

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

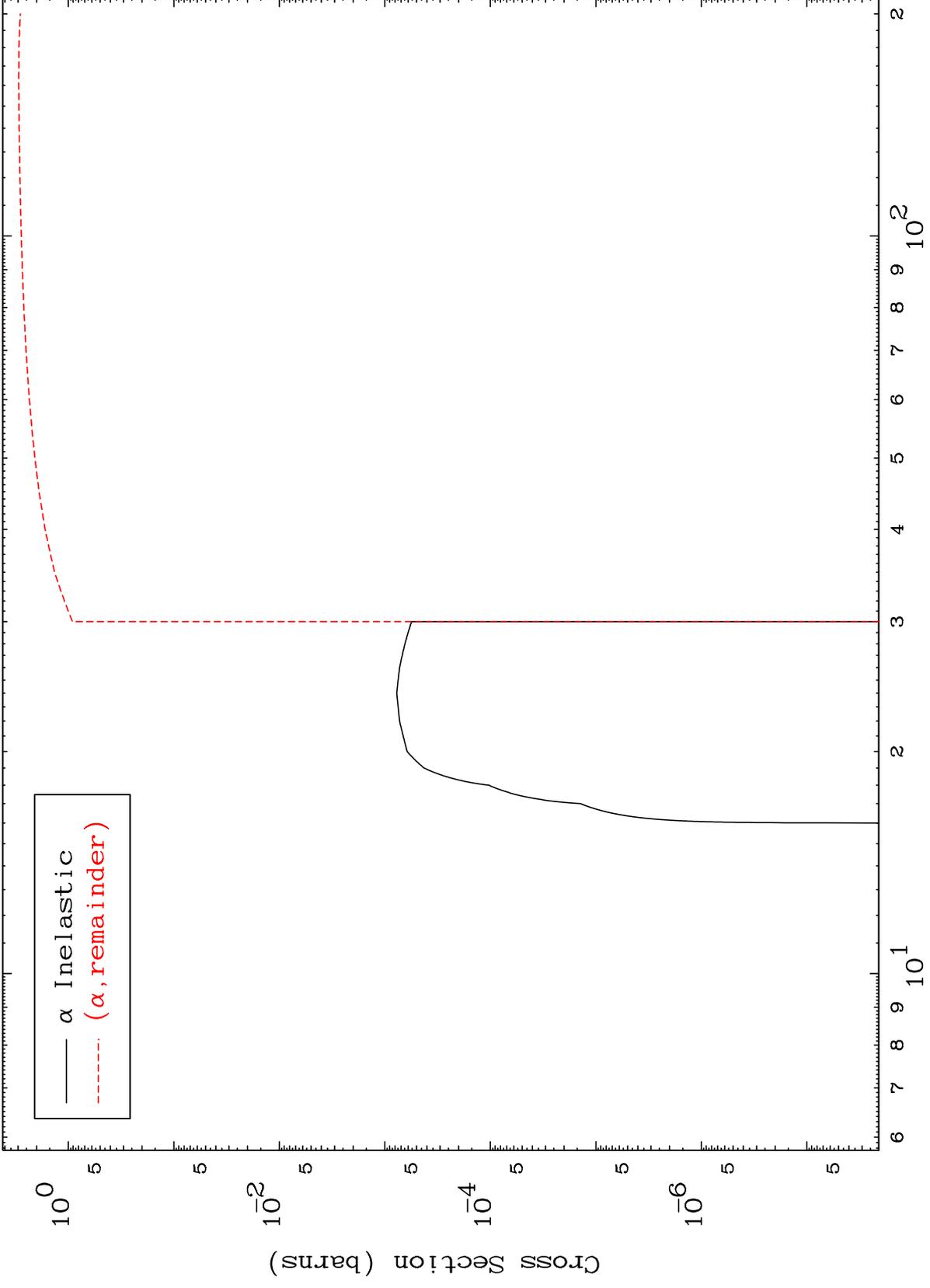
Press Mouse Button to Start

MAT 9216

$\alpha$  Major

92-U -231

0 Kelvin Cross Sections

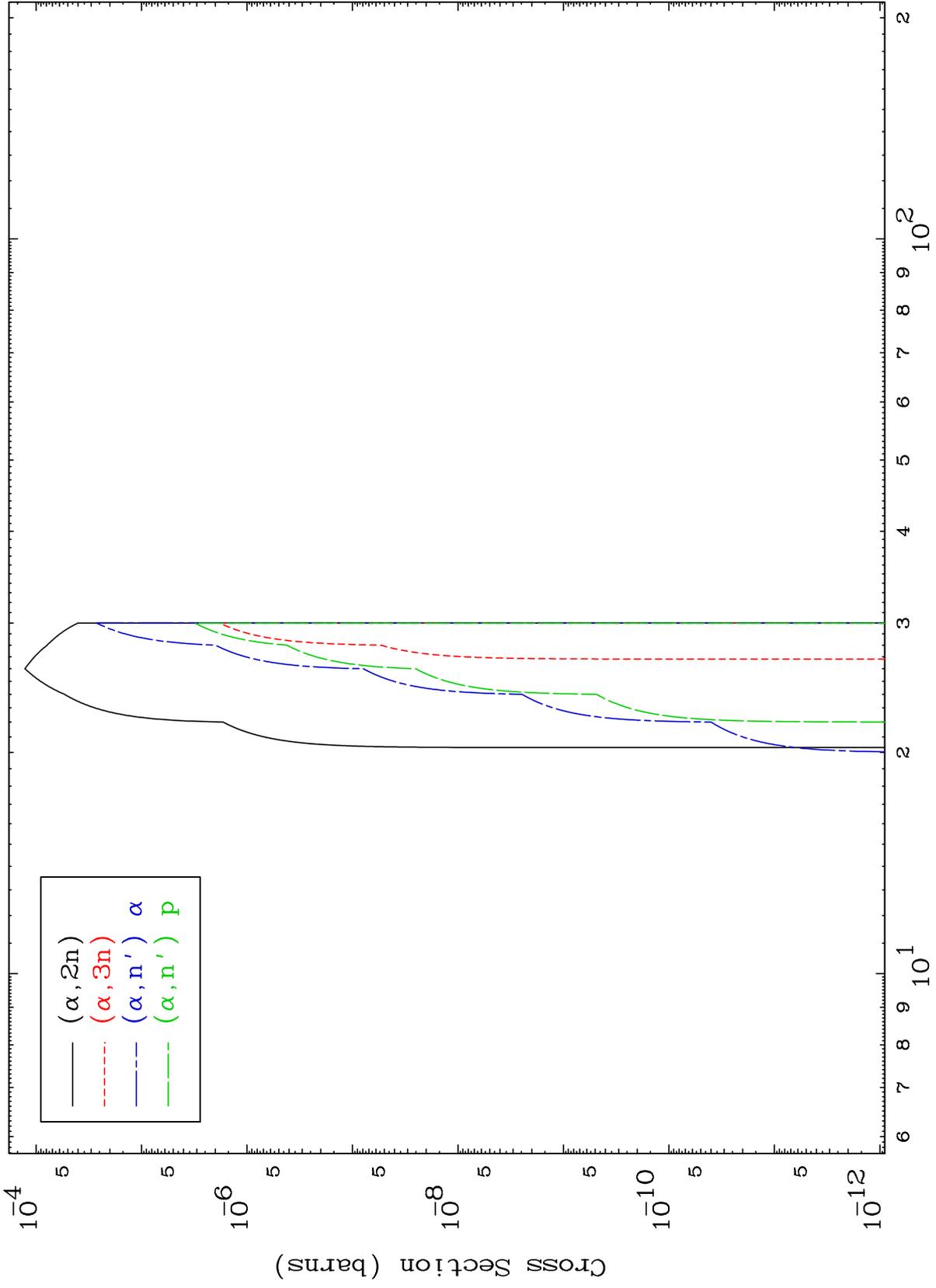


—  $\alpha$  Inelastic  
- - - ( $\alpha$ , remainder)

