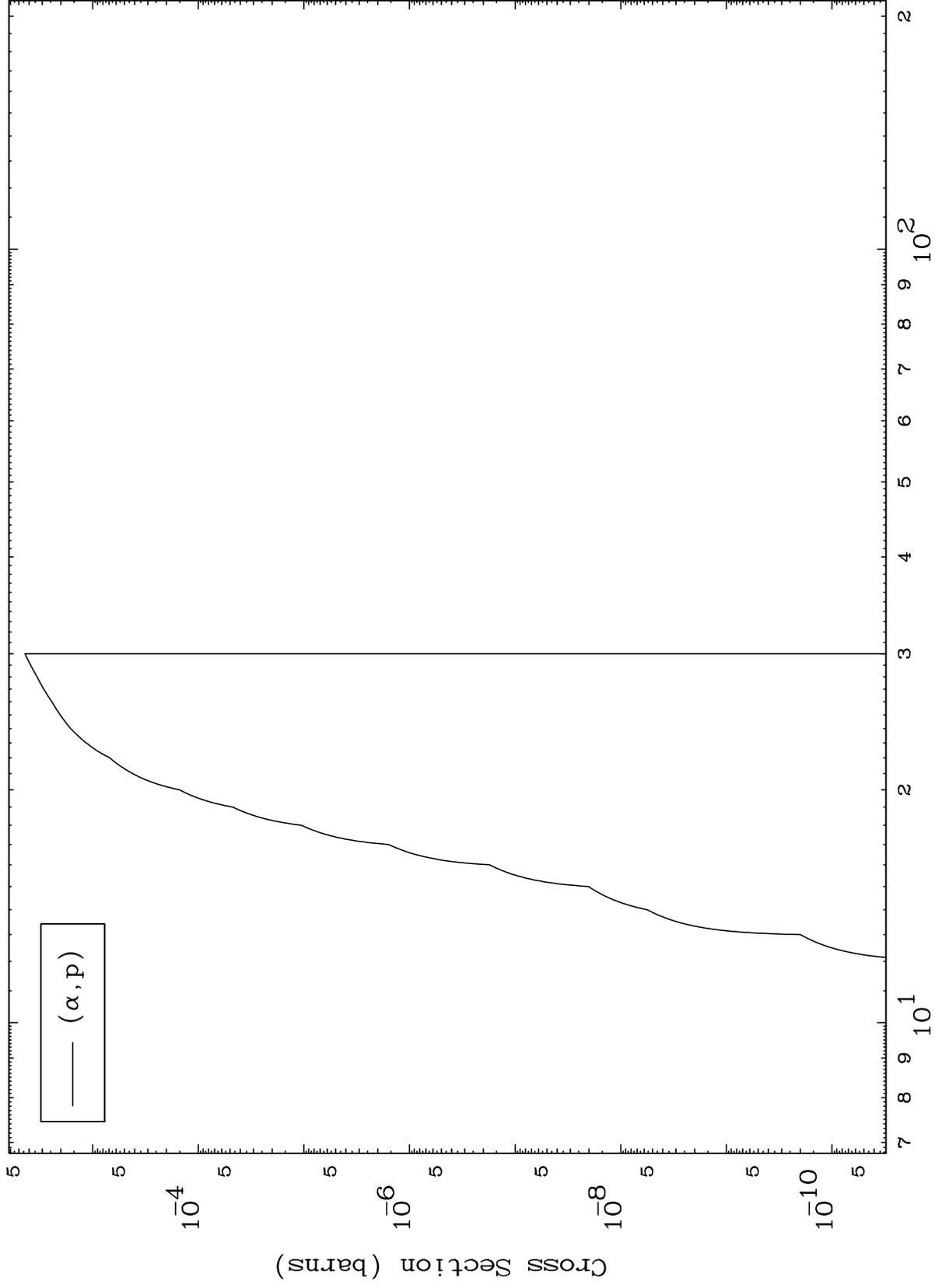
81-Tl-197



MAT 8107

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

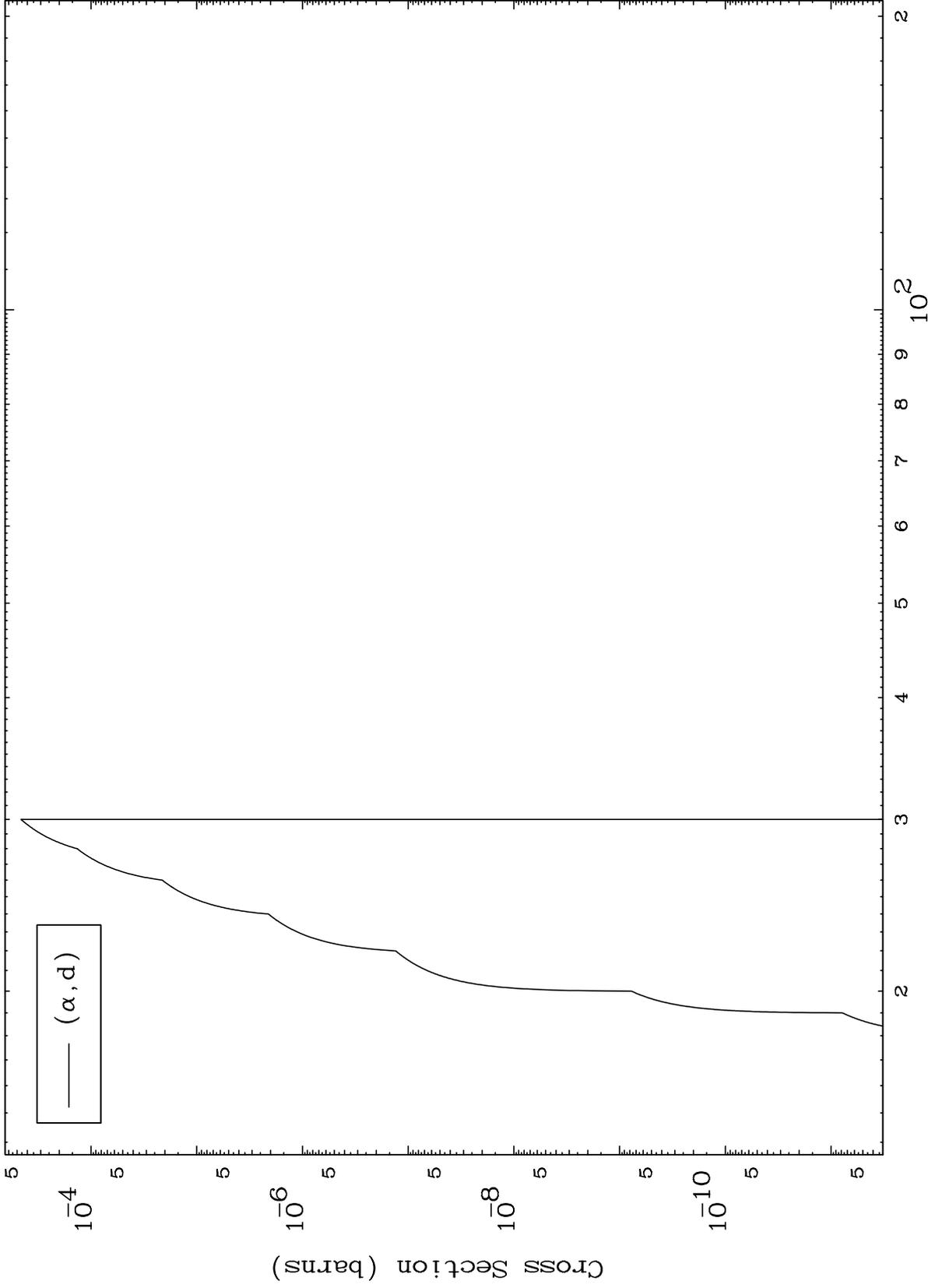
