

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

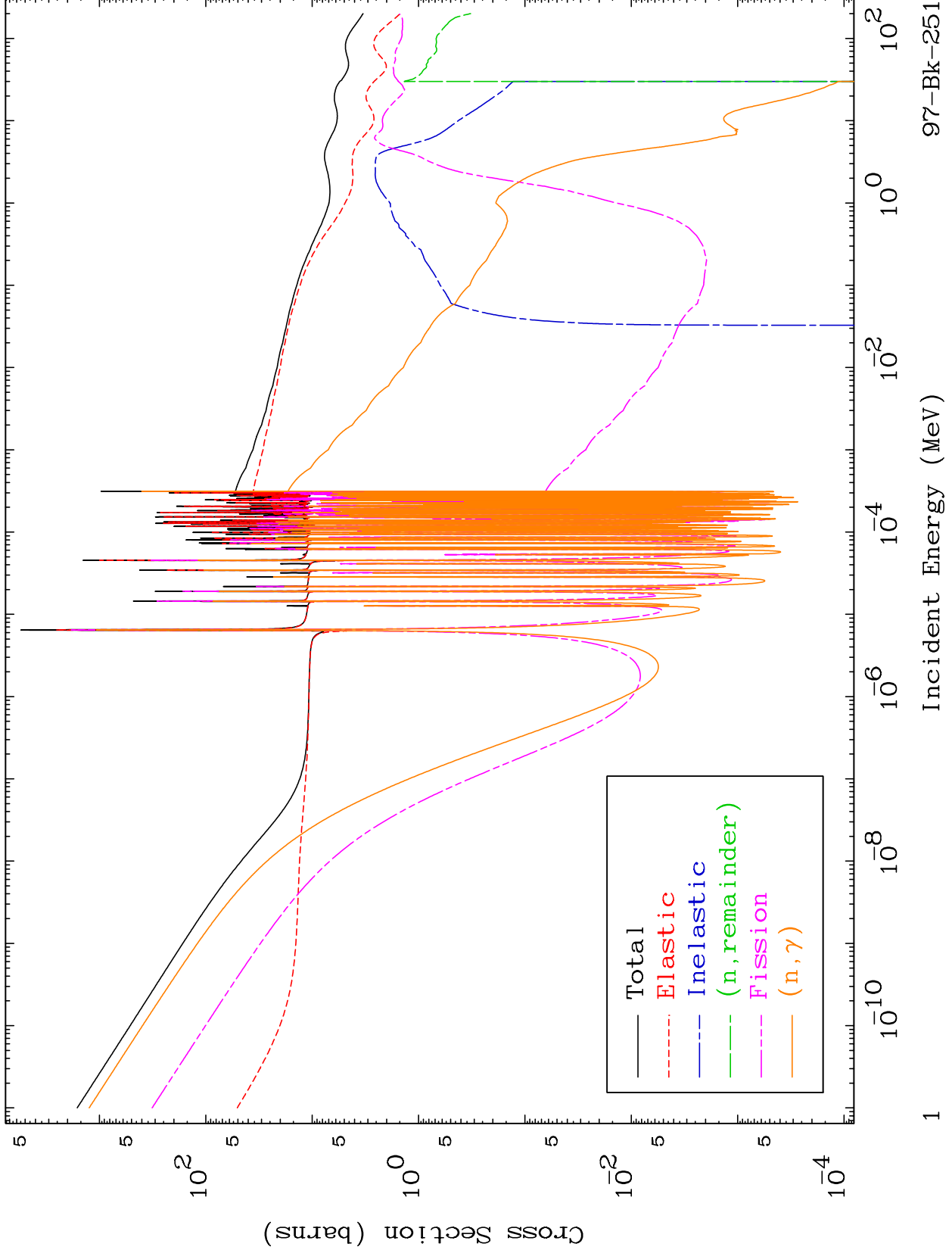
Press Mouse Button to Start

MAT 9758

Major

293 Kelvin Cross Sections

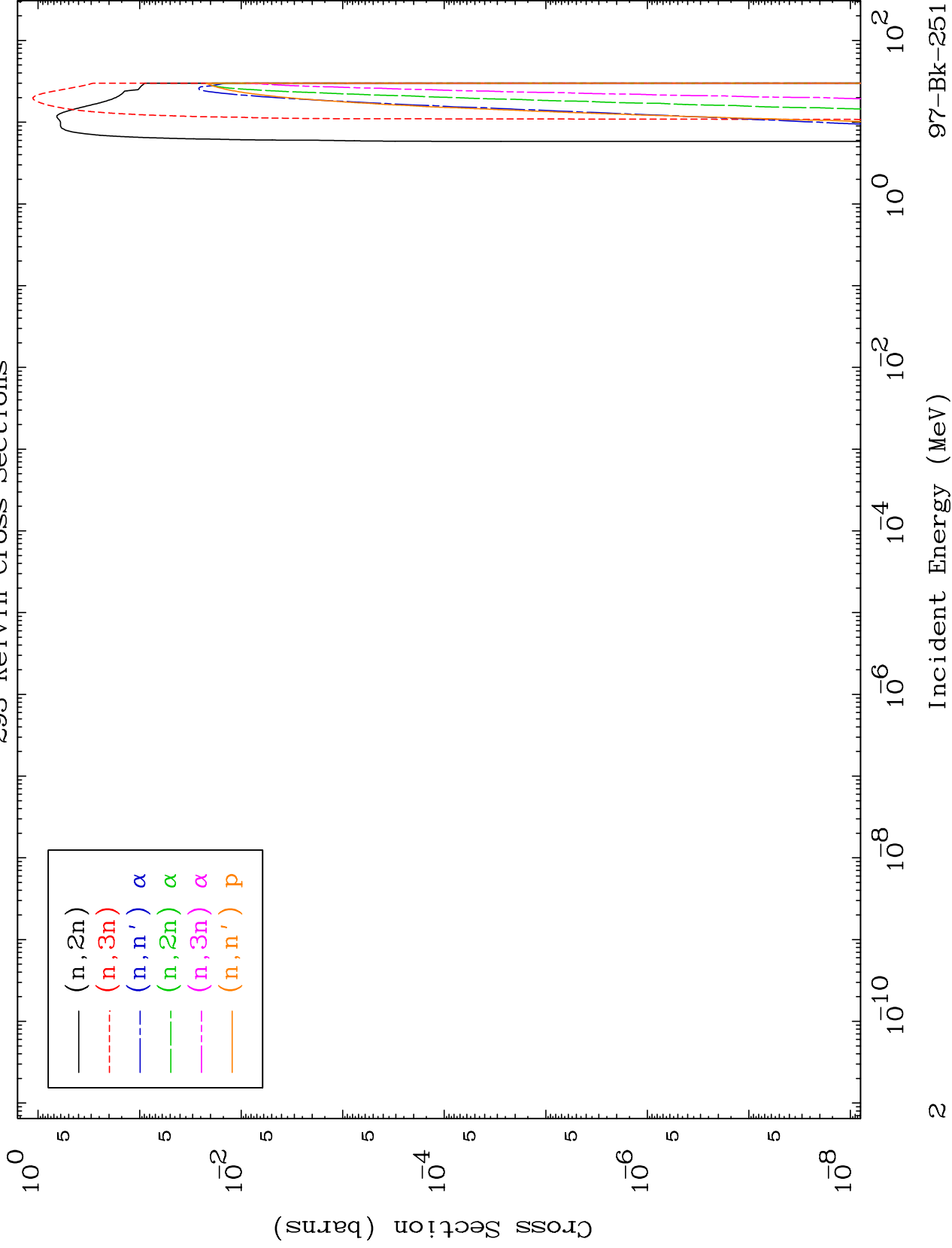
97-Bk-251



MAT 9758

Neutron Production  
293 Kelvin Cross Sections

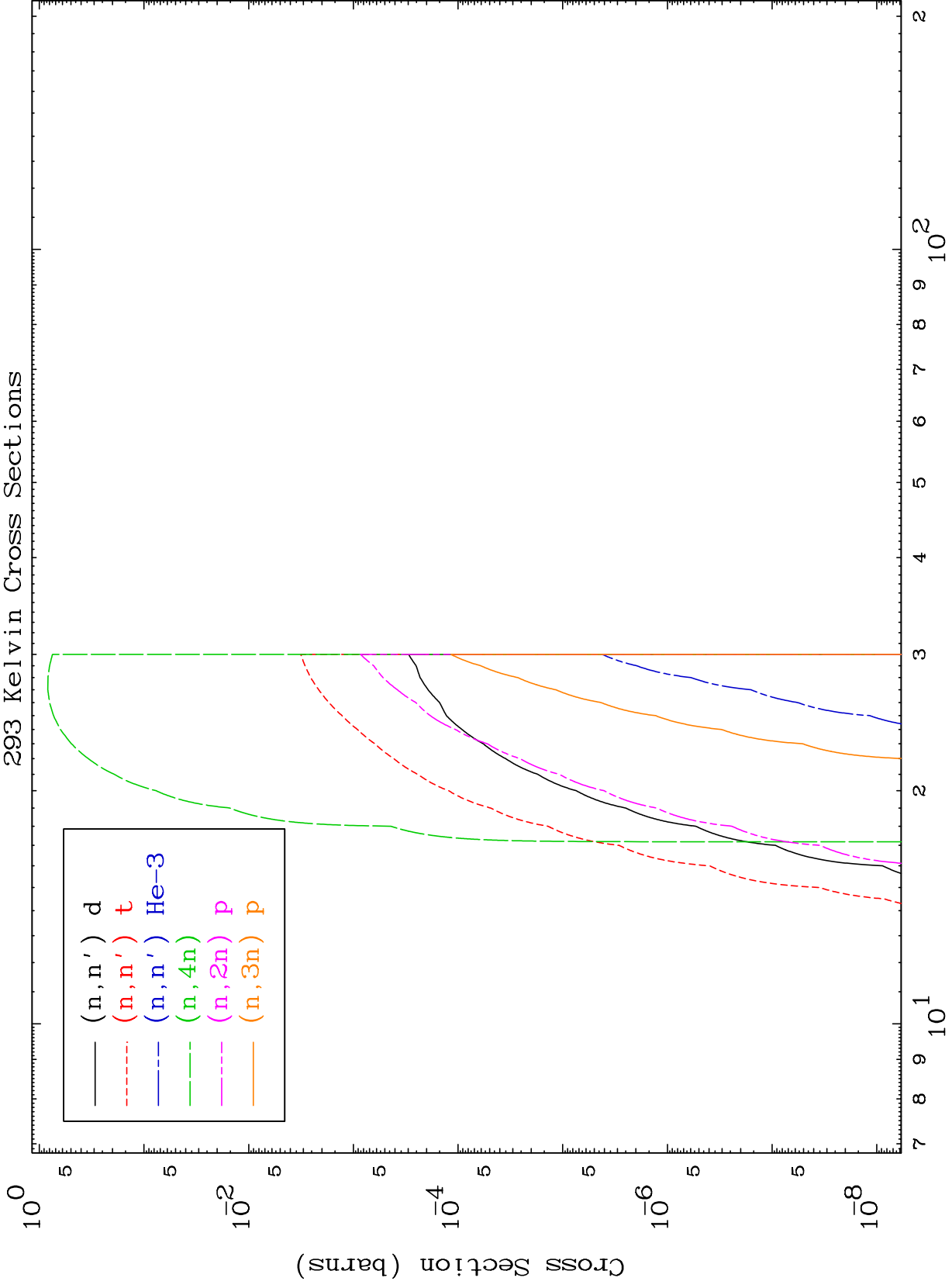
97-Bk-251



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Neutron Production  
293 Kelvin Cross Sections

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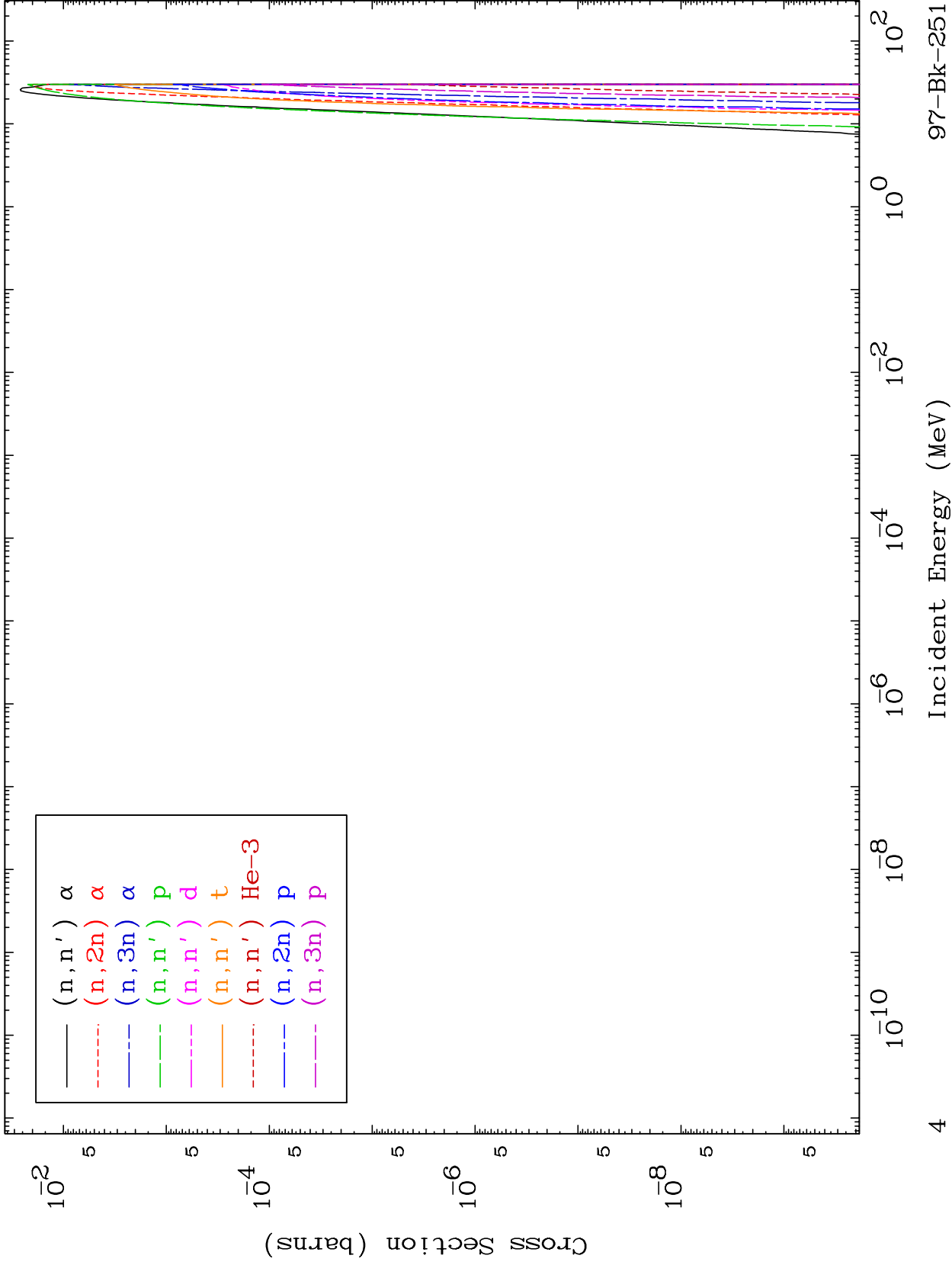
Incident Energy (MeV)