MAT 9216

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

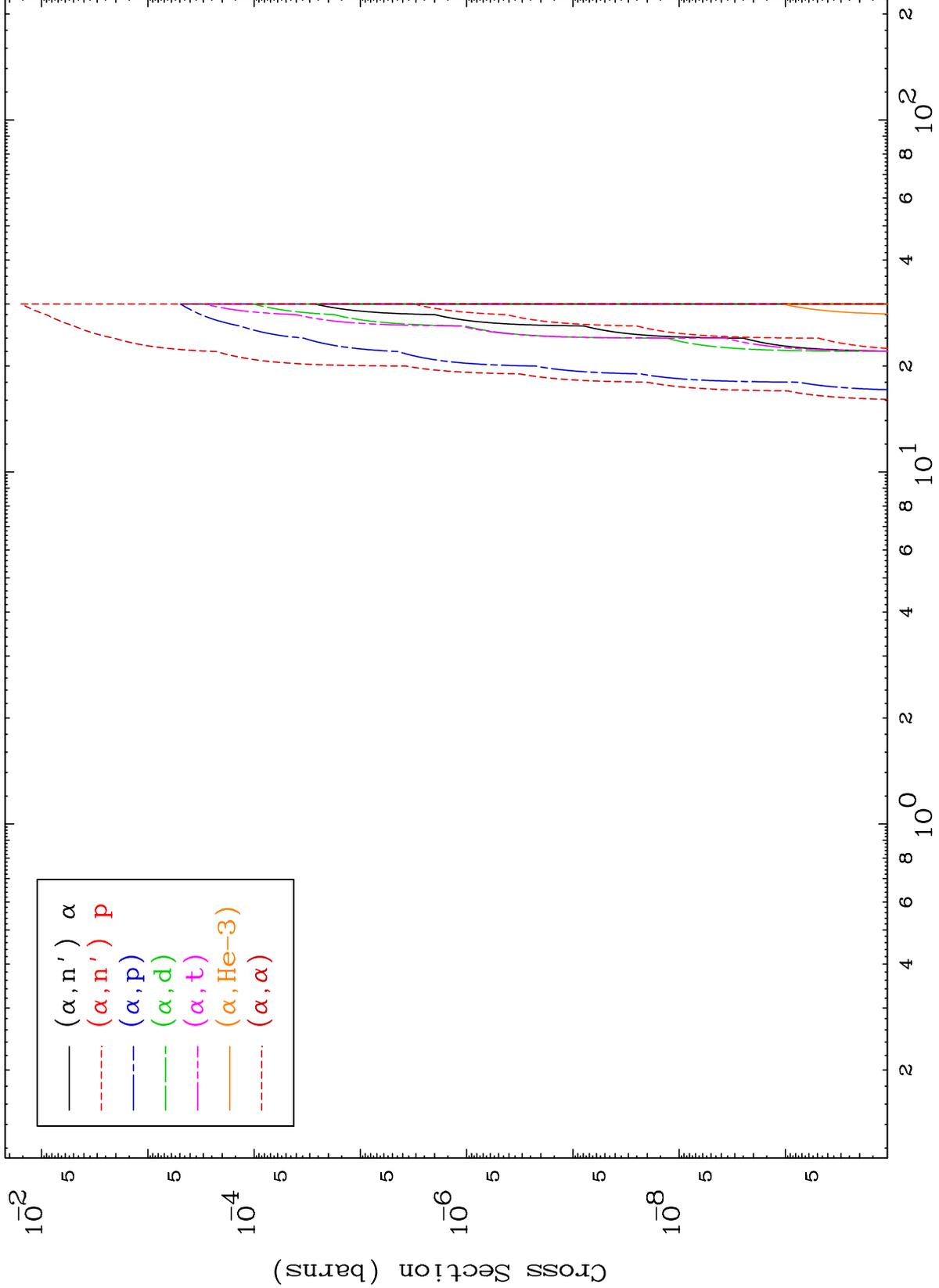
92-U -231



2

Incident Energy (MeV)

