

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

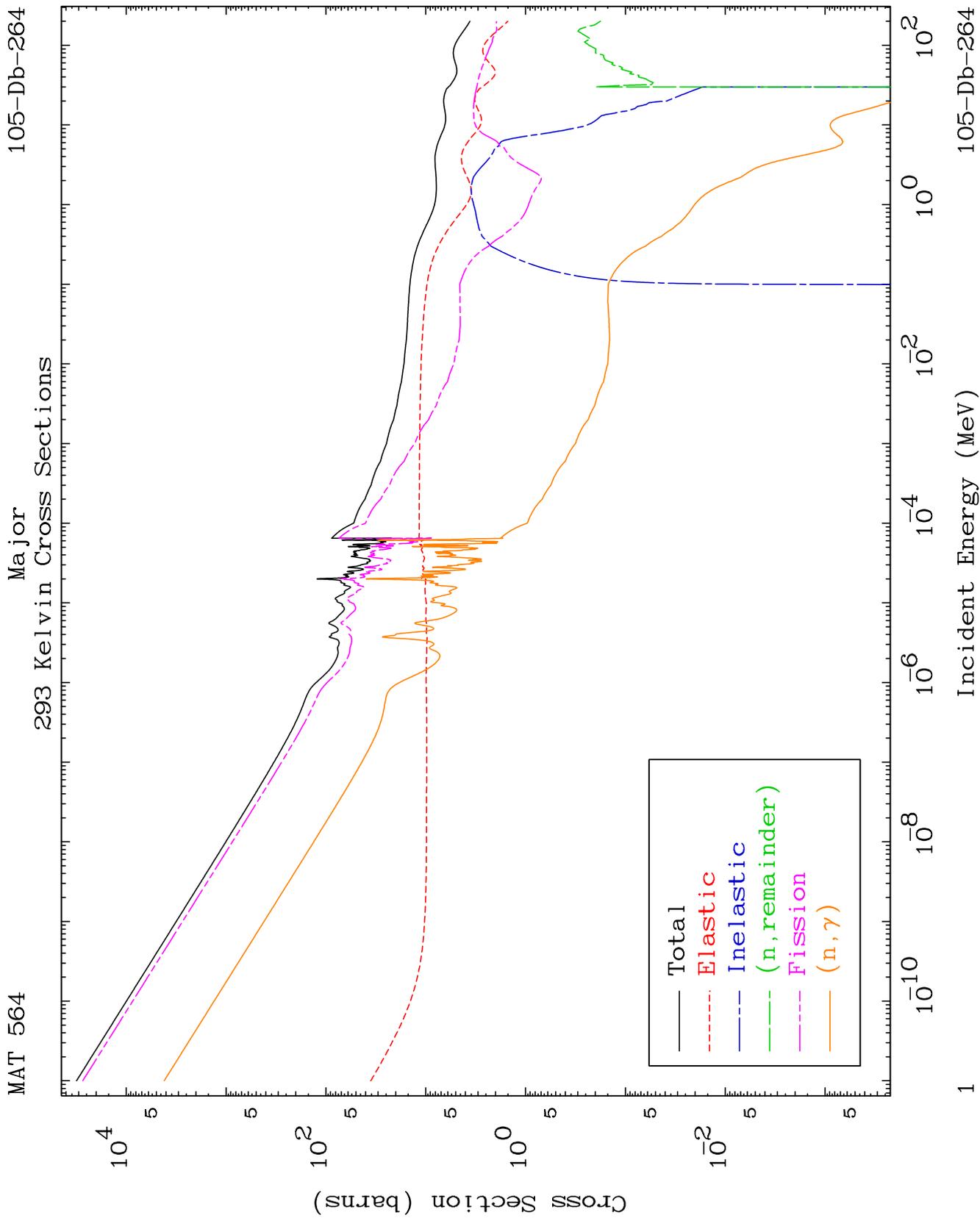
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Web:redcullen1.net/HOMEPAGE.NEW