81-T1-197

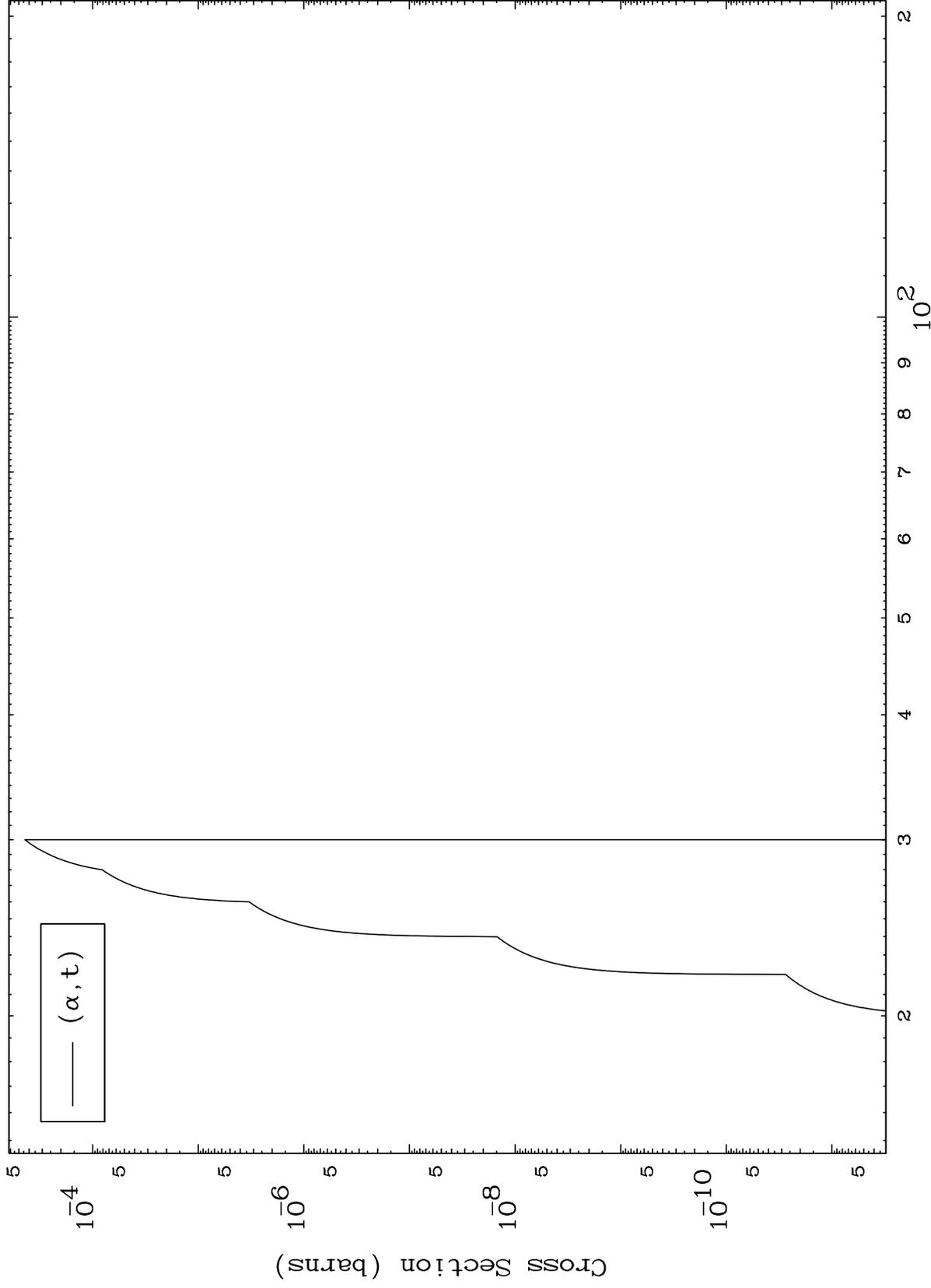


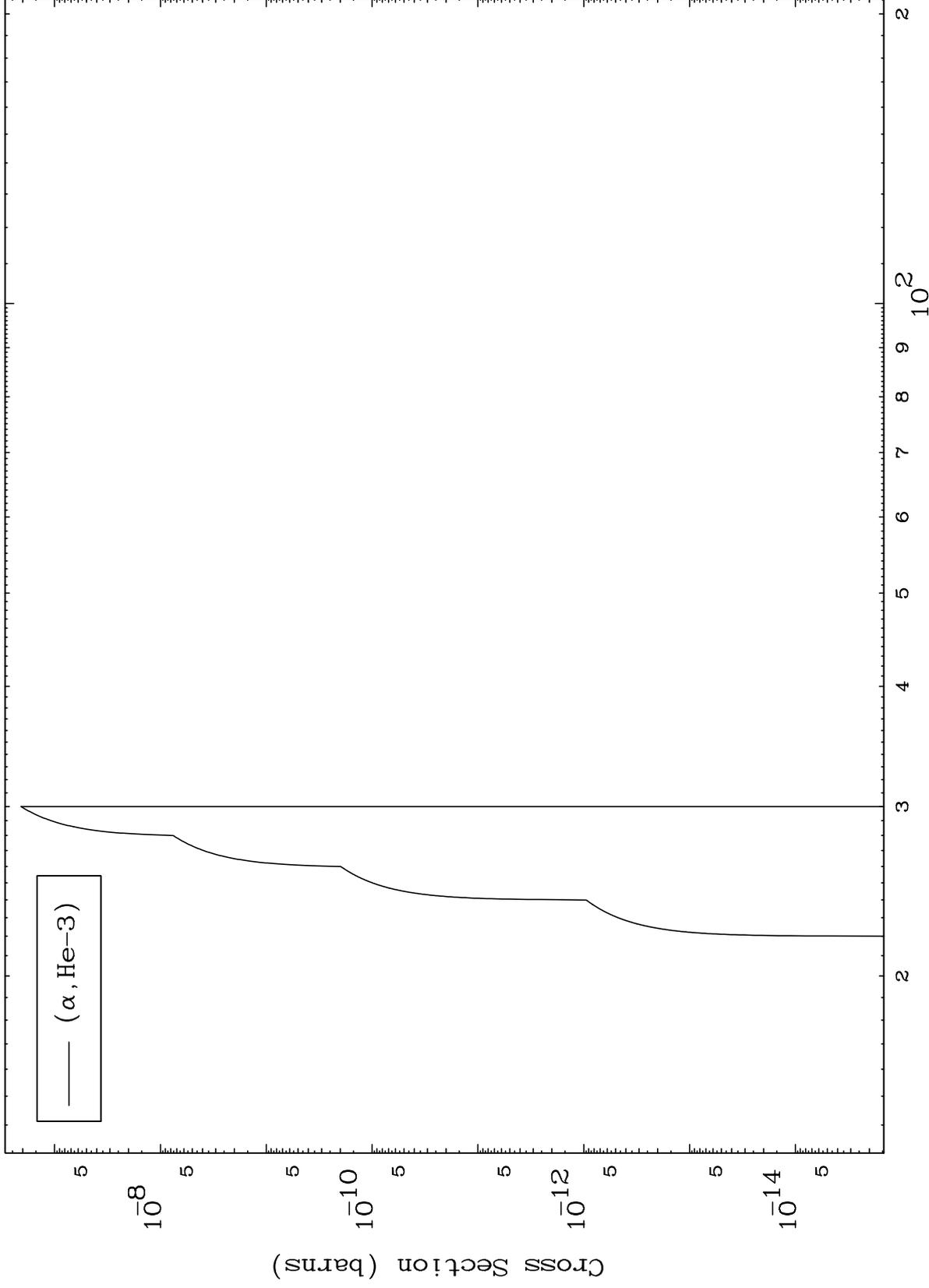
6

Incident Energy (MeV)