97-Bk-251

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Charged Particle  
293 Kelvin Cross Sections

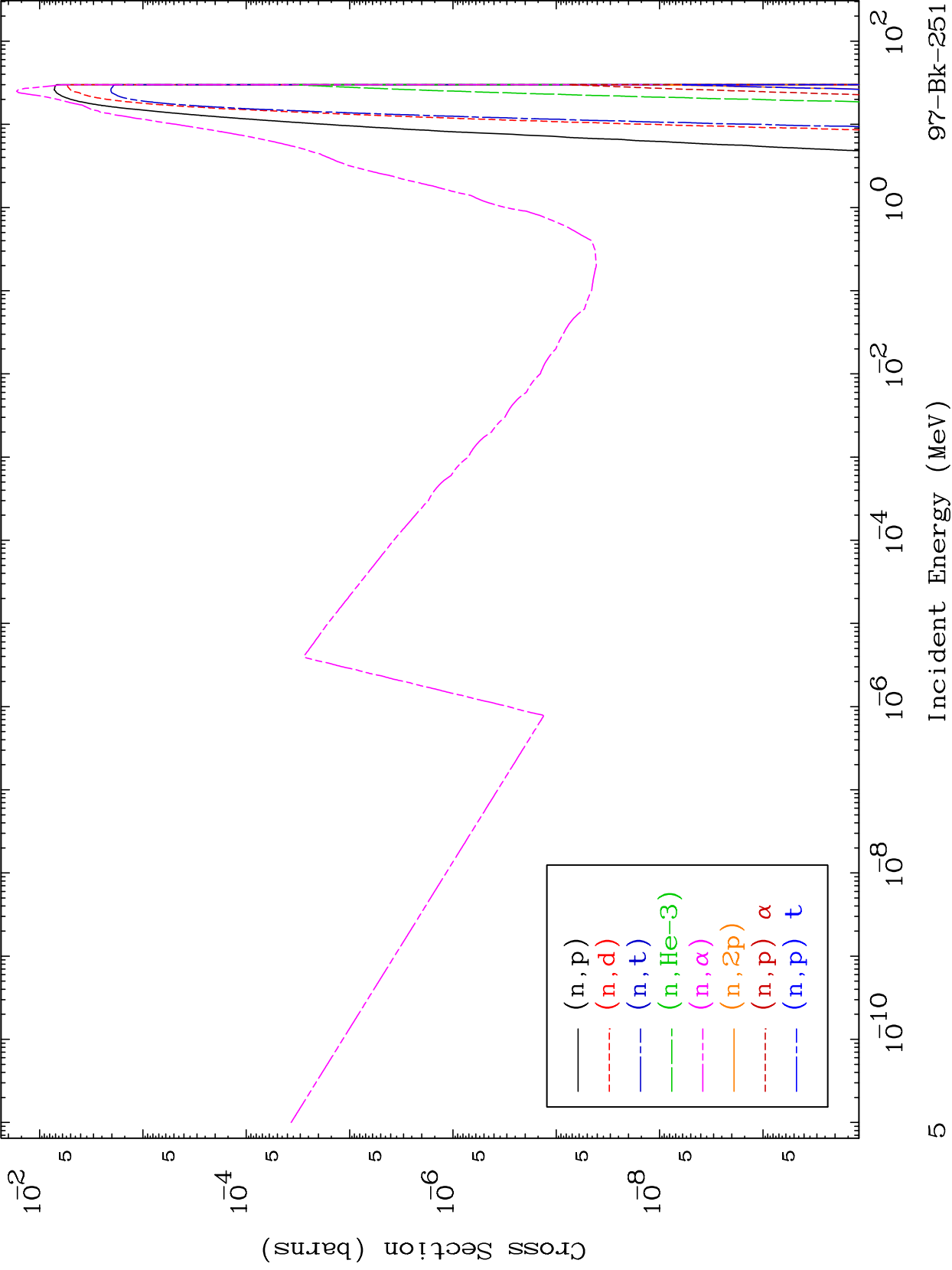
97-Bk-251



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Charged Particle  
293 Kelvin Cross Sections

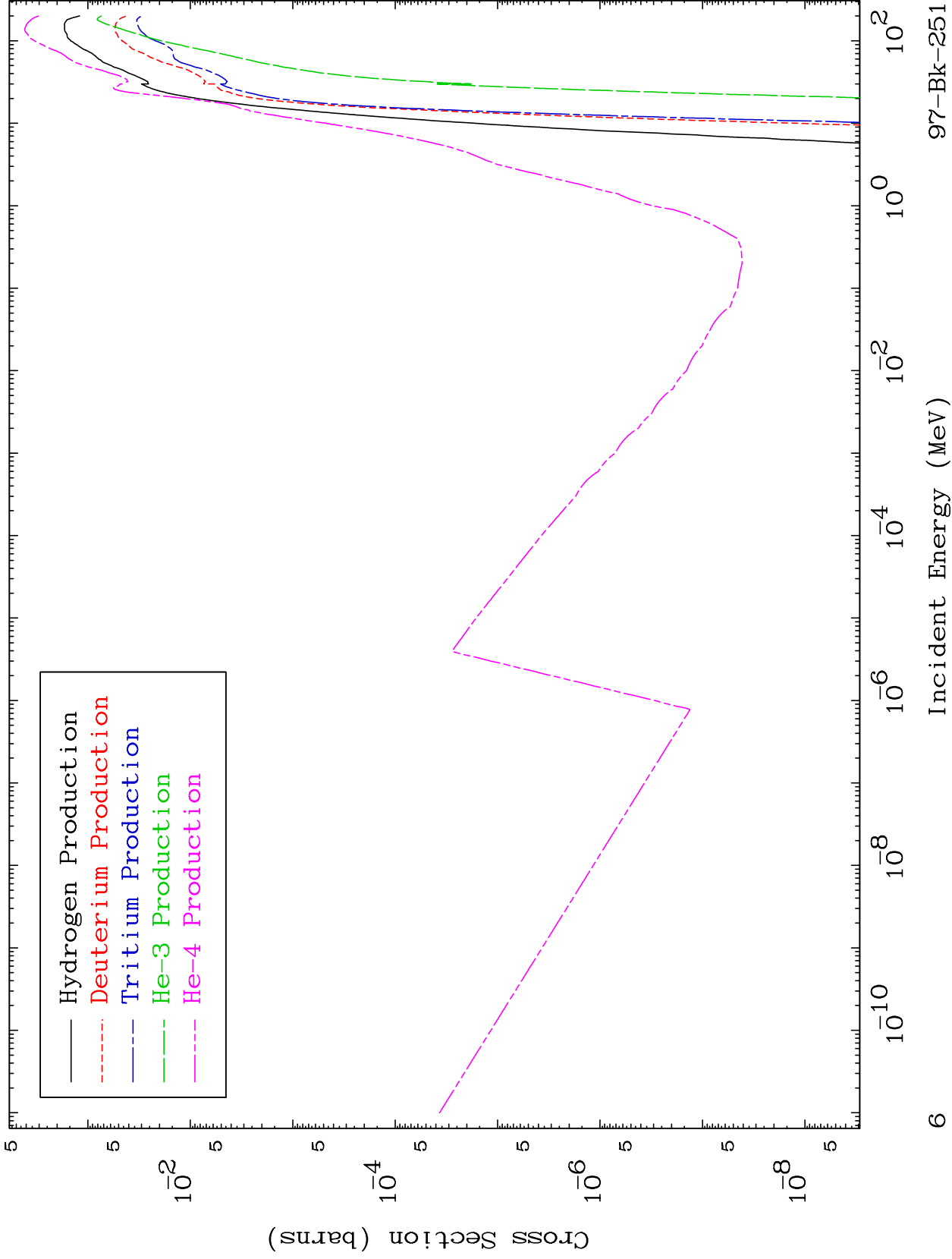
97-Bk-251



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Particle Production  
293 Kelvin Cross Sections

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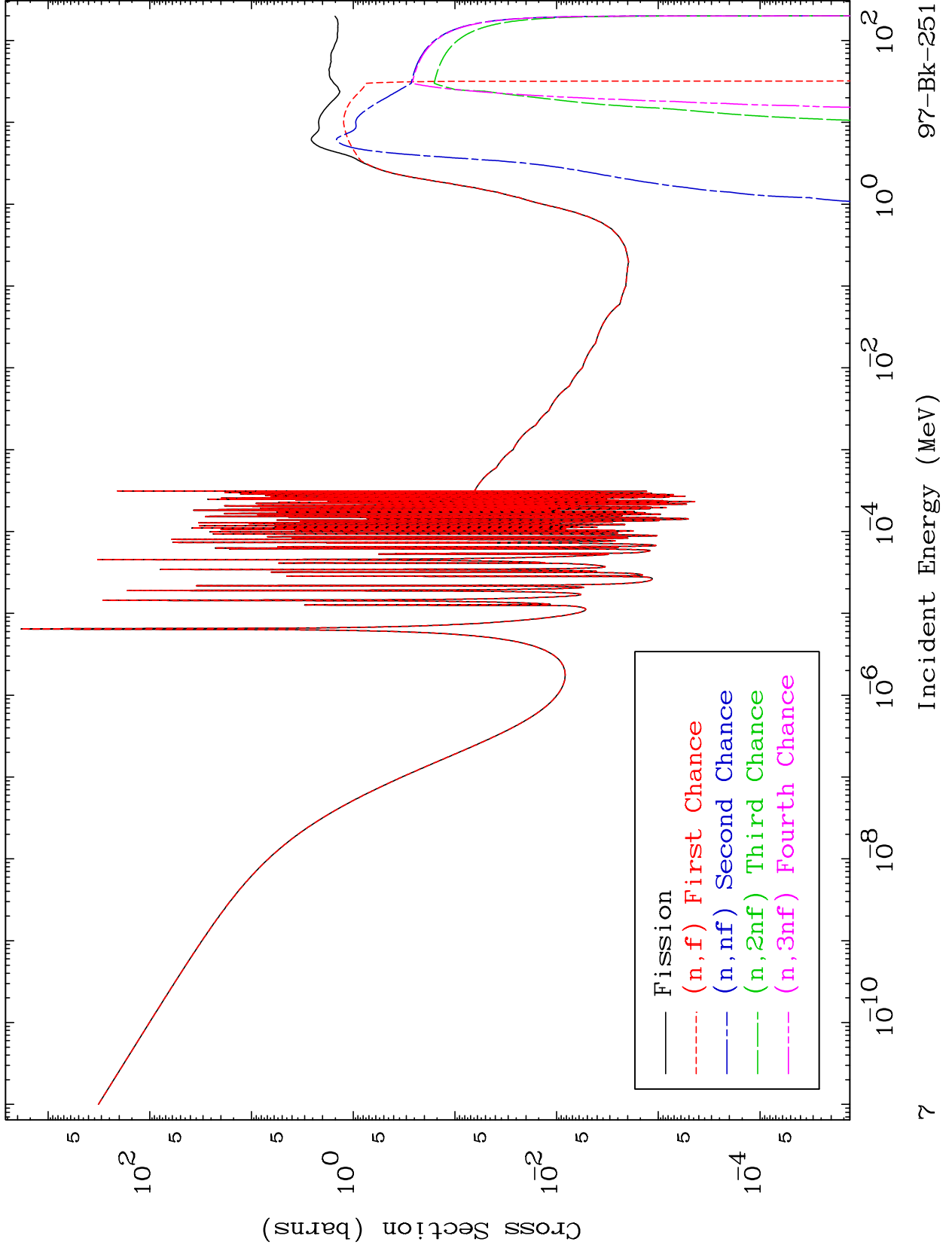
6