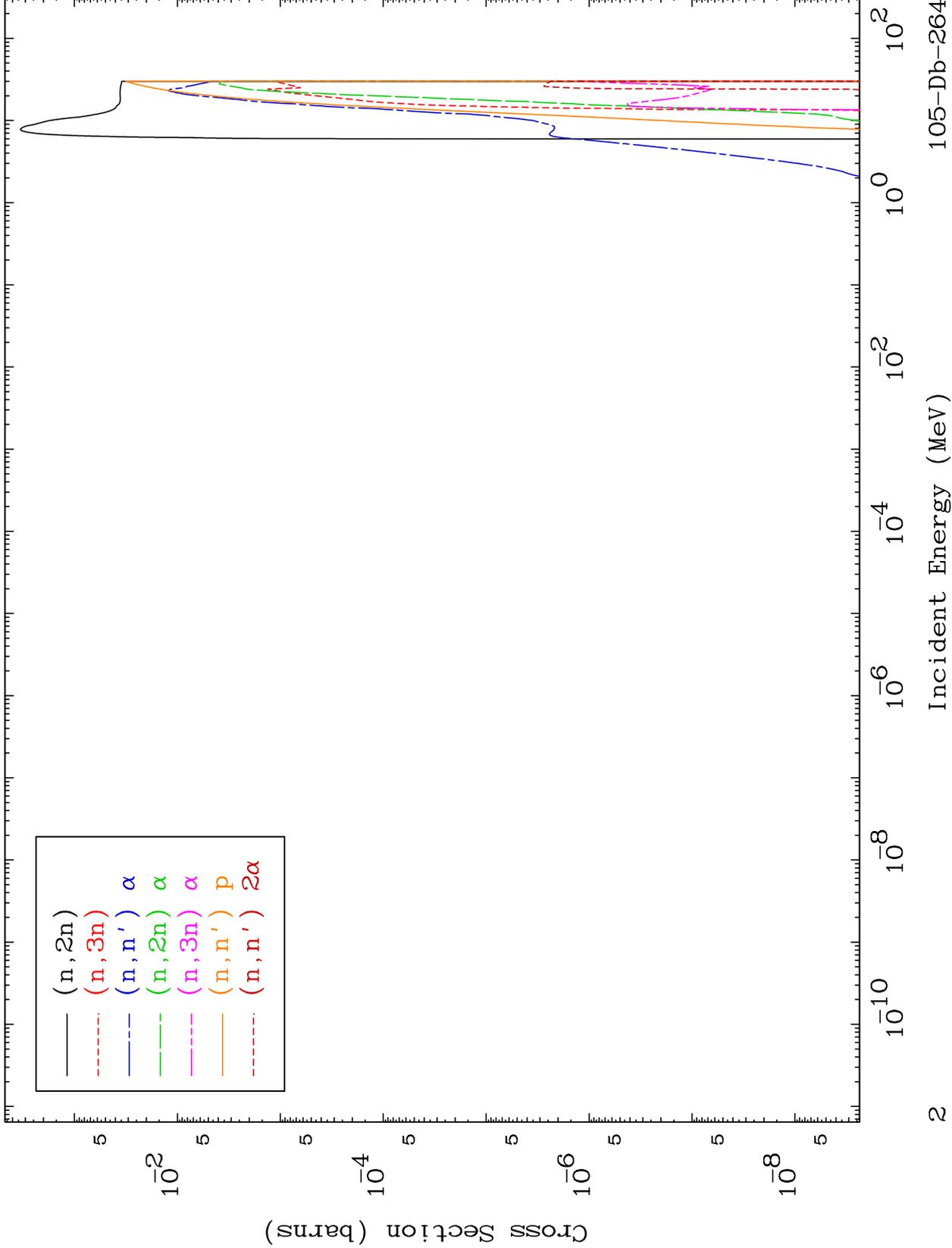
Press Mouse Button to Start



MAT 564

Neutron Production
293 Kelvin Cross Sections

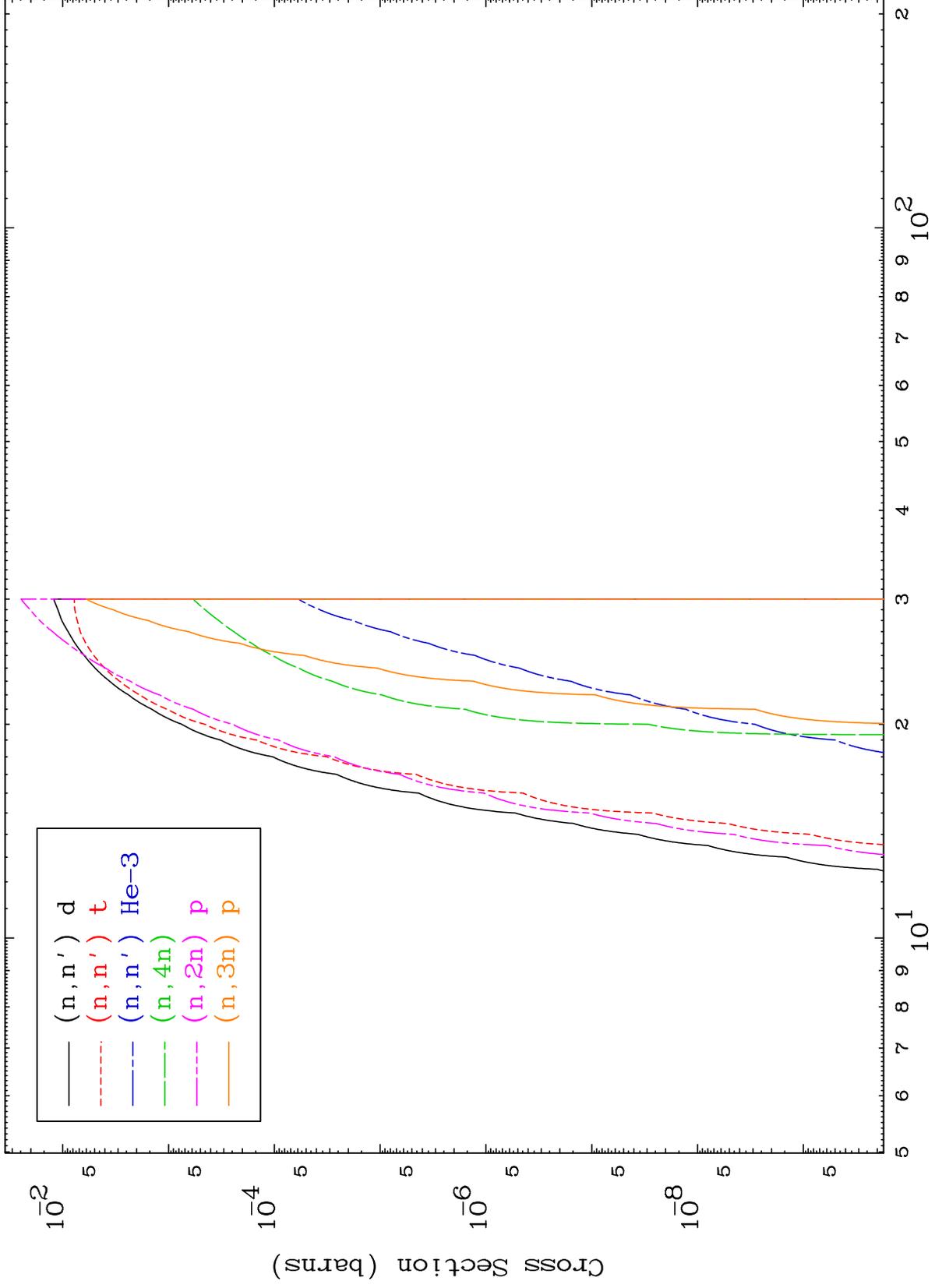
105-Db-264



MAT 564

Neutron Production
293 Kelvin Cross Sections

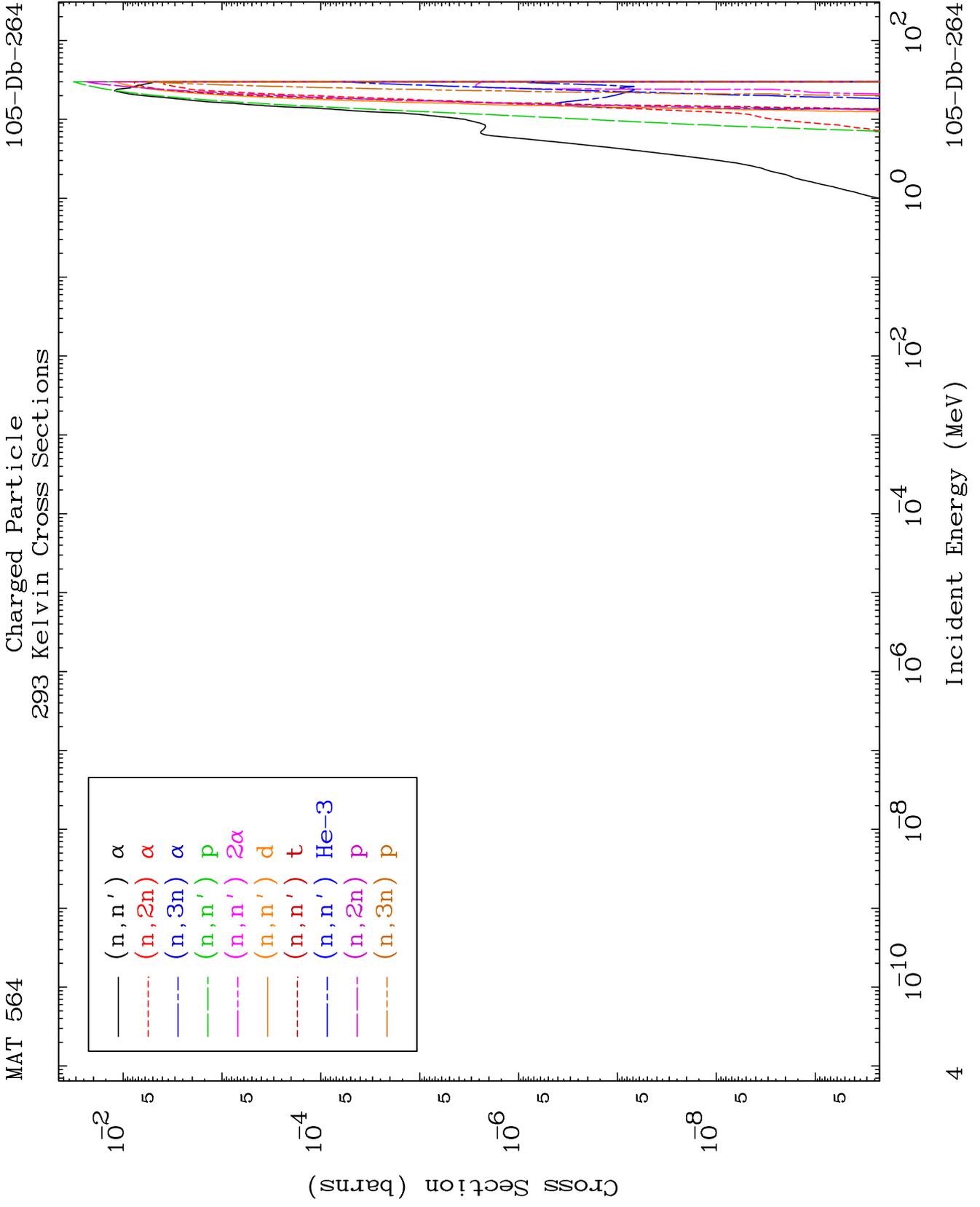
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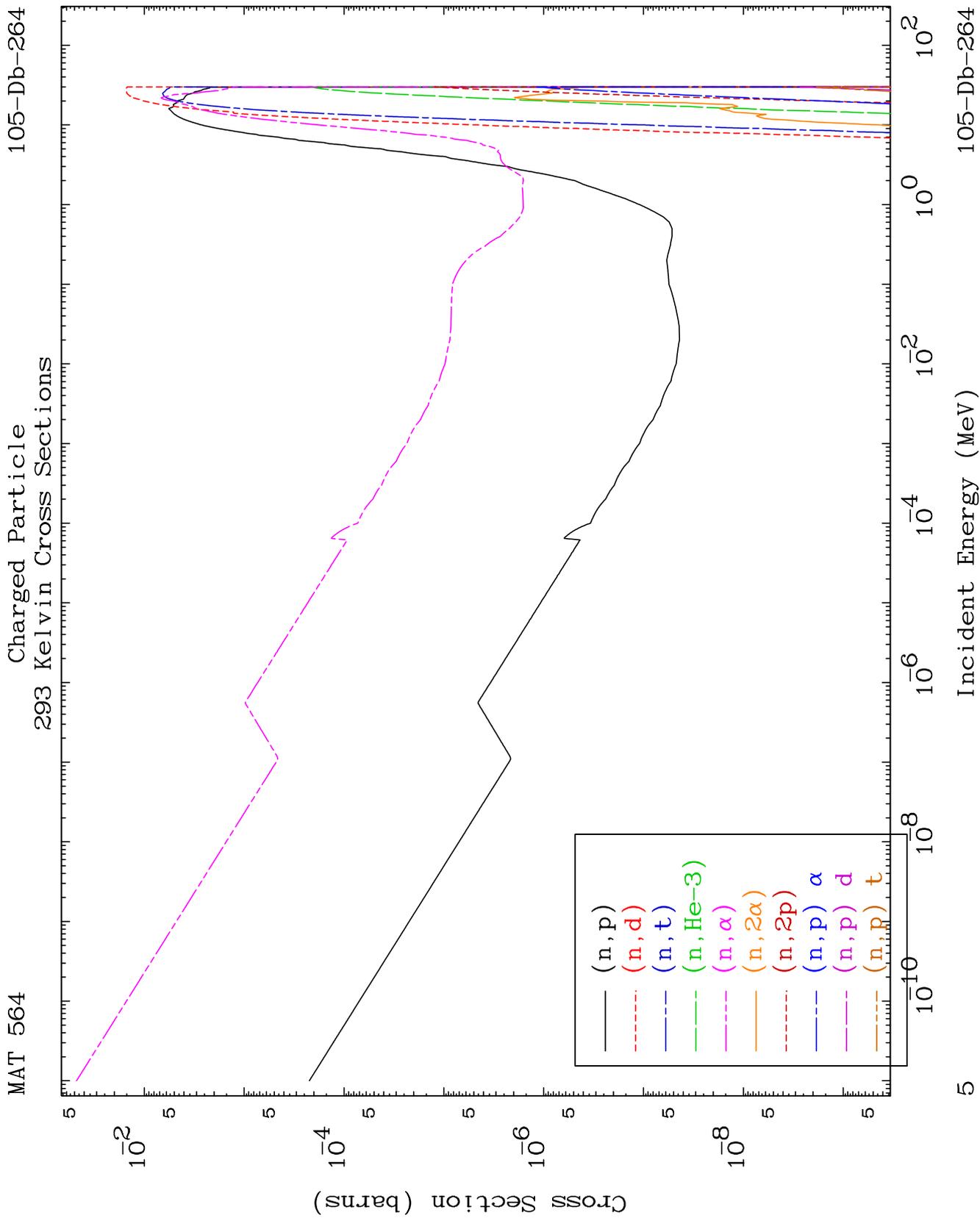


3

Incident Energy (MeV)