81-T1-197





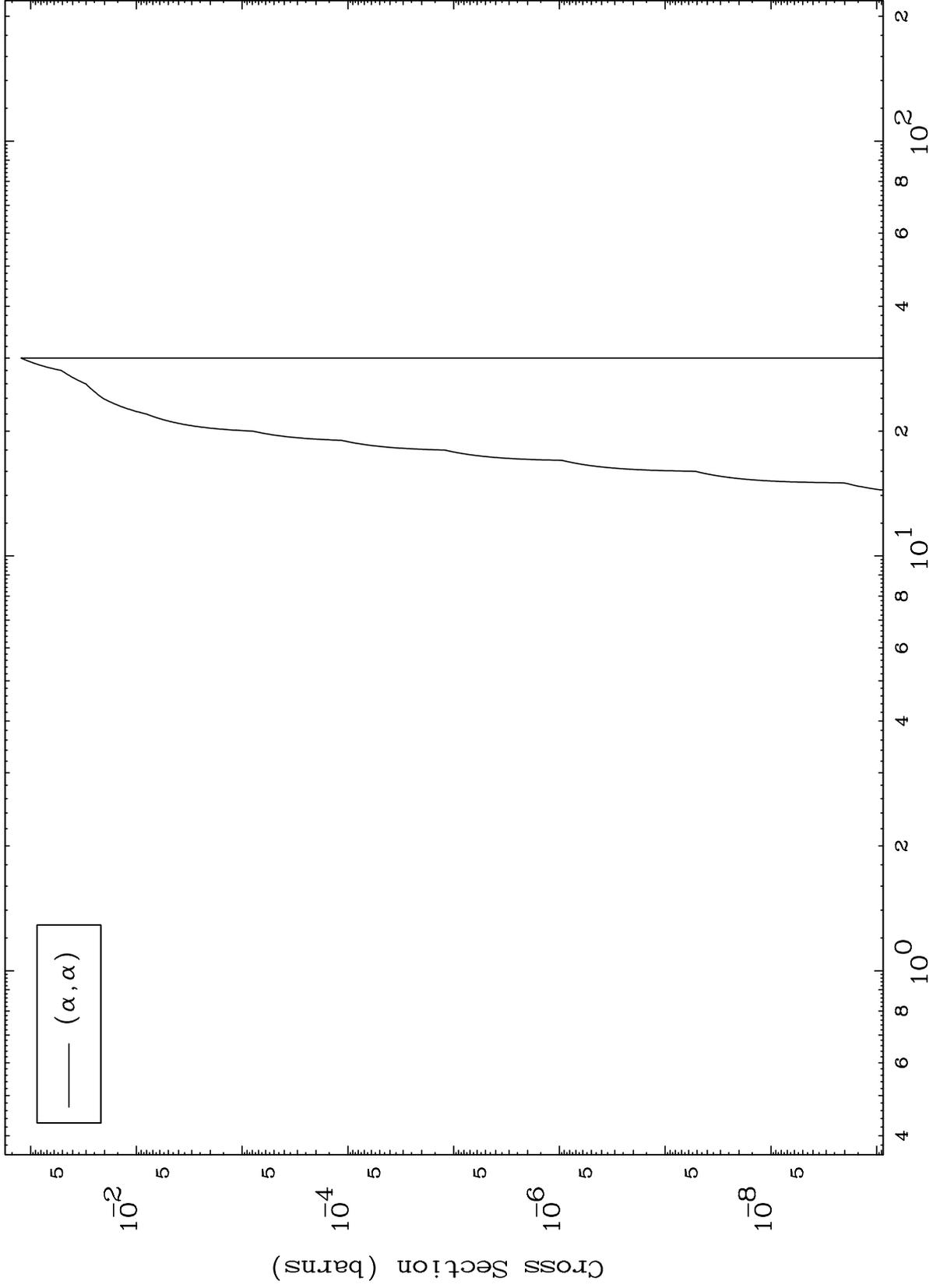


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( $\alpha, \alpha$ ) Levels

81-Tl-197

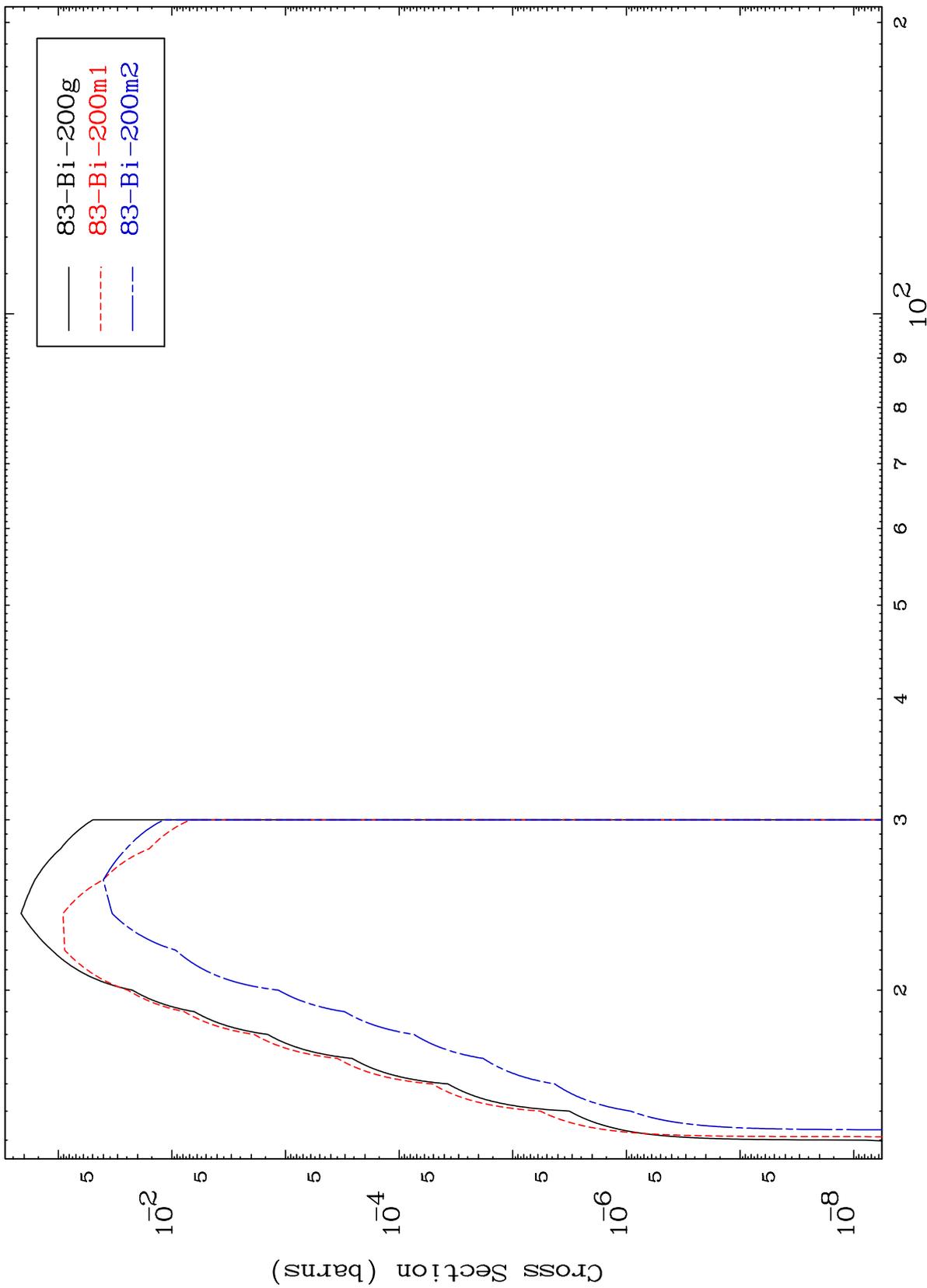
0 Kelvin Cross Sections



