

Program EVALPLOT  
(Version 2018-1)

by

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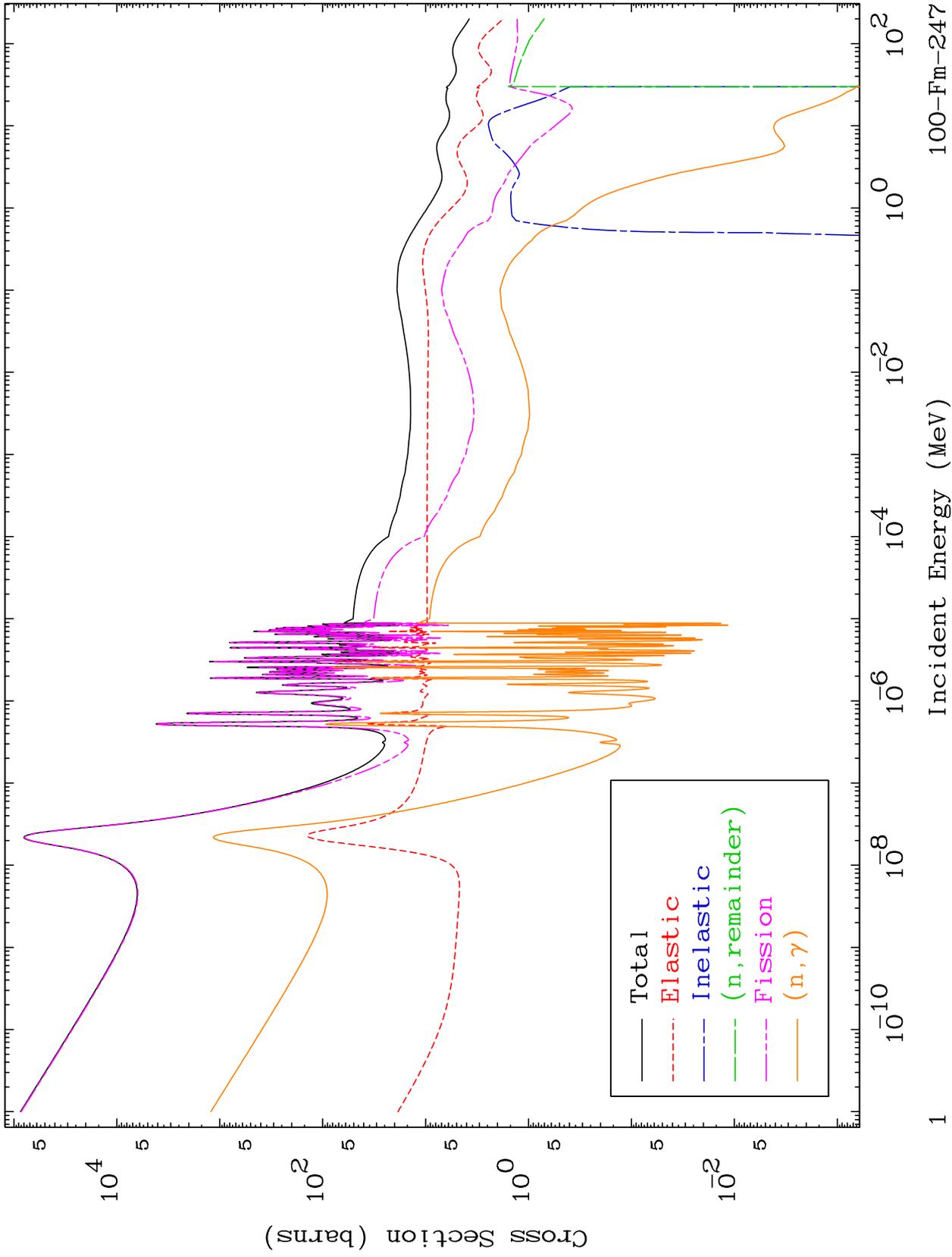
Press Mouse Button to Start