97-Bk-251

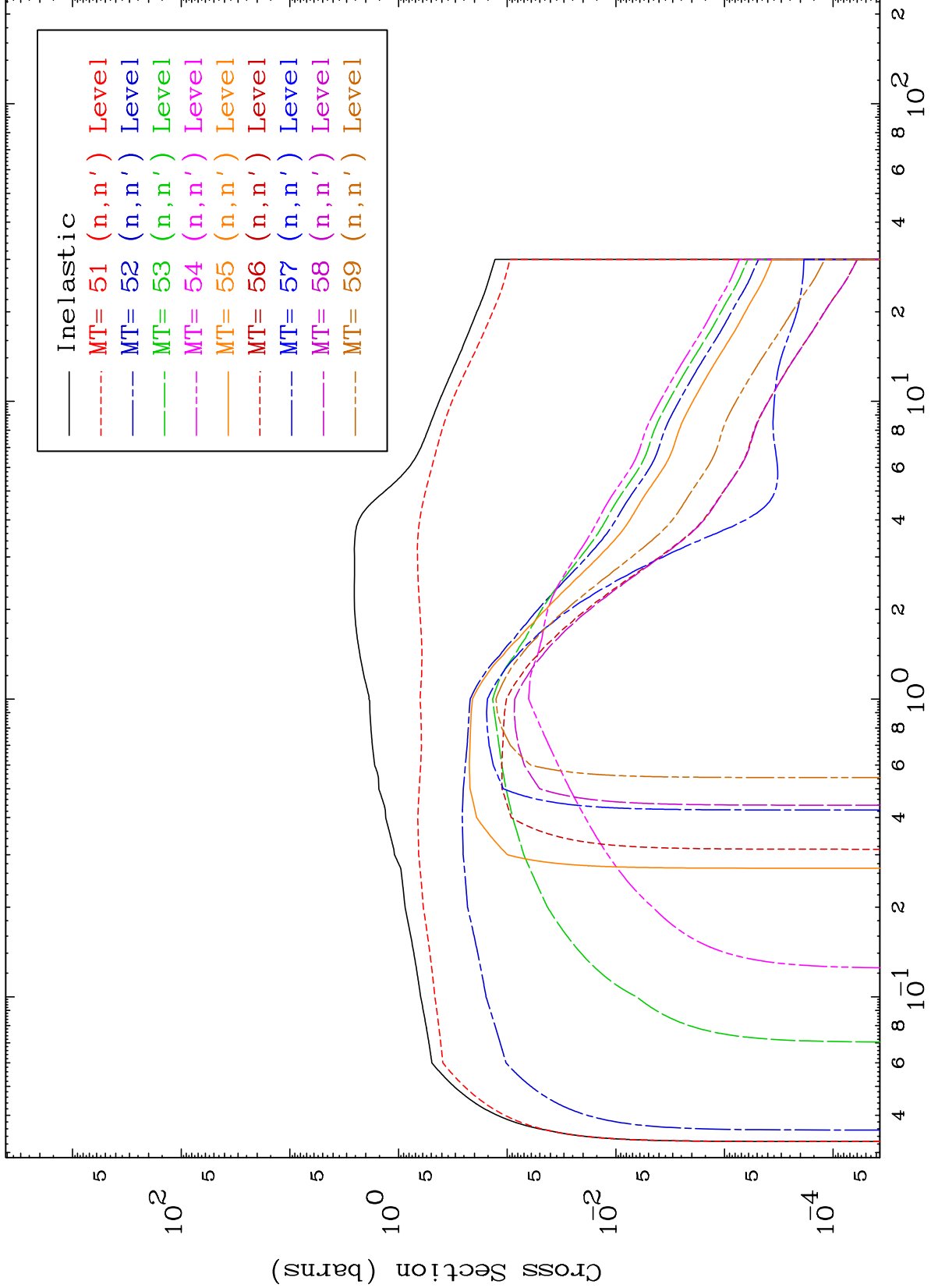
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Fission  
293 Kelvin Cross Sections

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293 Kelvin Cross Sections

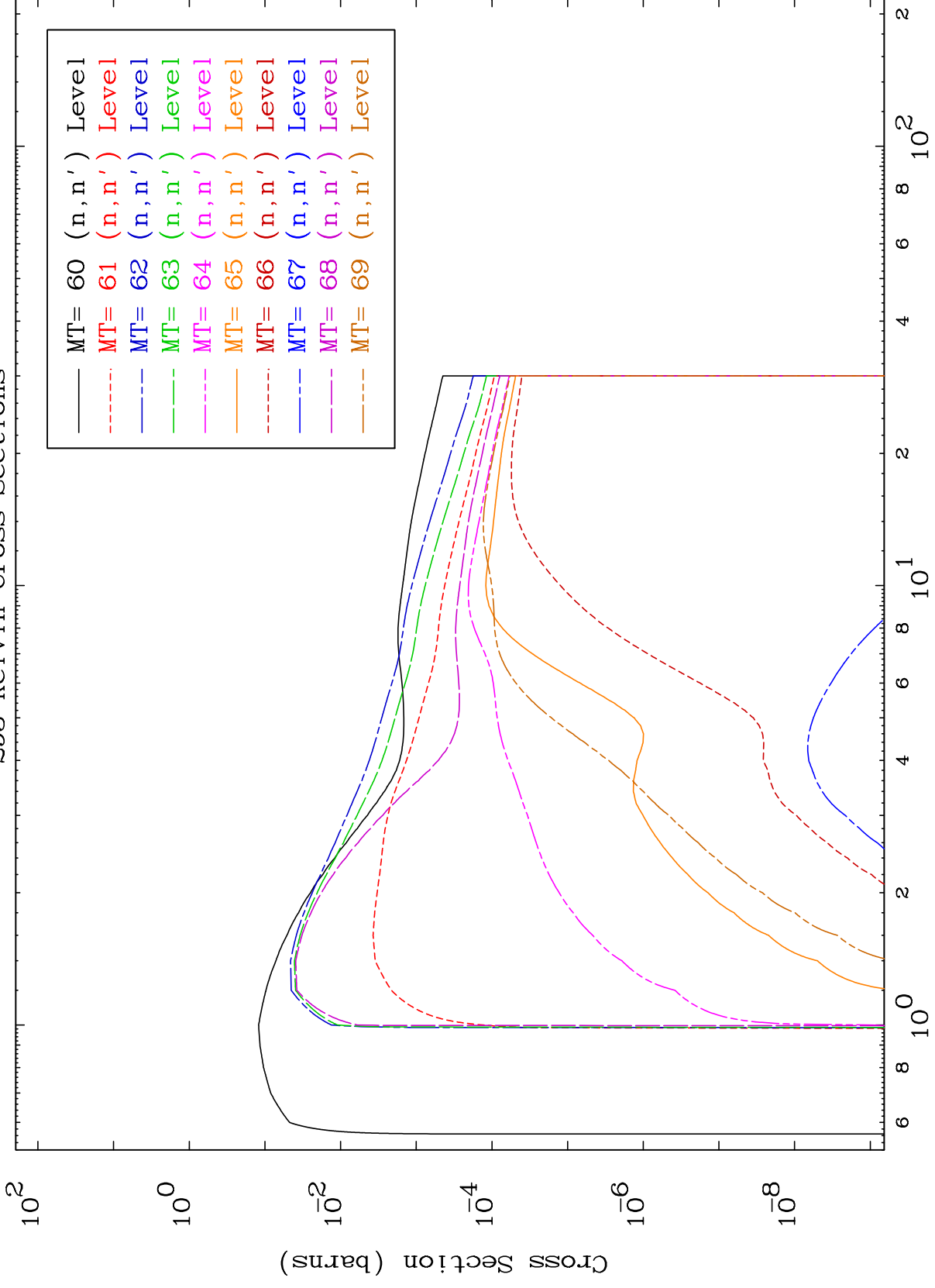


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(n,n') Level

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293 Kelvin Cross Sections



9

Incident Energy (MeV)

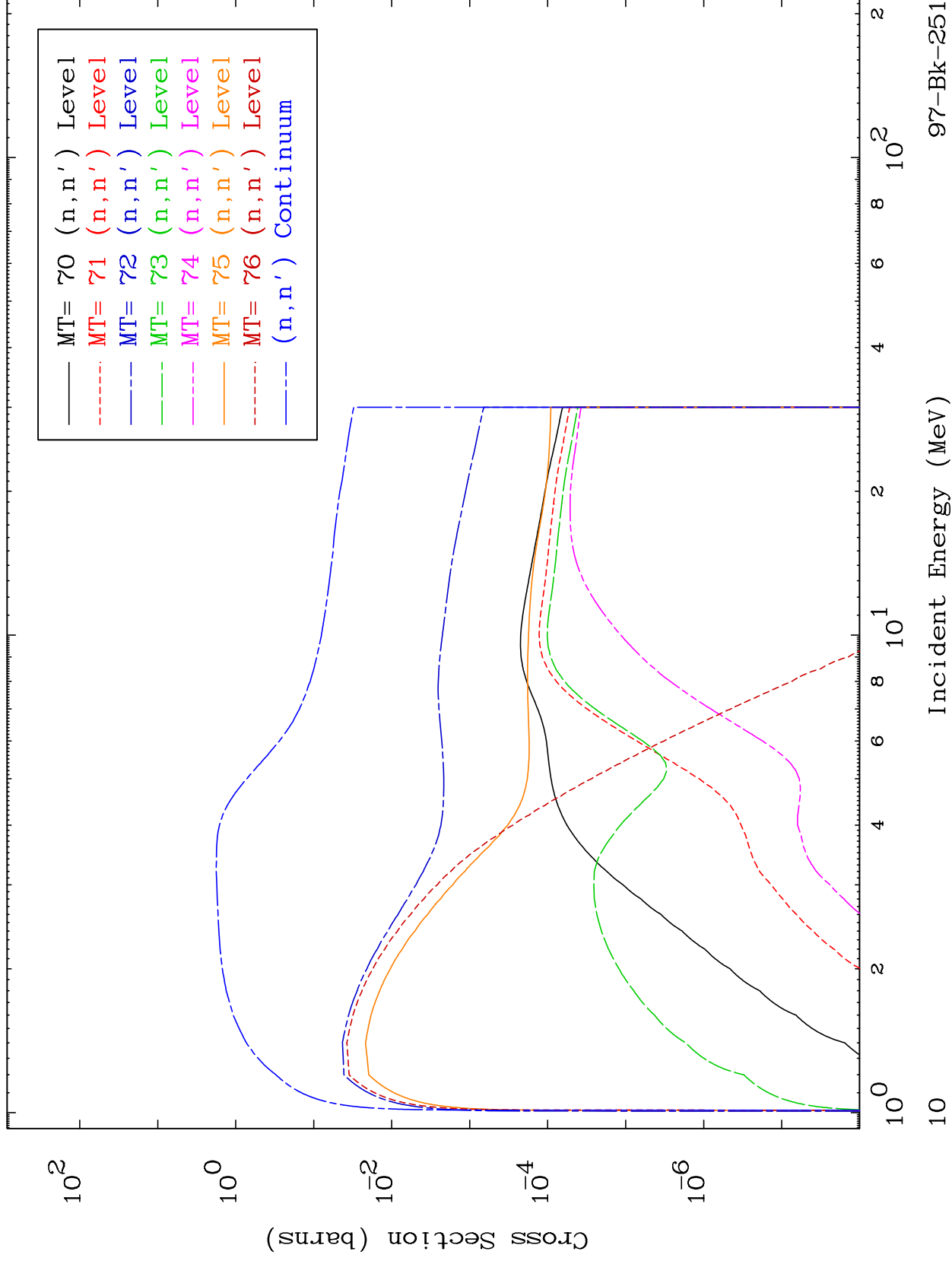
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(n,n') Level

293 Kelvin Cross Sections

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Incident Energy (MeV)

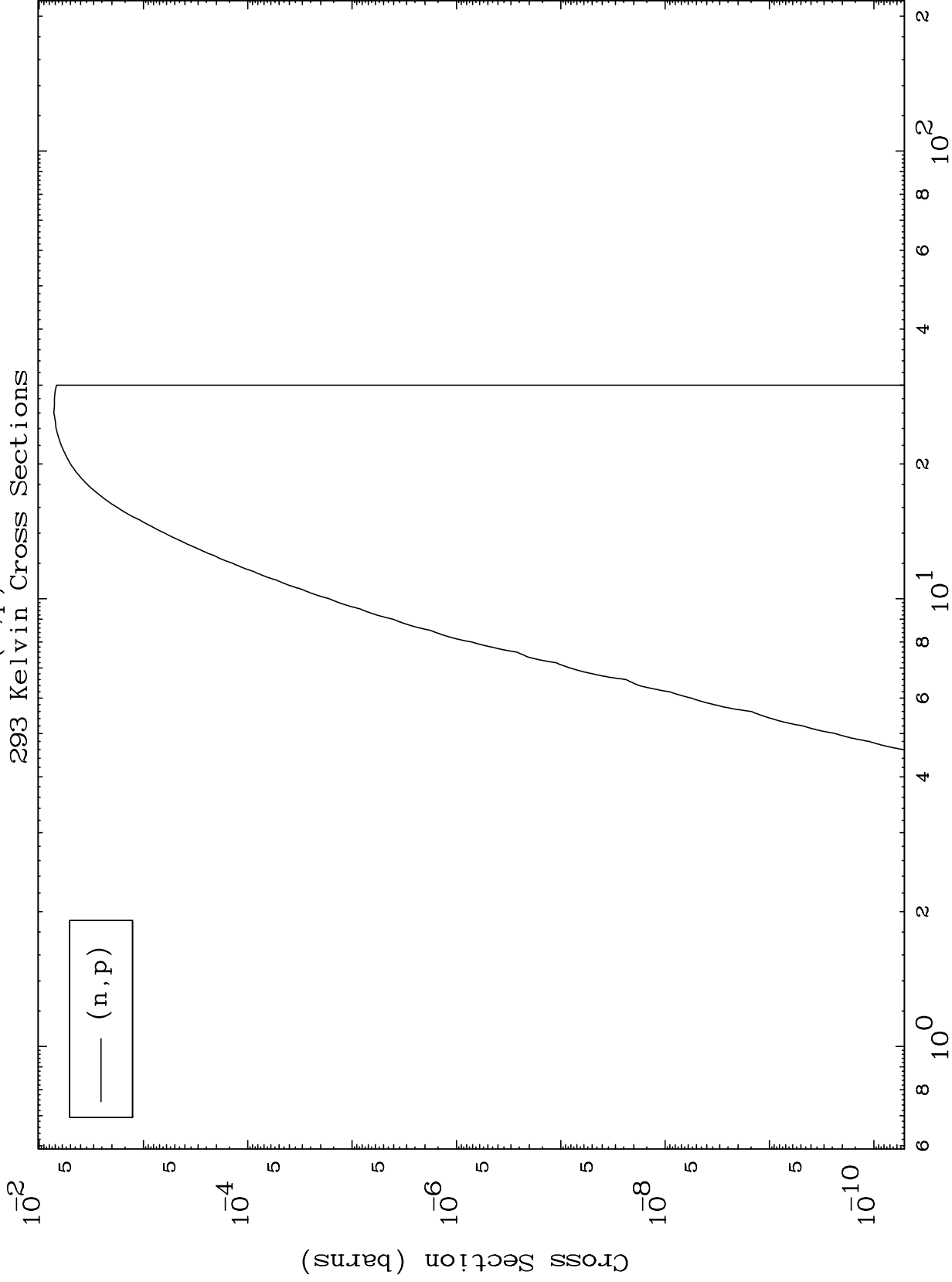
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(n,p) Levels