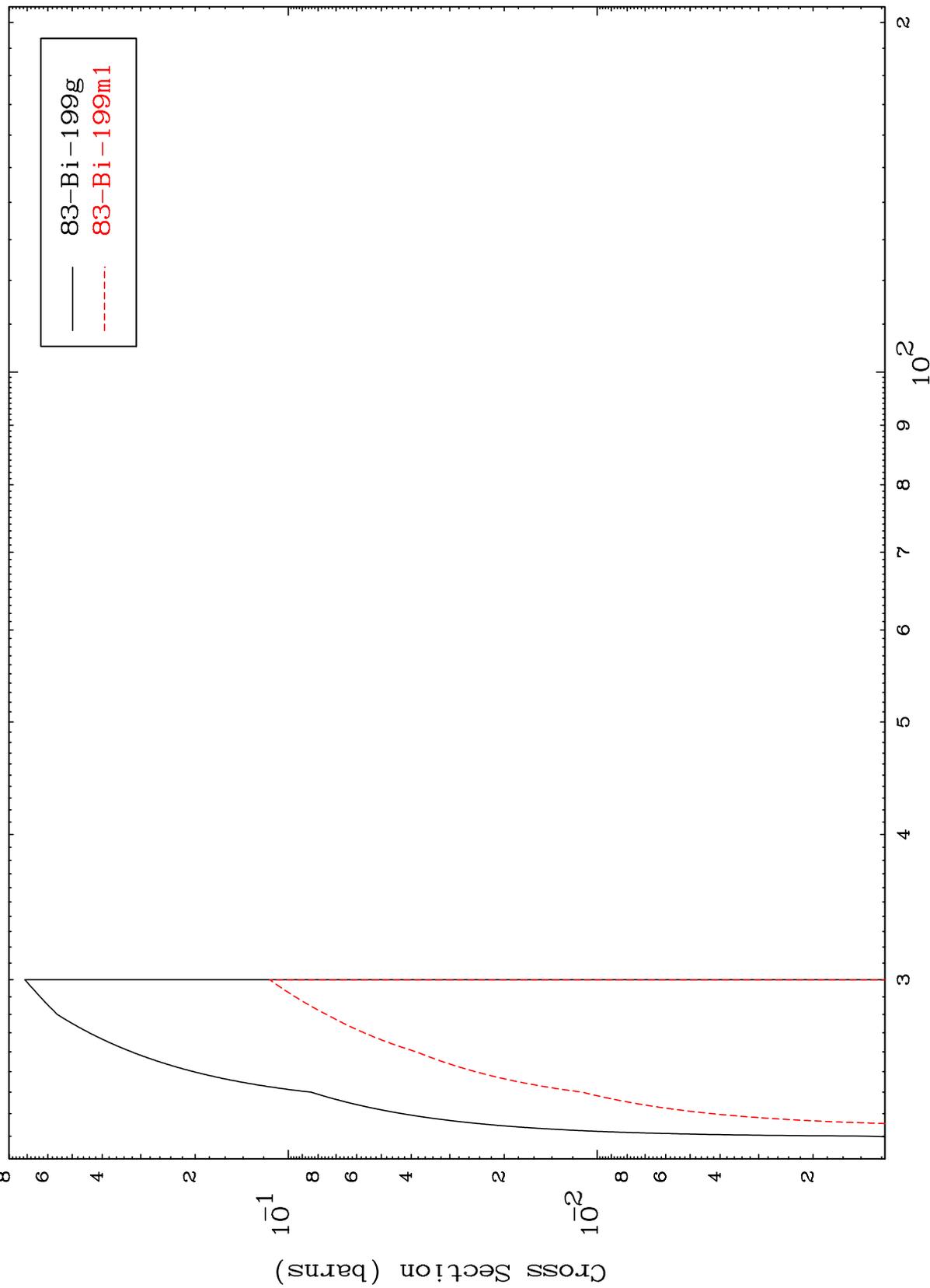
Incident Energy (MeV)

81-Tl-197

Radionuclide Production Cross Section  
 $\alpha$  Inelastic



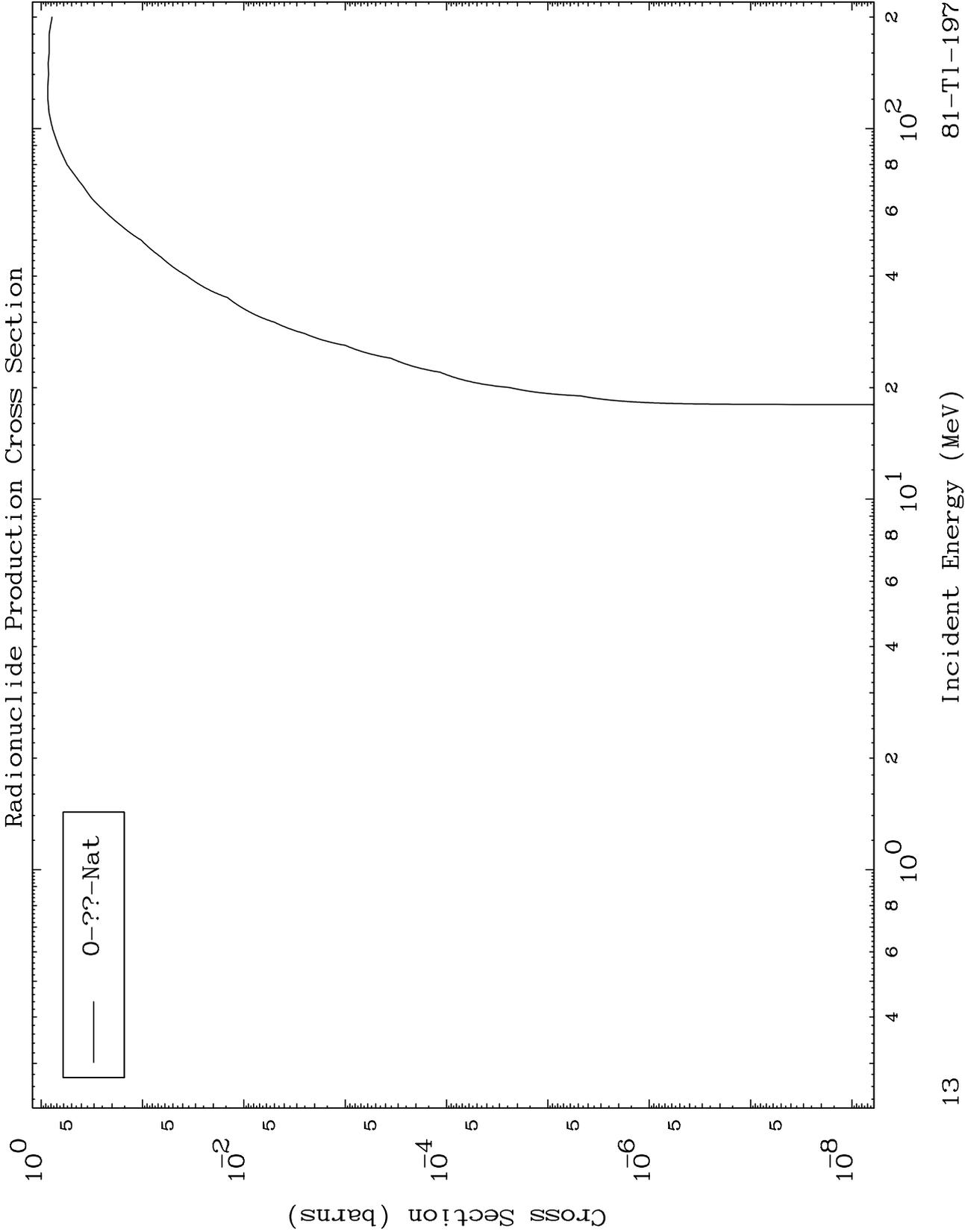
Radionuclide Production Cross Section  
( $\alpha, 2n$ )