MAT 9928

Major

293 Kelvin Cross Sections

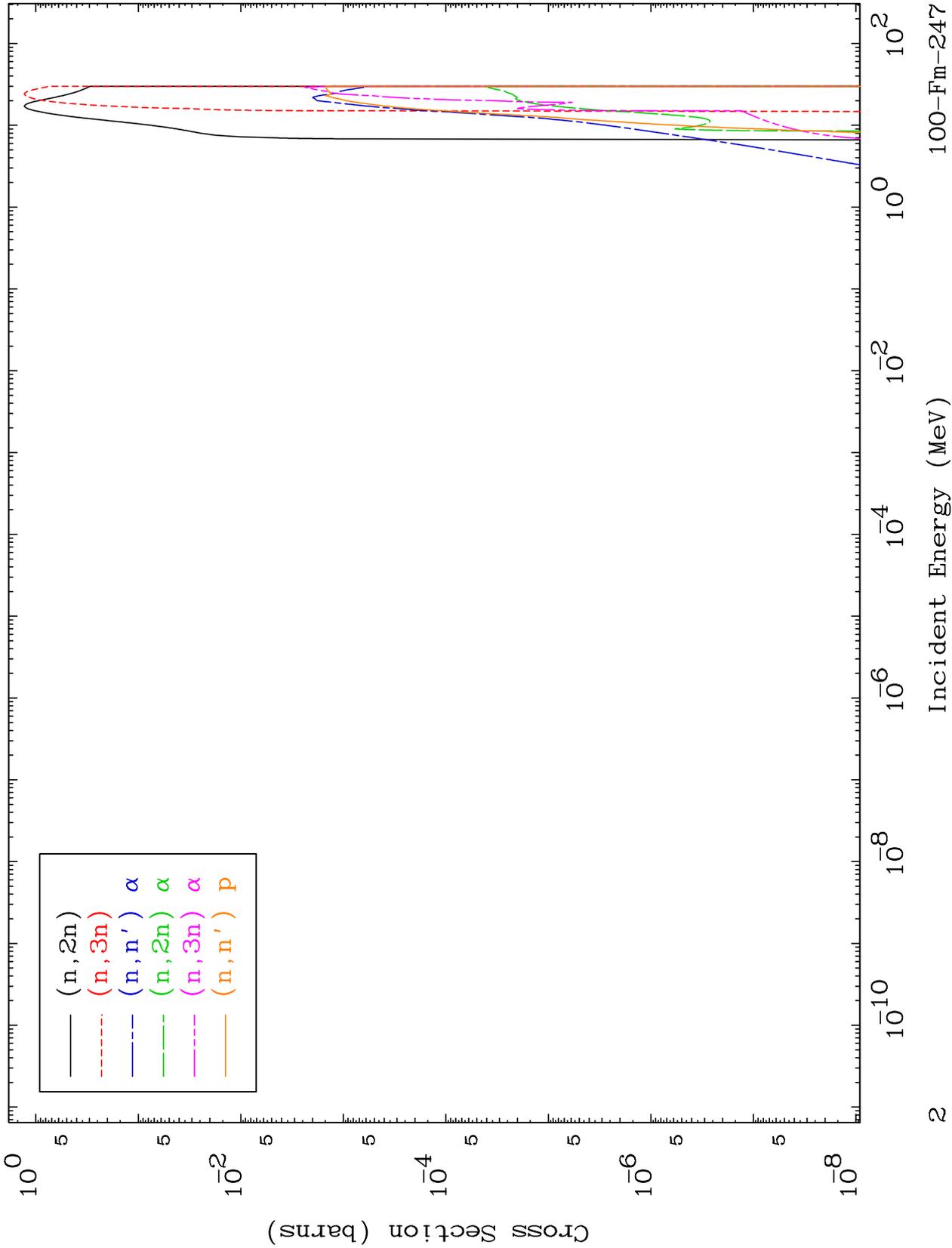
