

Program EVALPLOT
(Version 2018-1)

by

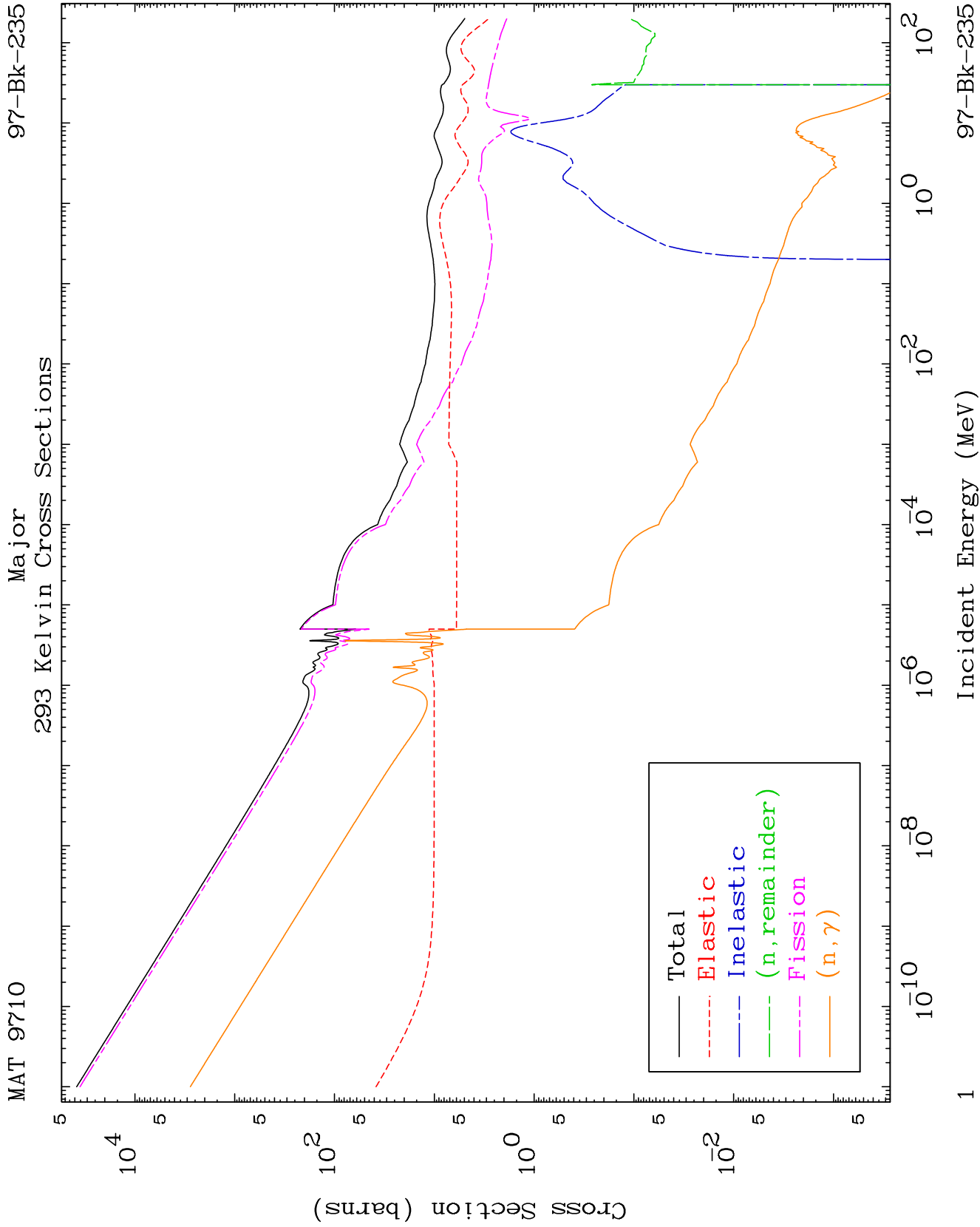
Dermott E. Cullen
(Present Contact Information)

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Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