105-Db-264





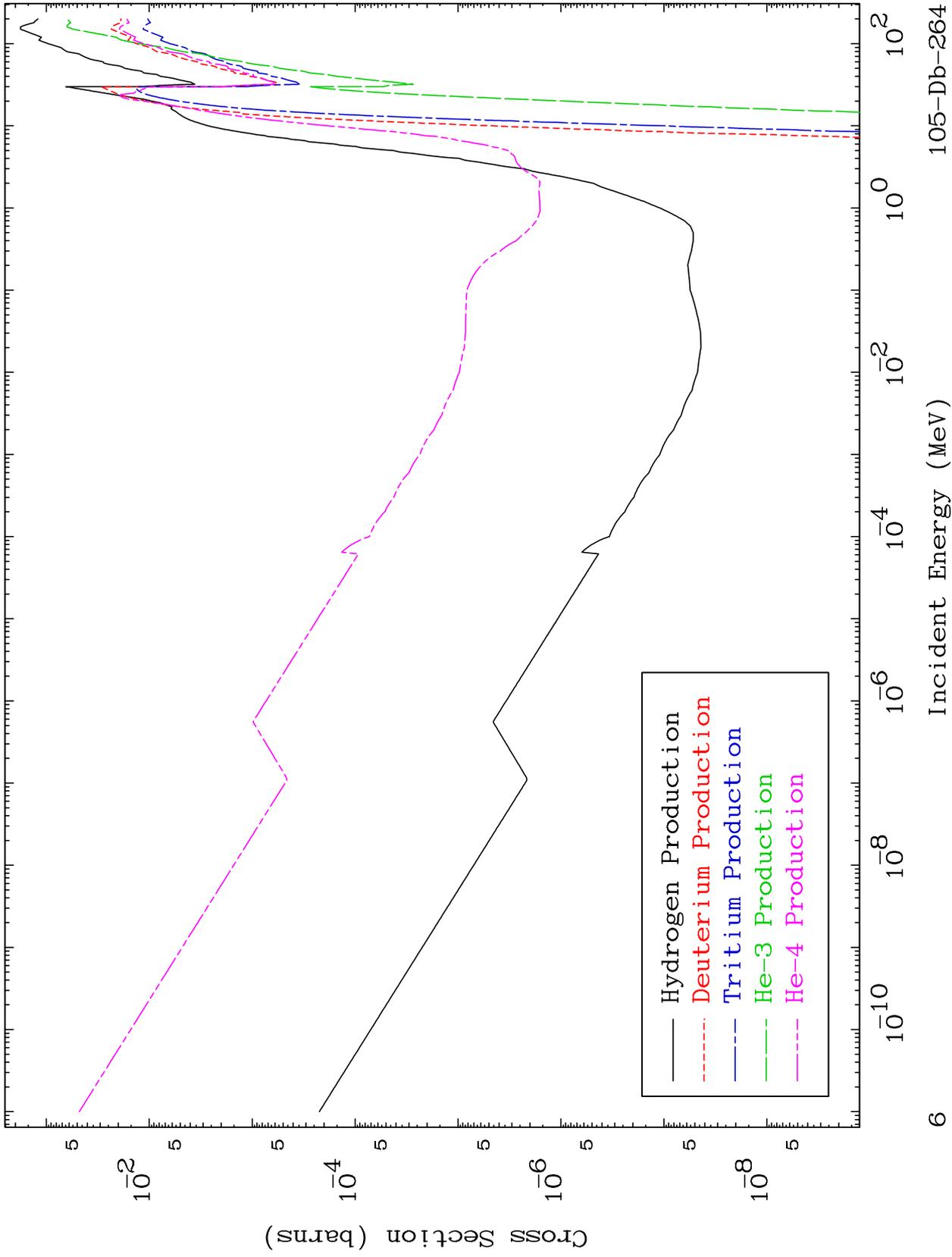
MAT 564

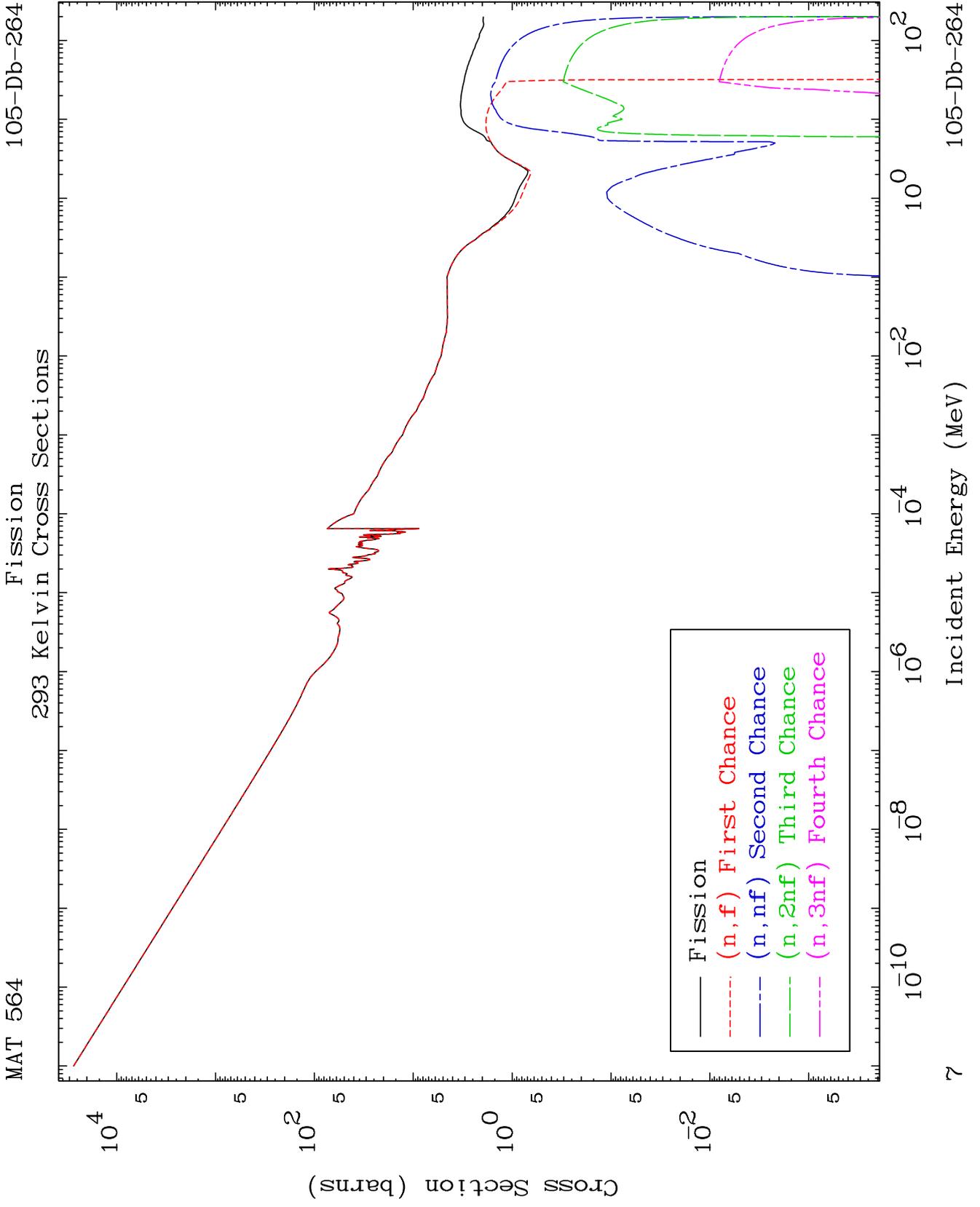
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MAT 564

Particle Production
293 Kelvin Cross Sections

105-Db-264

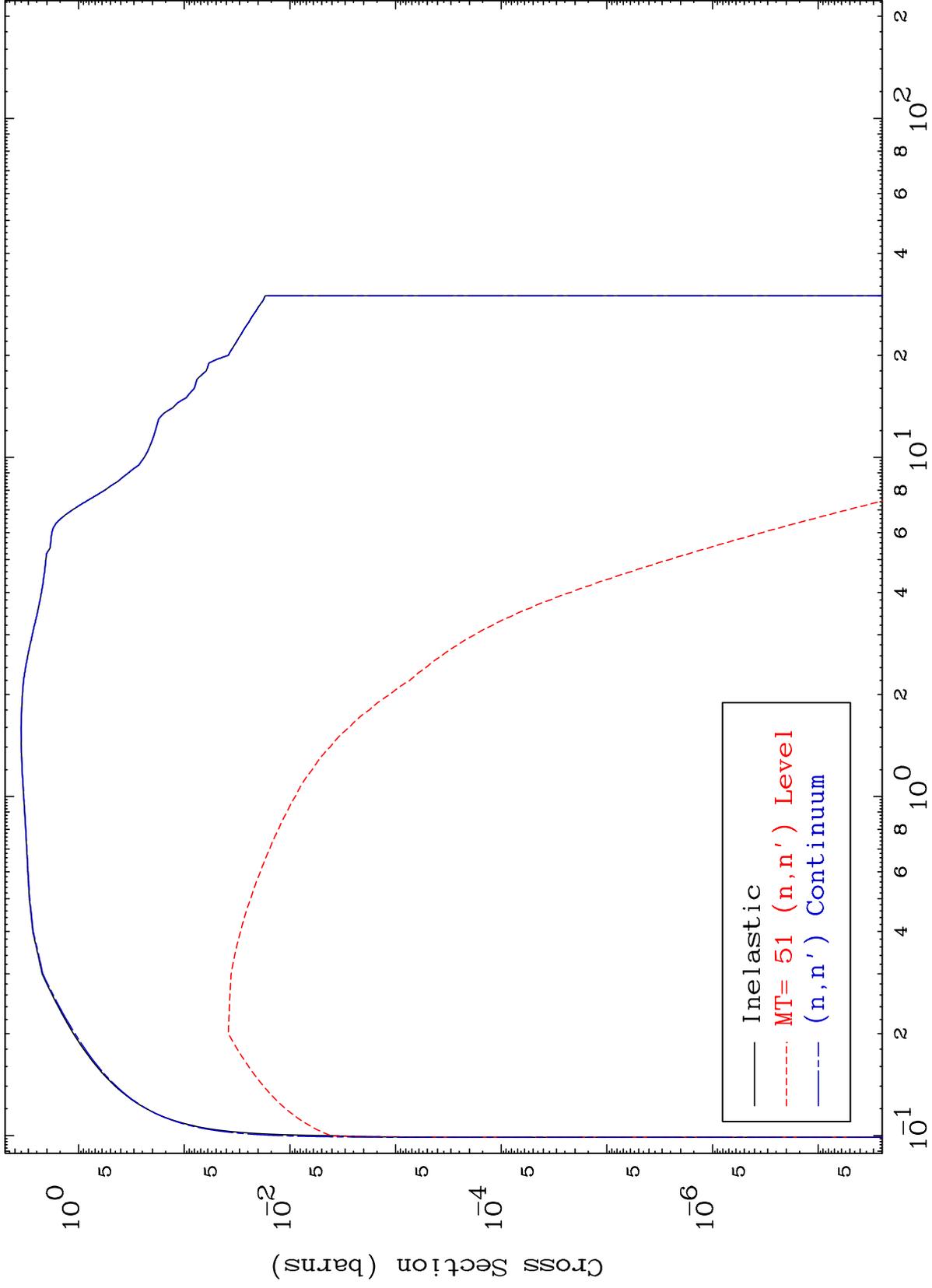




MAT 564

(n,n') Level
293 Kelvin Cross Sections

105-Db-264



Incident Energy (MeV)

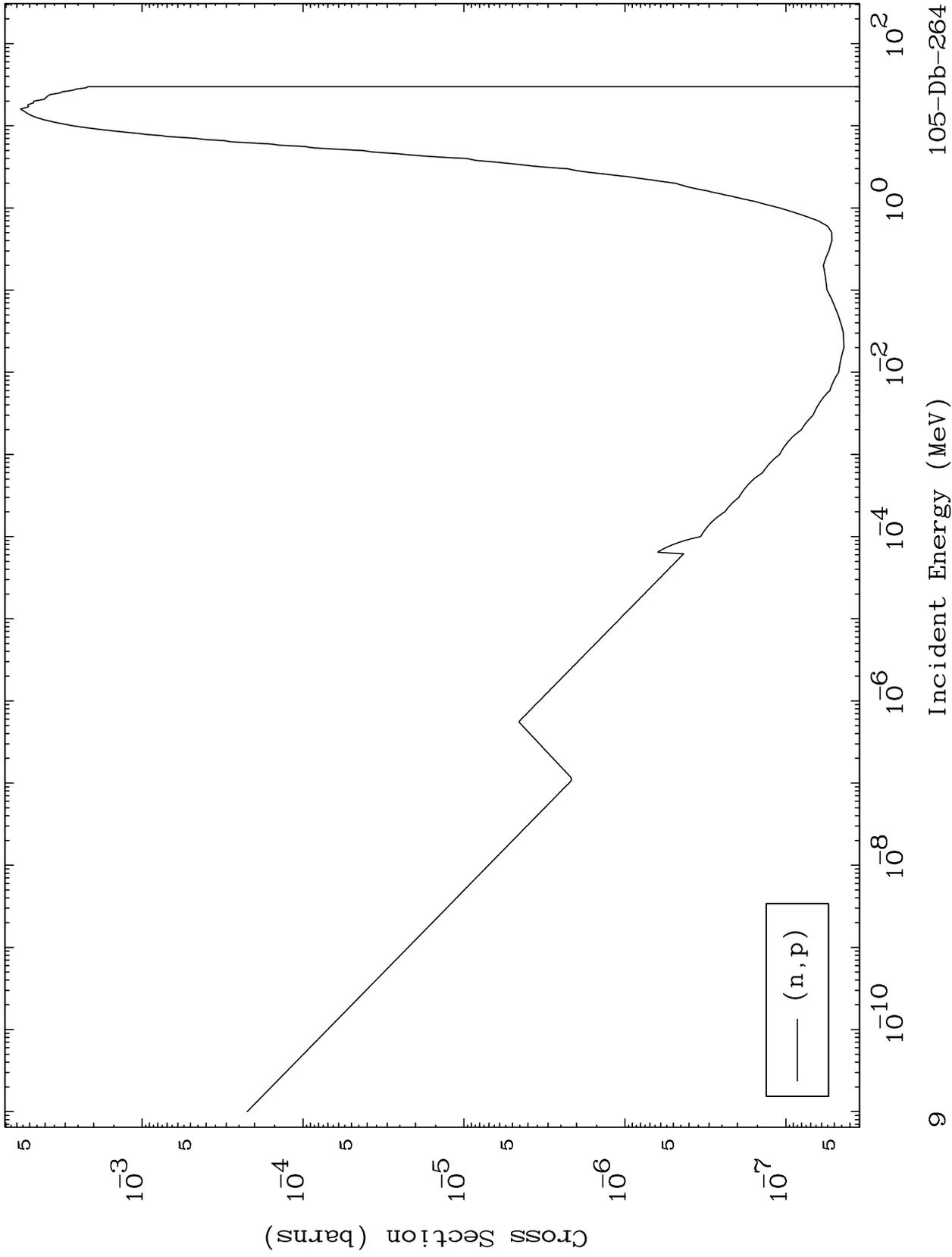
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8