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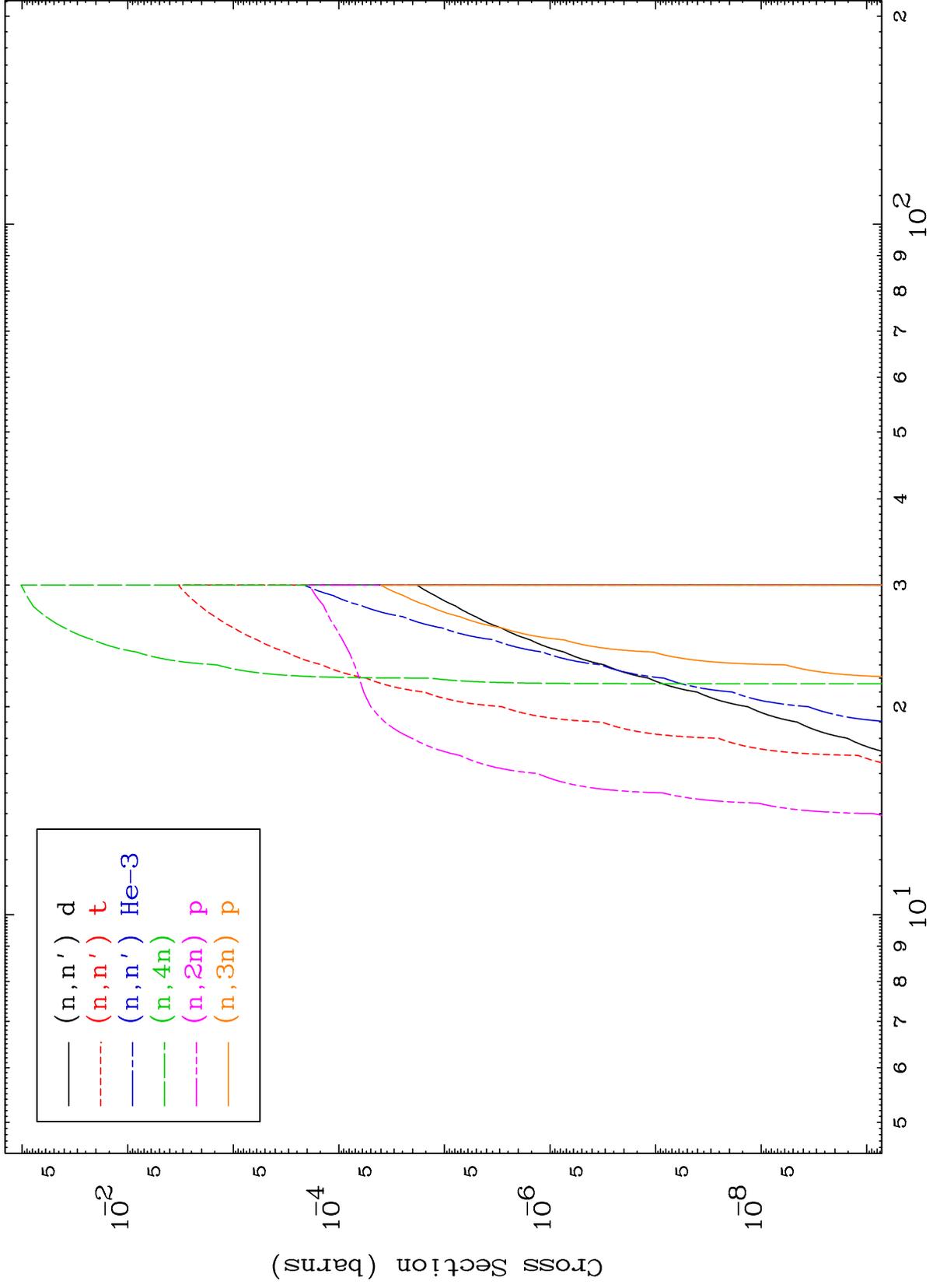