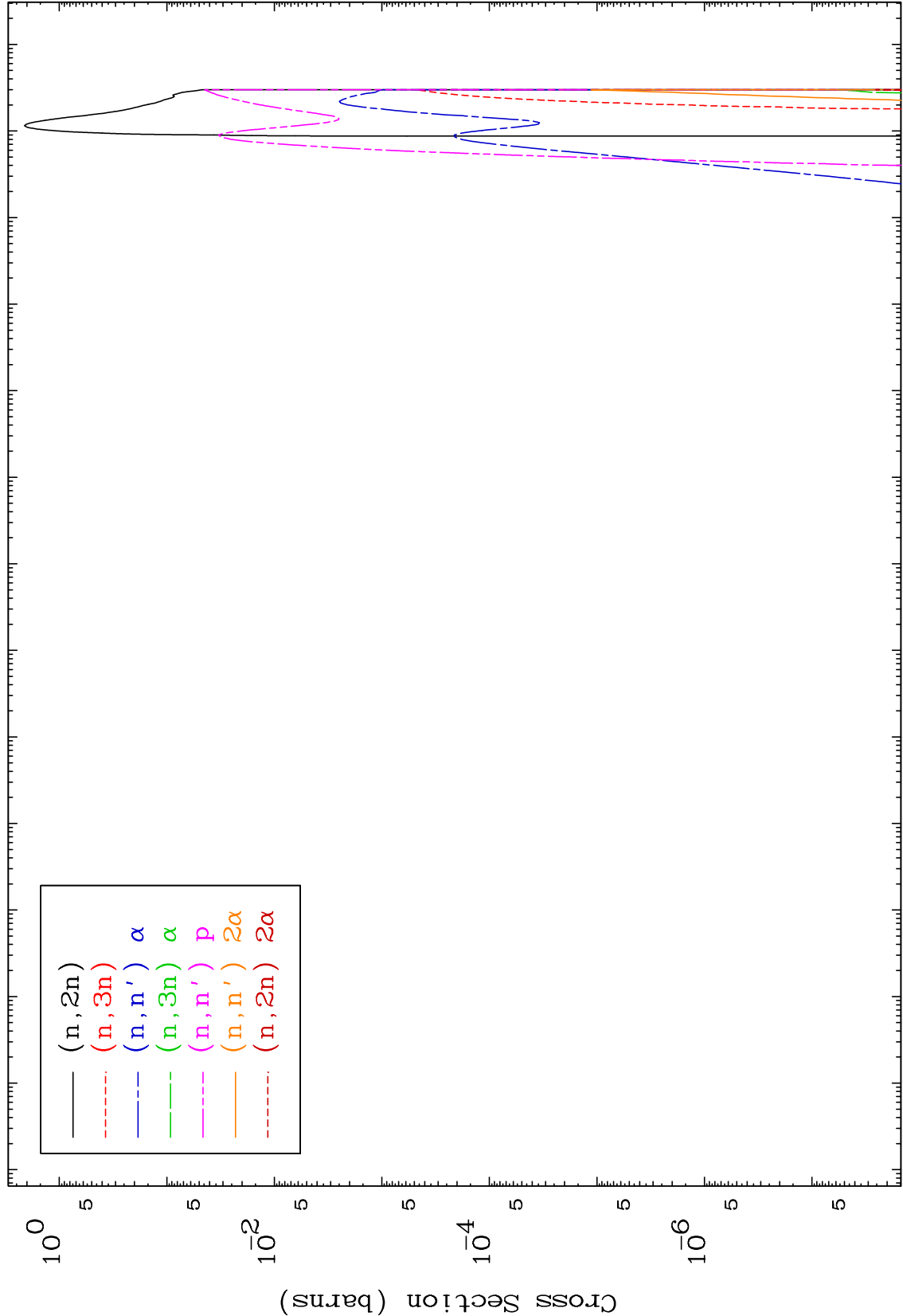
Press Mouse Button to Start



MAT 9710

Neutron Production
293 Kelvin Cross Sections

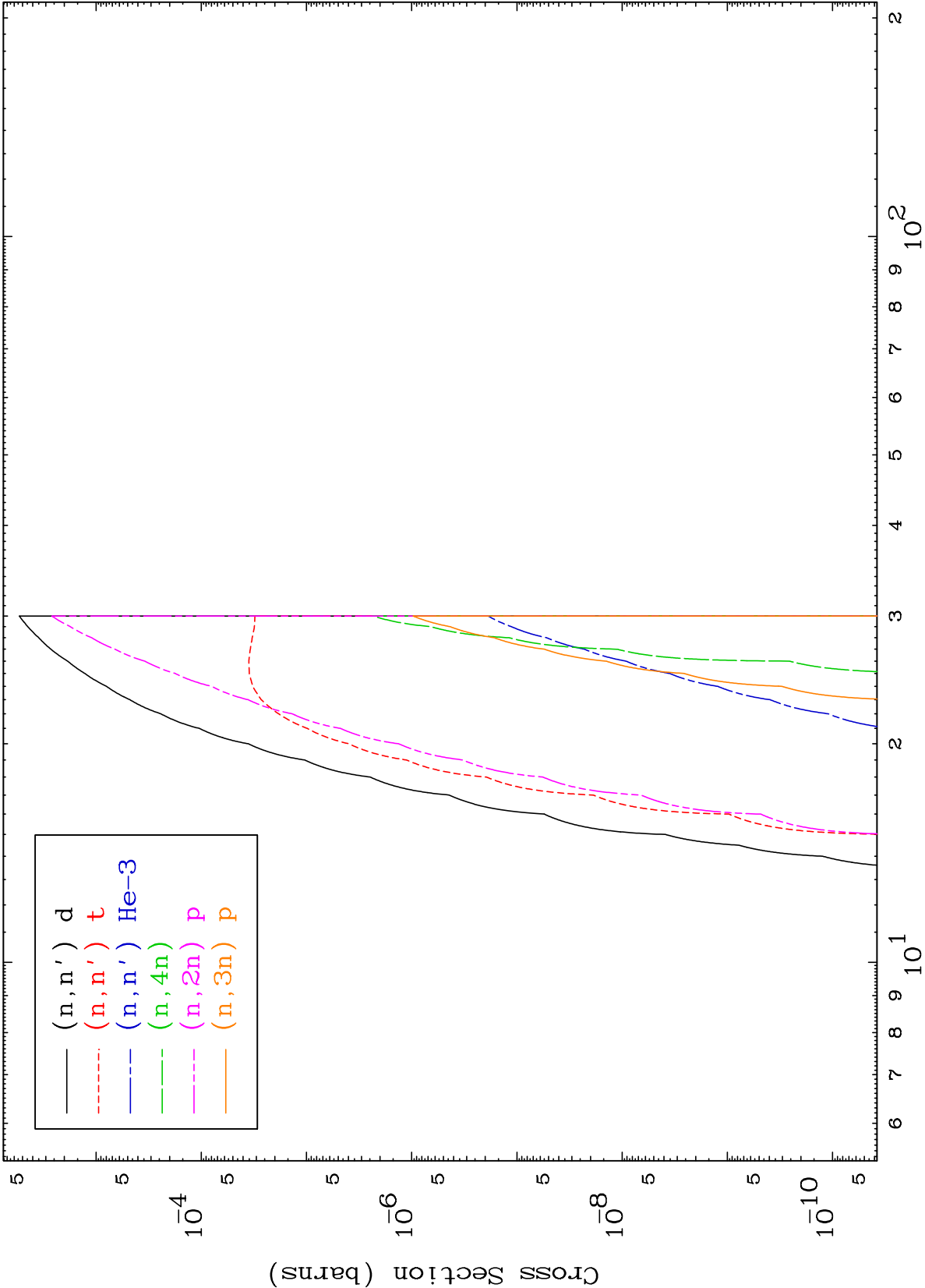
97-Bk-235



MAT 9710

Neutron Production
293 Kelvin Cross Sections

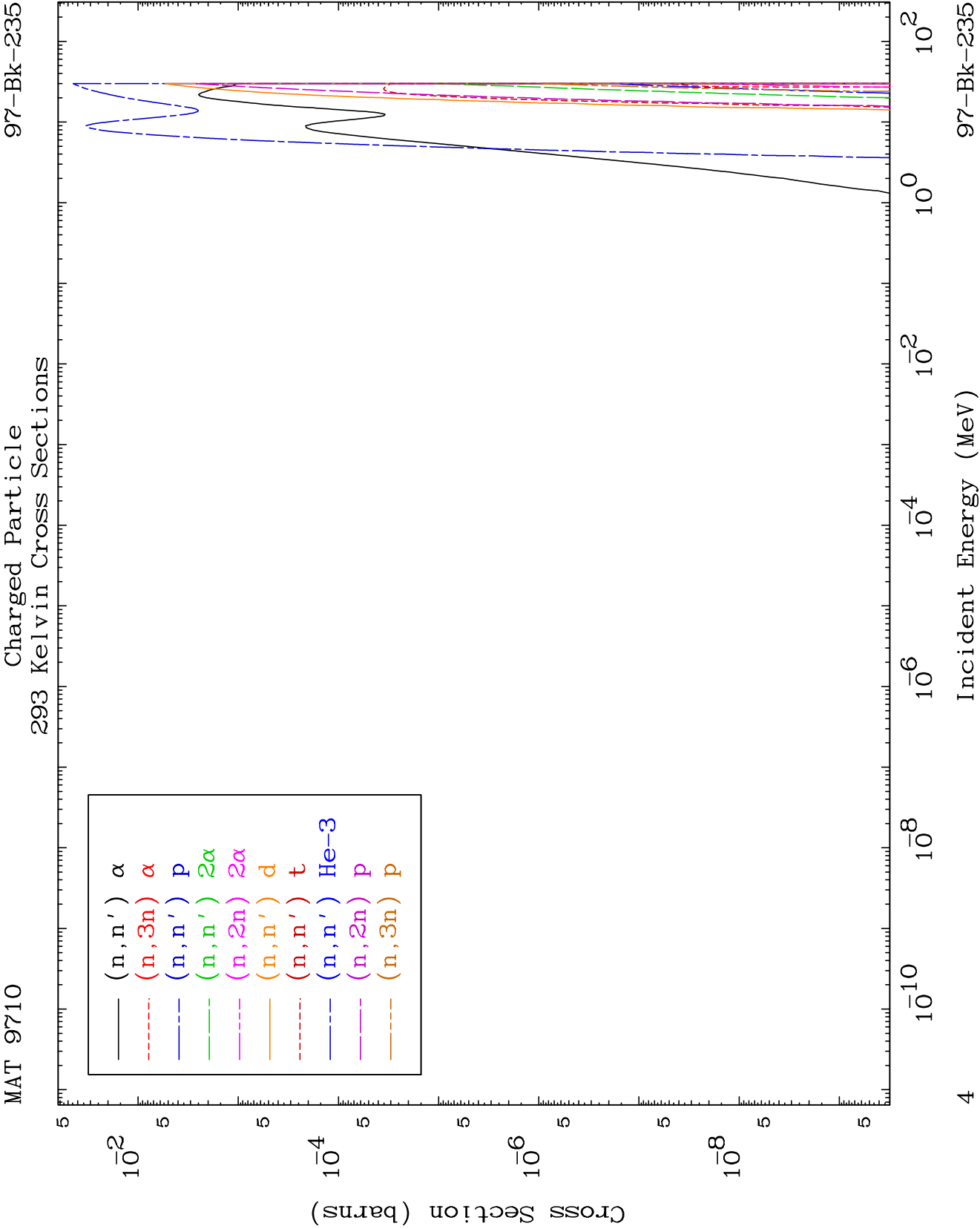
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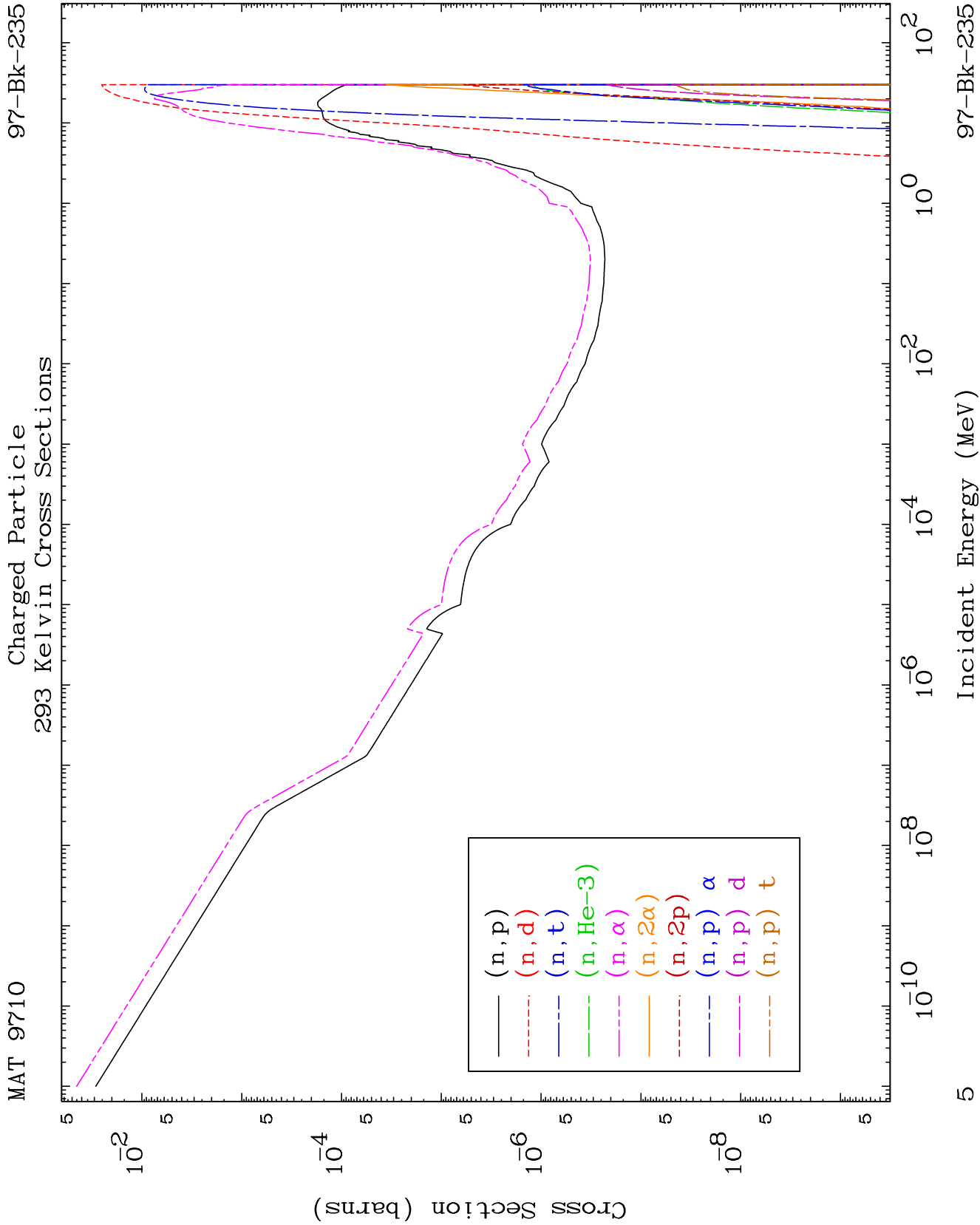