92-U -231

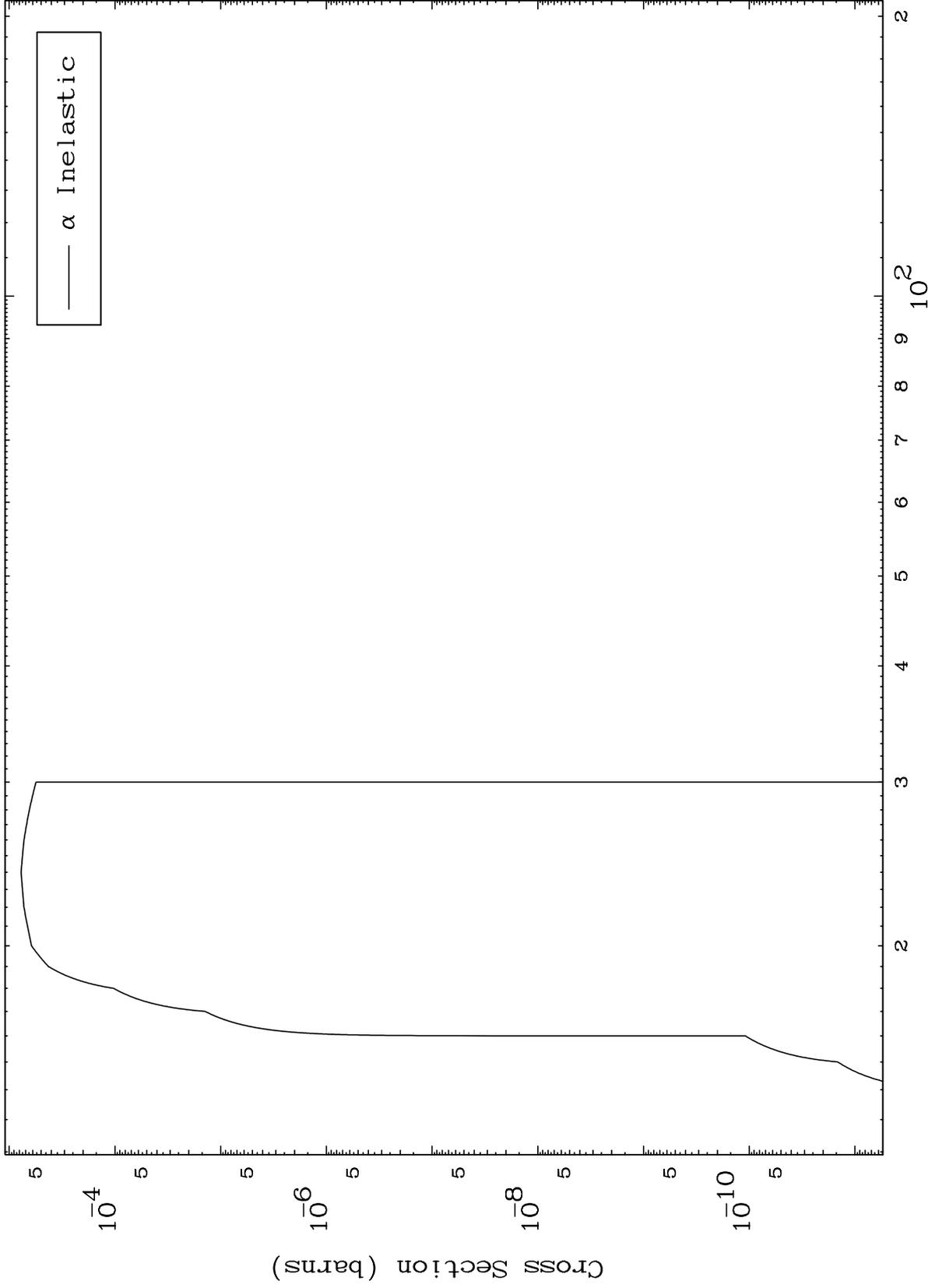


MAT 9216

( $\alpha, n'$ ) Level

92-U -231

0 Kelvin Cross Sections



4

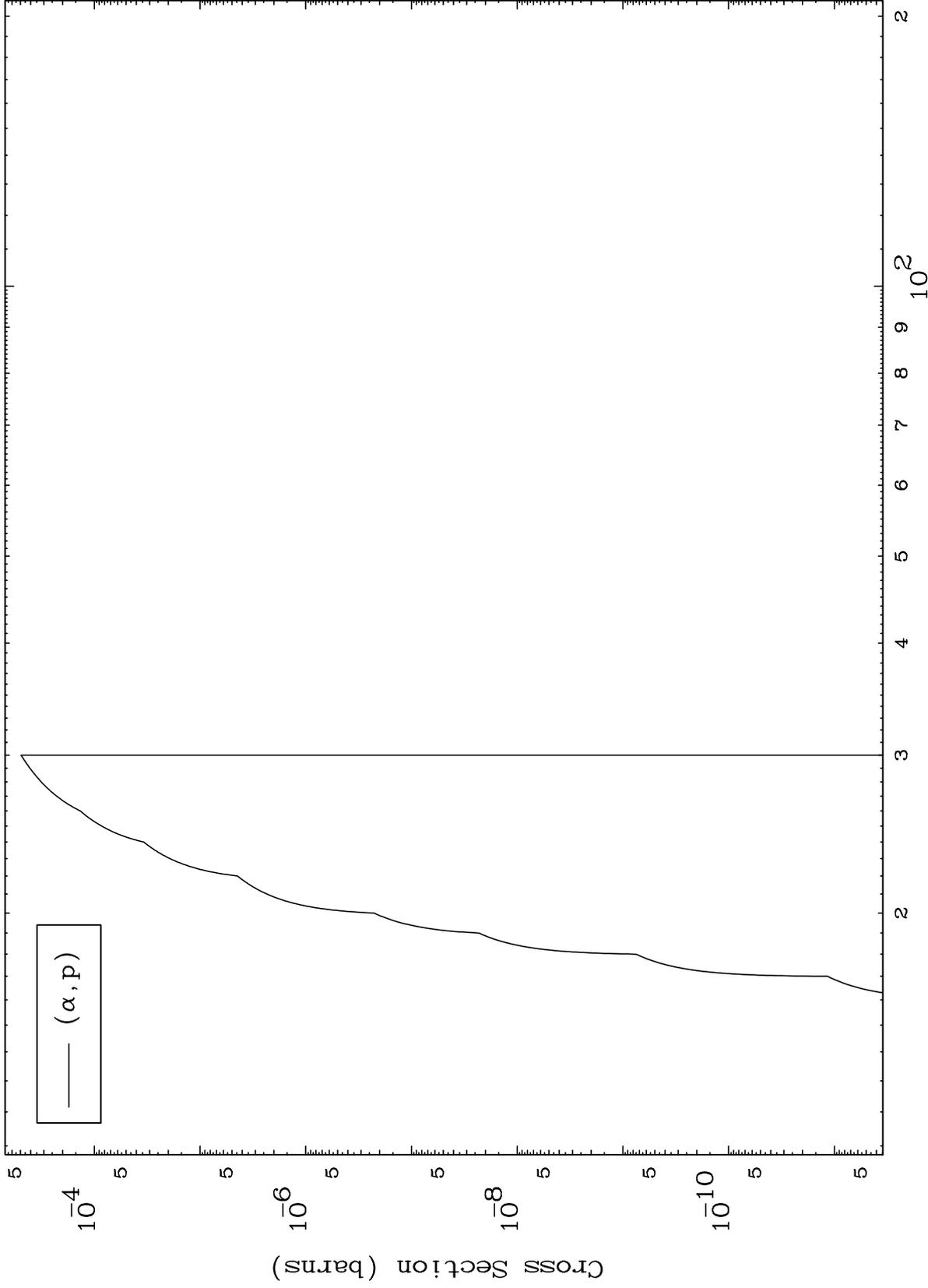
Incident Energy (MeV)