MAT 9928

Neutron Production  
293 Kelvin Cross Sections

100-Fm-247

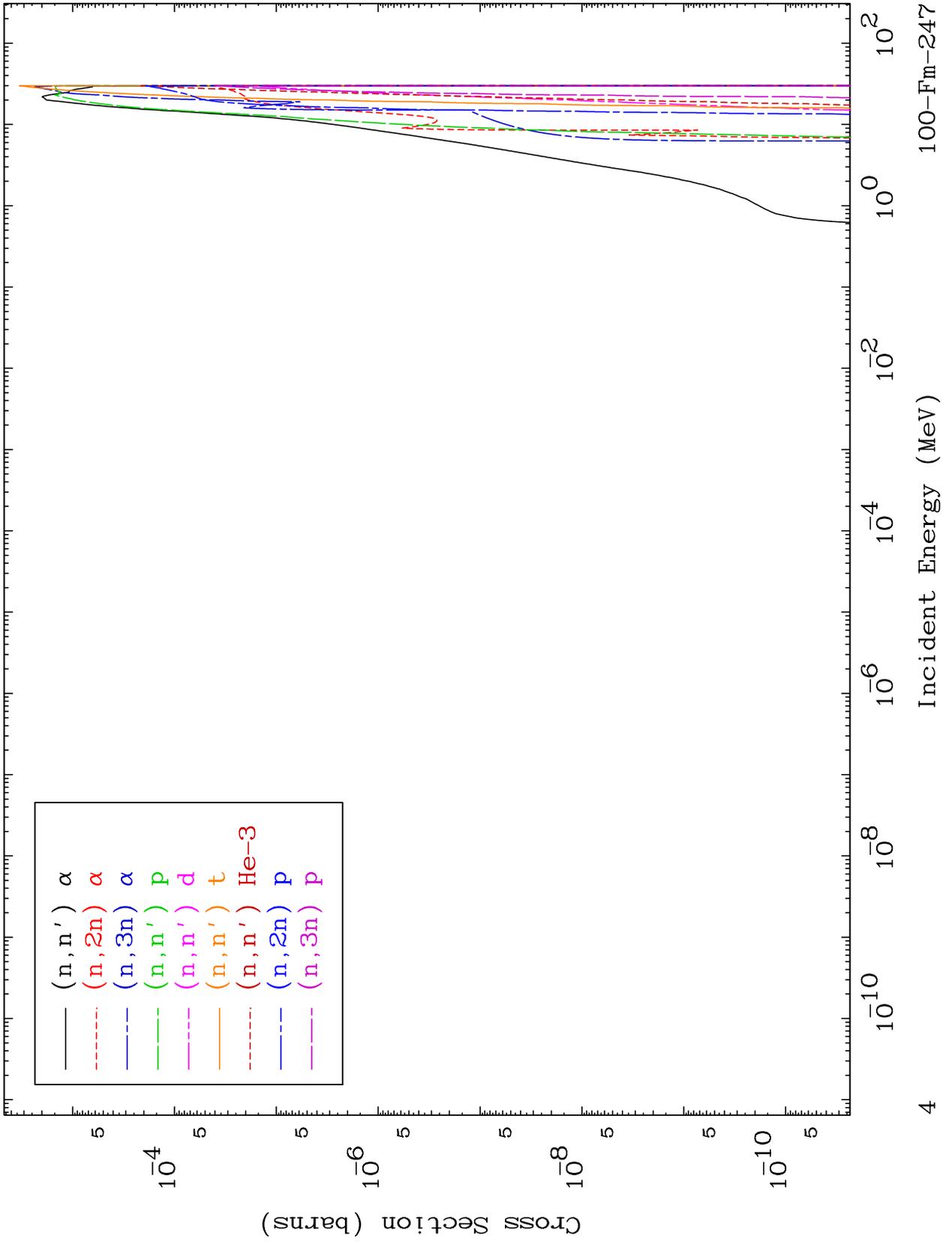




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Charged Particle  
293 Kelvin Cross Sections