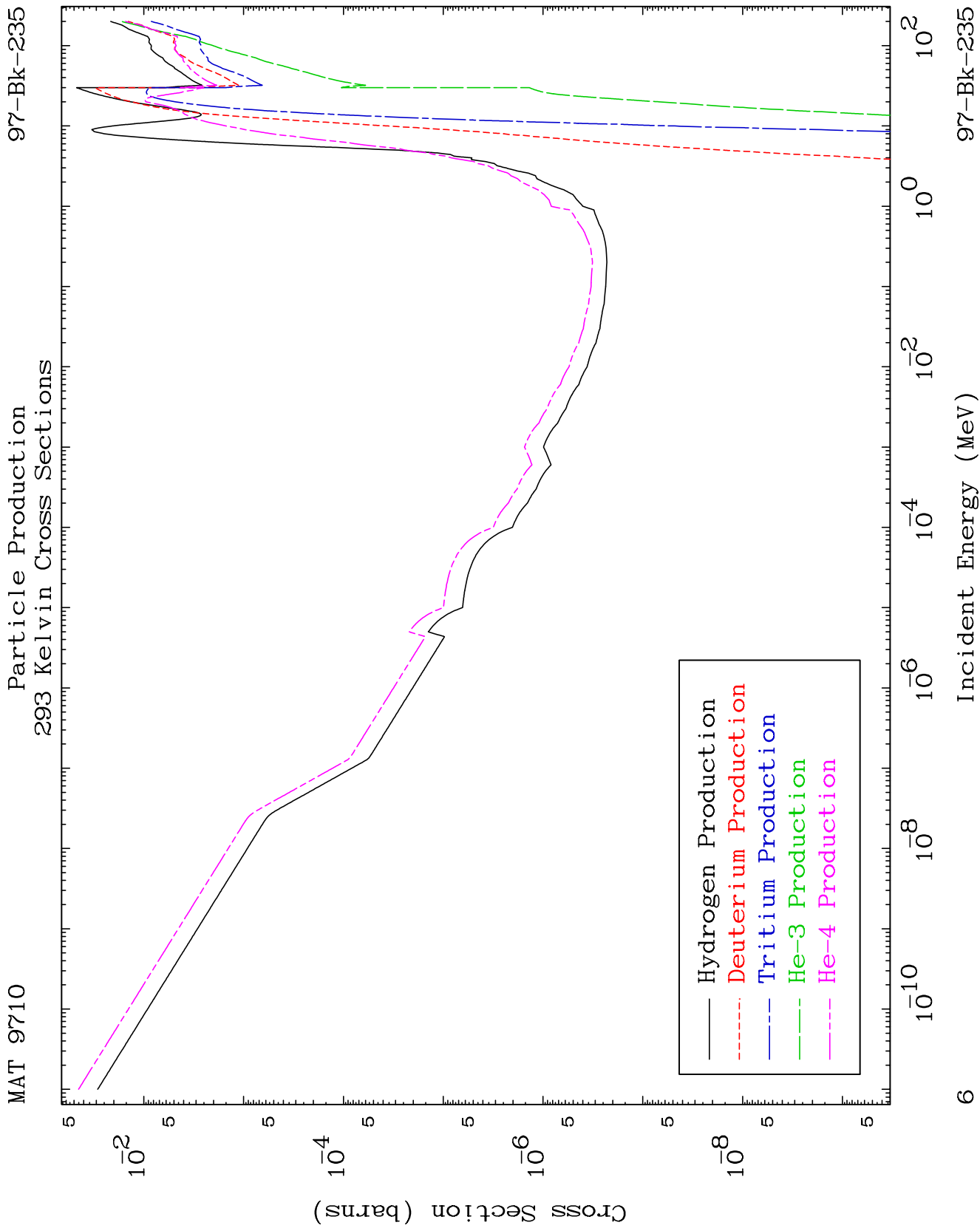
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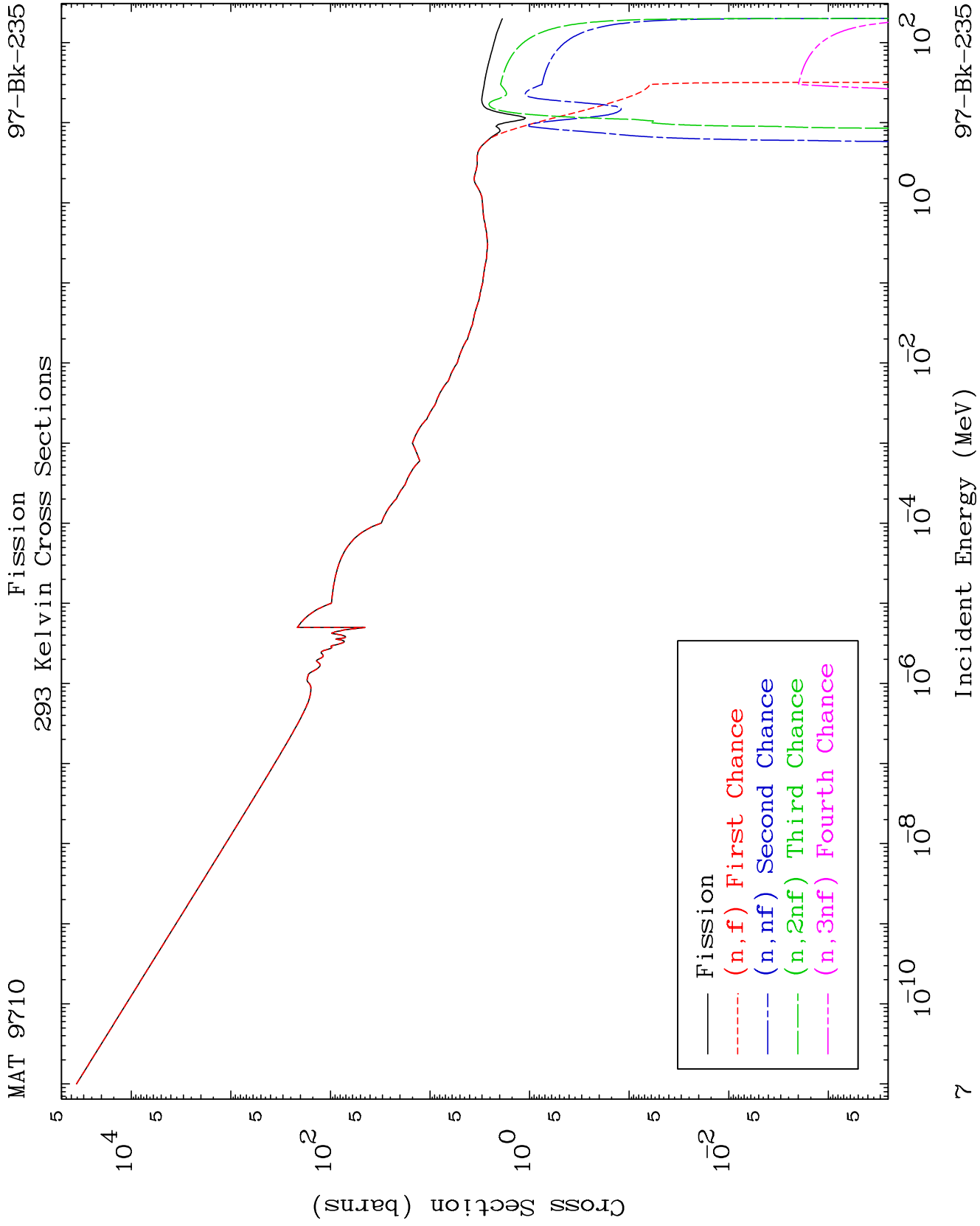
Incident Energy (MeV)

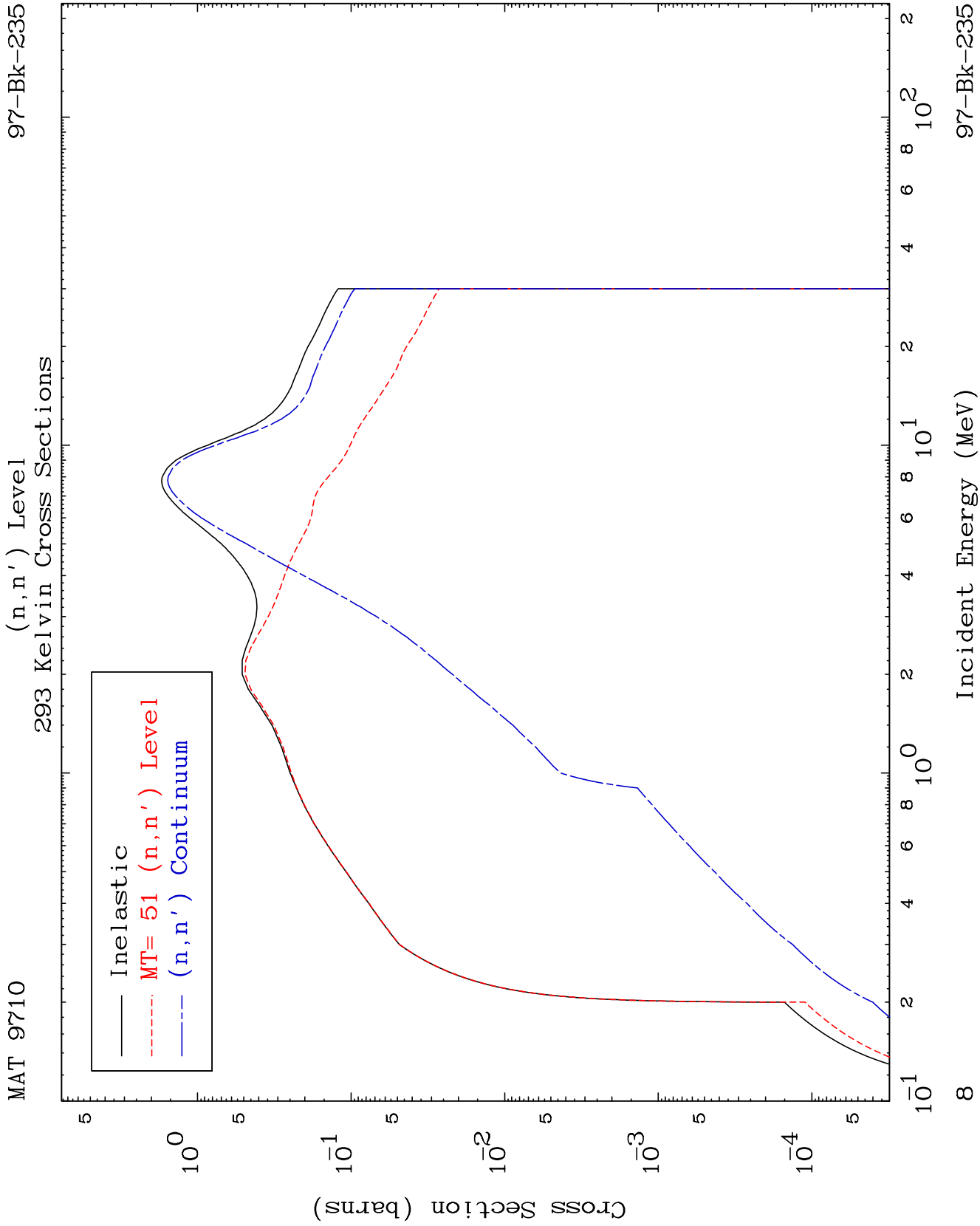
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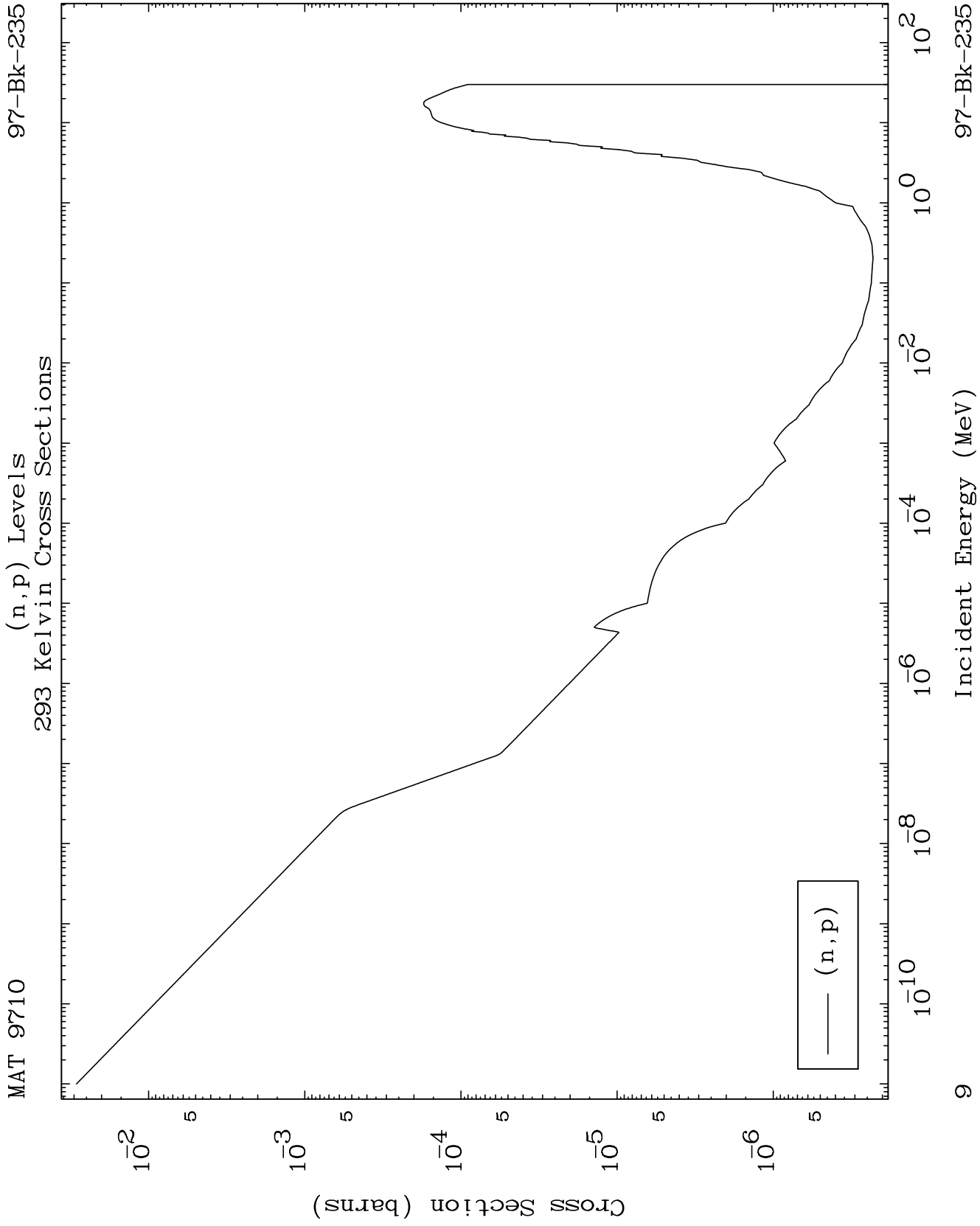