97-Bk-251

293 Kelvin Cross Sections



11

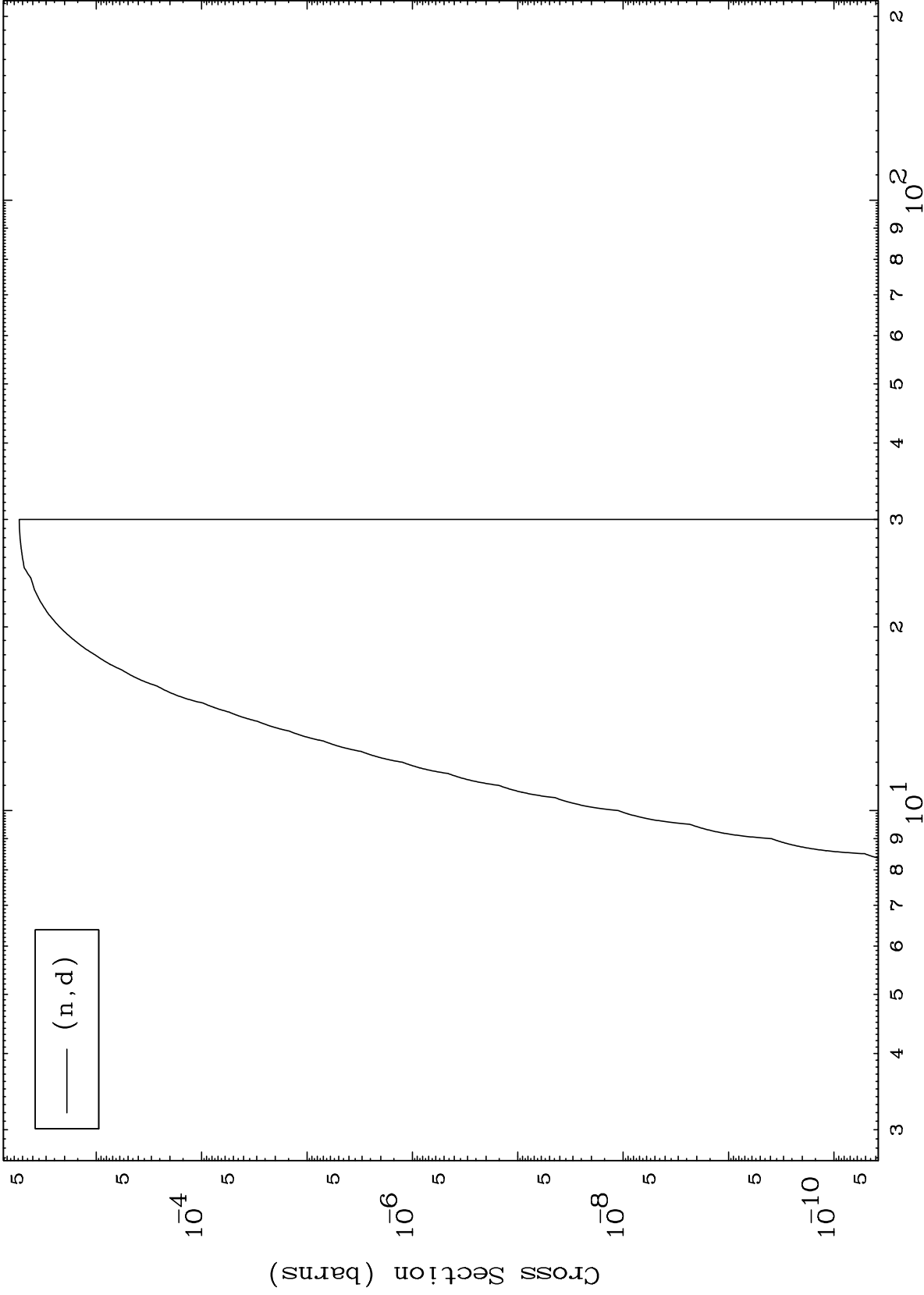
Incident Energy (MeV)

97-Bk-251

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(n,d) Levels  
293 Kelvin Cross Sections

97-Bk-251



12

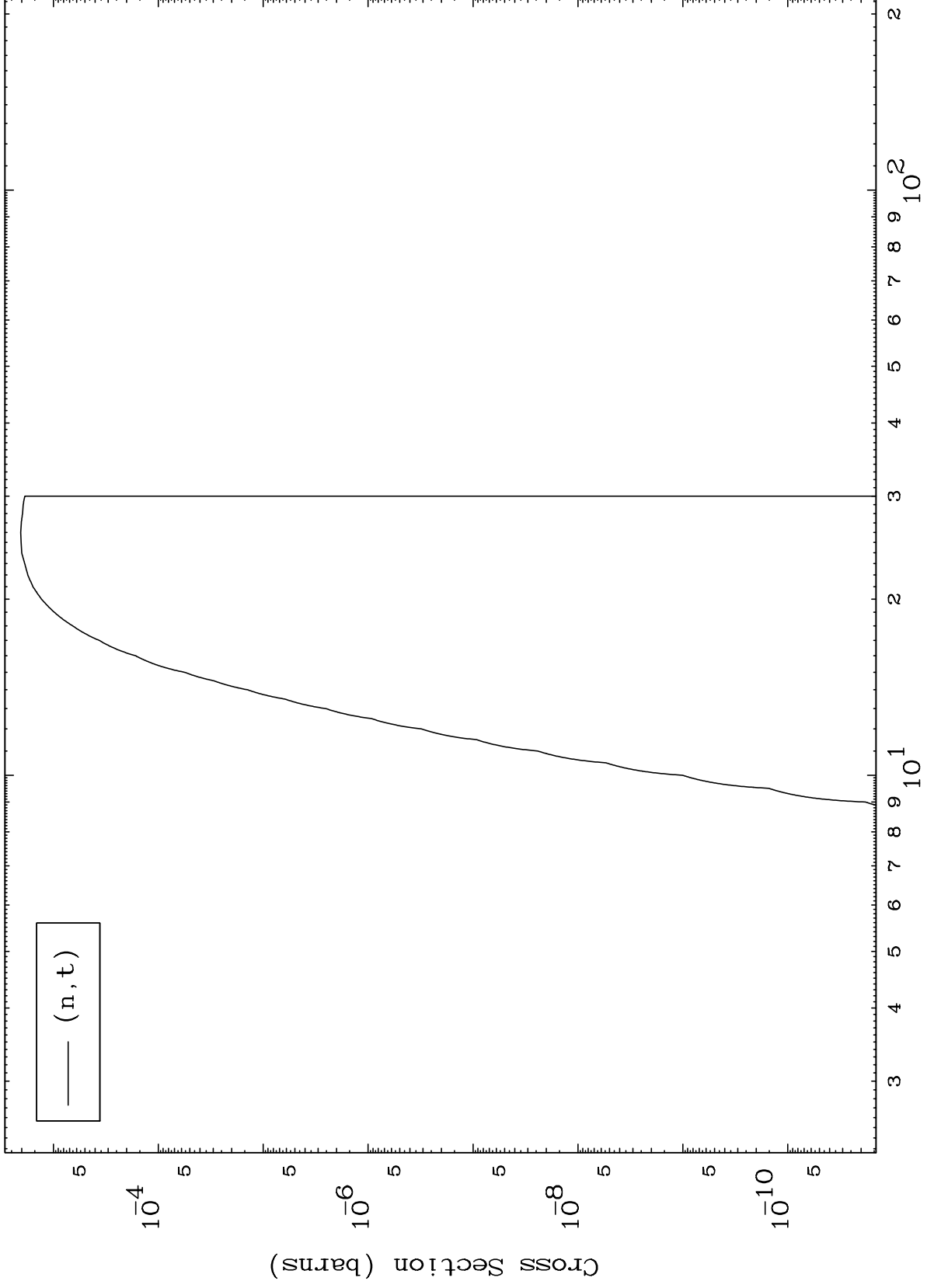
Incident Energy (MeV)

97-Bk-251

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(n,t) Levels  
293 Kelvin Cross Sections

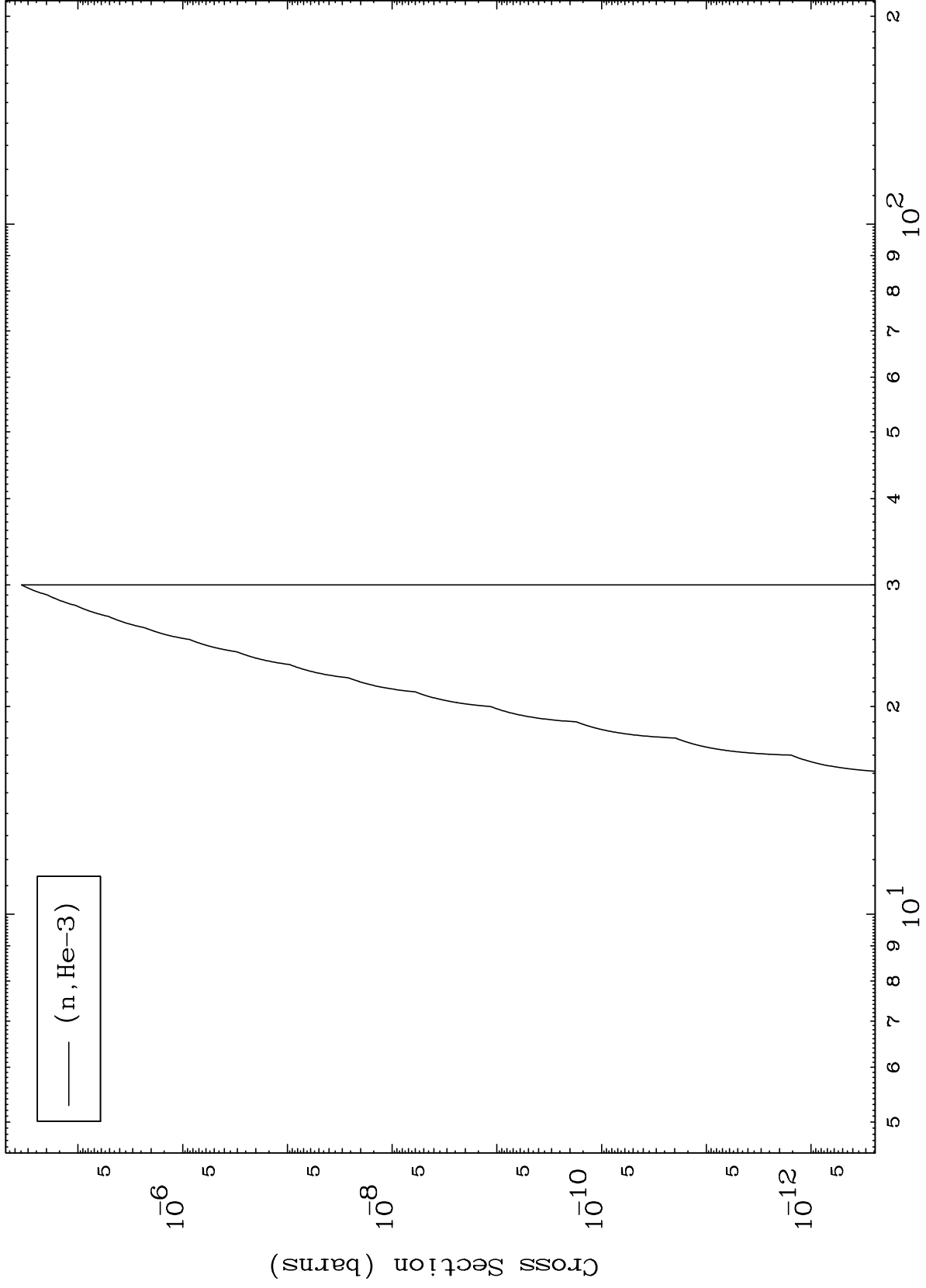
97-Bk-251



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(n,He3) Levels  
293 Kelvin Cross Sections