MAT 564

(n,p) Levels
293 Kelvin Cross Sections

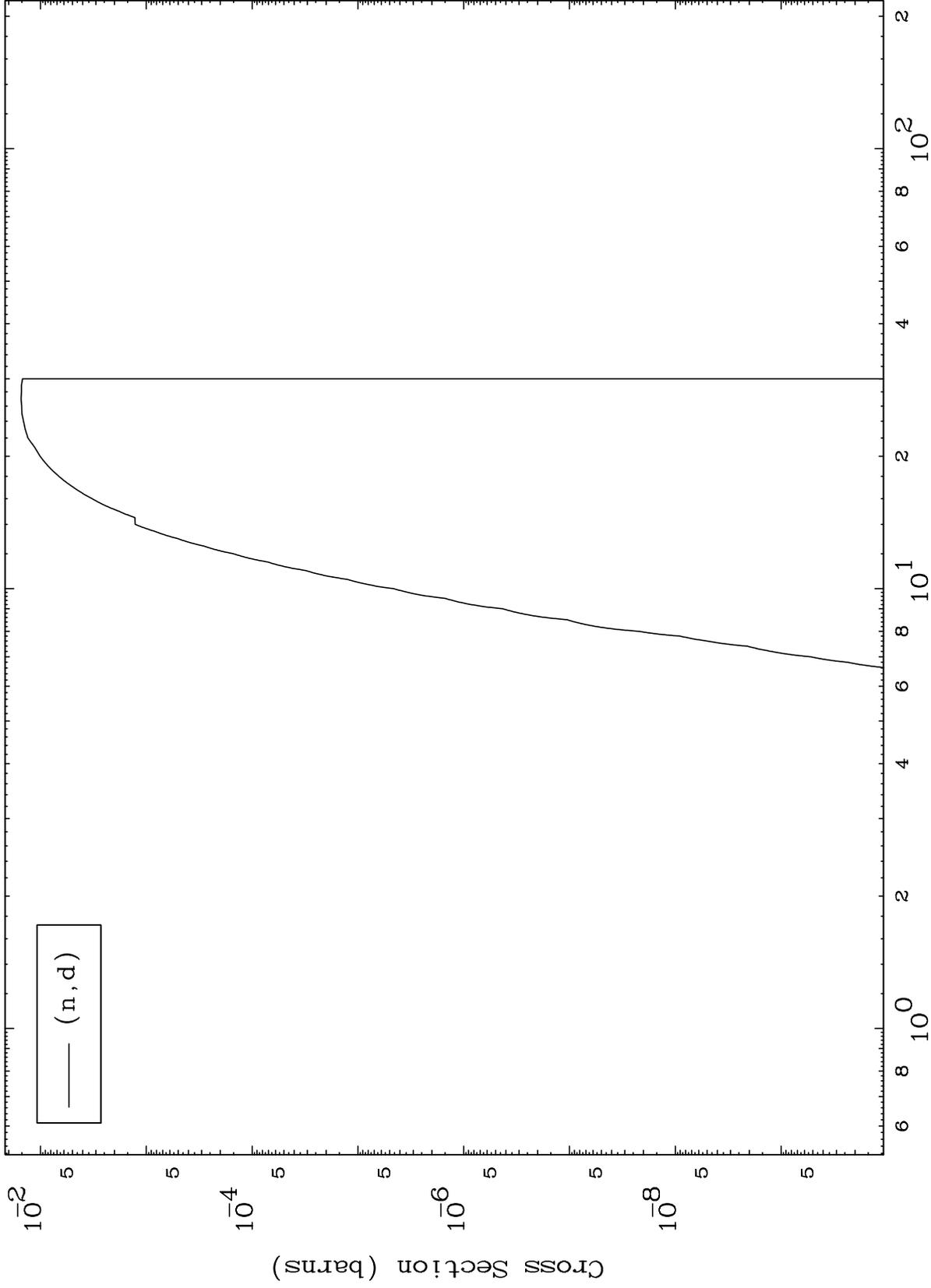
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MAT 564

(n,d) Levels
293 Kelvin Cross Sections

105-Db-264



10

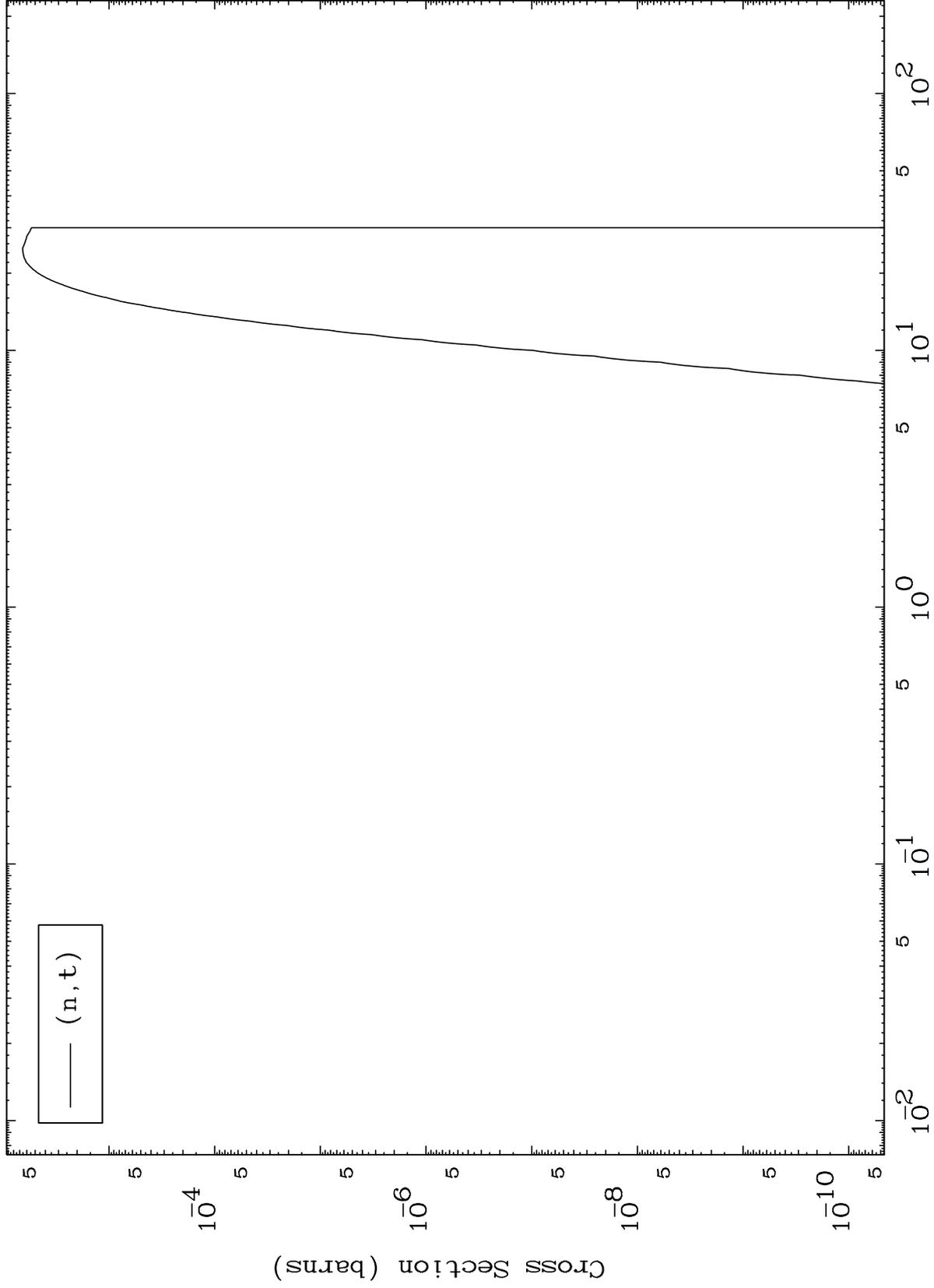
Incident Energy (MeV)