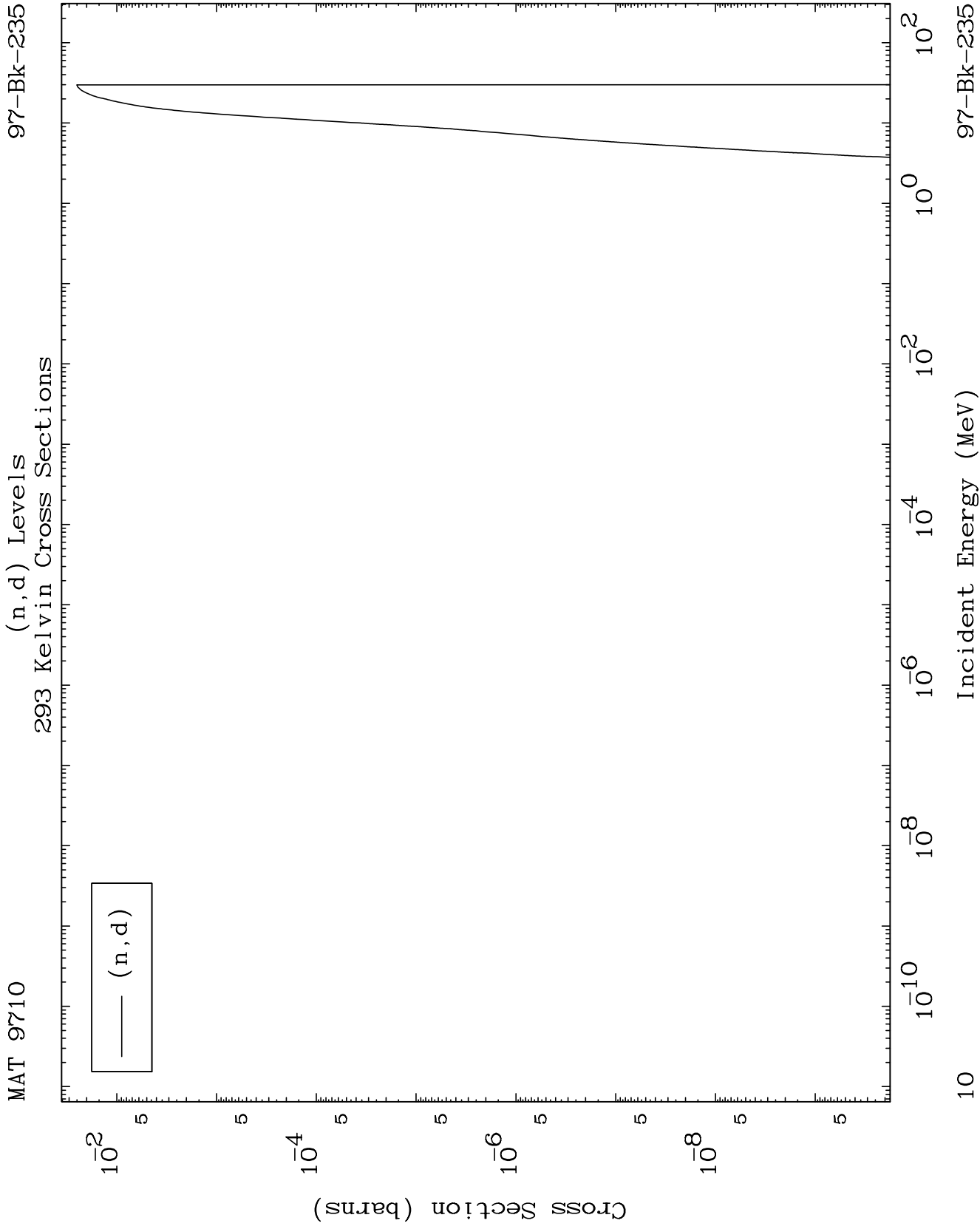


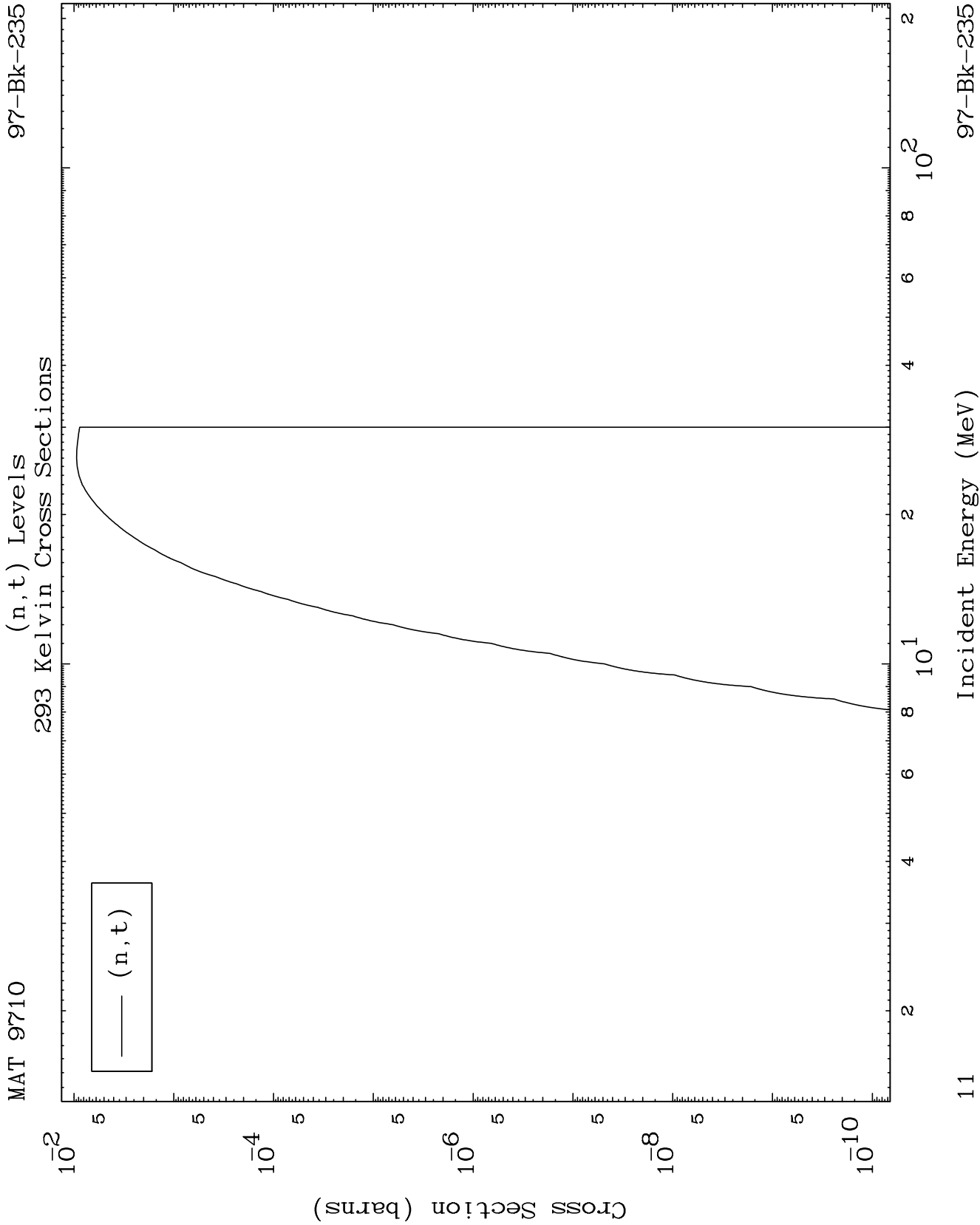












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(n, He3) Levels

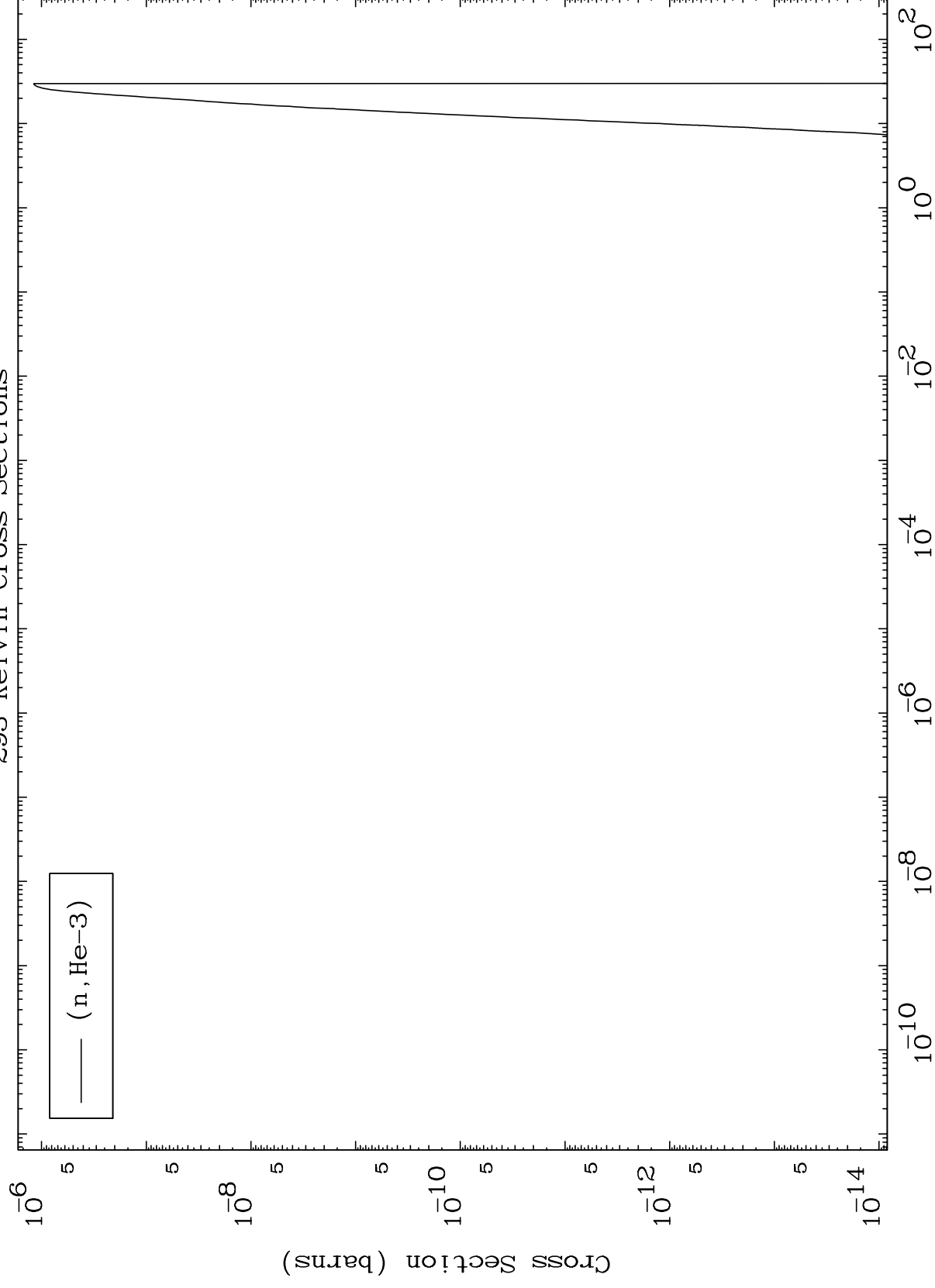
293 Kelvin Cross Sections