97-Bk-251



14

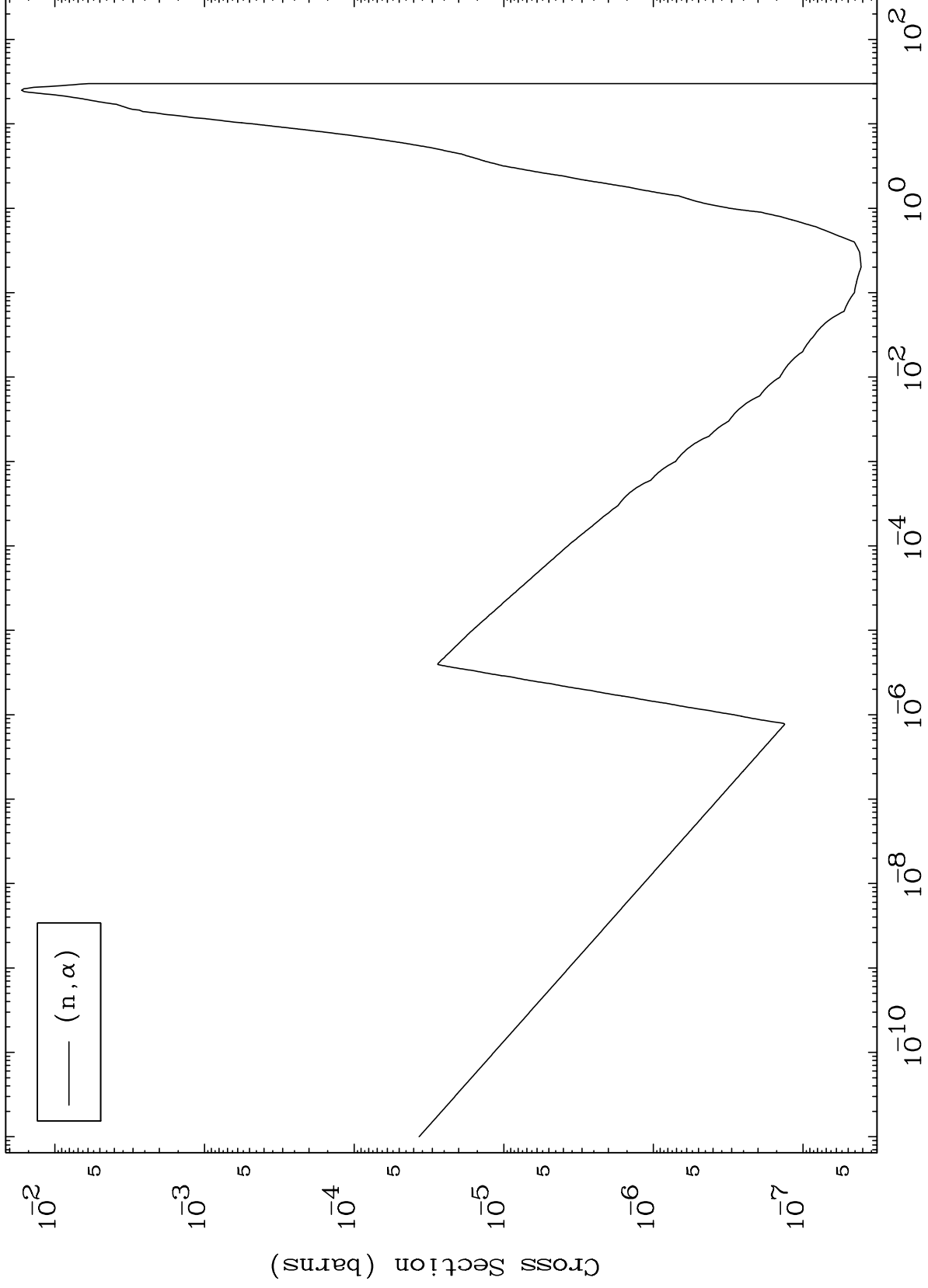
Incident Energy (MeV)

97-Bk-251

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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

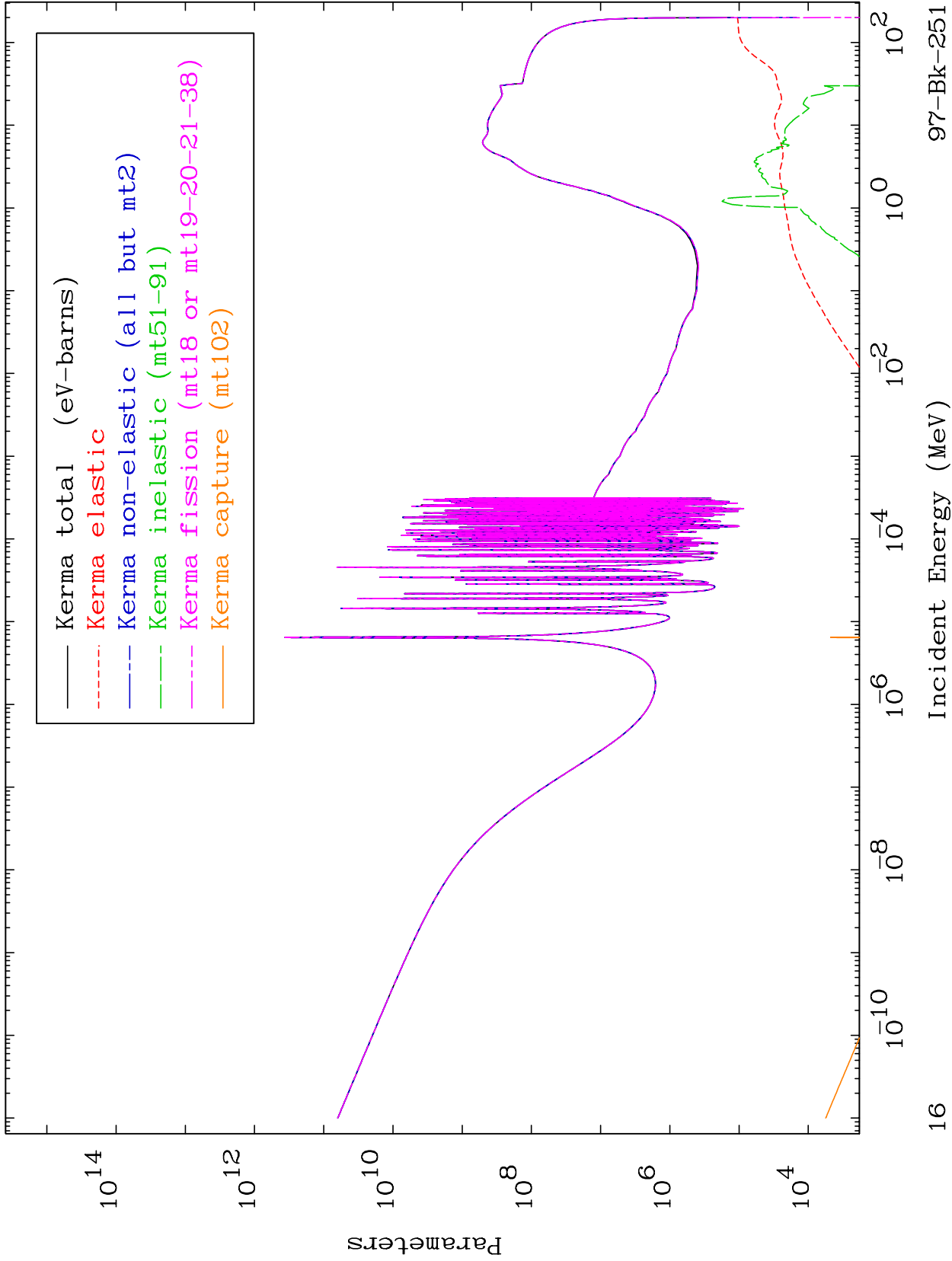
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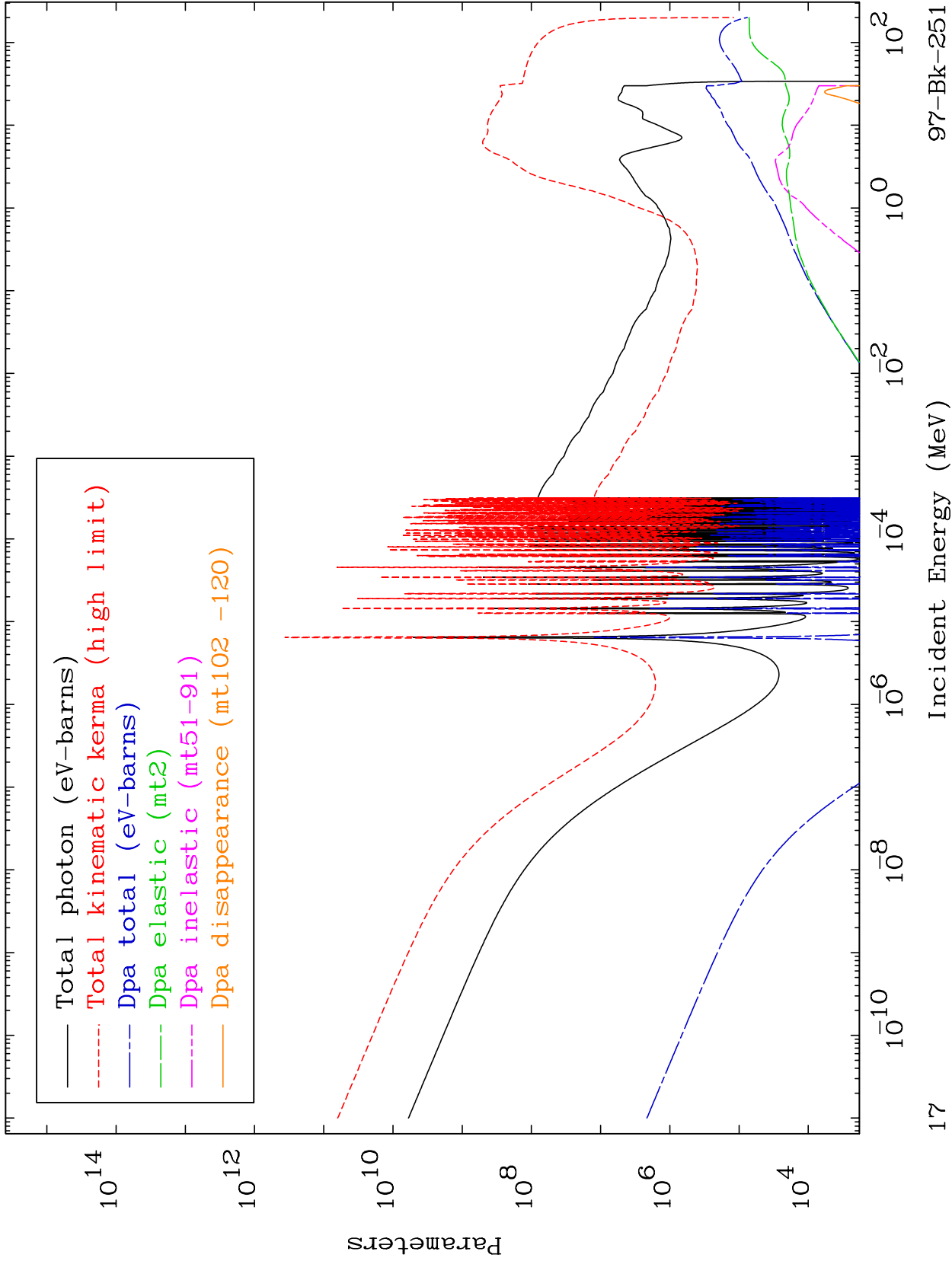