92-U -231

MAT 9216

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

92-U -231



5

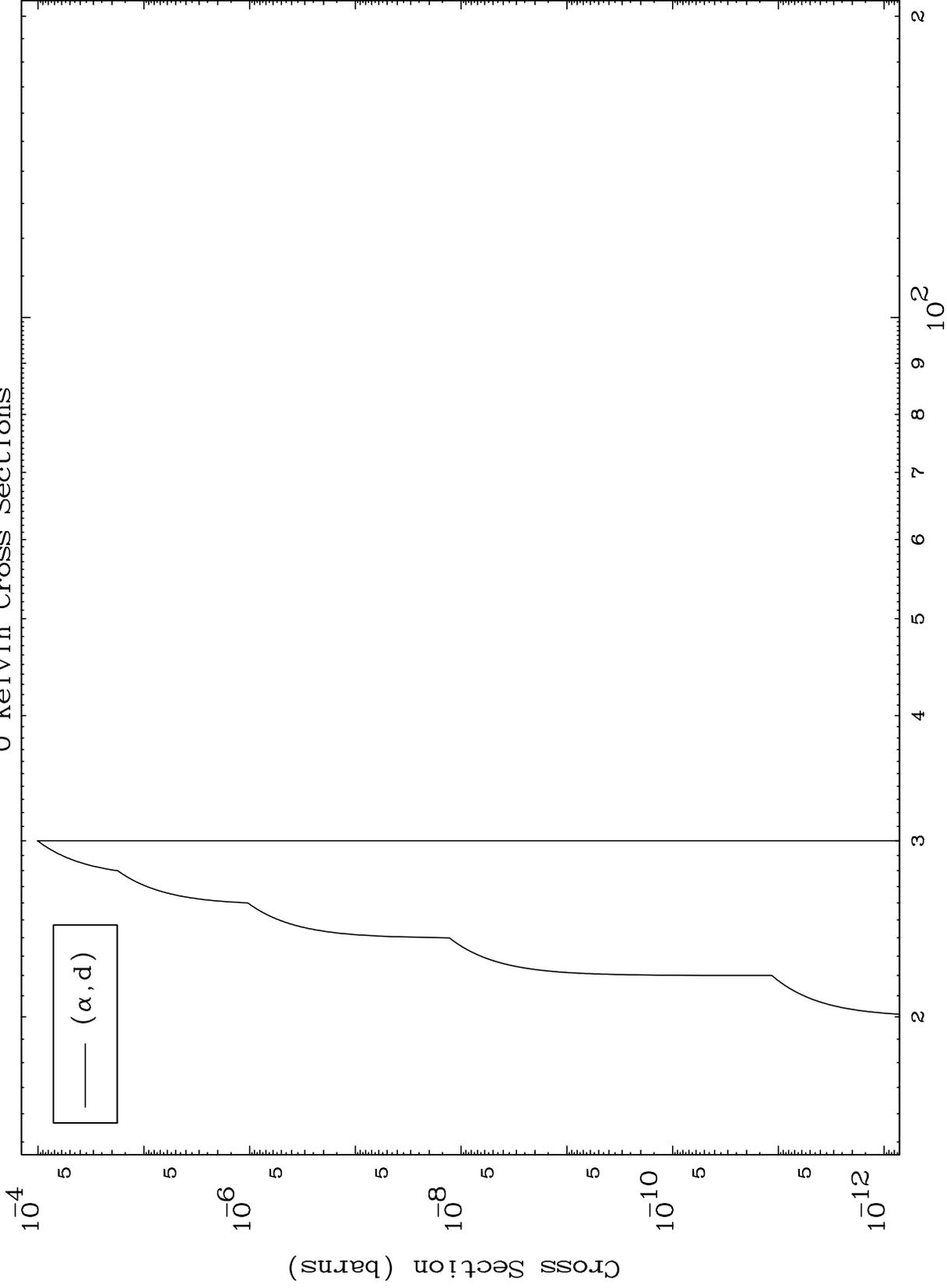
Incident Energy (MeV)