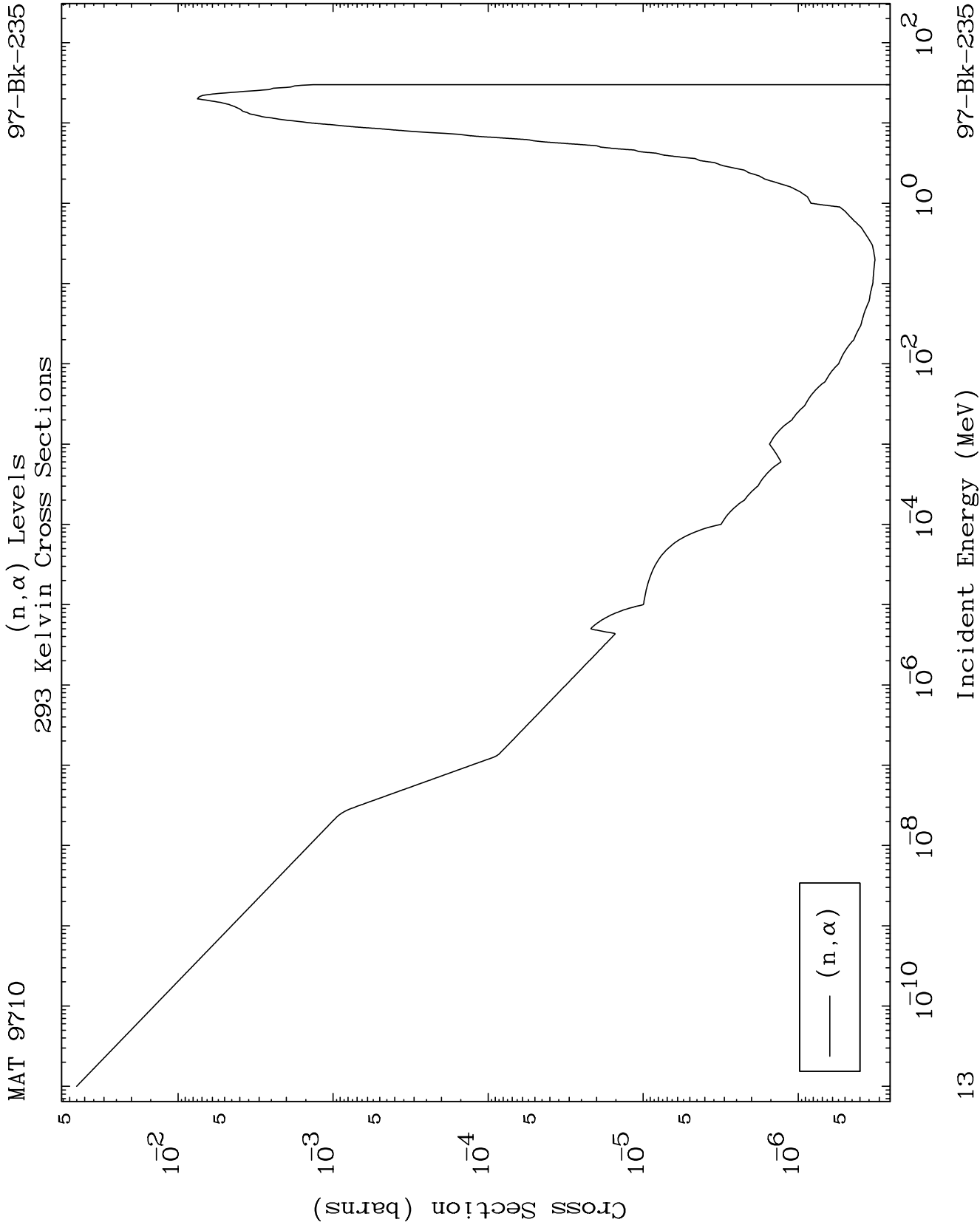
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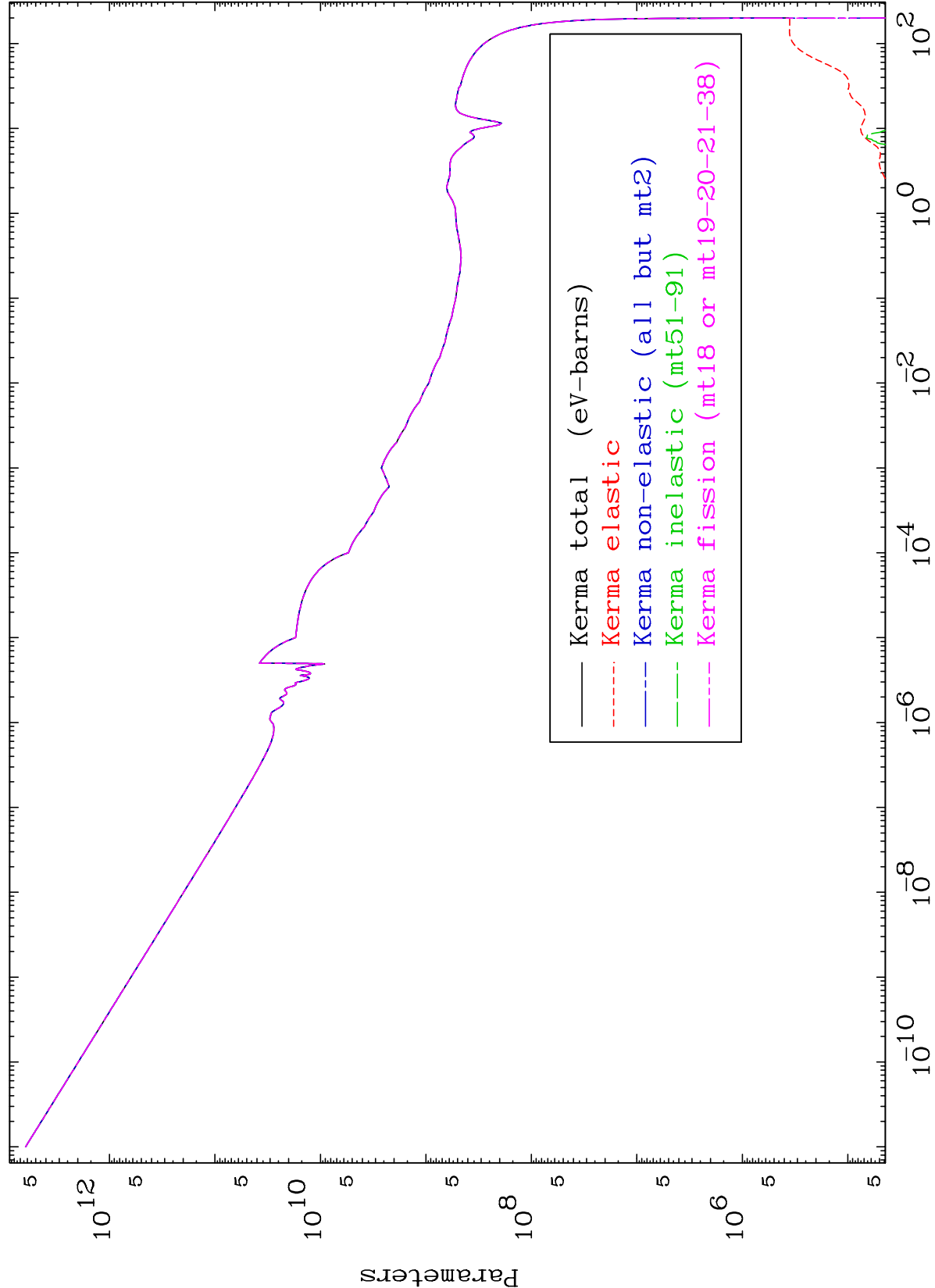
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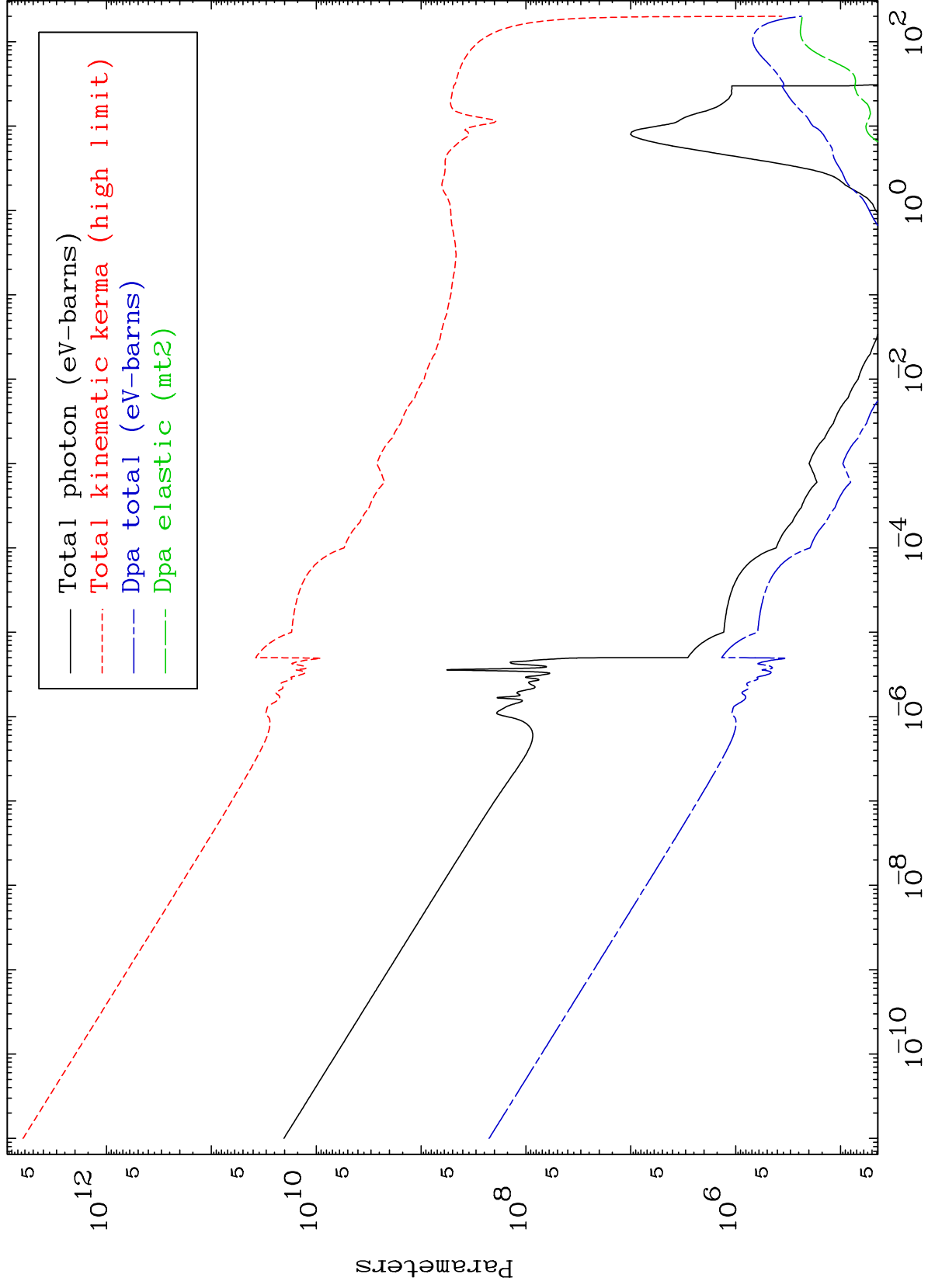
Incident Energy (MeV)