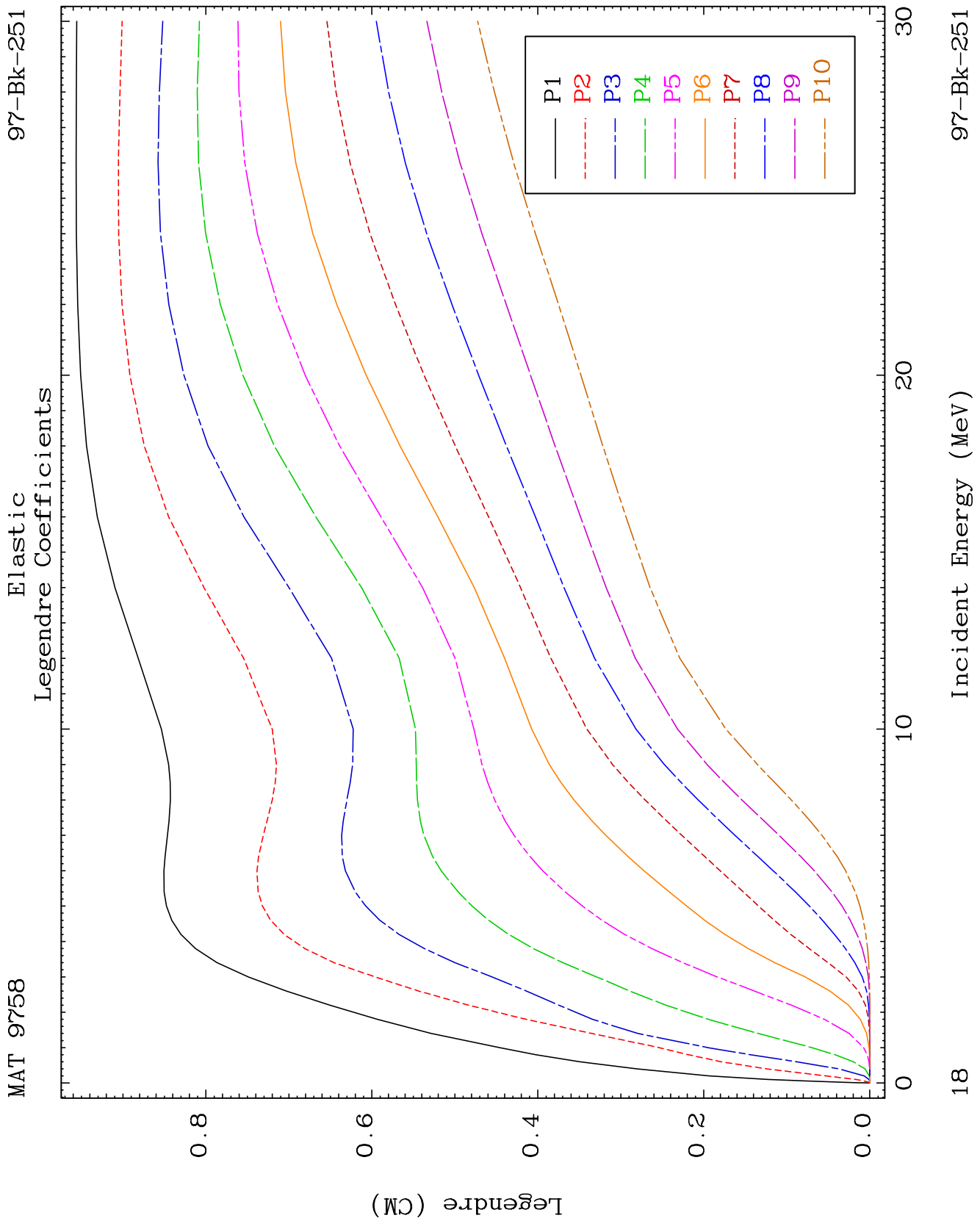
15

Incident Energy (MeV)

97-Bk-251



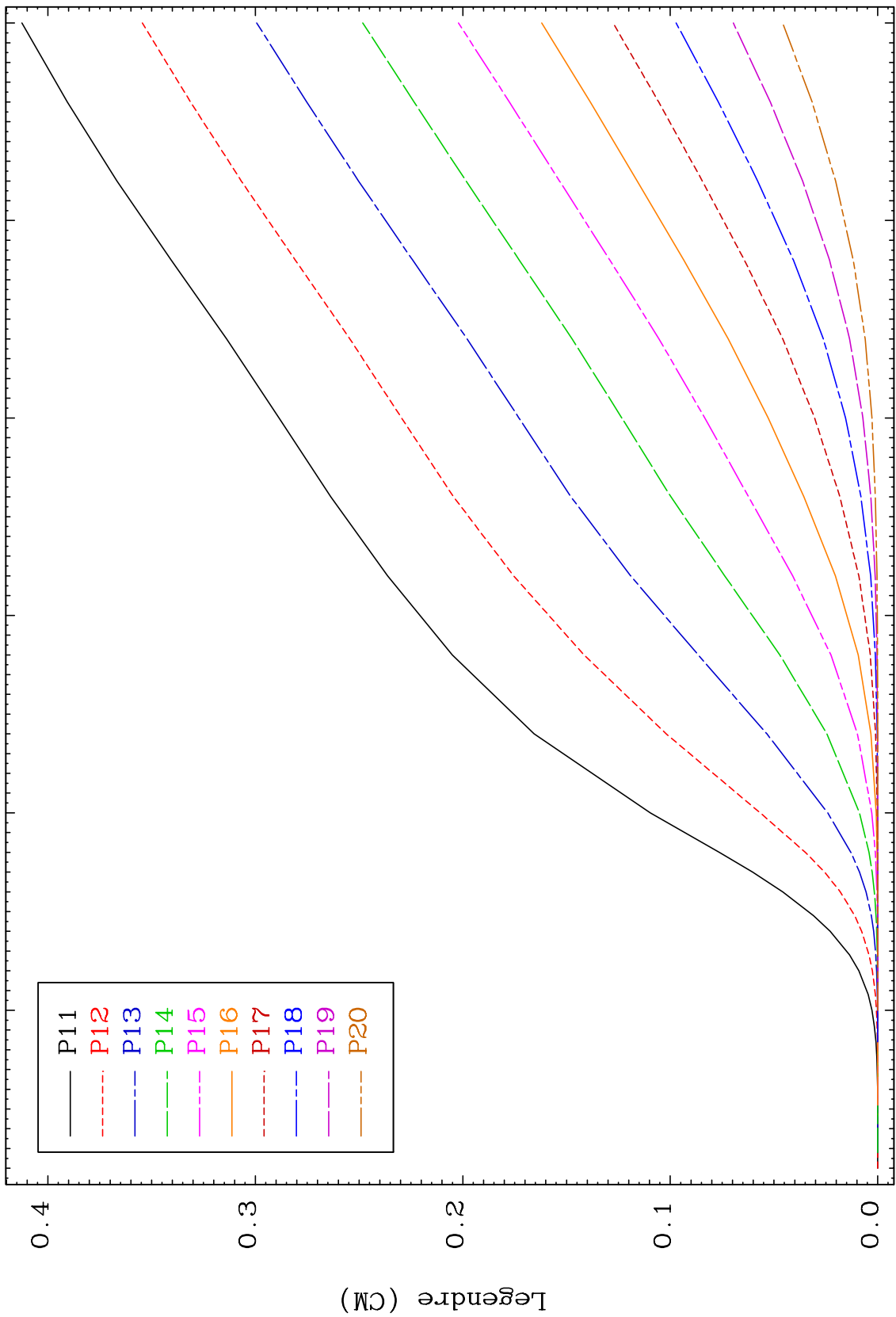




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Elastic Legendre Coefficients

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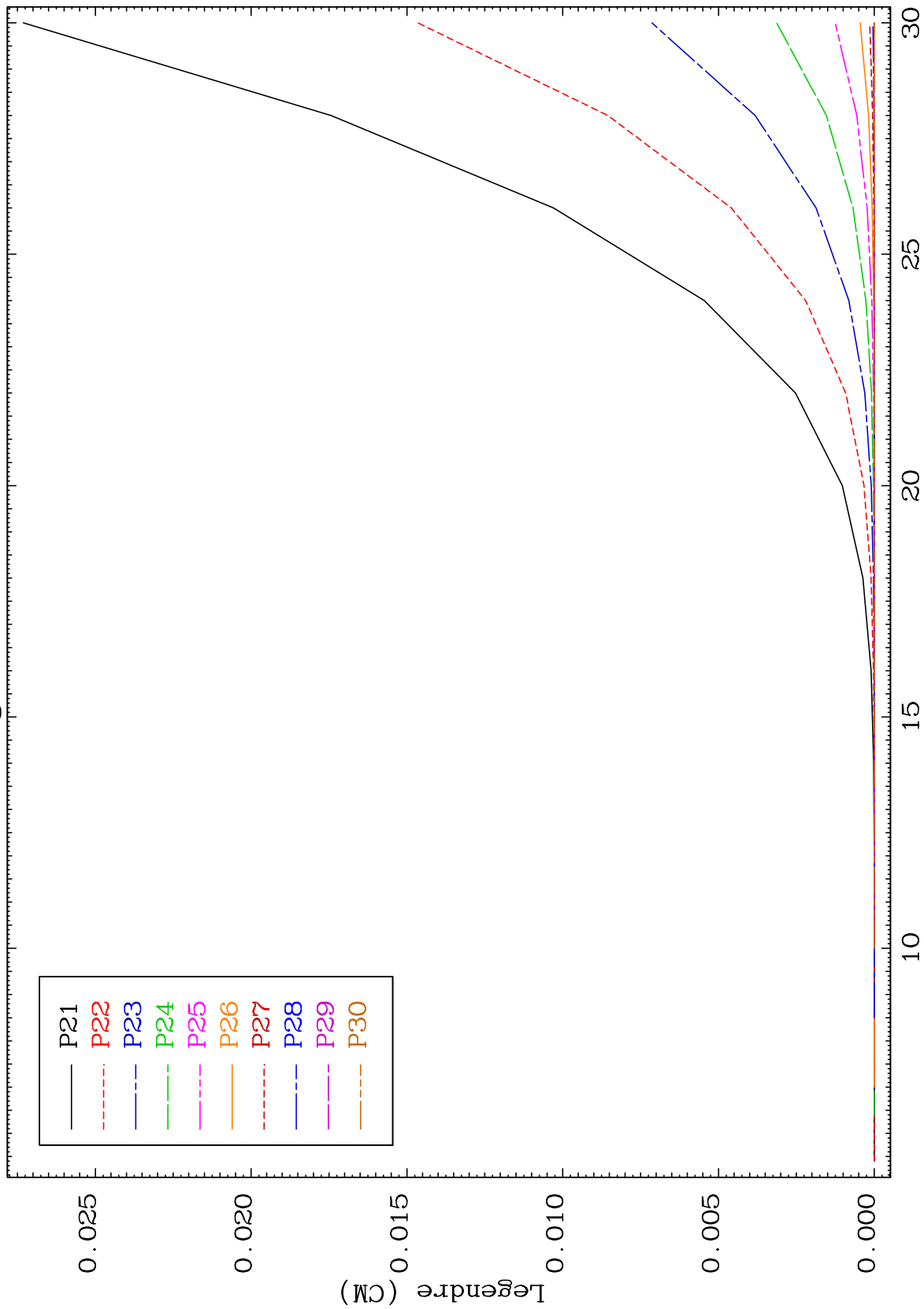
Incident Energy (MeV)

97-Bk-251

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### Elastic Legendre Coefficients

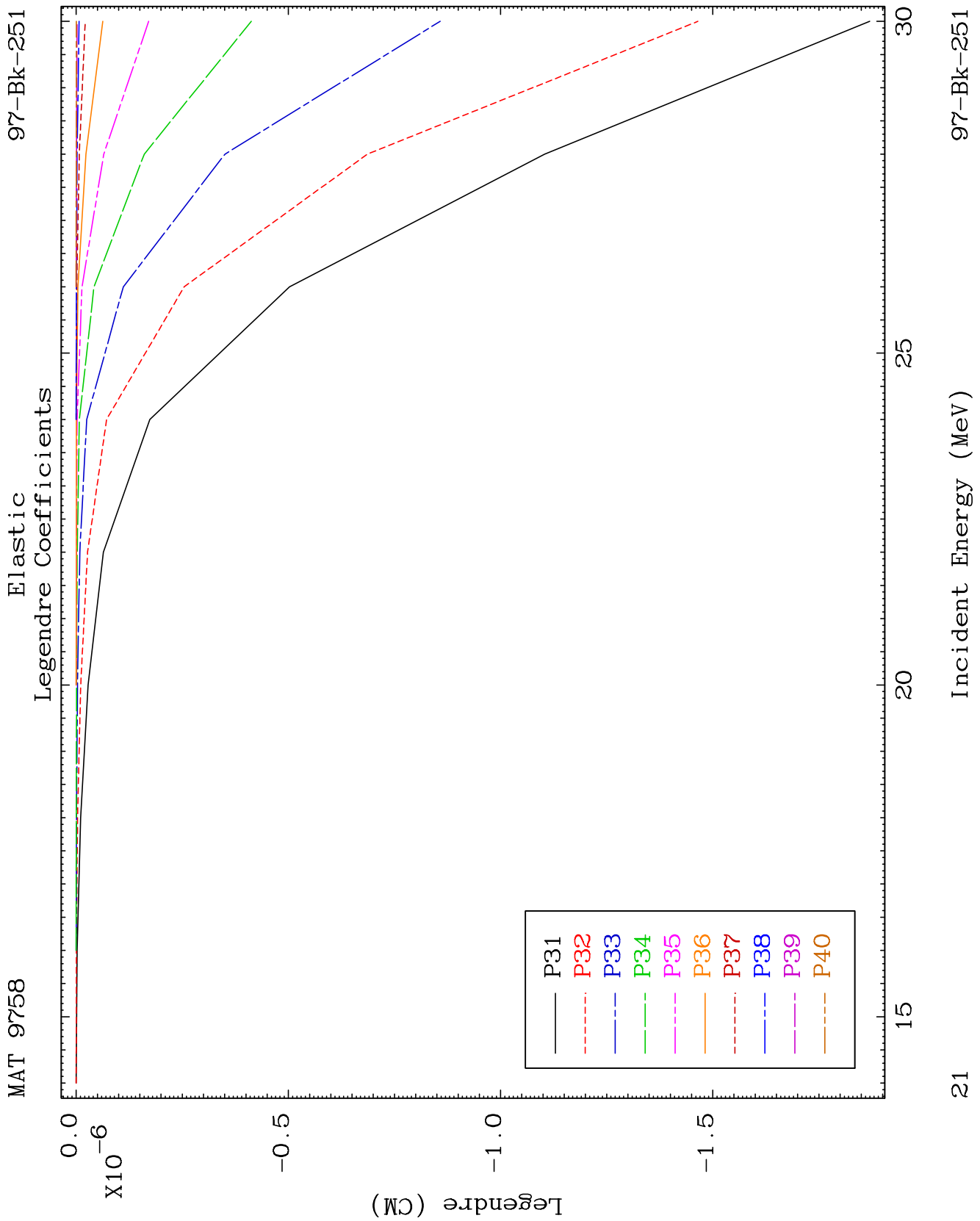
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20

Incident Energy (MeV)