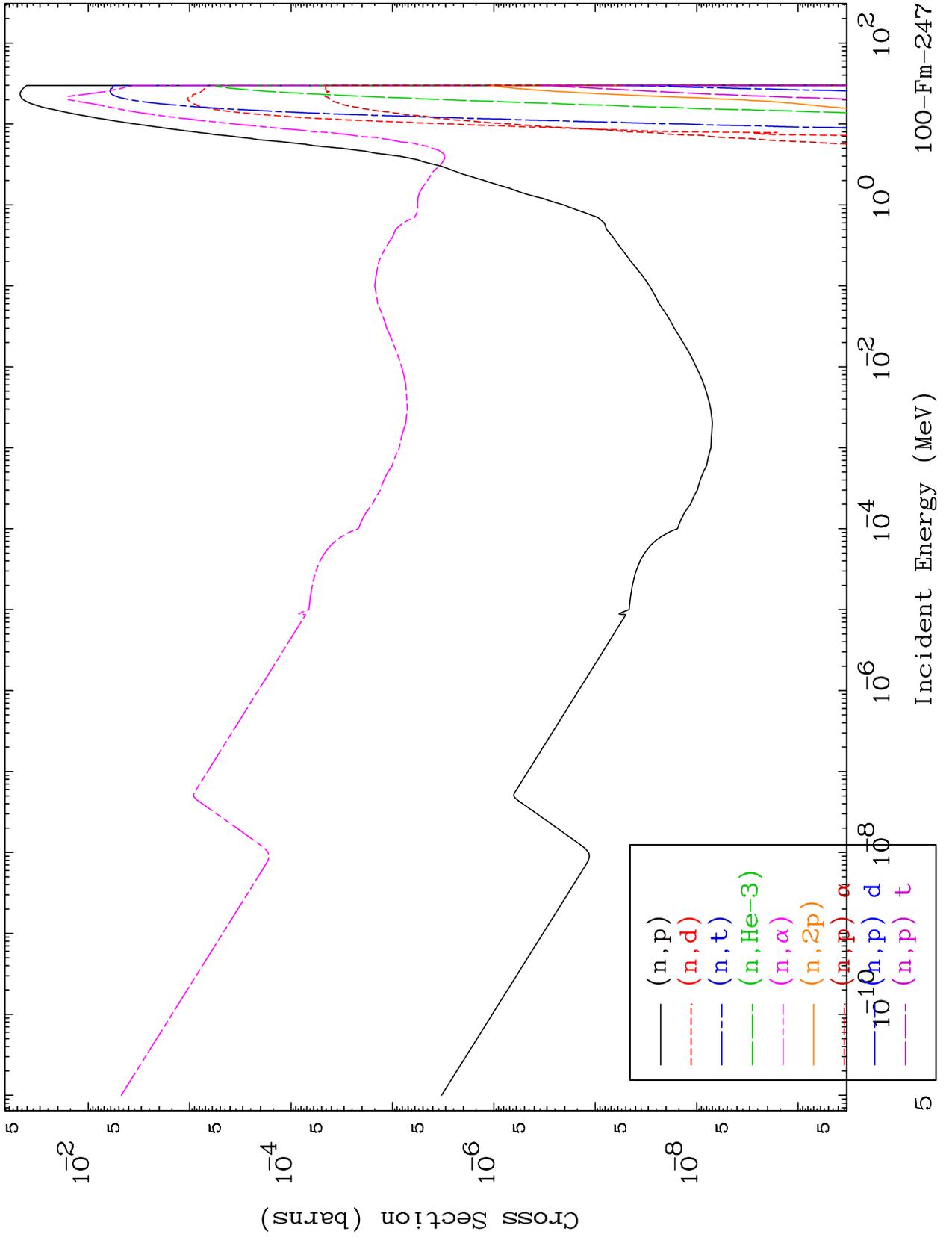
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Charged Particle  
293 Kelvin Cross Sections