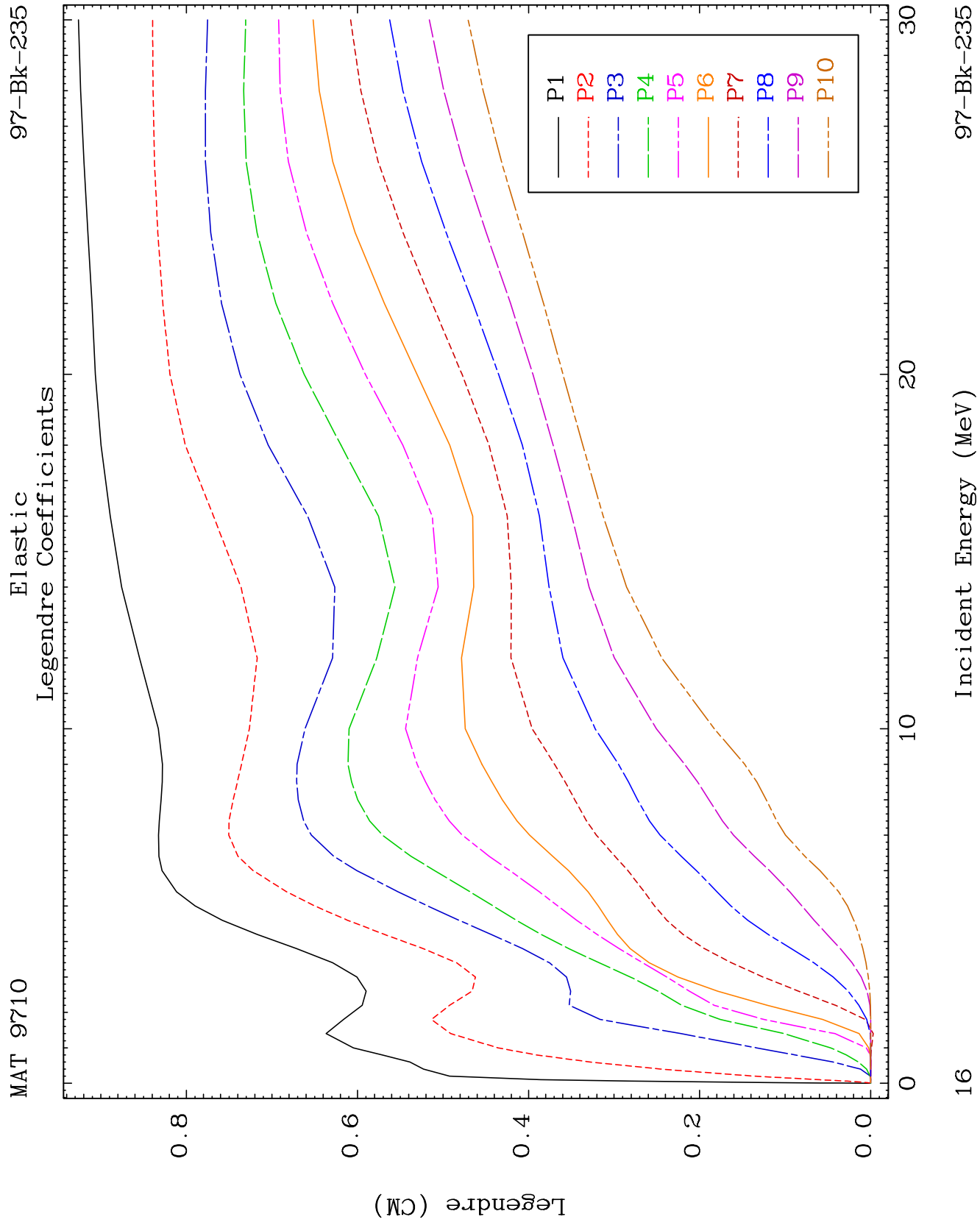
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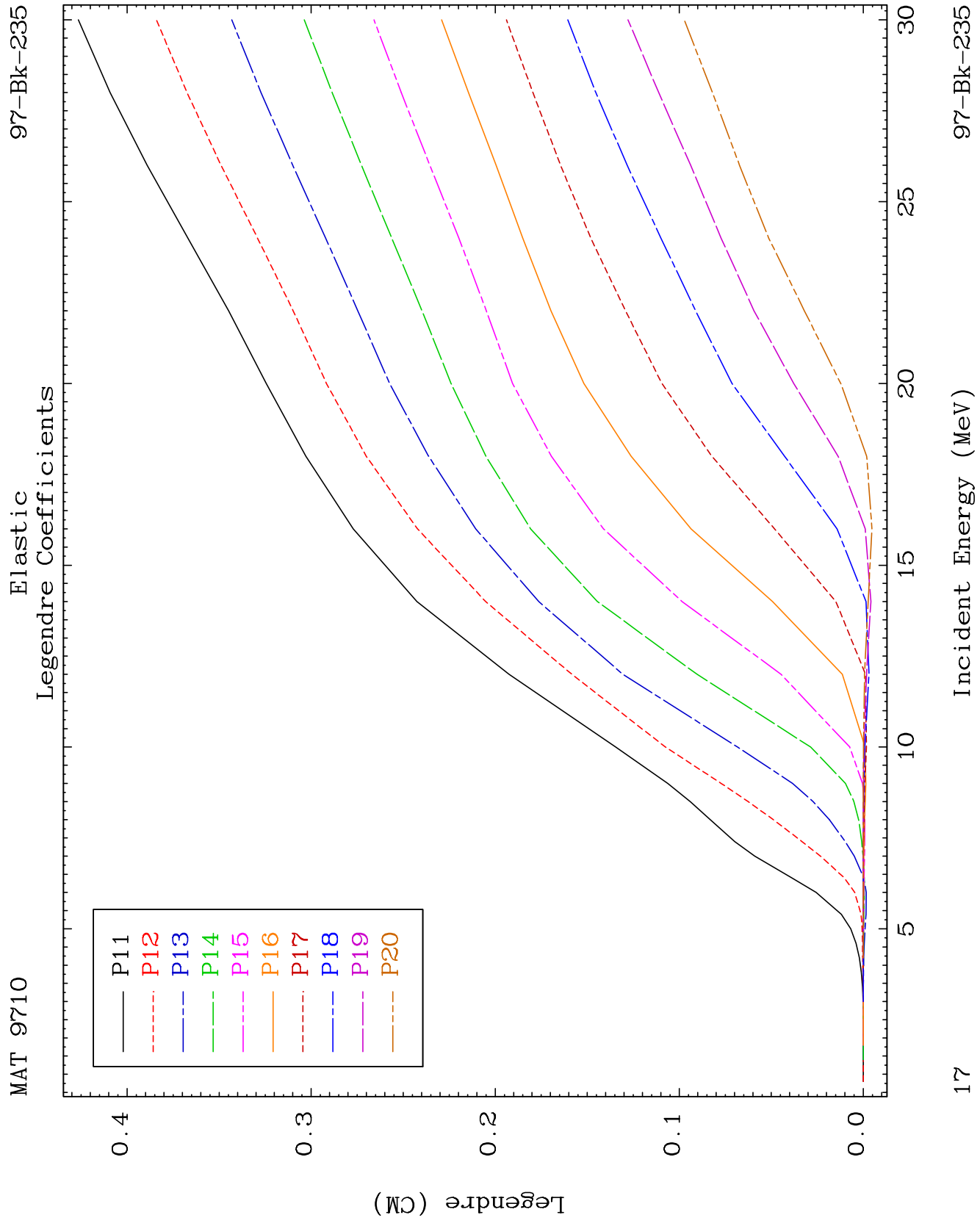