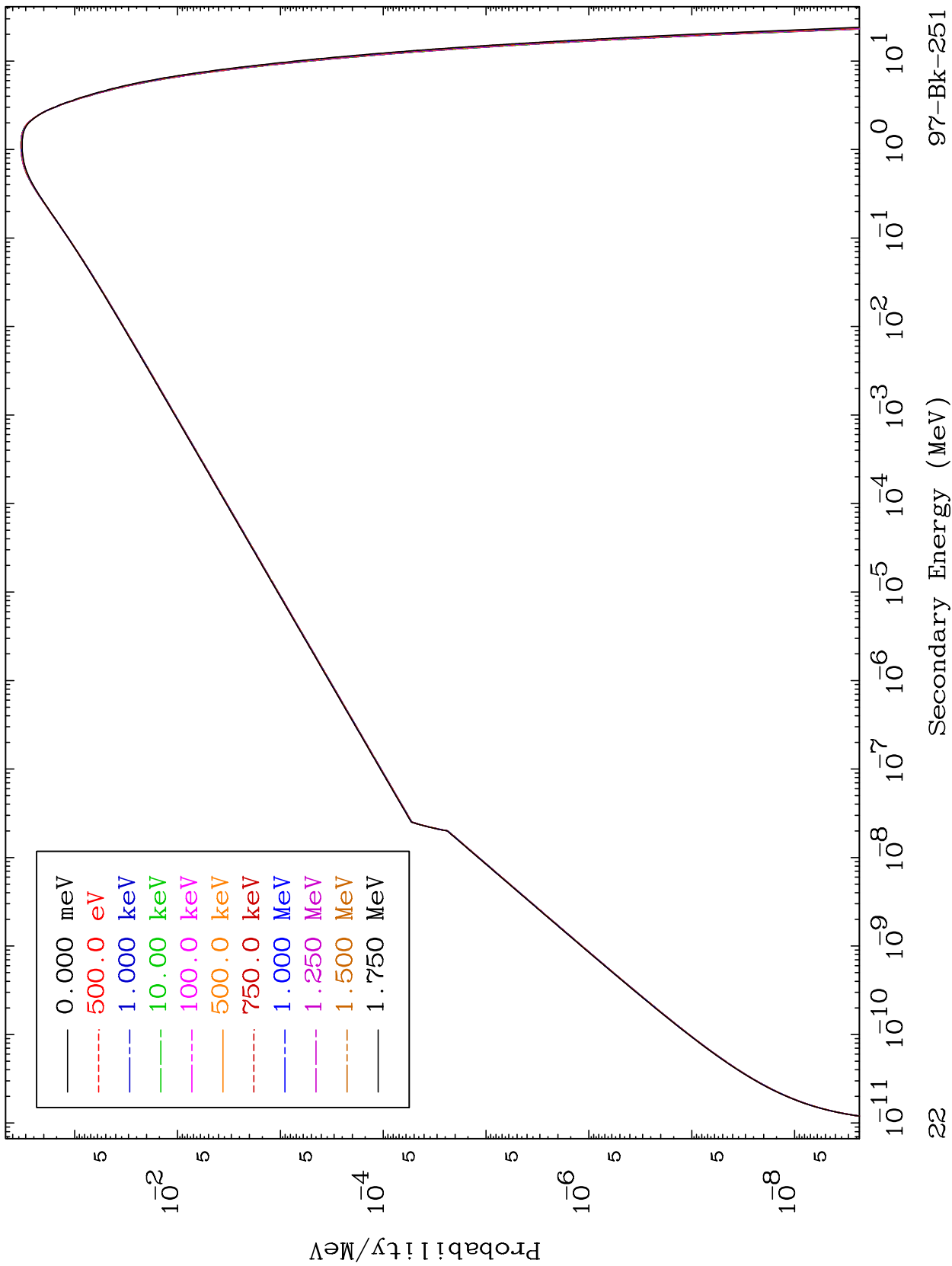
97-Bk-251



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### Fission Energy Distribution

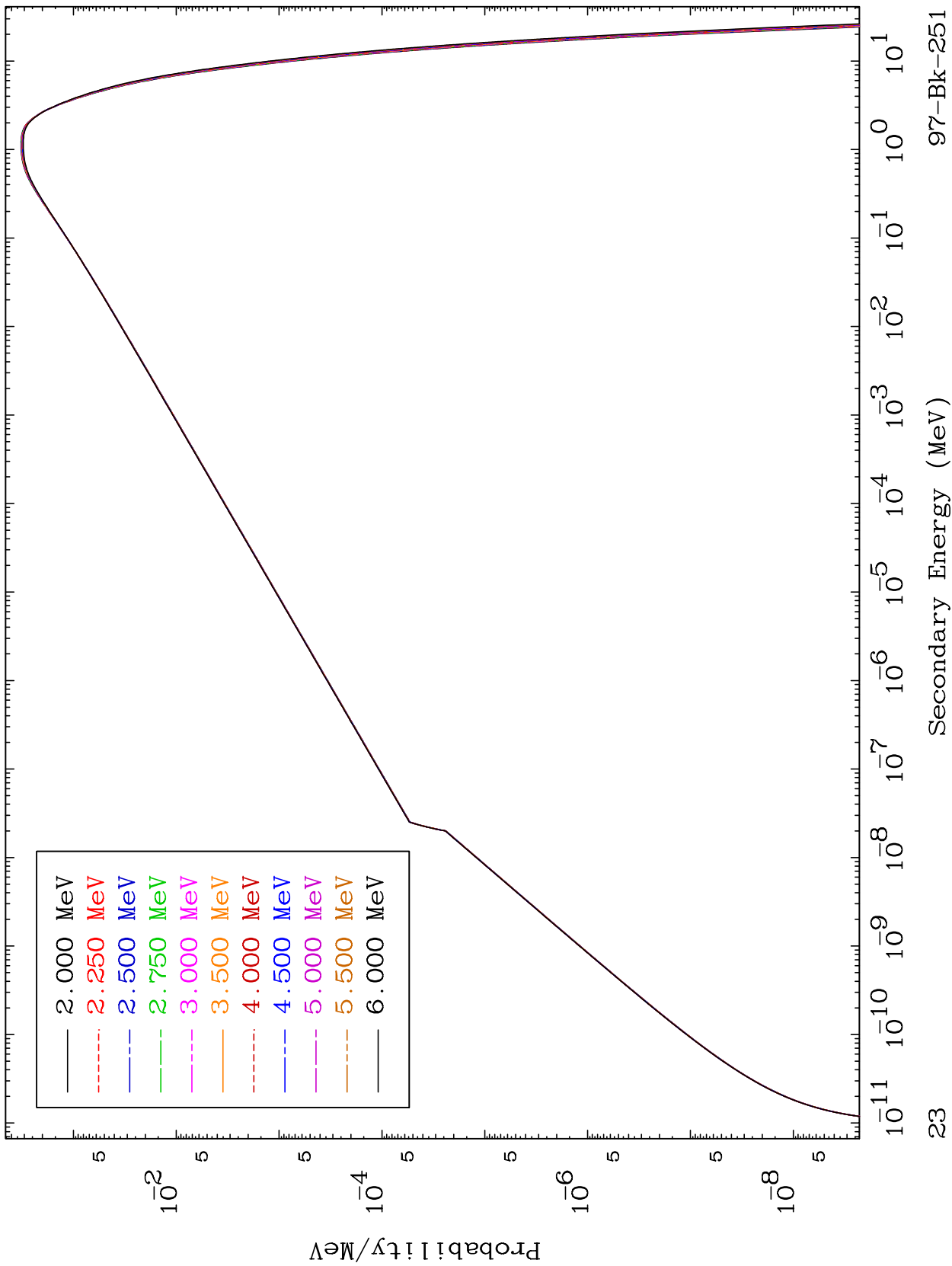
97-Bk-251



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### Fission Energy Distribution

97-Bk-251



97-Bk-251

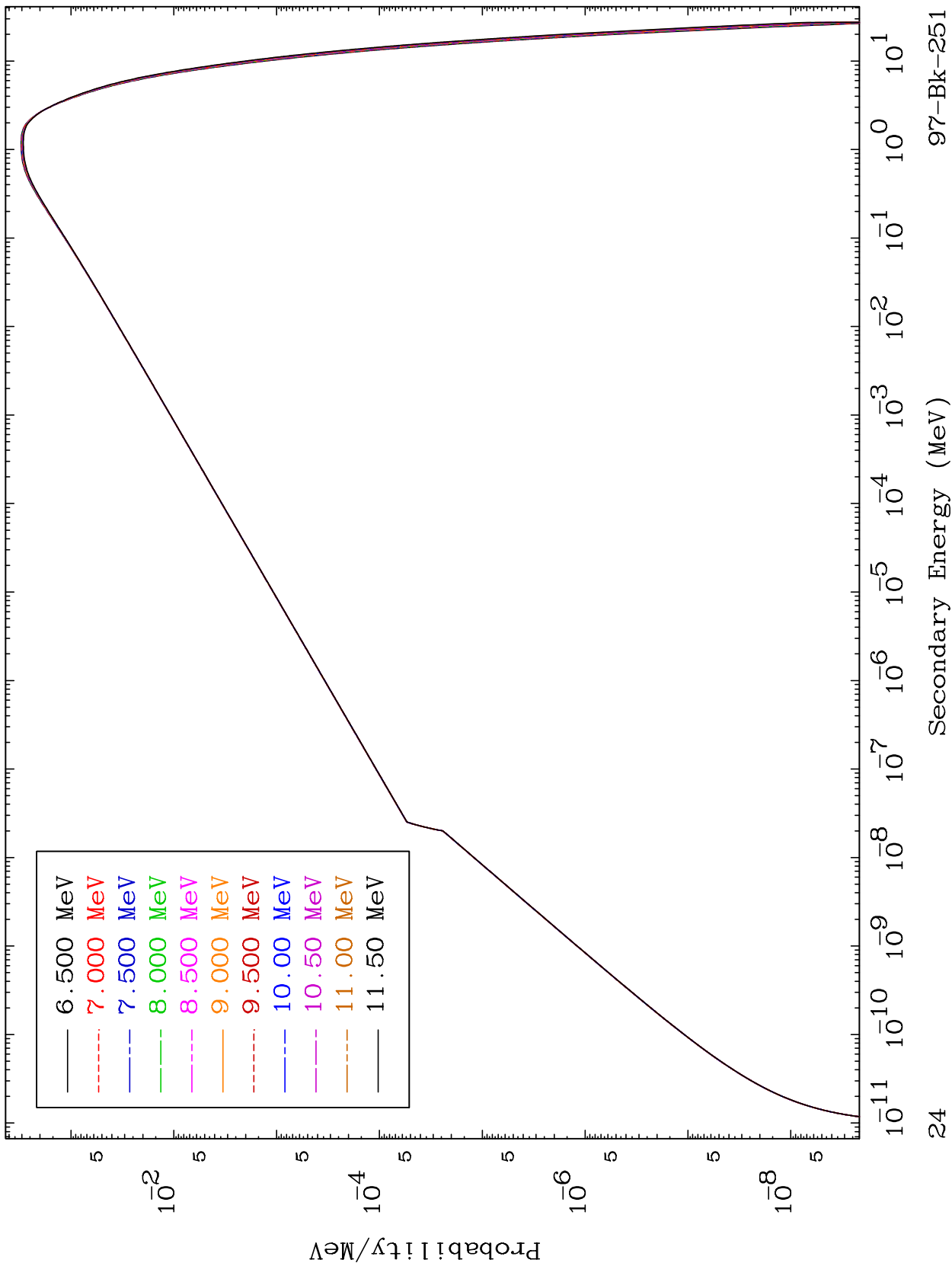
Secondary Energy (MeV)

23

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### Fission Energy Distribution

97-Bk-251



97-Bk-251

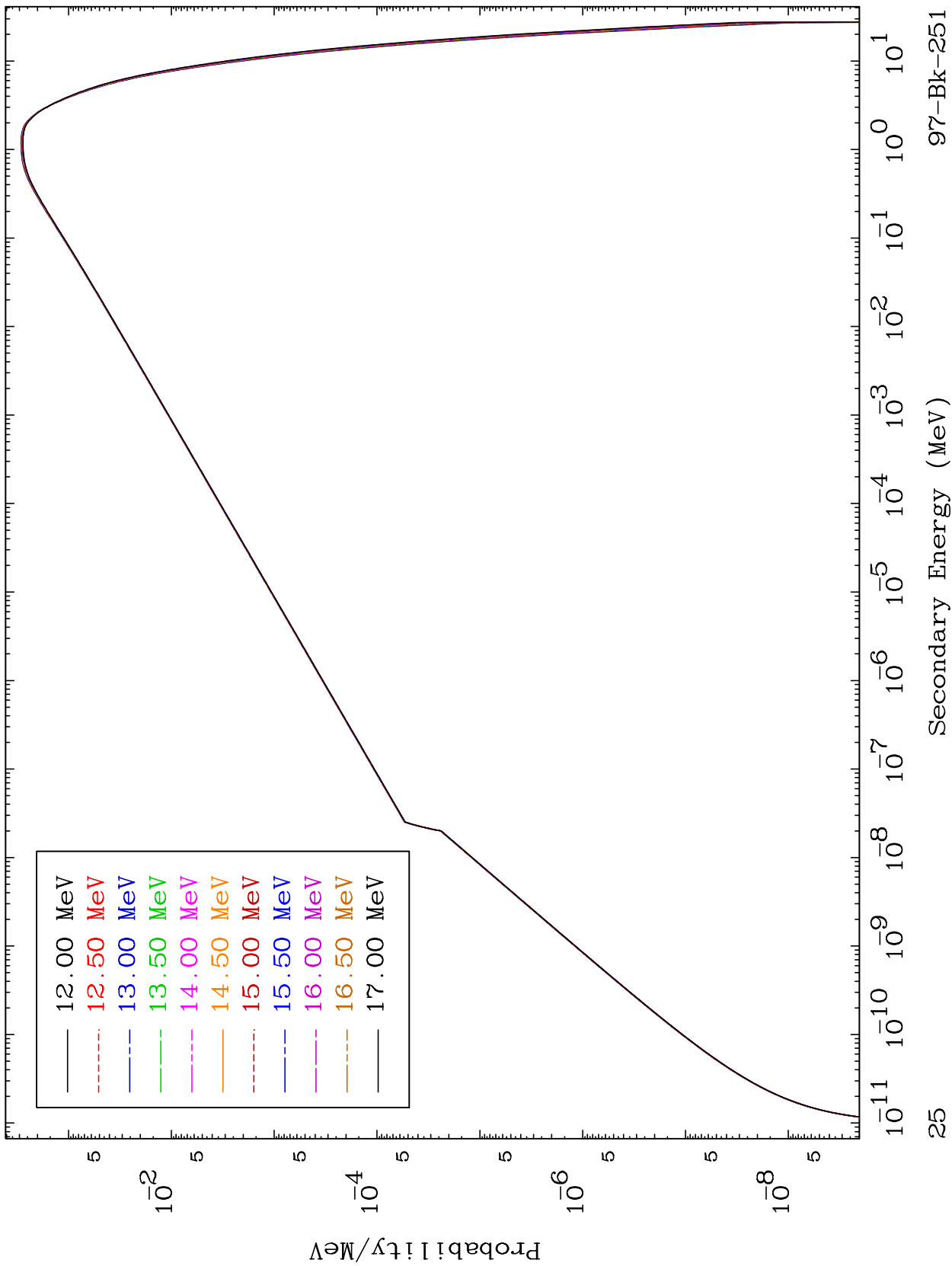
Secondary Energy (MeV)