105-Db-264

MAT 564

(n, t) Levels
293 Kelvin Cross Sections

105-Db-264



11

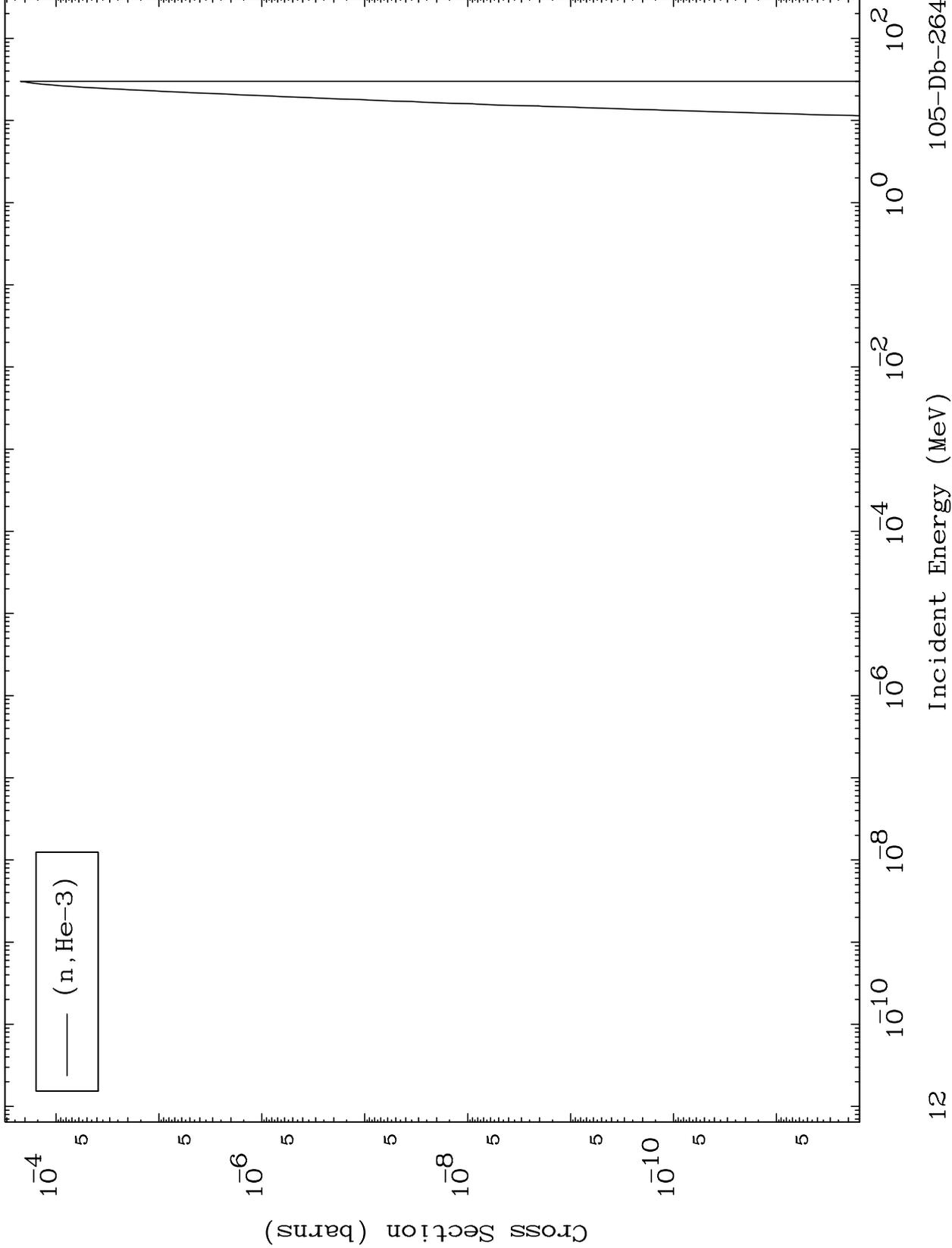
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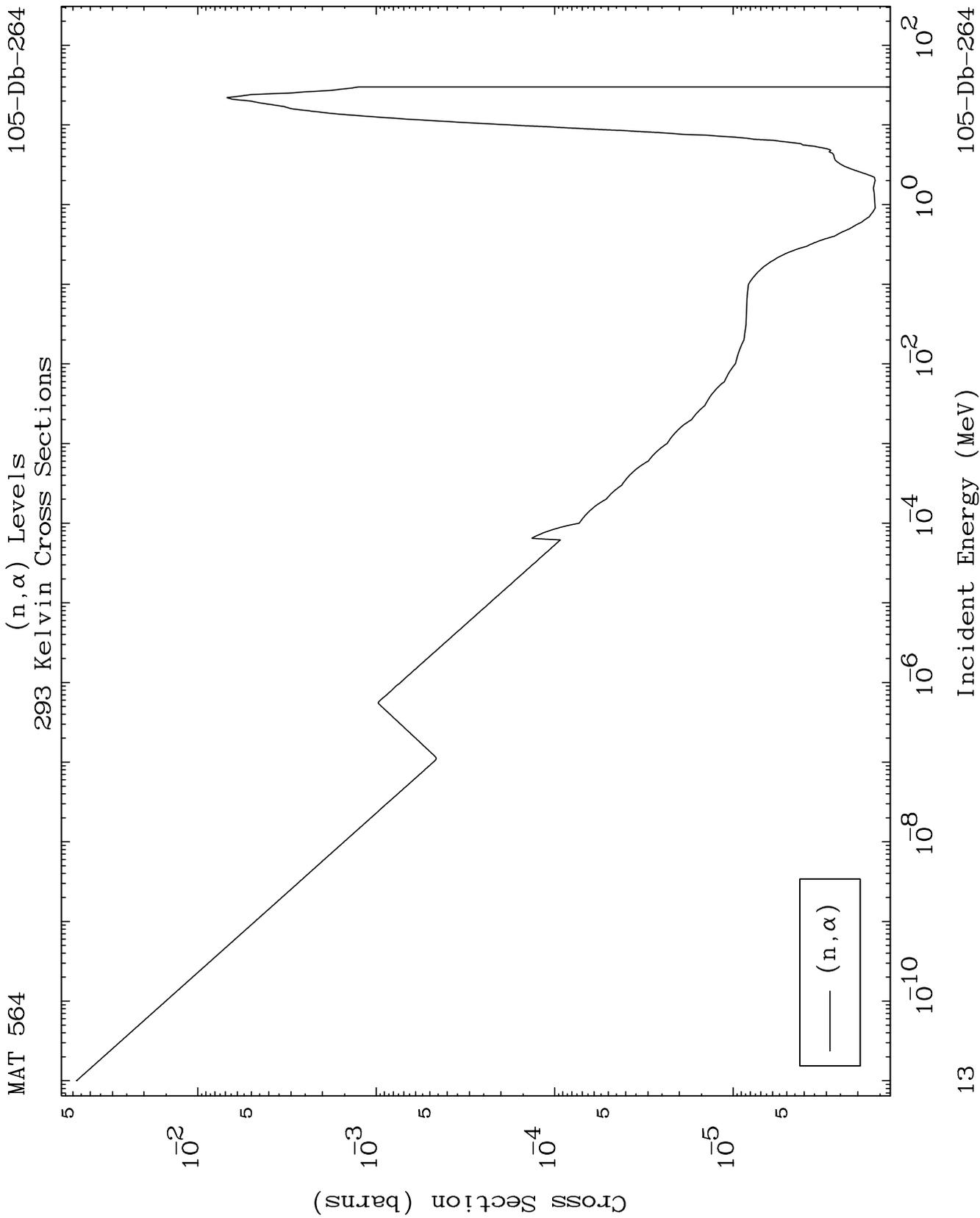
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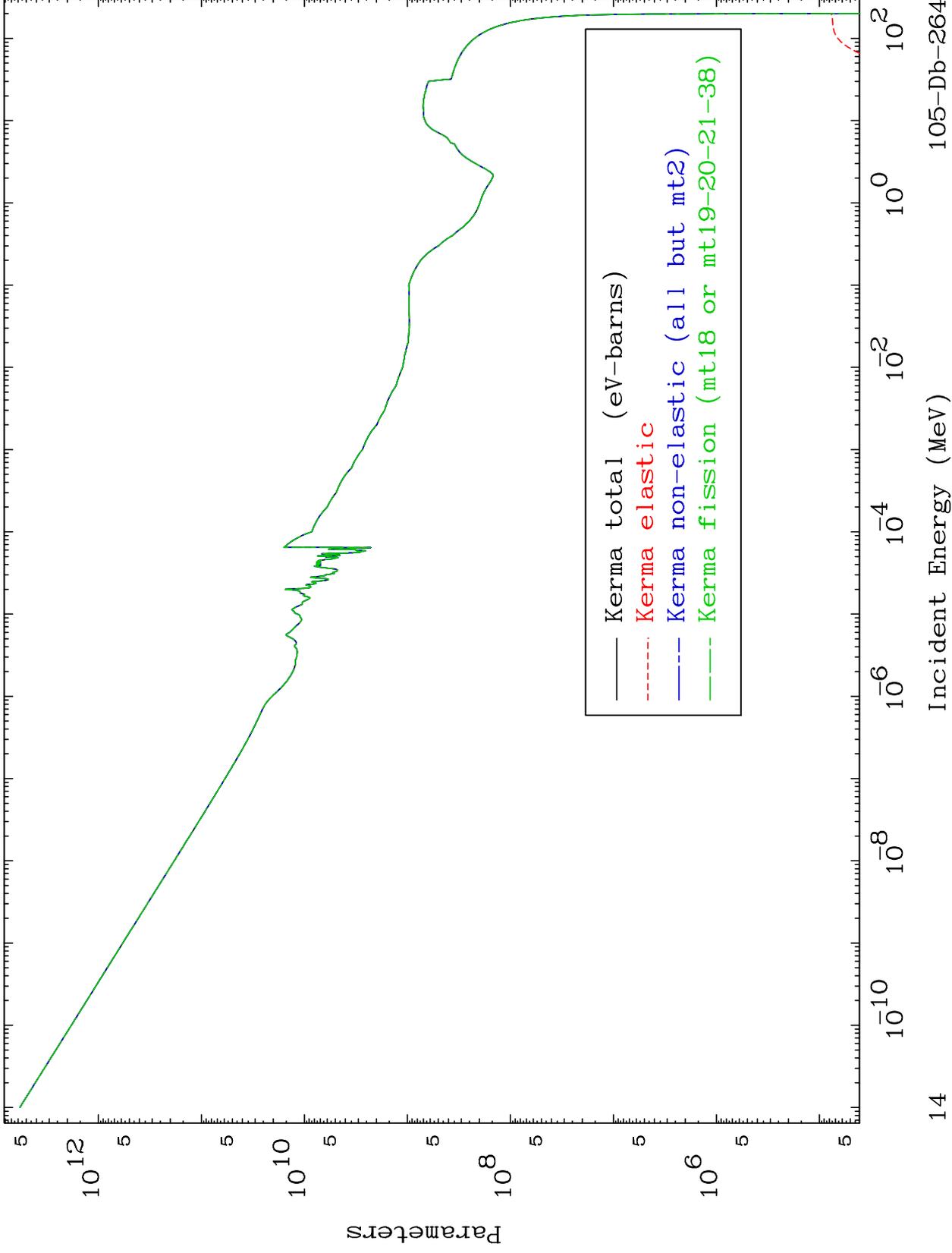
MAT 564

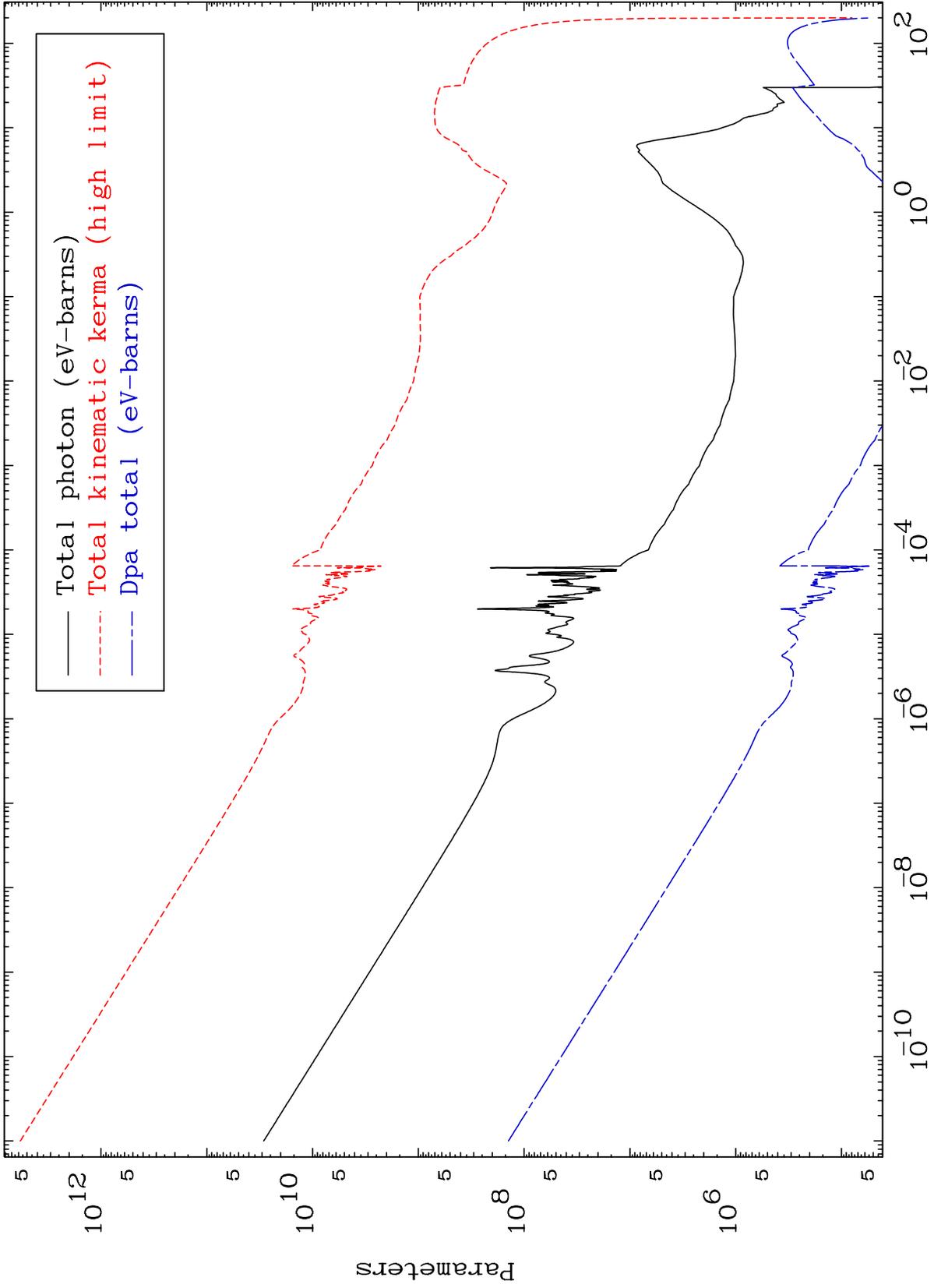
(n,He3) Levels
293 Kelvin Cross Sections