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$\alpha$  Fission

81-Tl-197

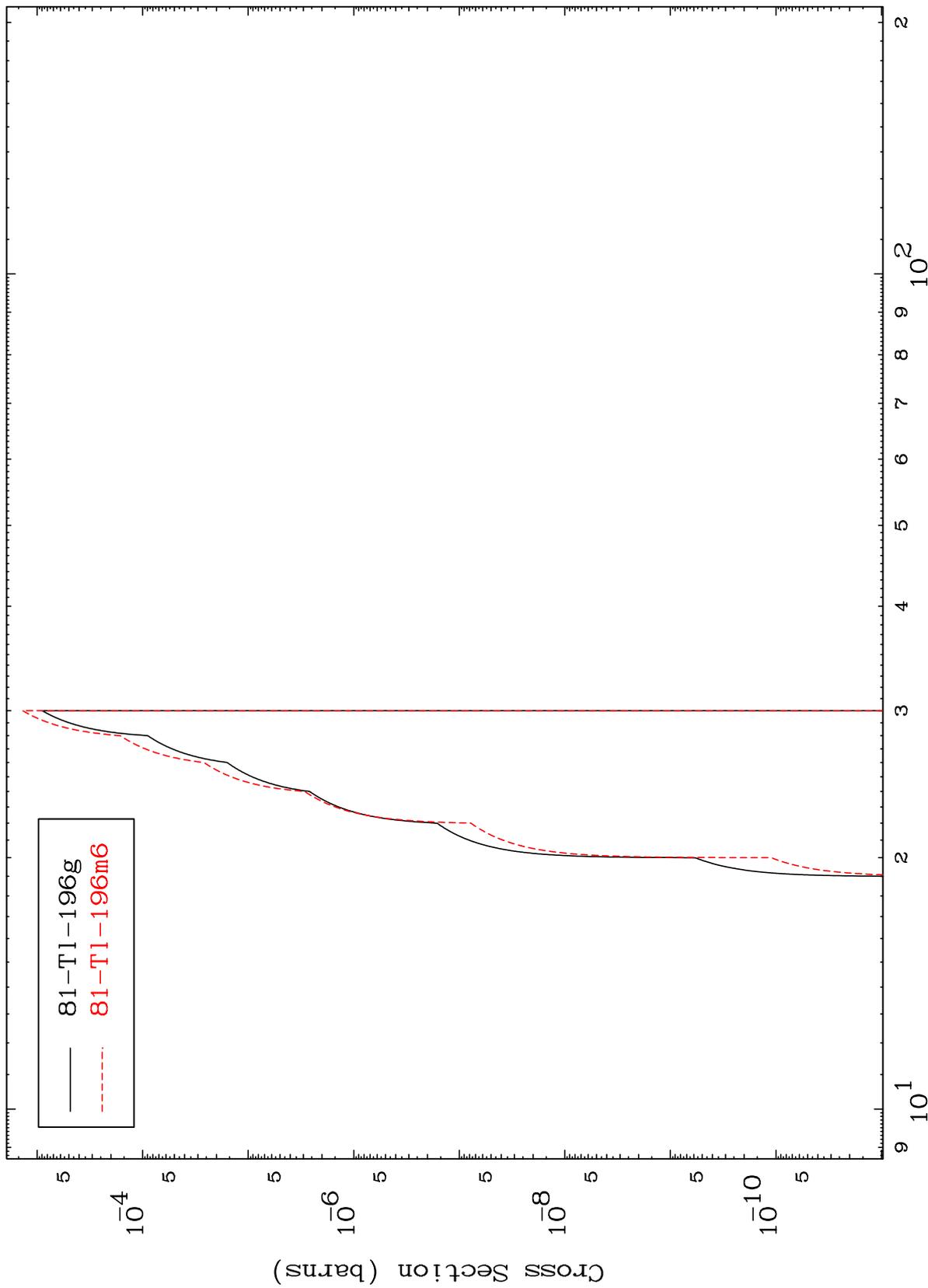


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( $\alpha, n'$ )  $\alpha$

81-Tl-197

Radionuclide Production Cross Section



81-Tl-196g  
81-Tl-196m6

Incident Energy (MeV)