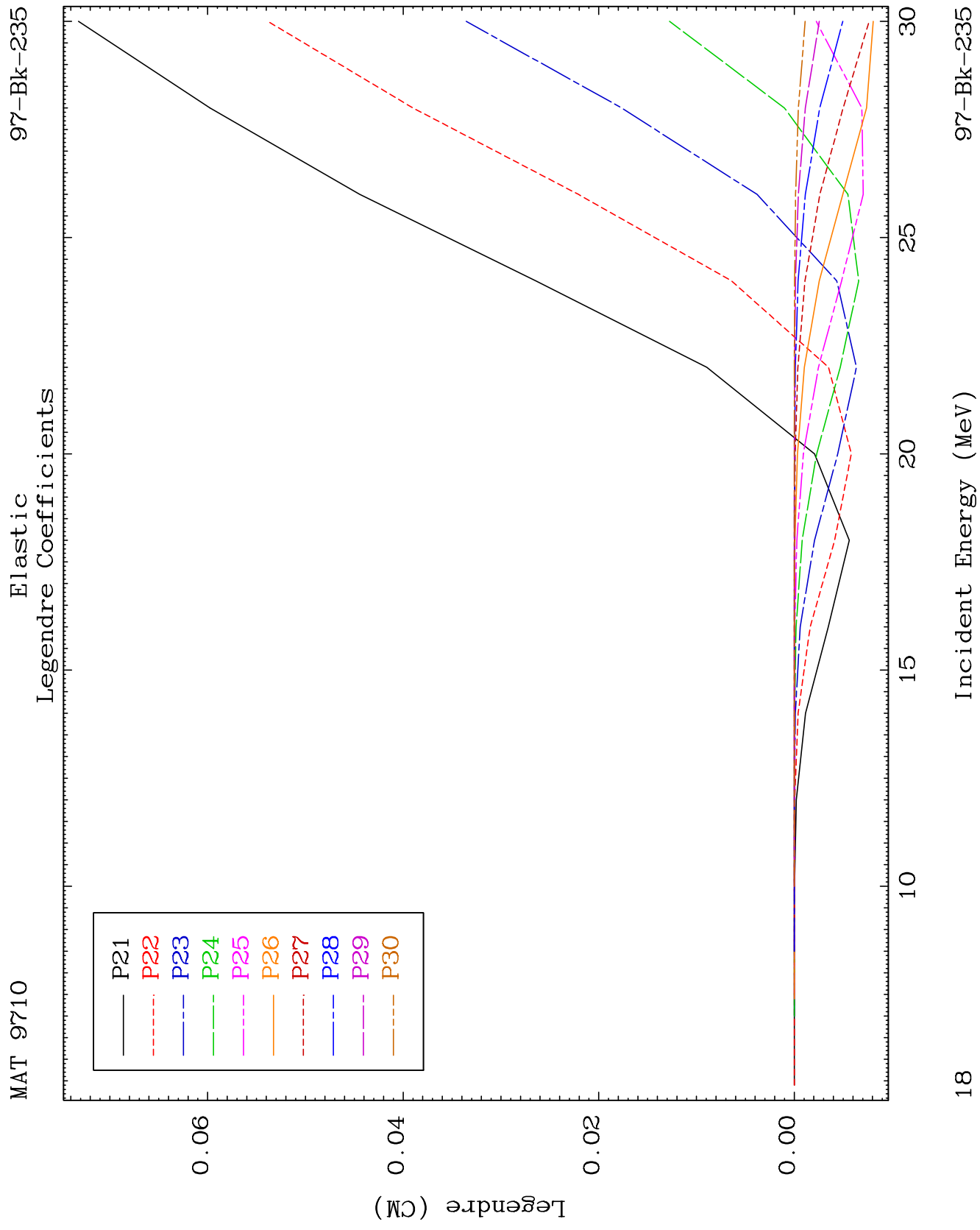


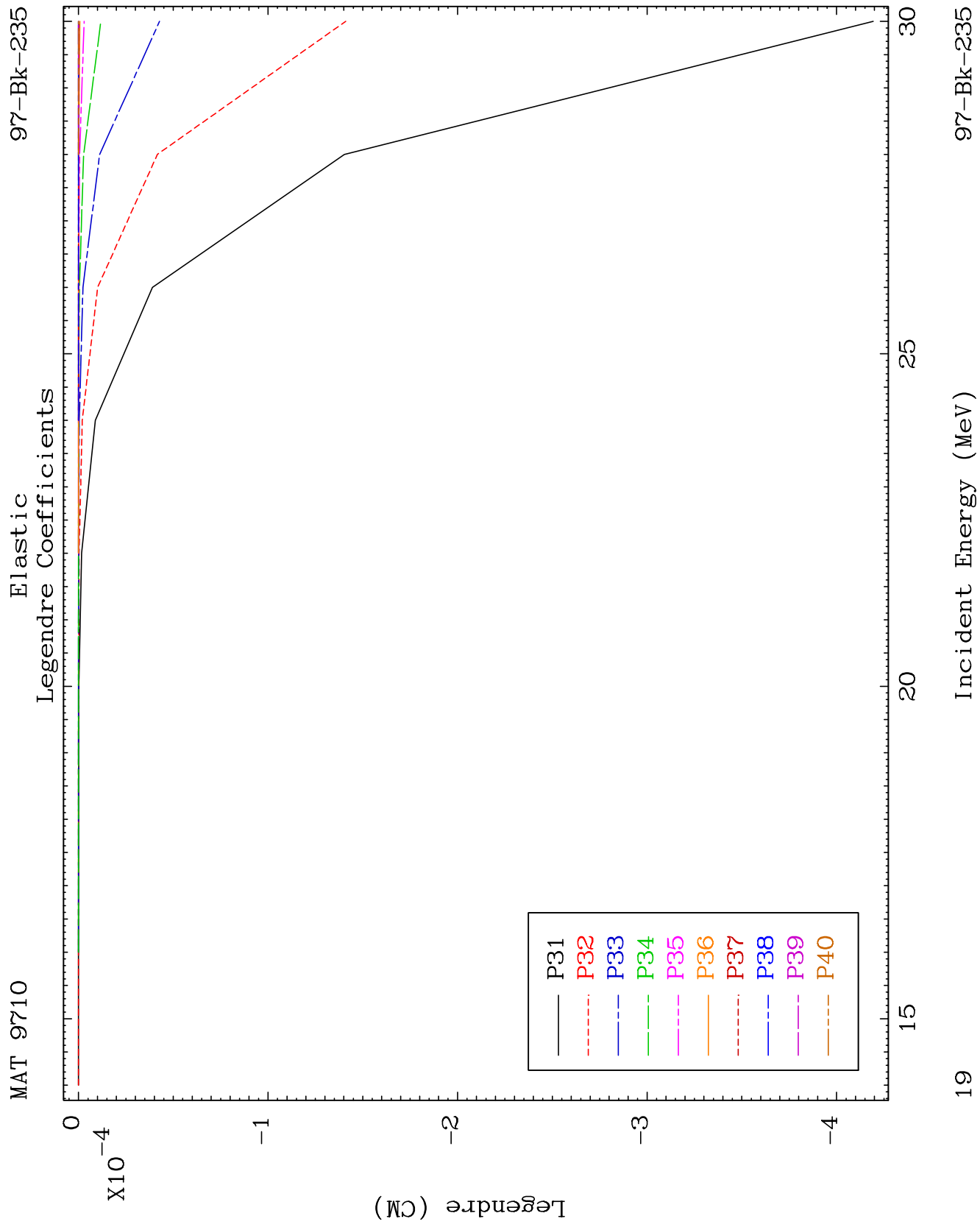


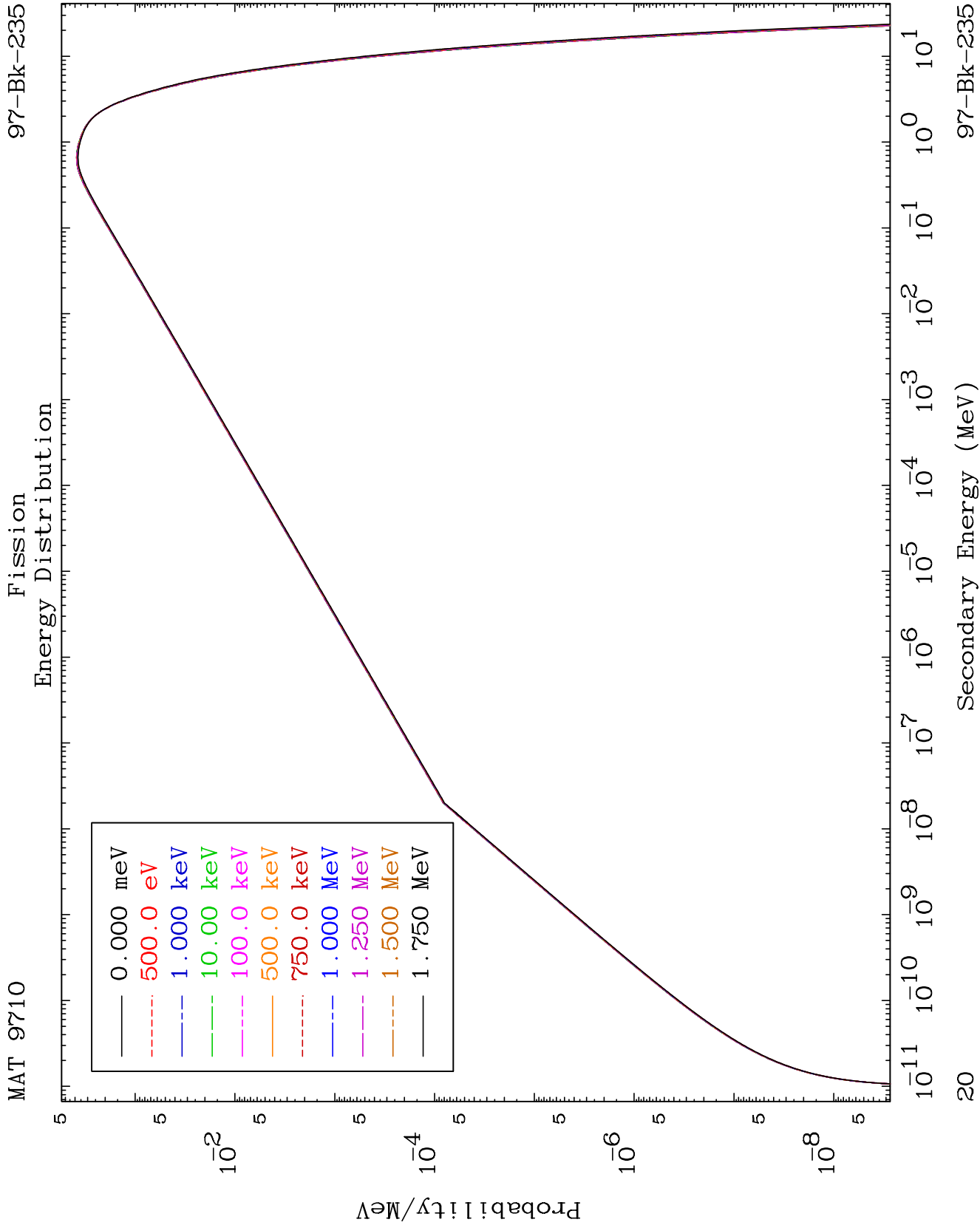


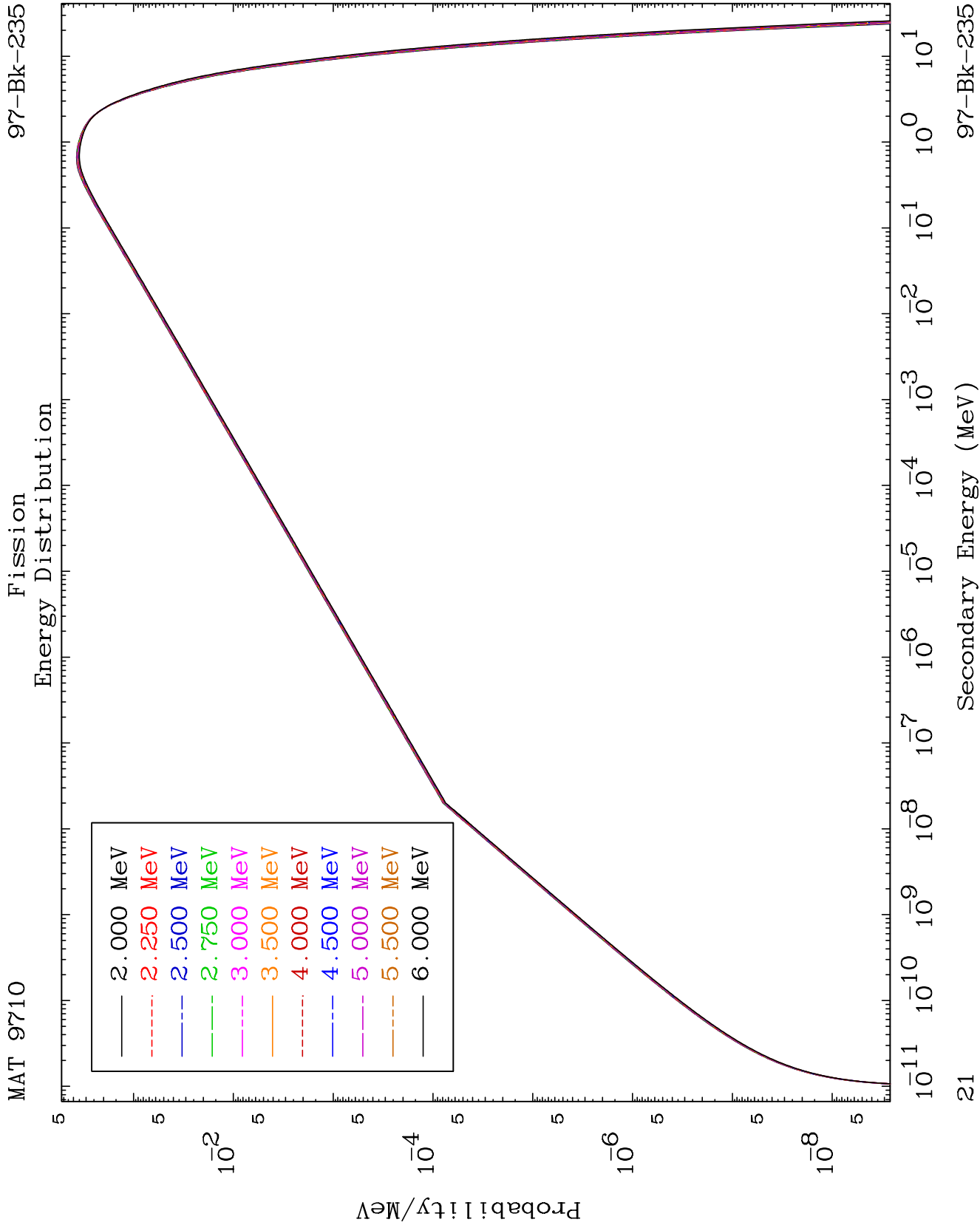








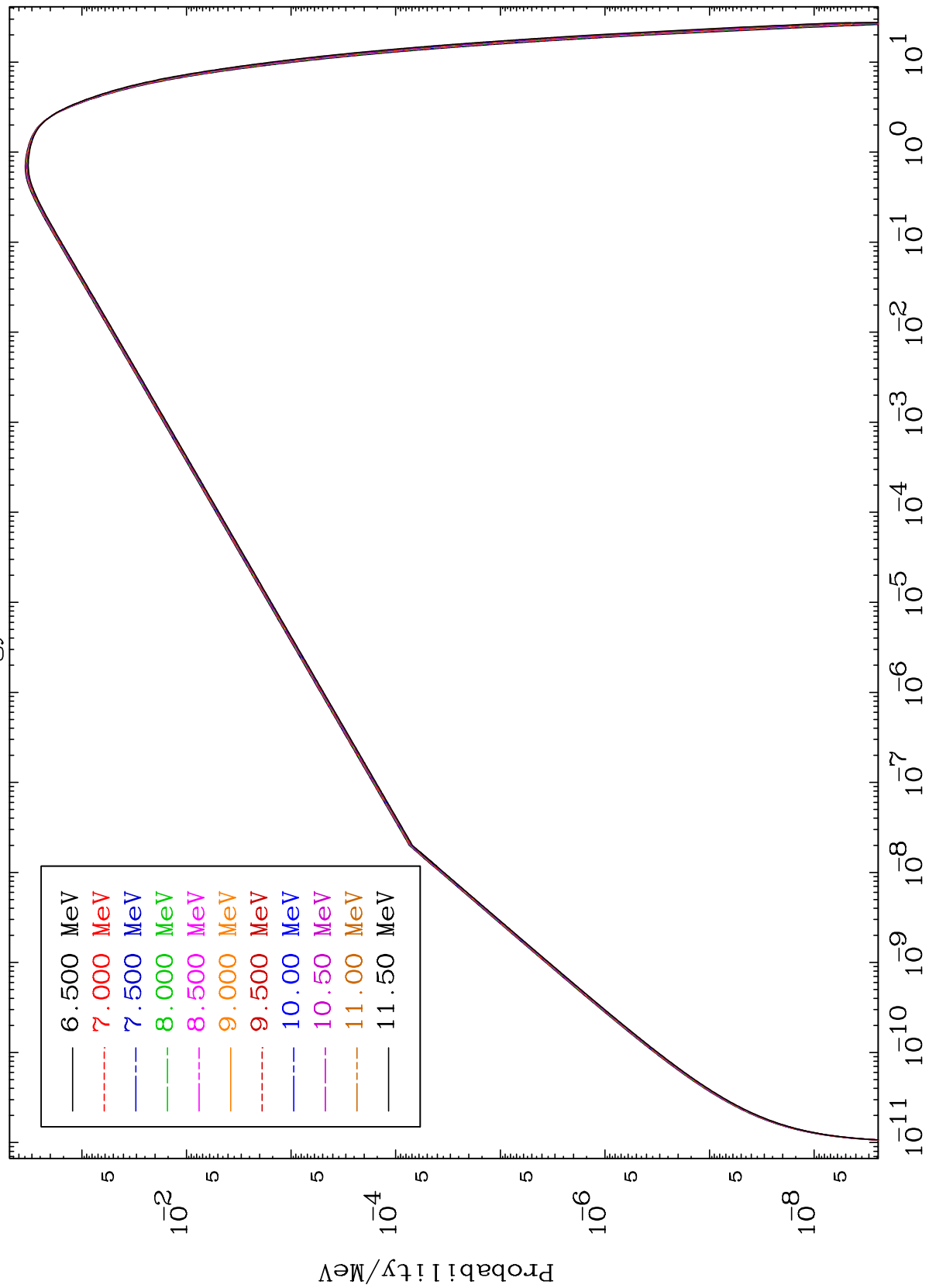




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Fission
Energy Distribution