24

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### Fission Energy Distribution

97-Bk-251



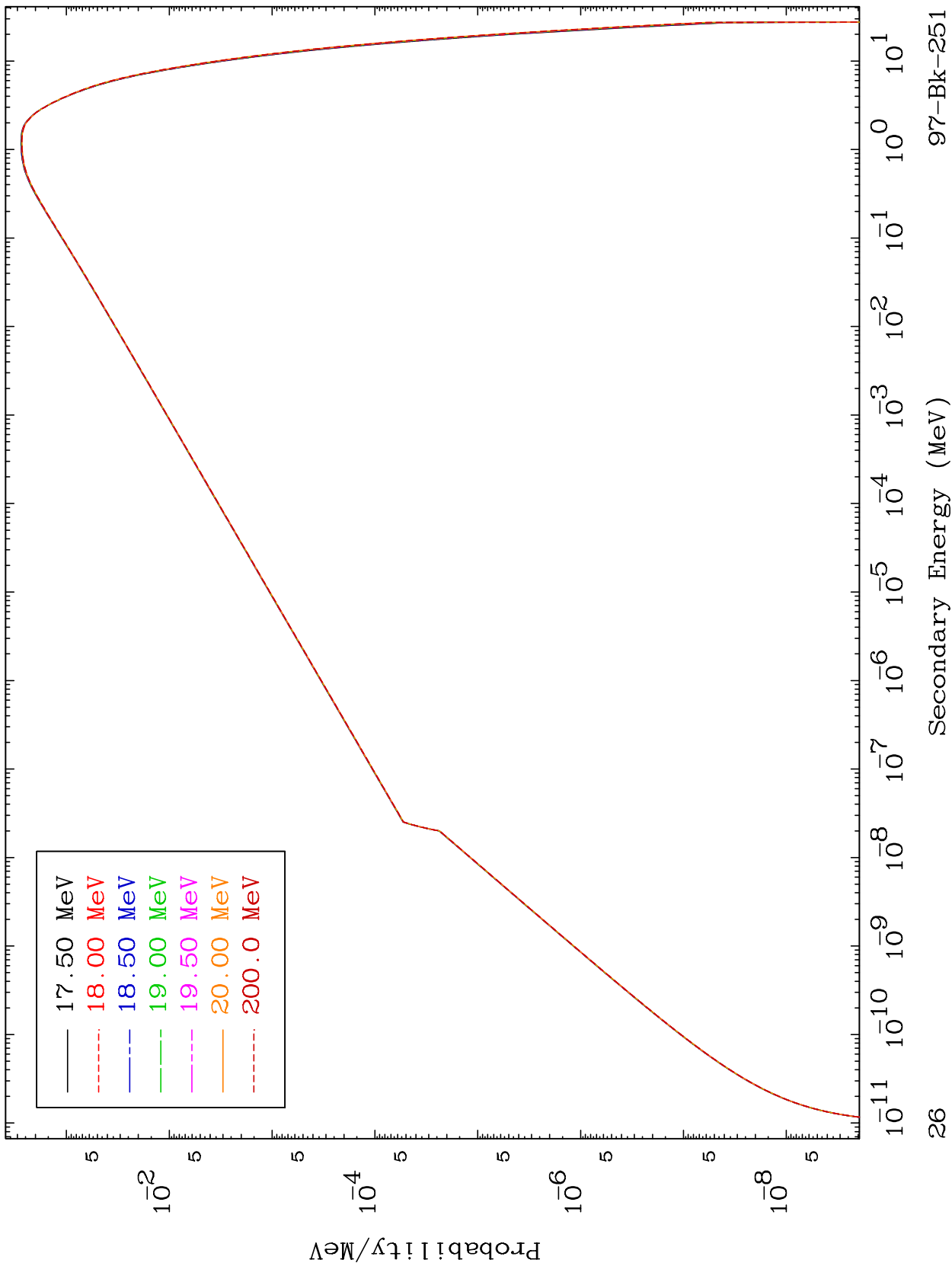
97-Bk-251

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Fission  
Energy Distribution

97-Bk-251

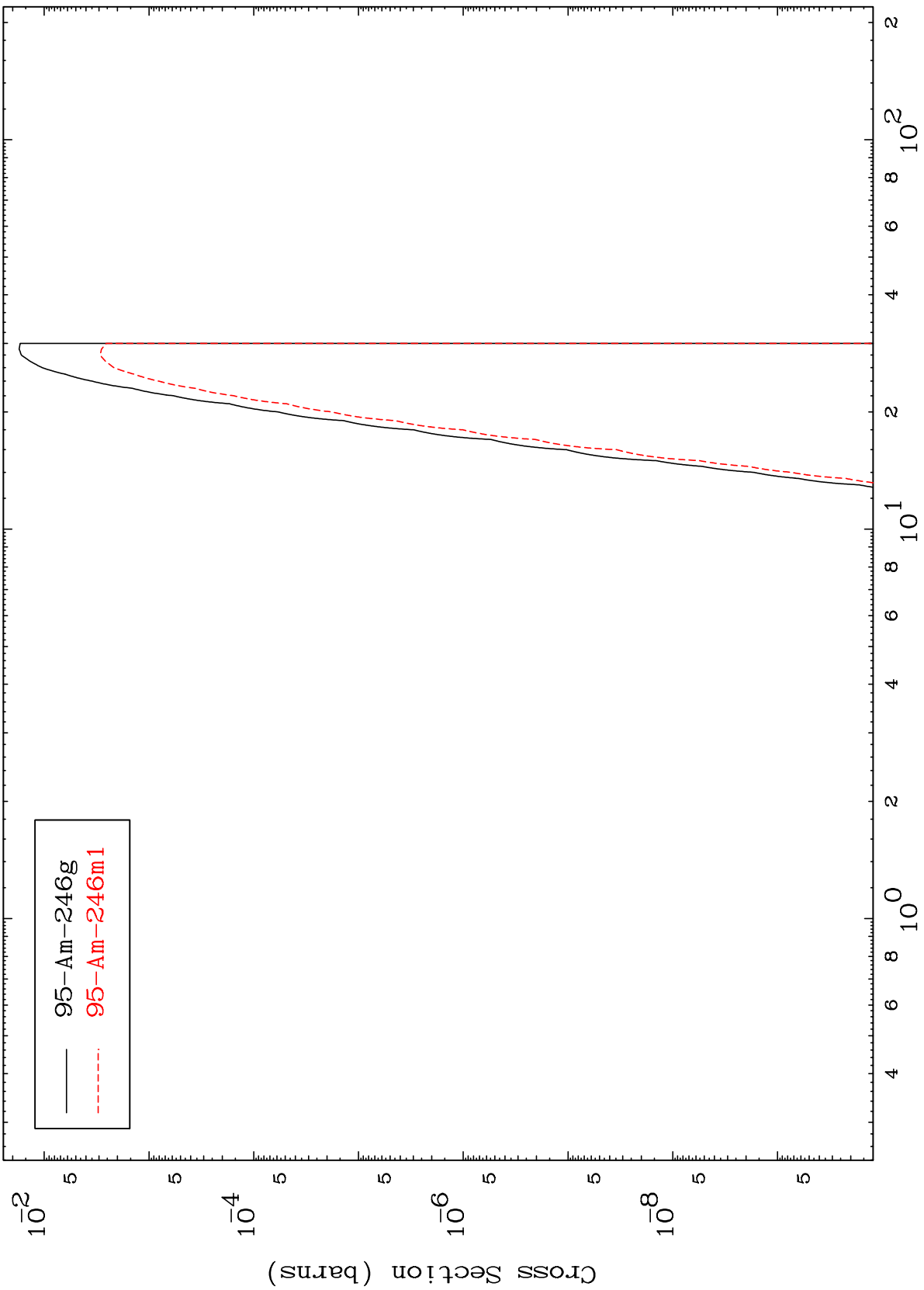


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(n,2n)  $\alpha$

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Radionuclide Production Cross Section

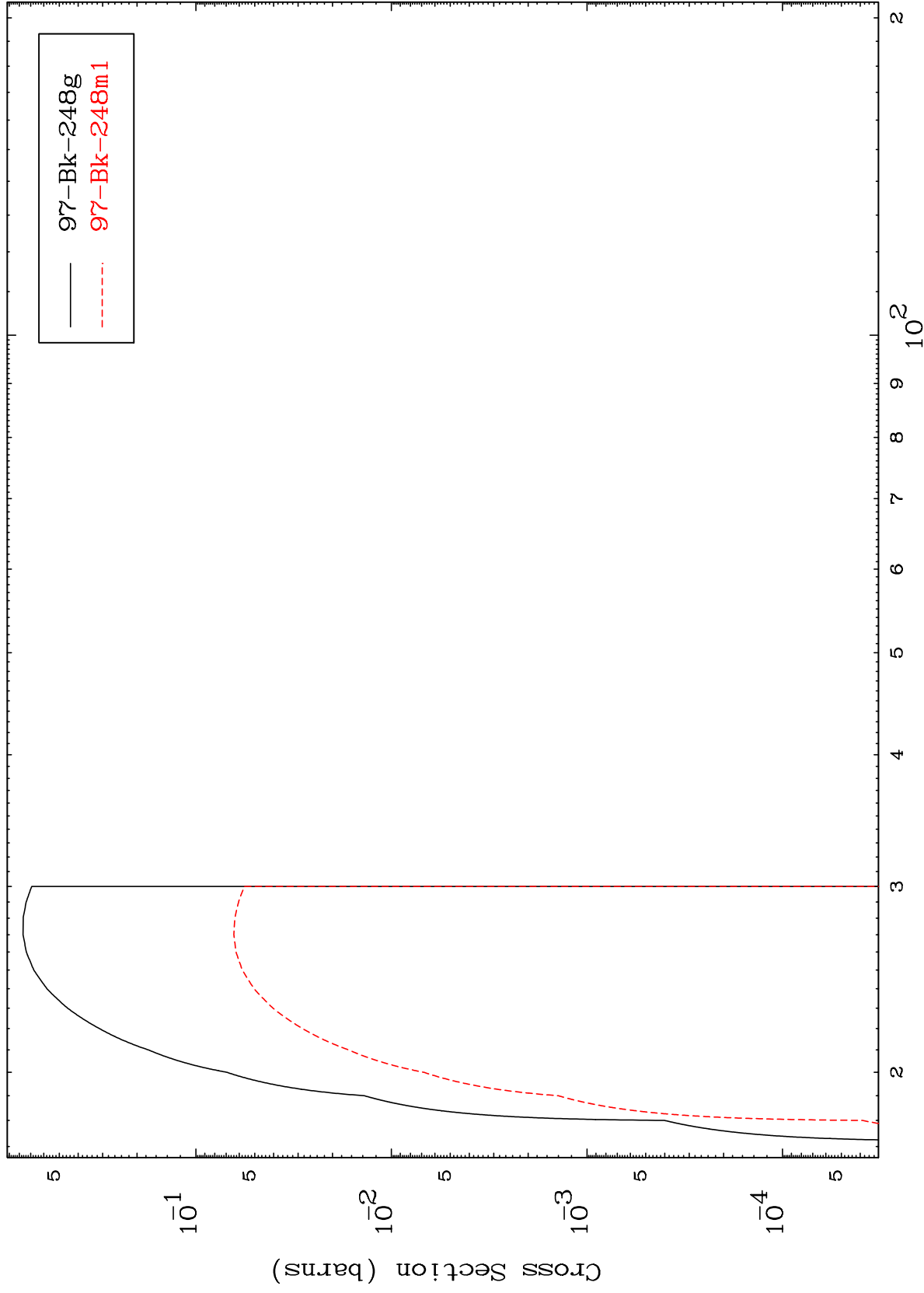


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(n,4n)

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Radionuclide Production Cross Section



28

Incident Energy (MeV)

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