81-Tl-197

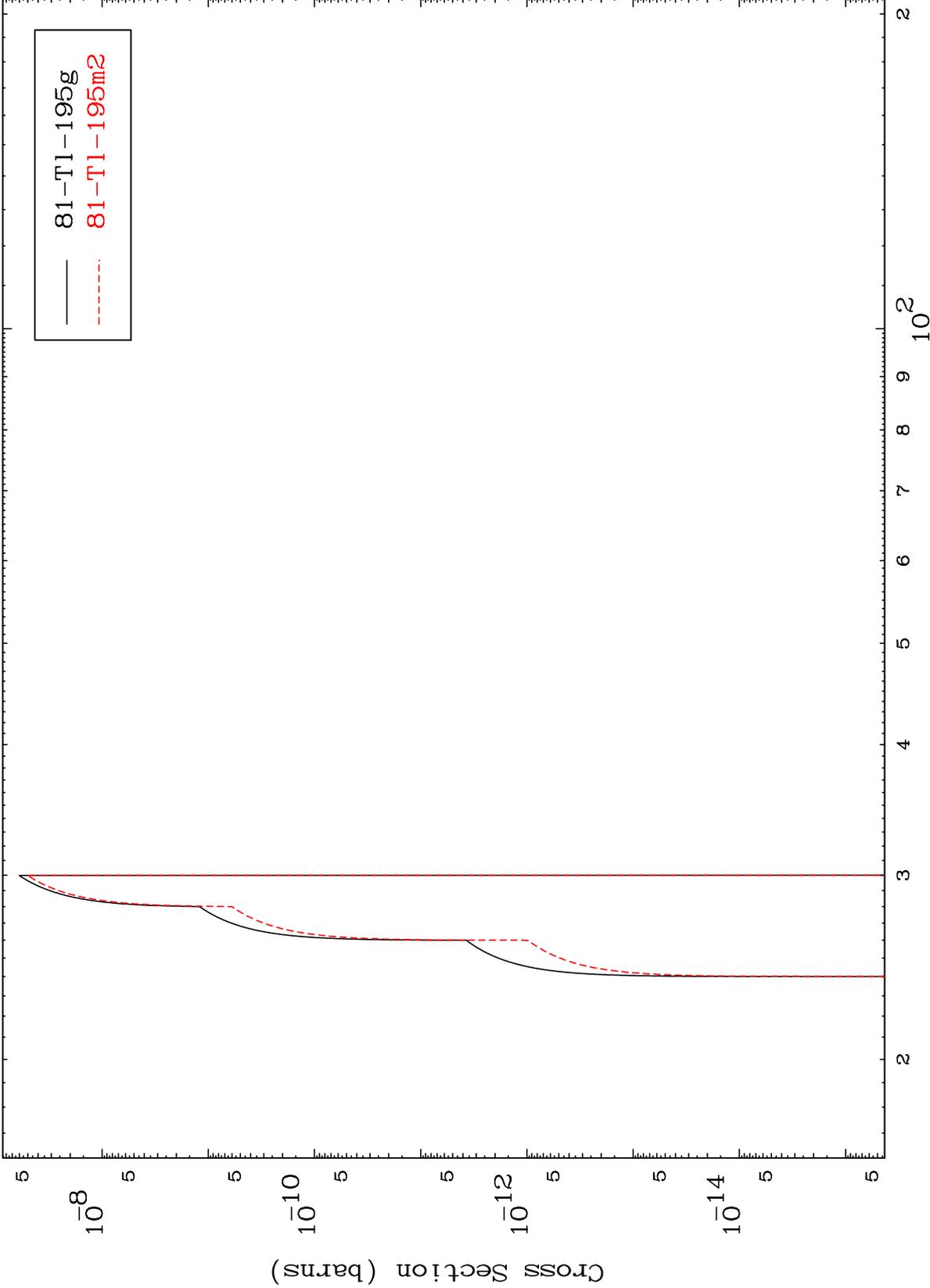
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$(\alpha, 2n)$   $\alpha$

81-Tl-197

Radionuclide Production Cross Section

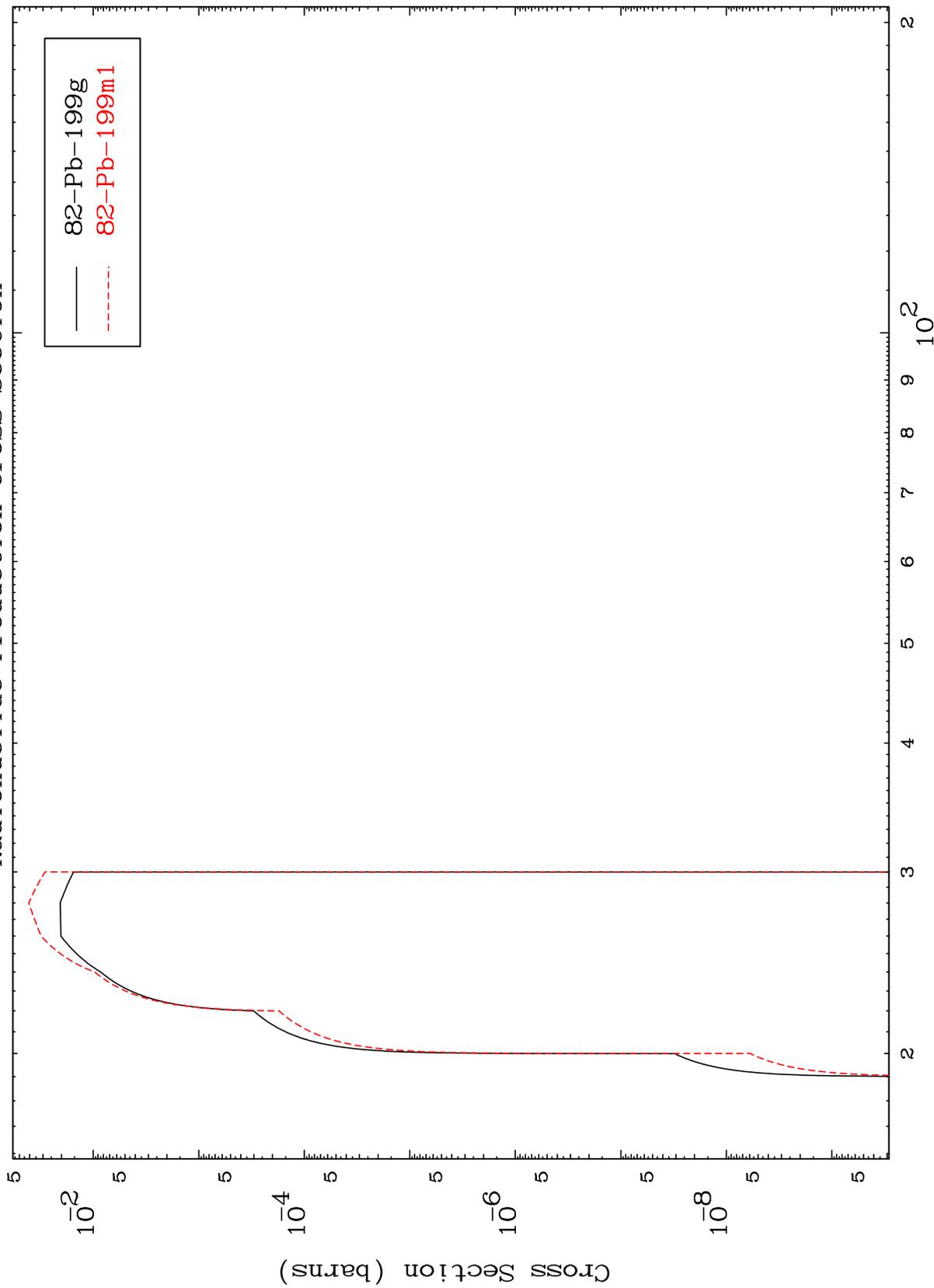


15

Incident Energy (MeV)