92-U -231

MAT 9216

92-U -231

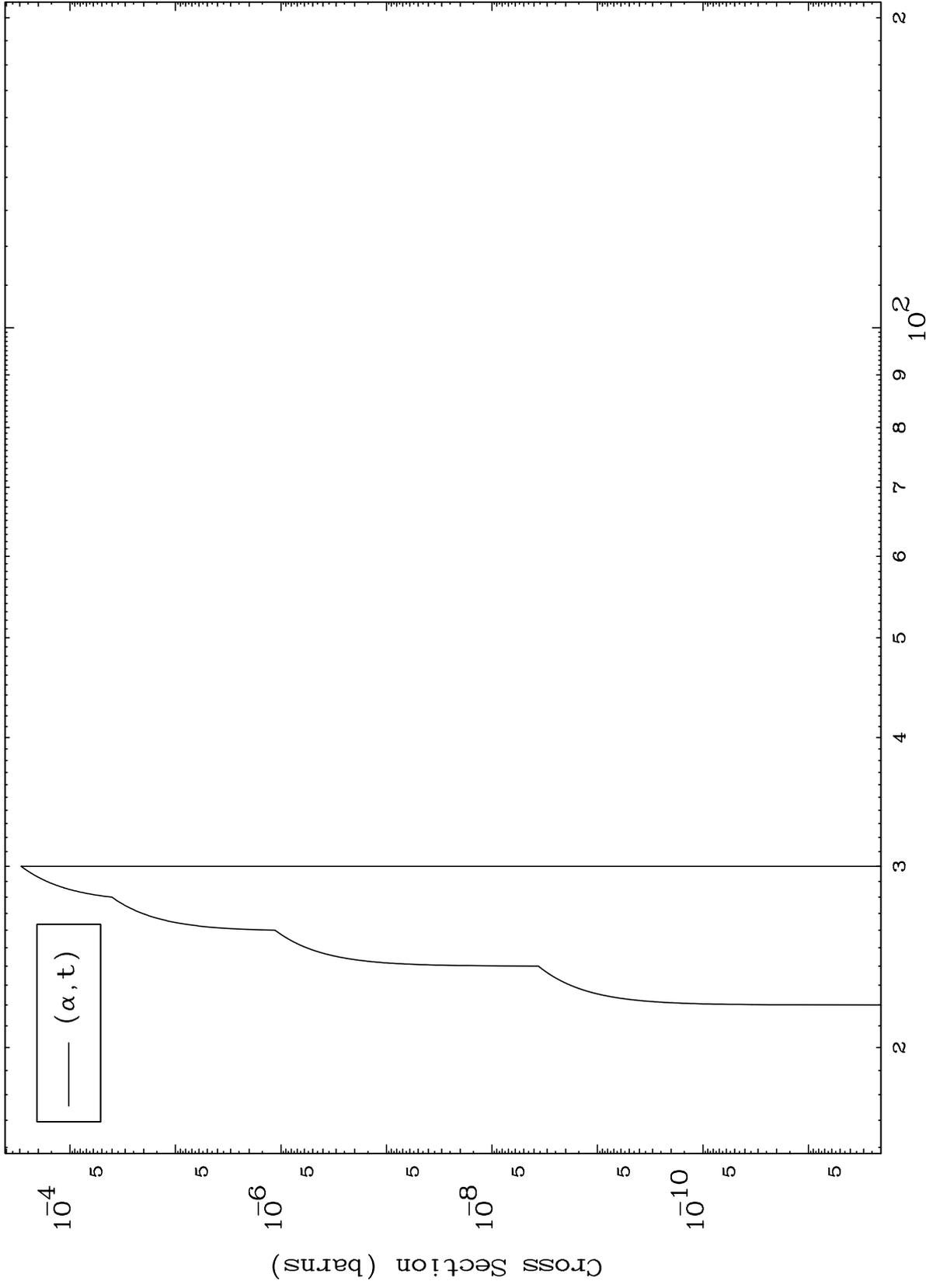
( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