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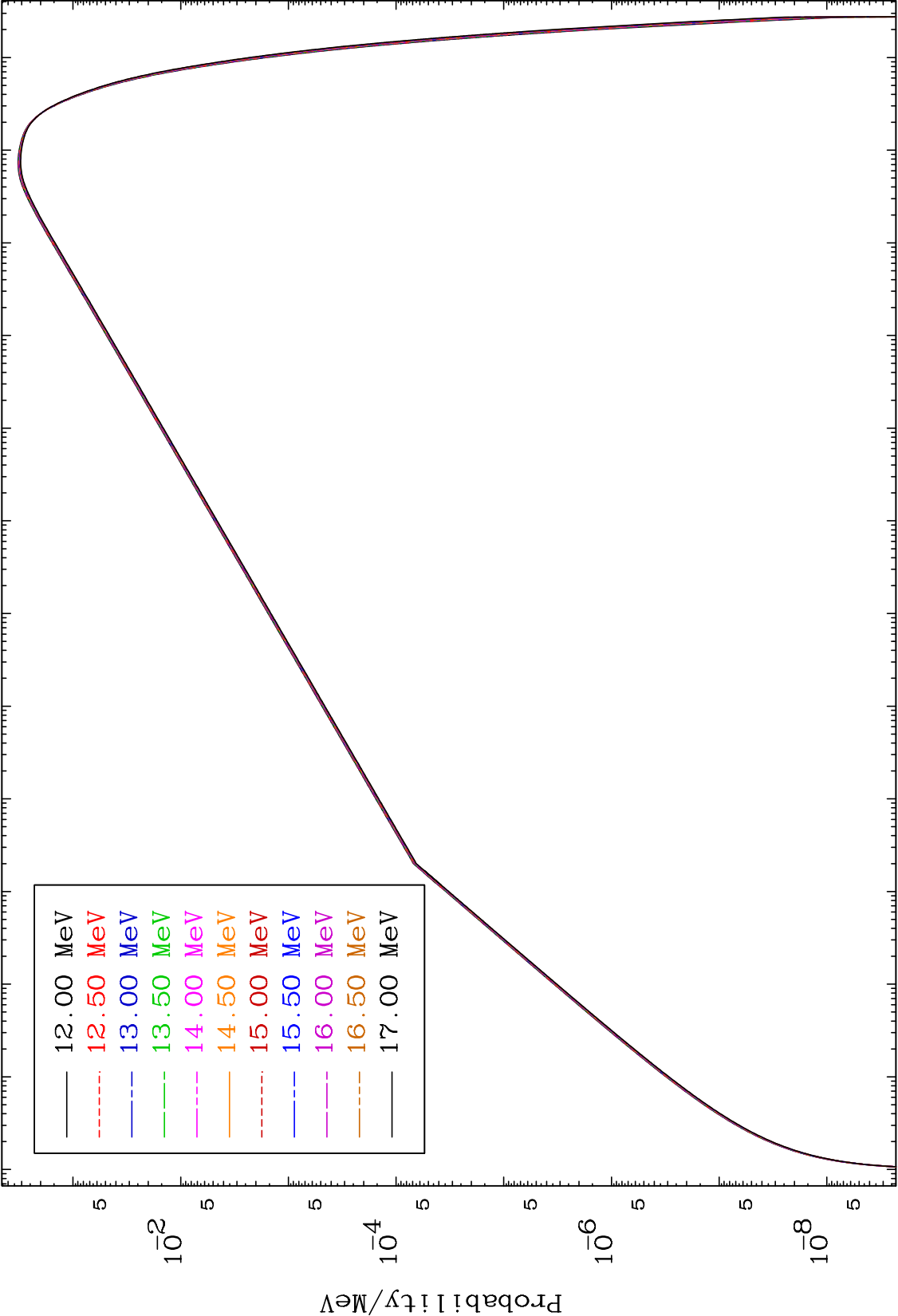


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Fission

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Energy Distribution



Secondary Energy (MeV)

97-Bk-235

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Fission
Energy Distribution

97-Bk-235