81-Tl-197

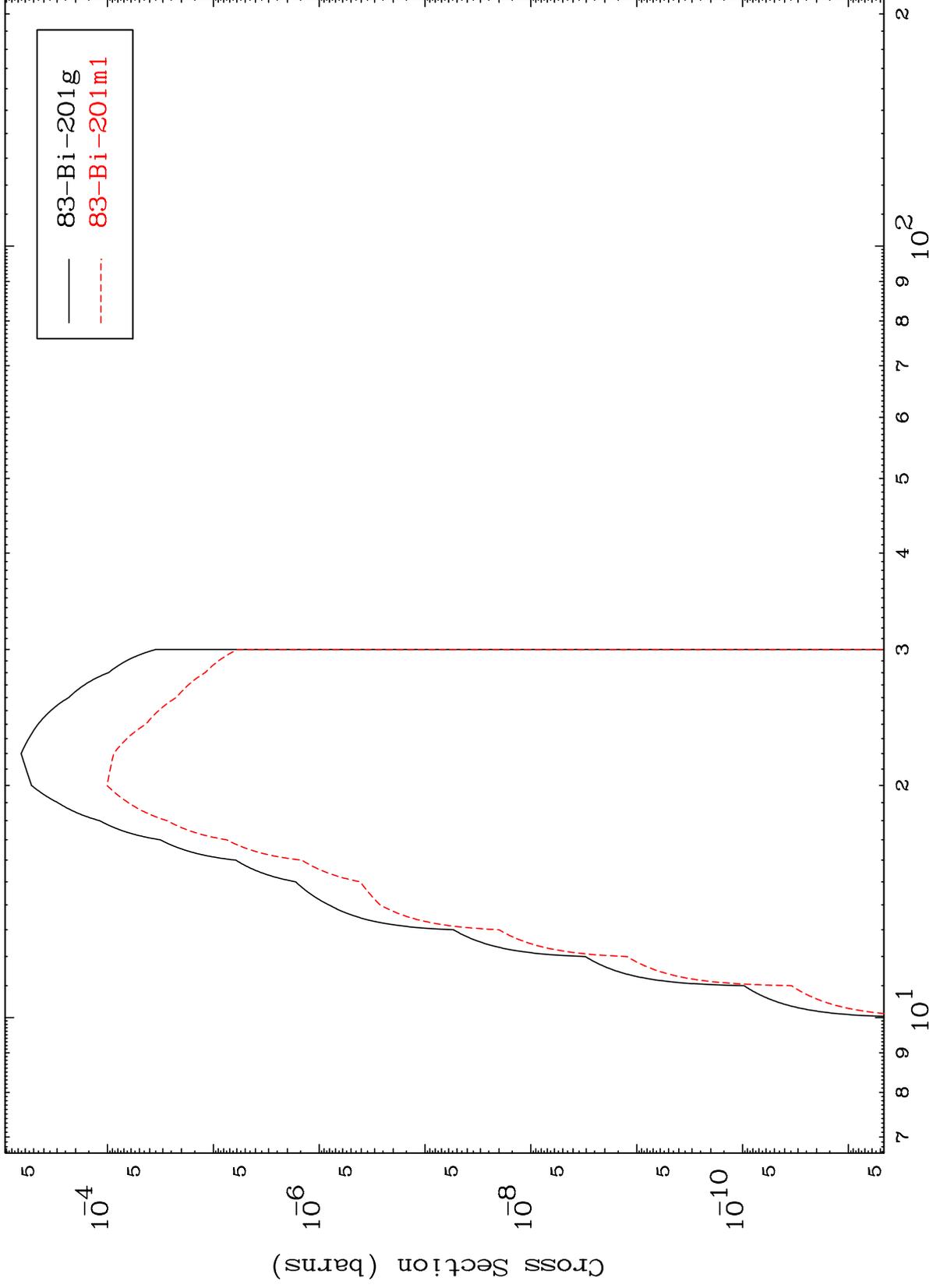
Radionuclide Production Cross Section



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81-Tl-197

$(\alpha, \gamma)$   
Radionuclide Production Cross Section



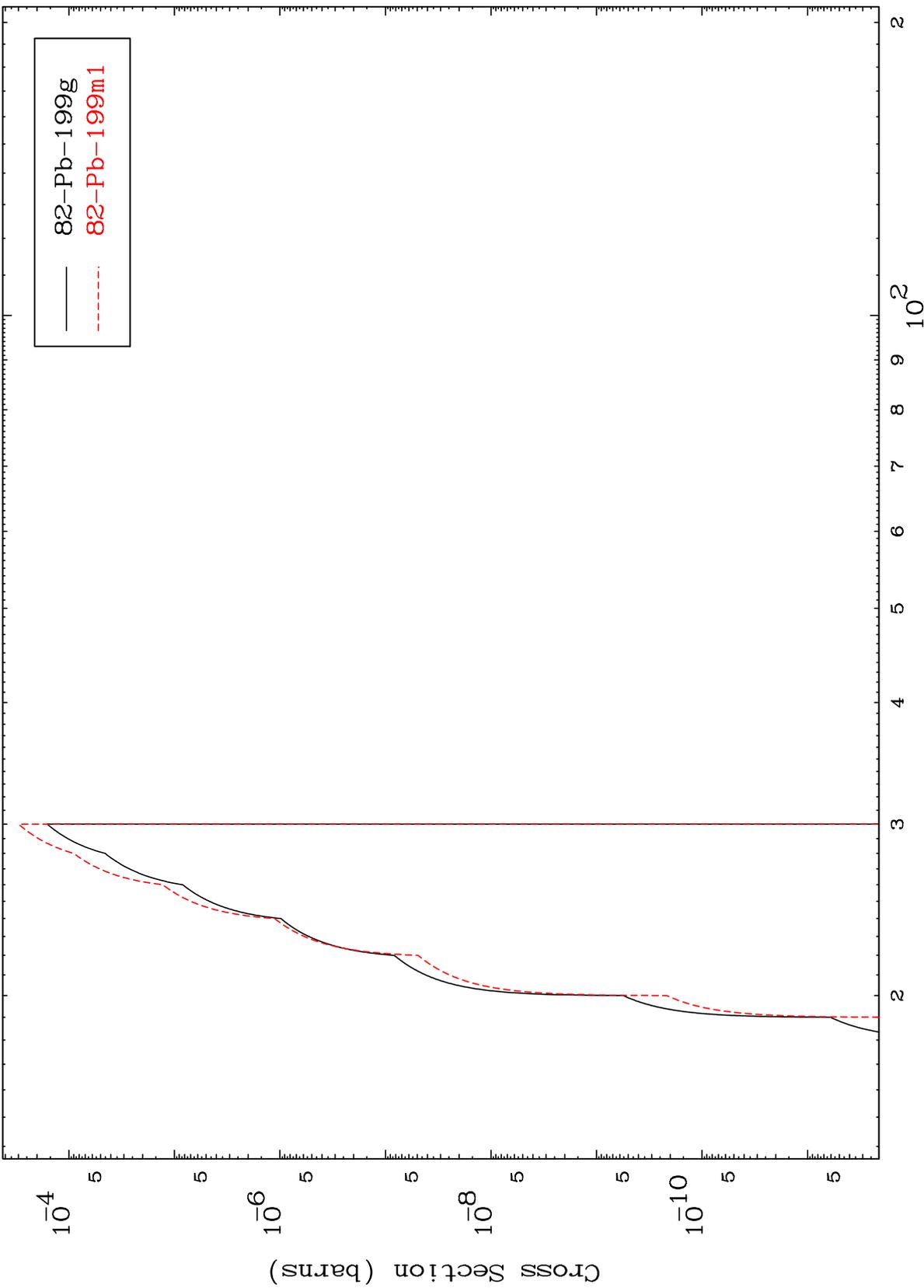
— 83-Bi-201g  
- - - 83-Bi-201m1

17

Incident Energy (MeV)