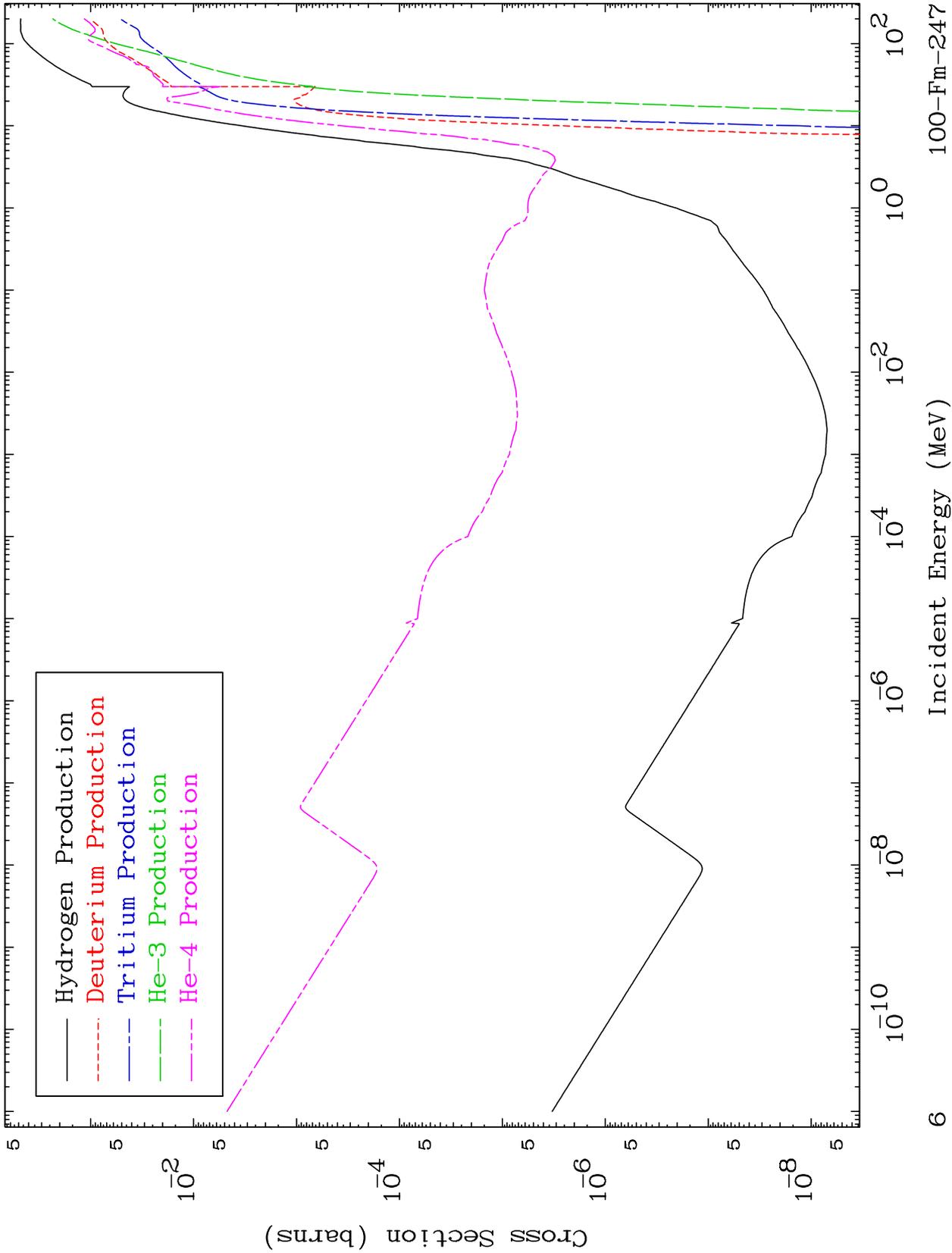
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Particle Production  
293 Kelvin Cross Sections