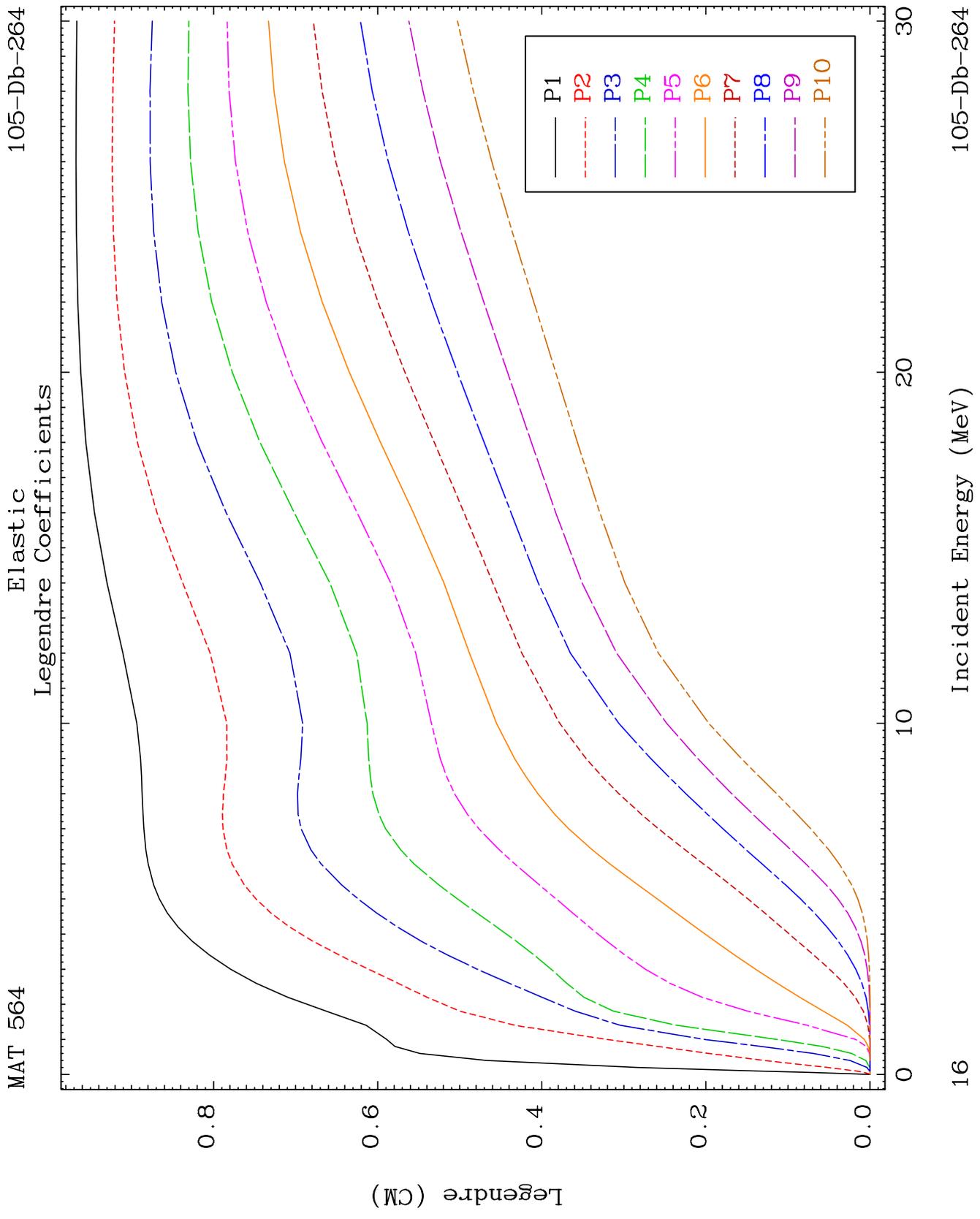
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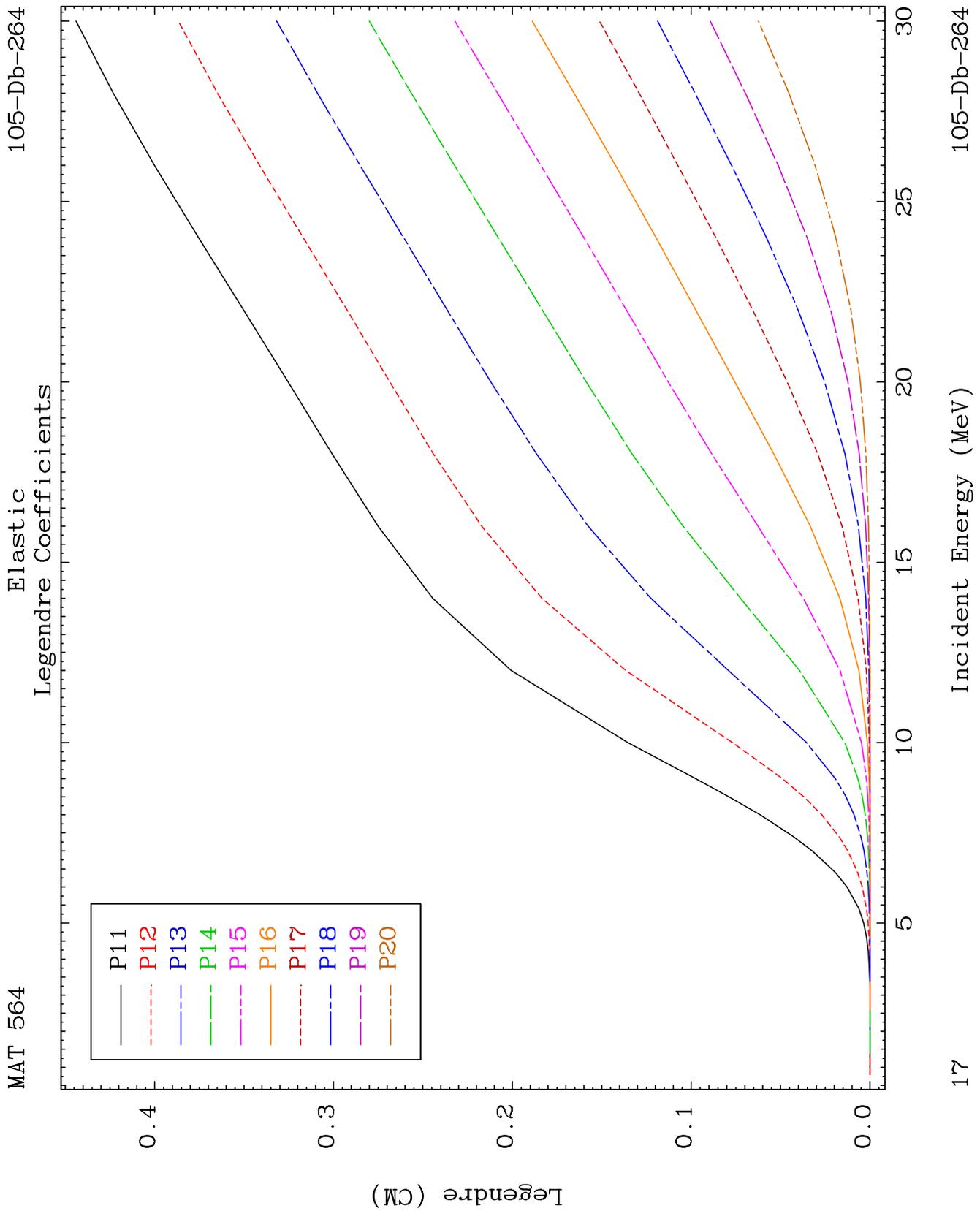








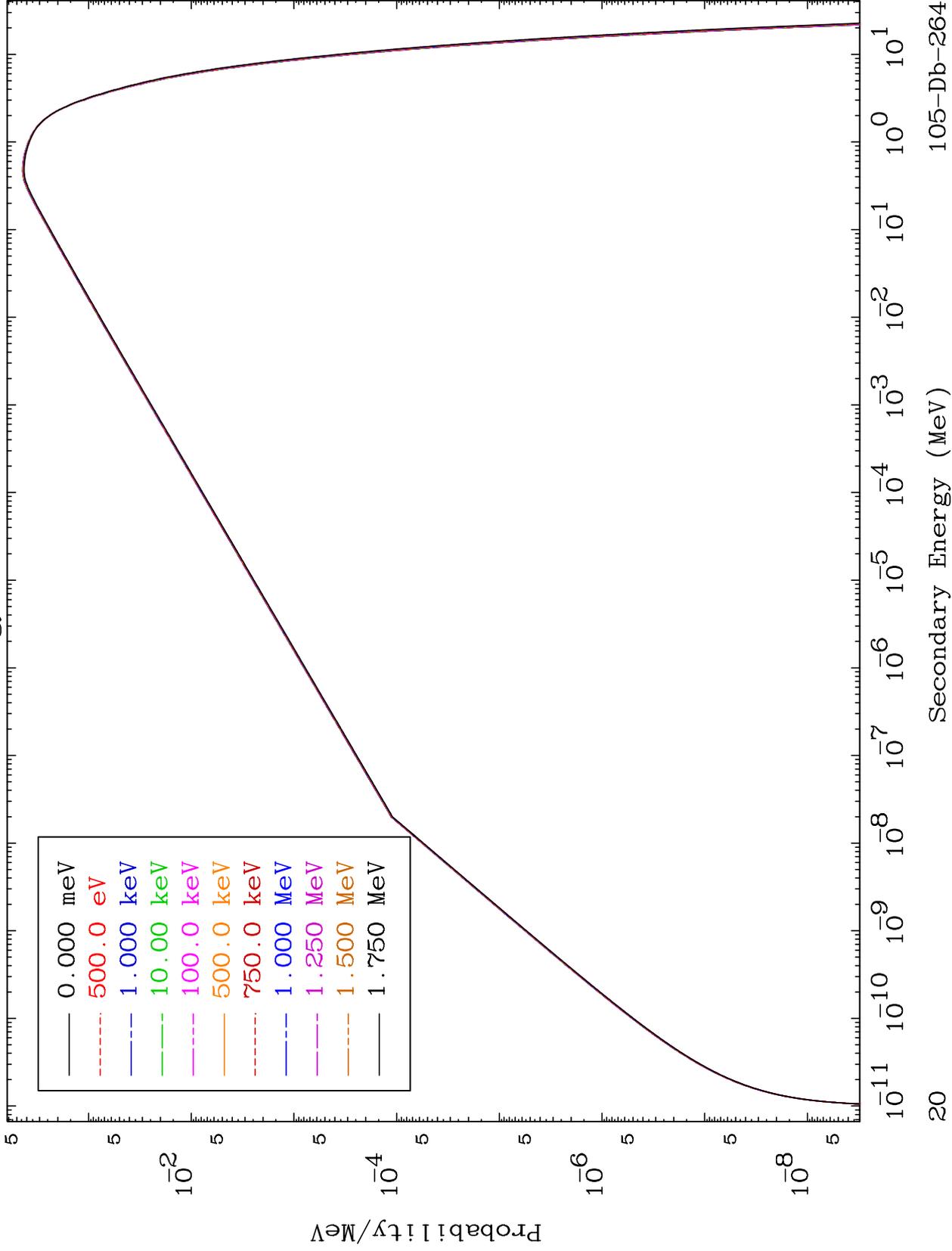




MAT 564

Fission Energy Distribution

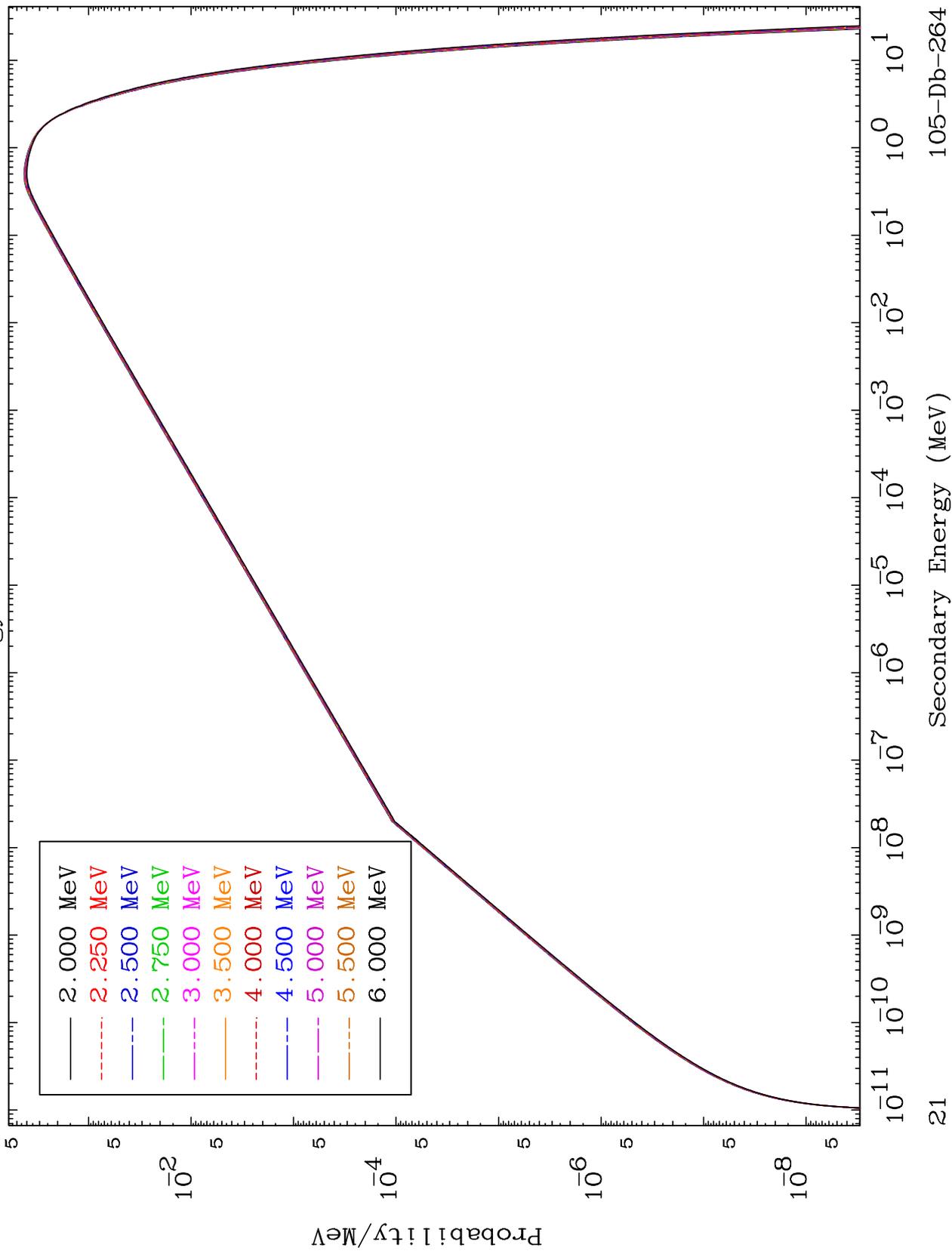
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MAT 564