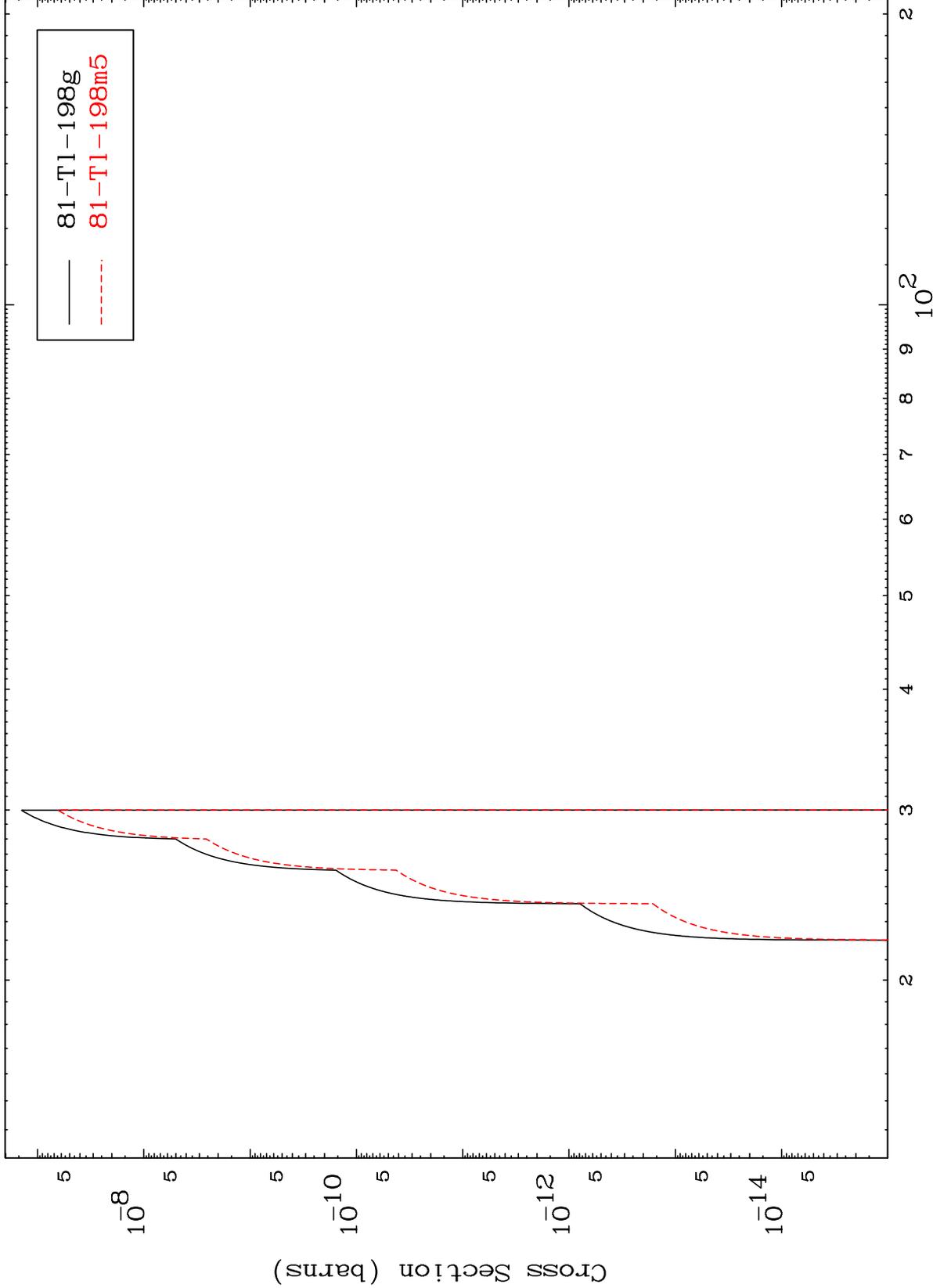
81-Tl-197

Radionuclide Production Cross Section



82-Pb-199g  
82-Pb-199m1

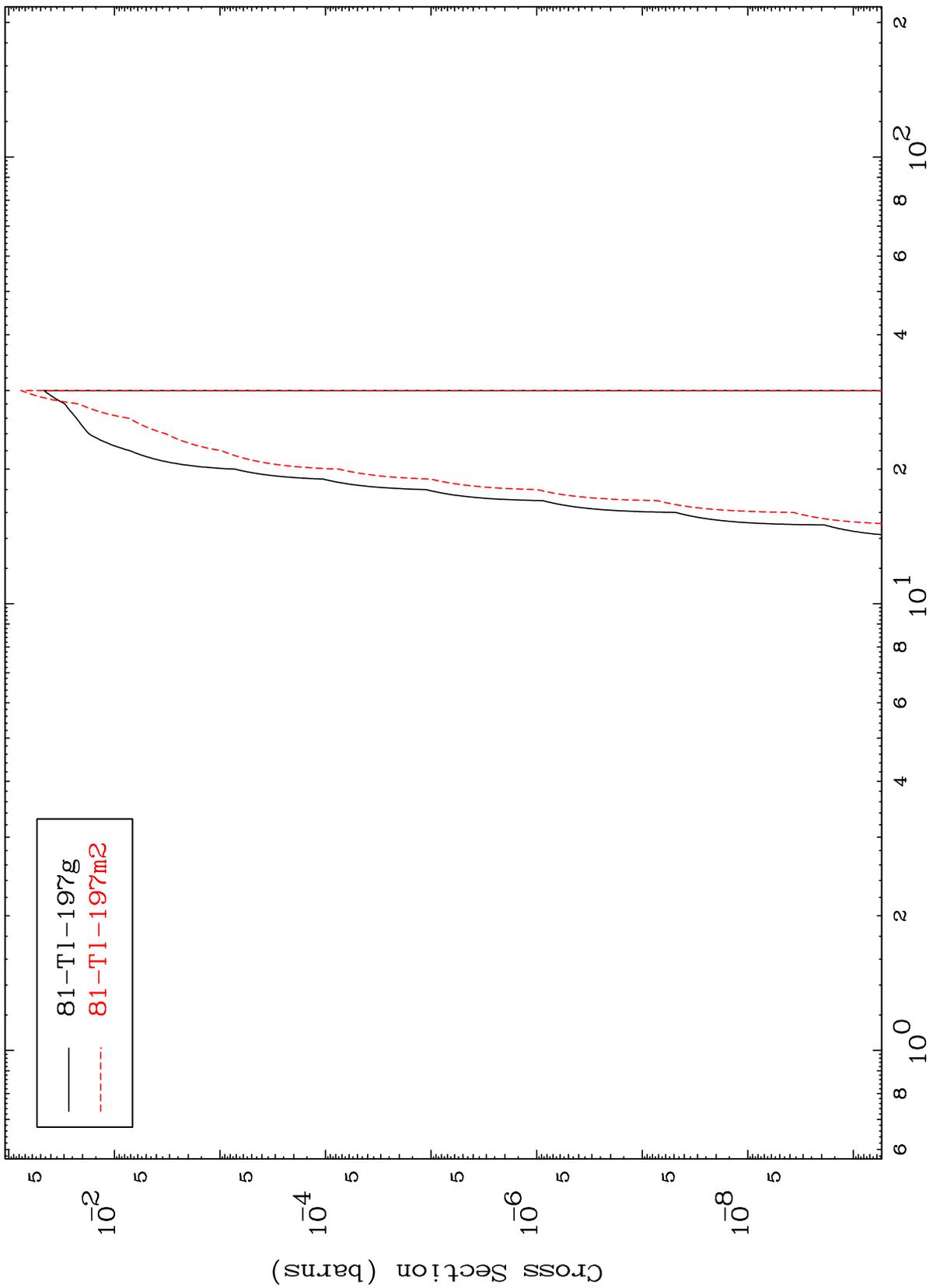
Radionuclide Production Cross Section



MAT 8107

81-Tl-197

( $\alpha, \alpha$ )  
Radionuclide Production Cross Section



20

Incident Energy (MeV)

81-Tl-197