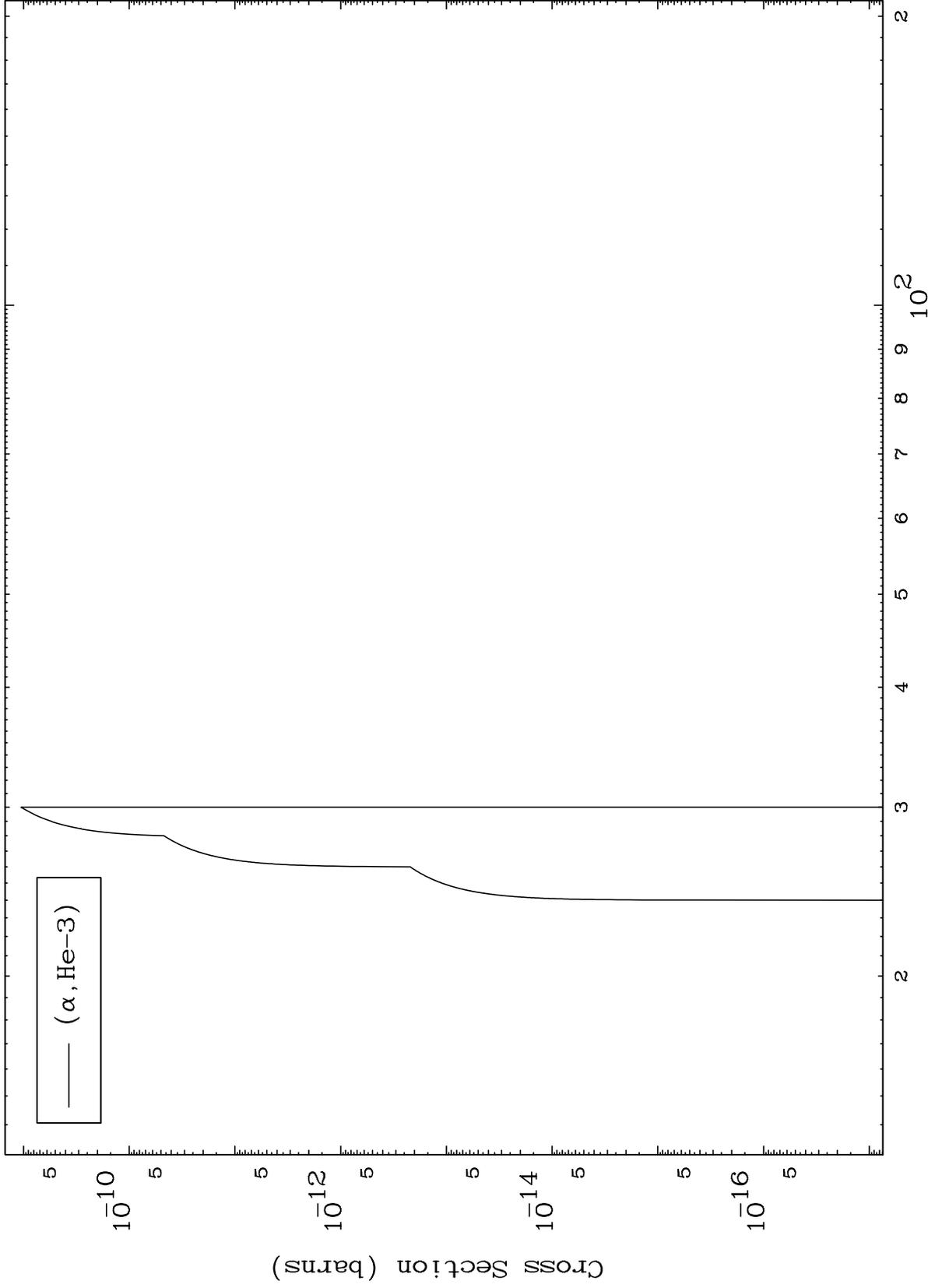


92-U -231

Incident Energy (MeV)