Fission Energy Distribution

105-Db-264

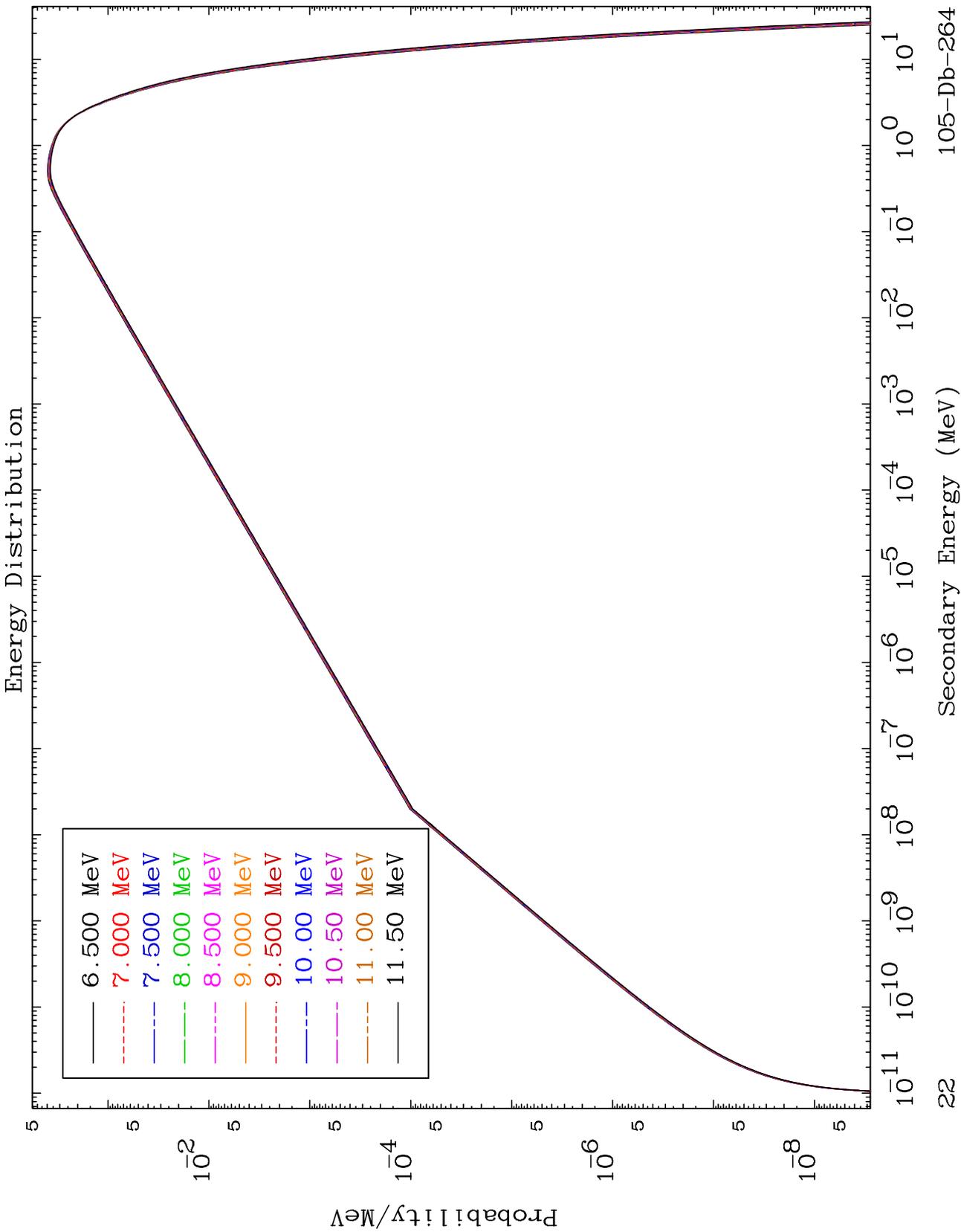


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MAT 564

Fission

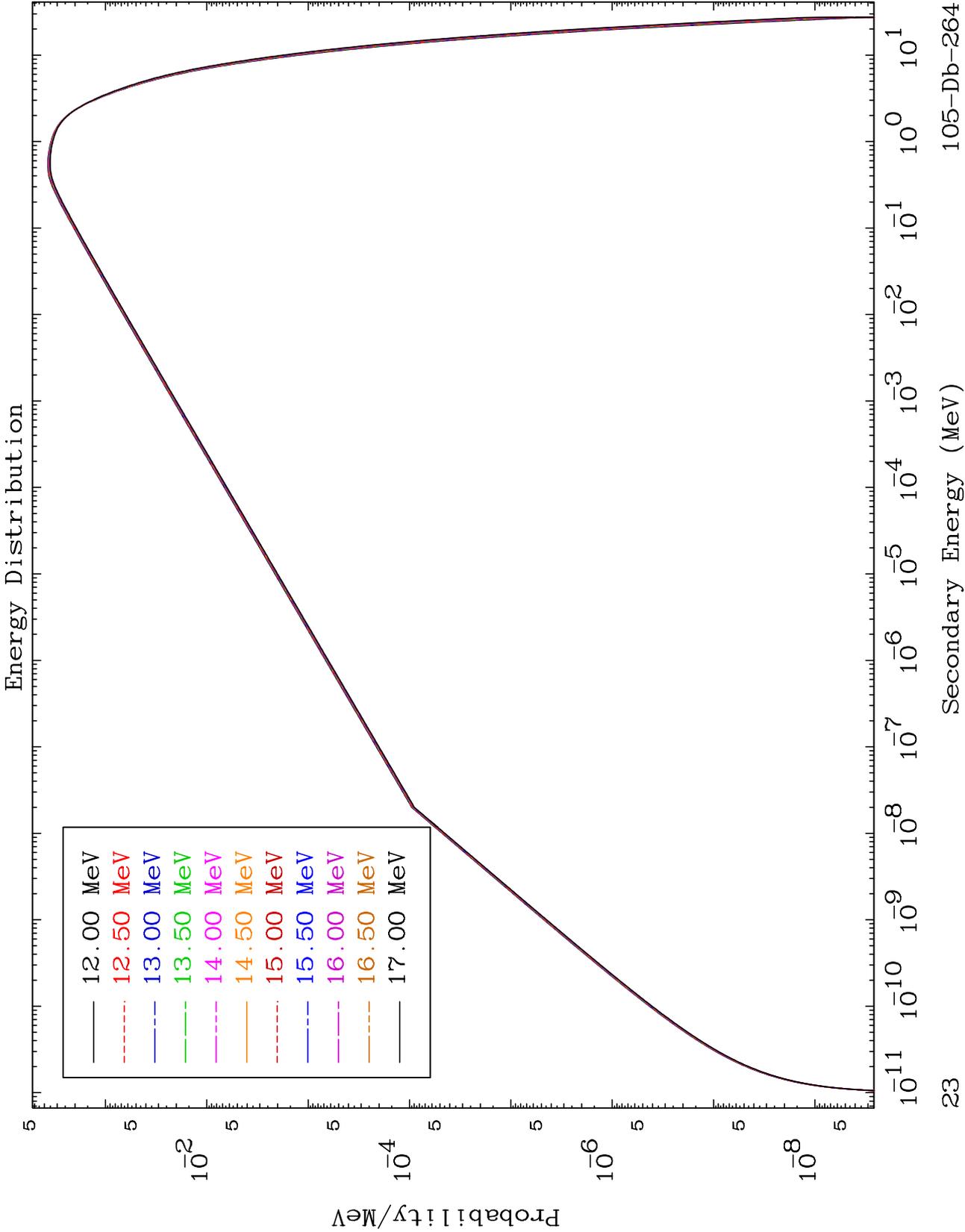
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MAT 564

Fission

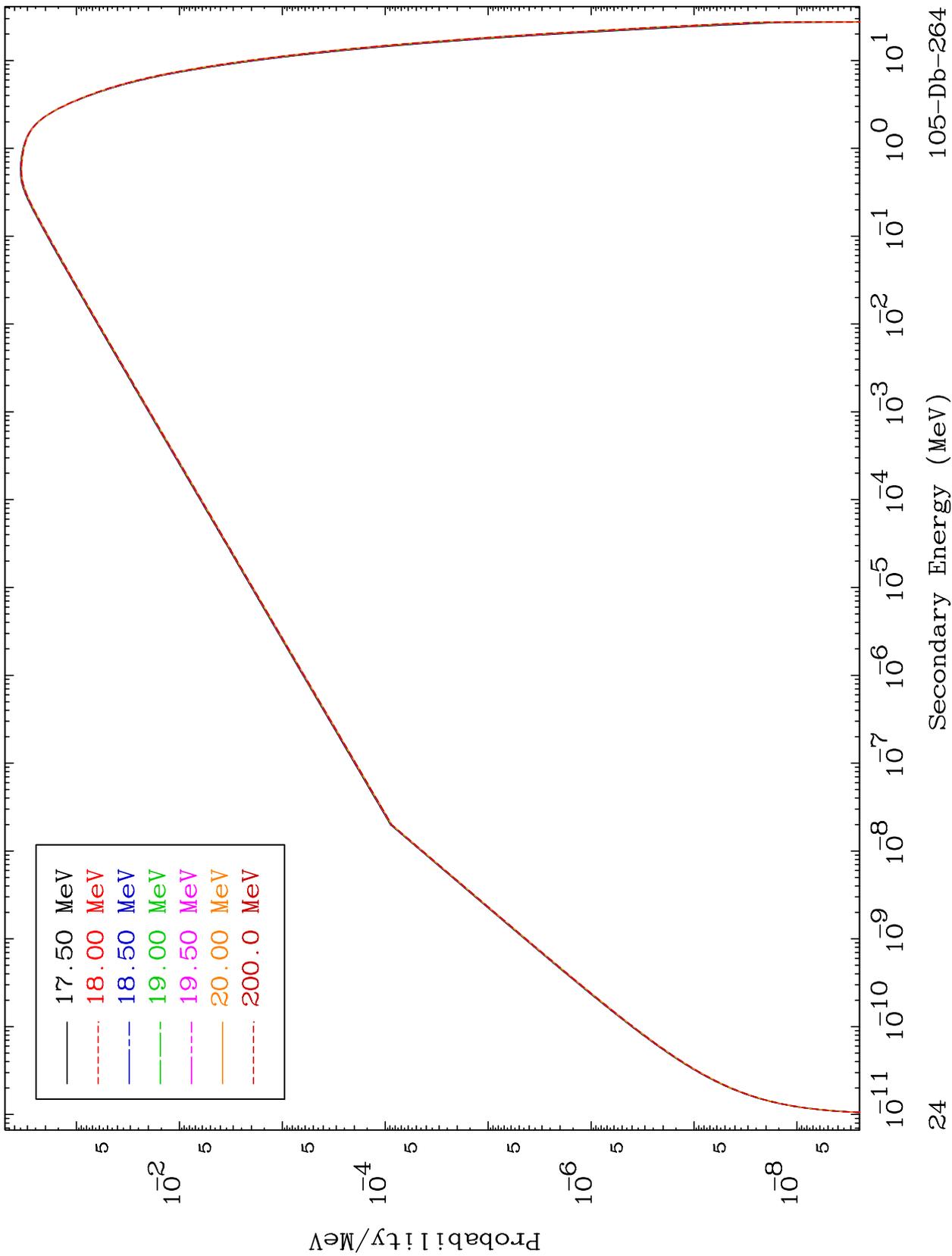
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MAT 564