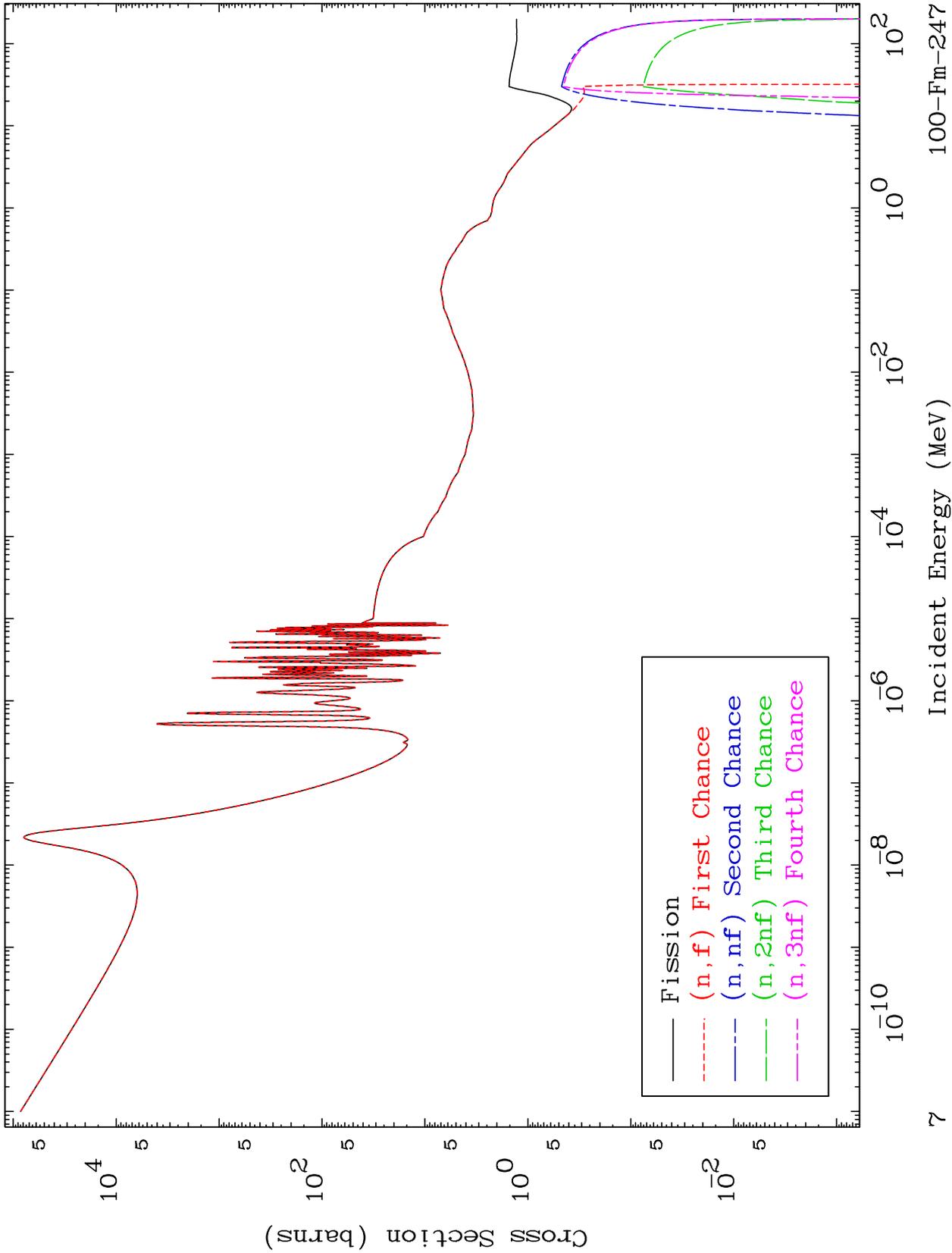
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Fission  
293 Kelvin Cross Sections