6



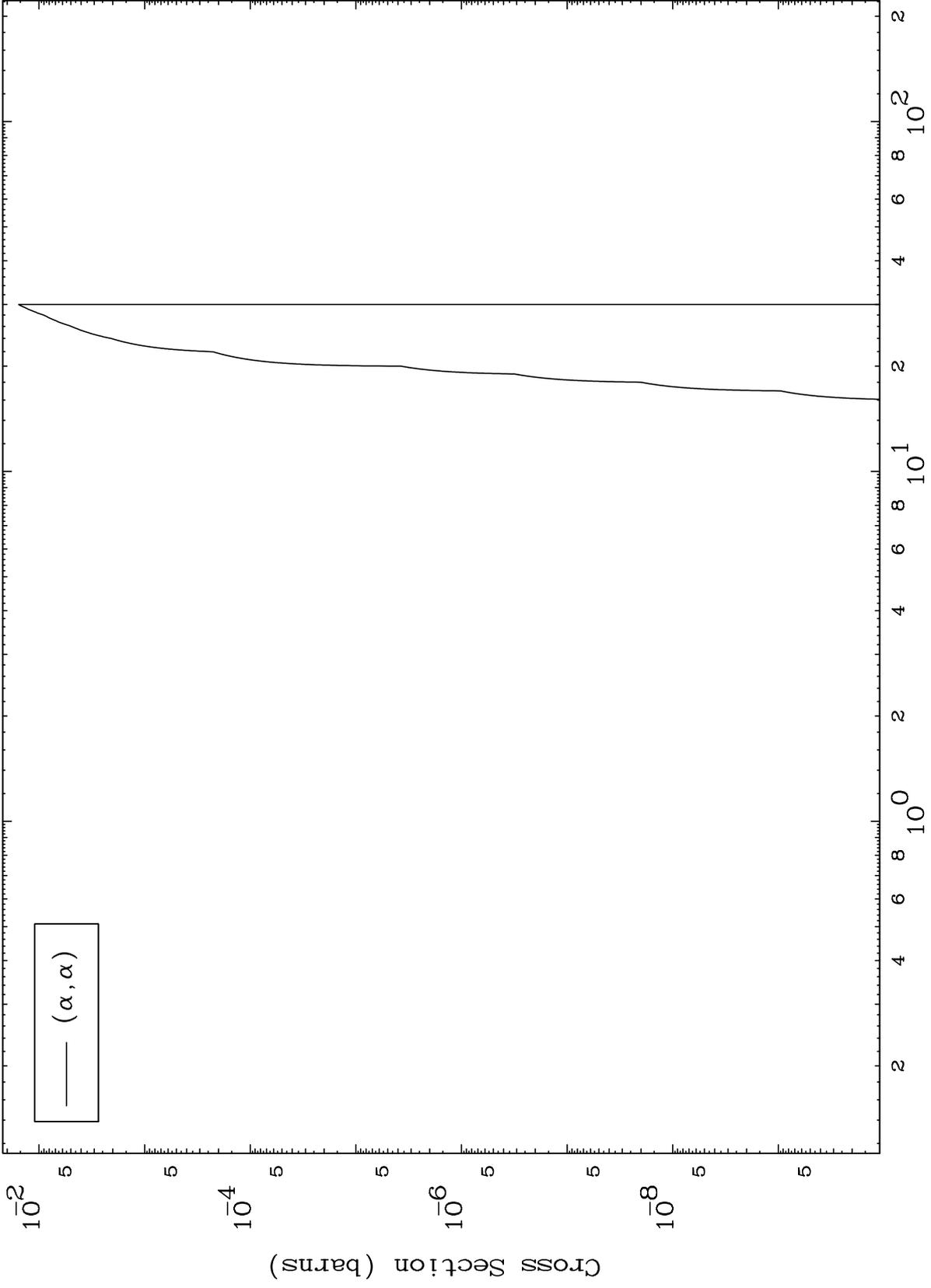


MAT 9216

( $\alpha, \alpha$ ) Levels

92-U -231

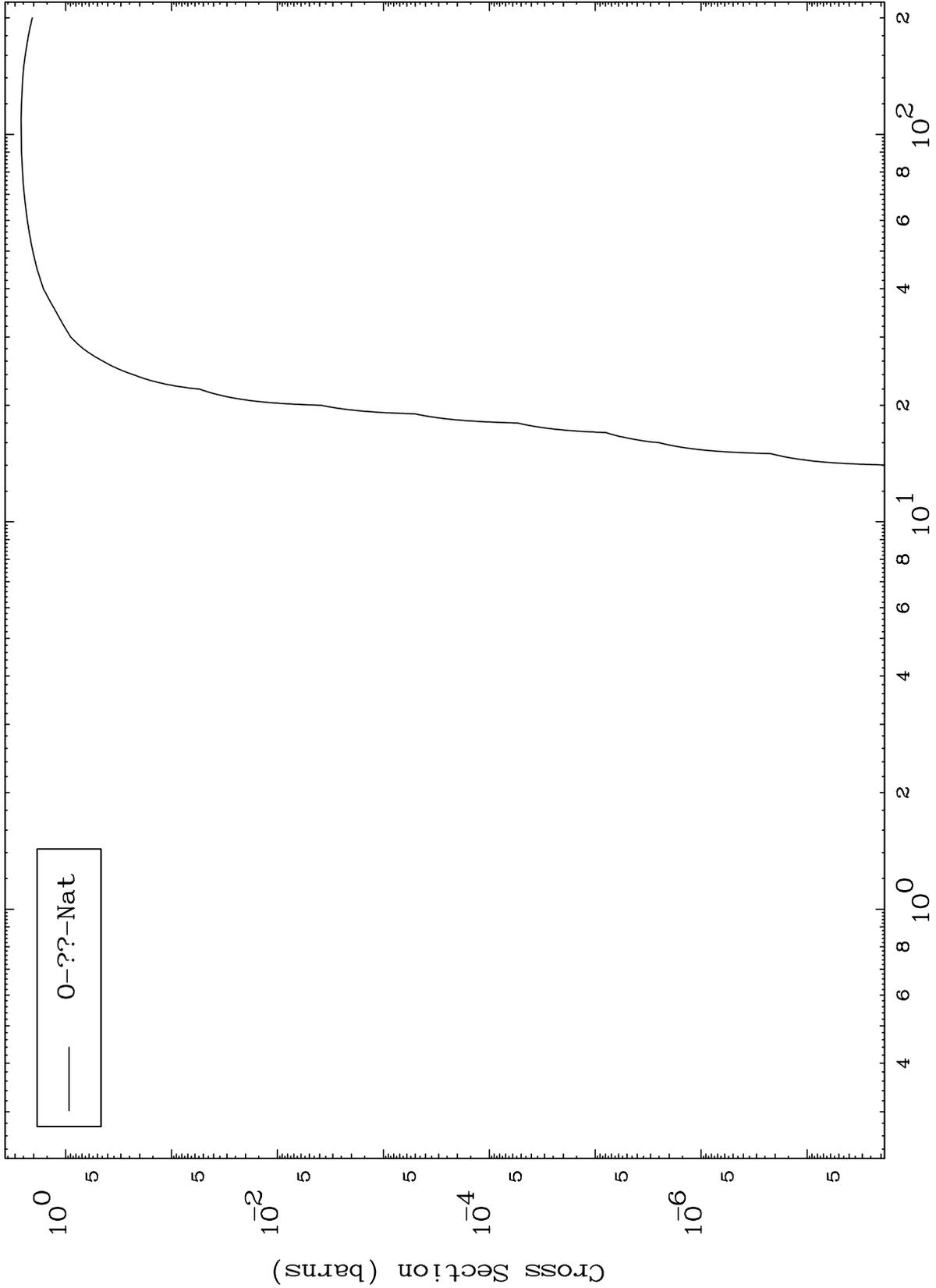
0 Kelvin Cross Sections



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92-U -231

$\alpha$  Fission  
Radionuclide Production Cross Section



10

92-U -231