Fission Energy Distribution

105-Db-264

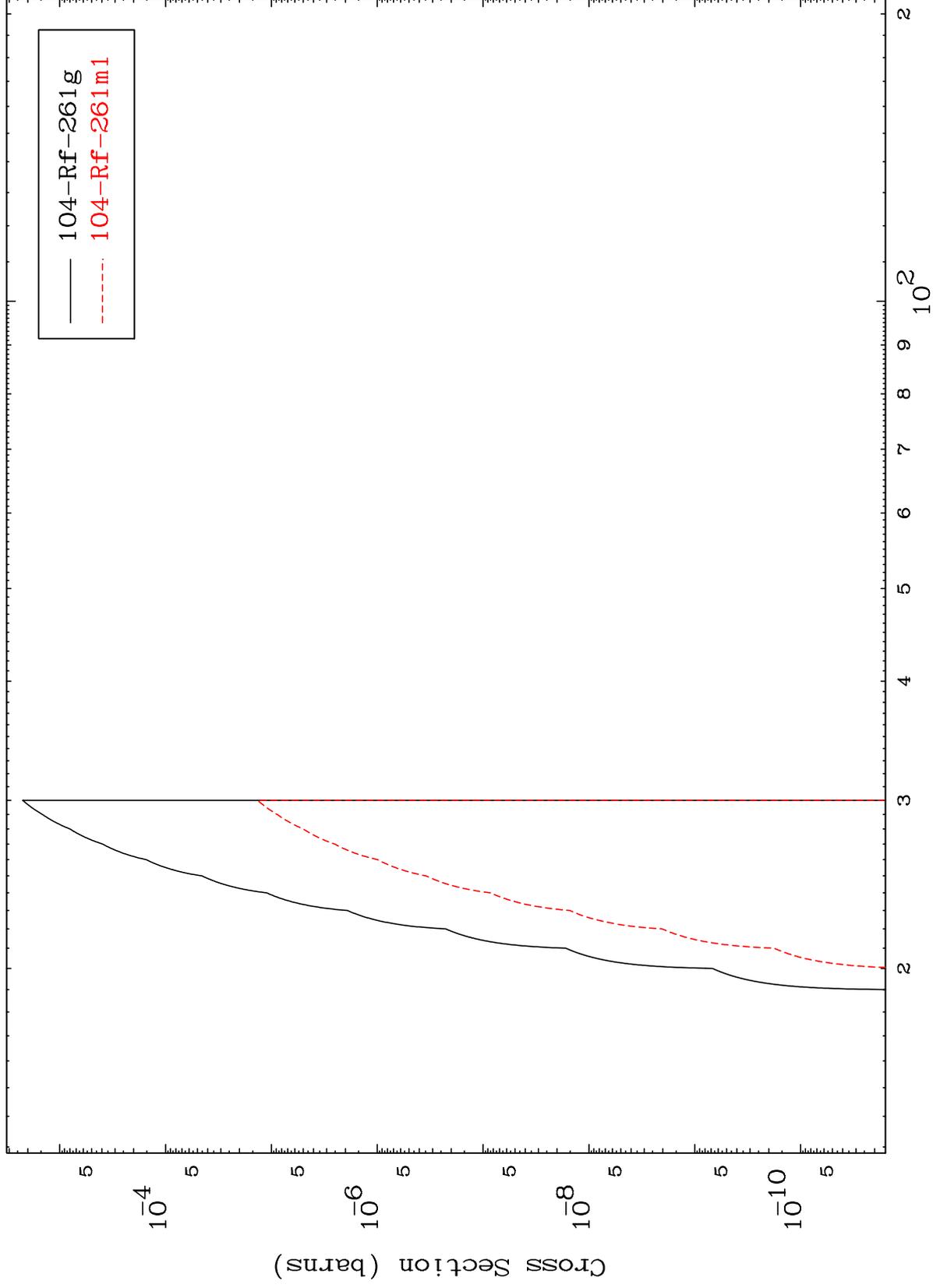


MAT 564

(n,2n) d

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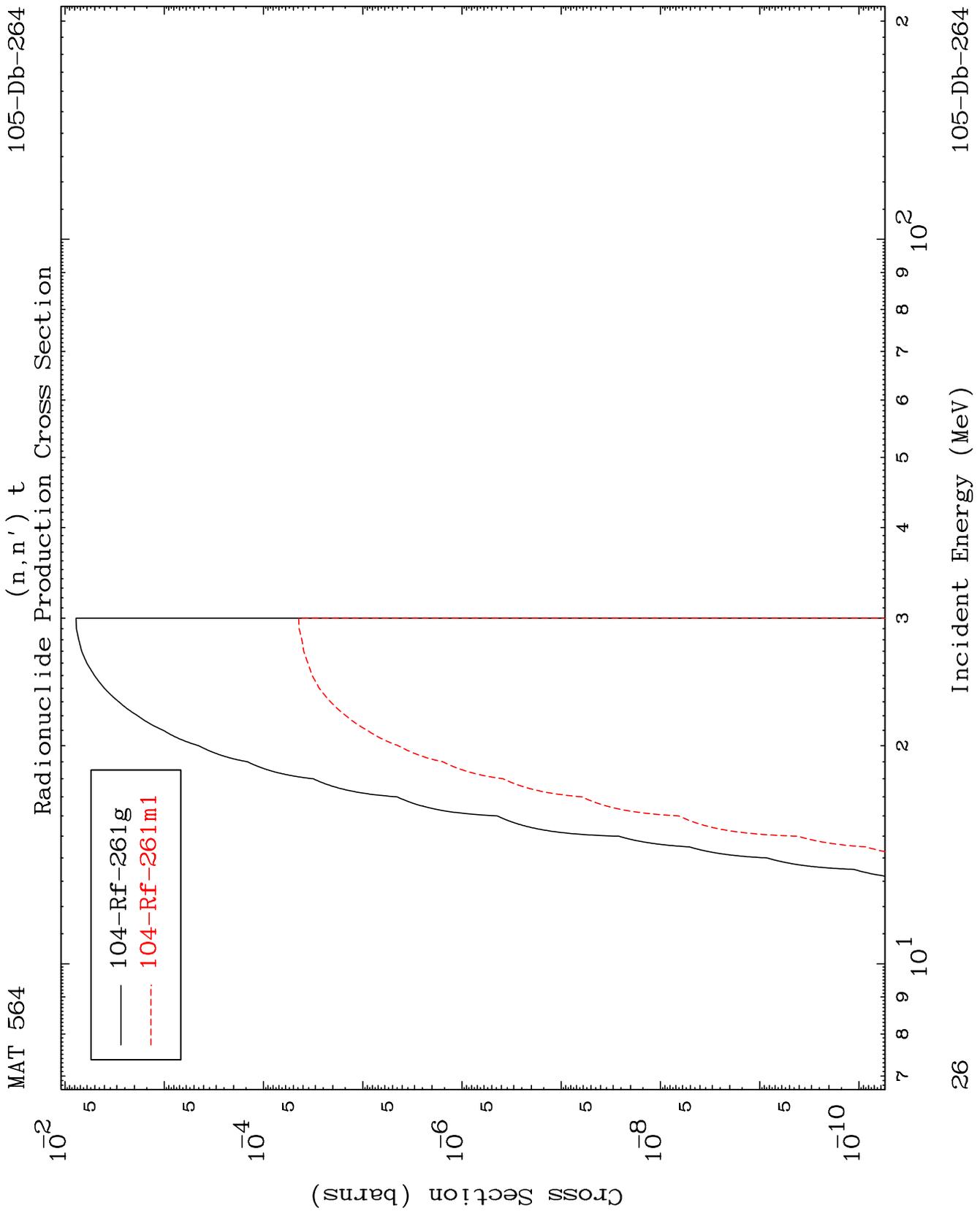
Radionuclide Production Cross Section



25

Incident Energy (MeV)

105-Db-264

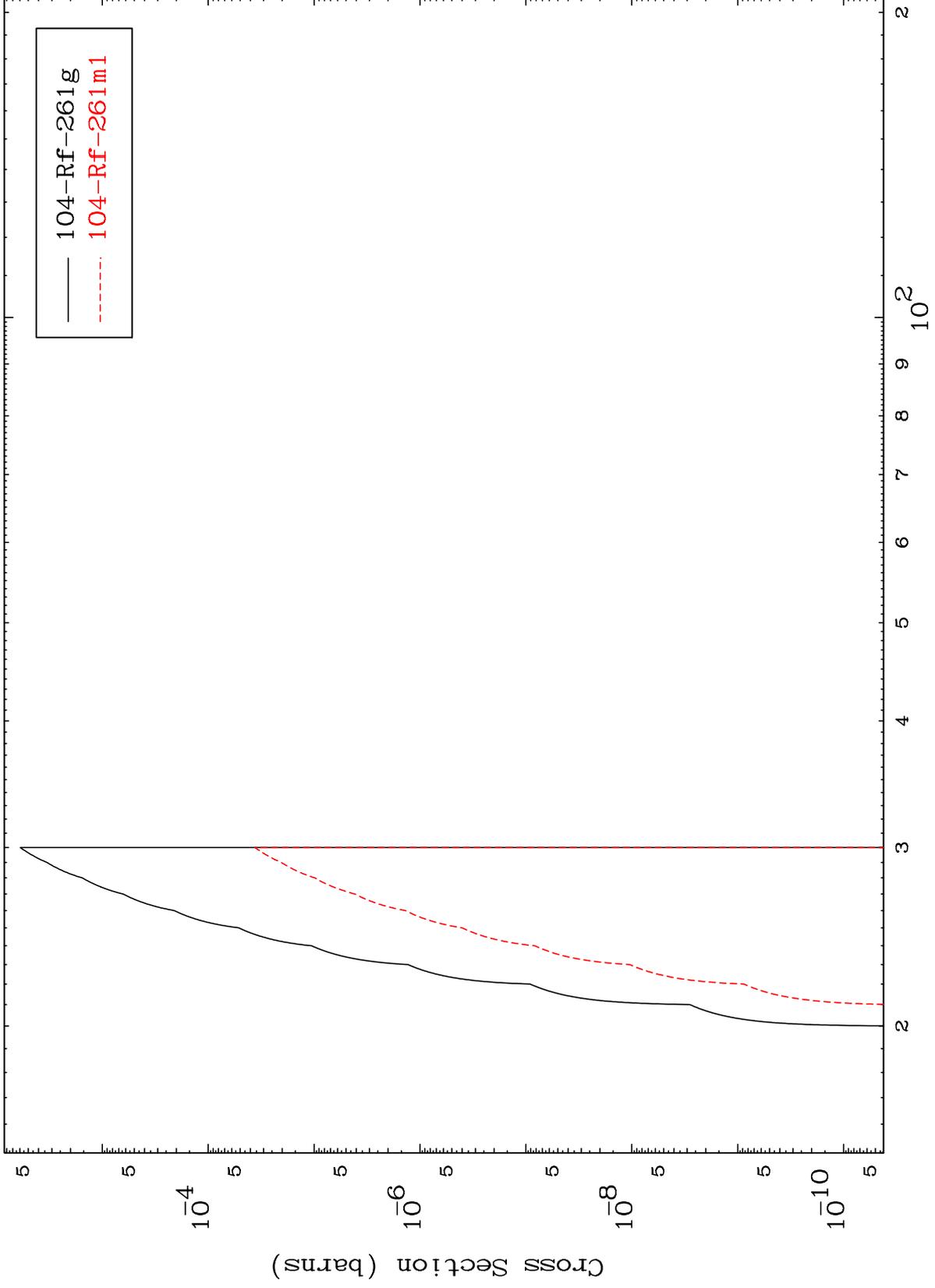


MAT 564

(n,3n) p

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Radionuclide Production Cross Section



27

Incident Energy (MeV)

105-Db-264