100-Fm-247

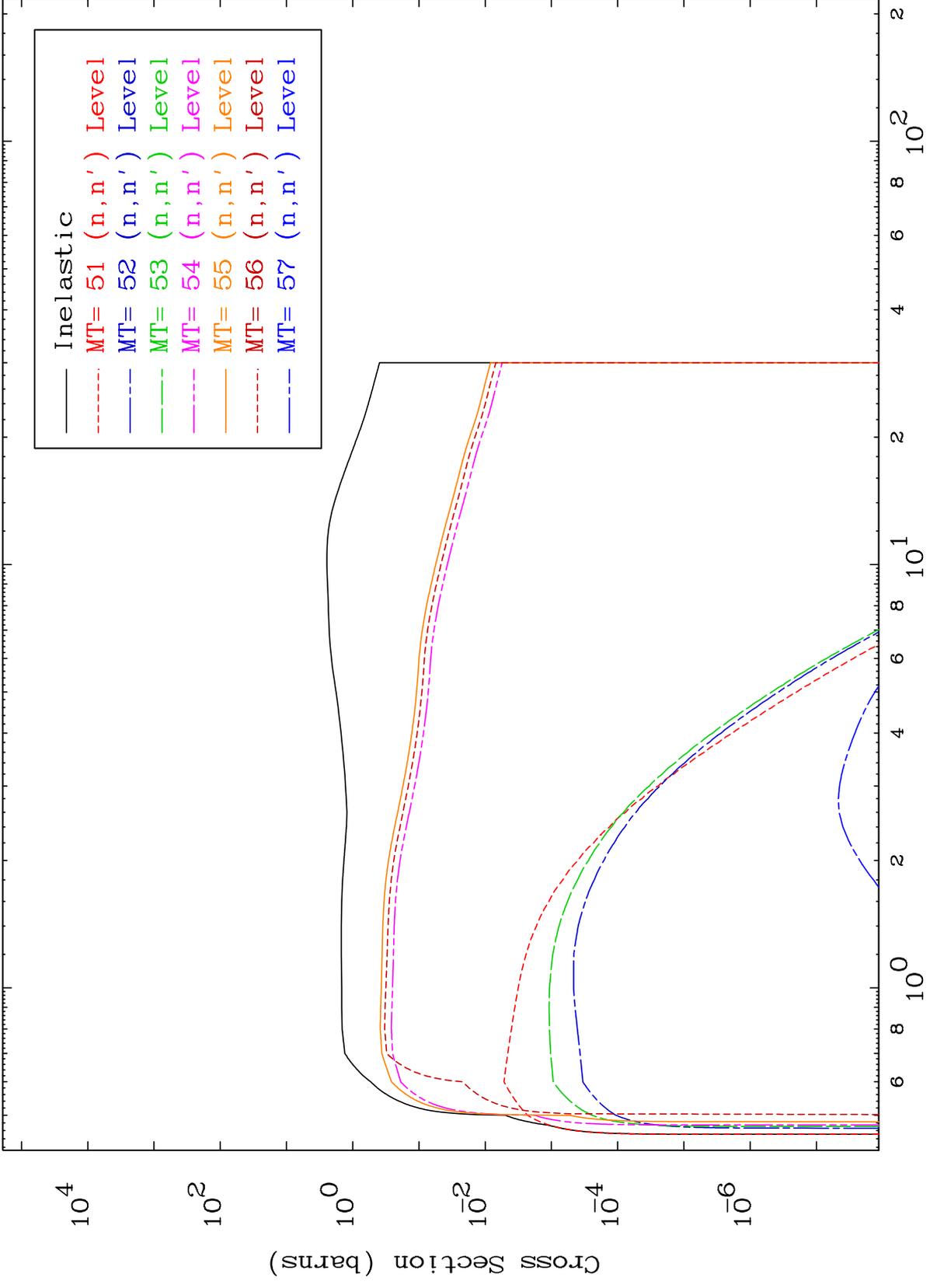


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(n,n') Level

100-Fm-247

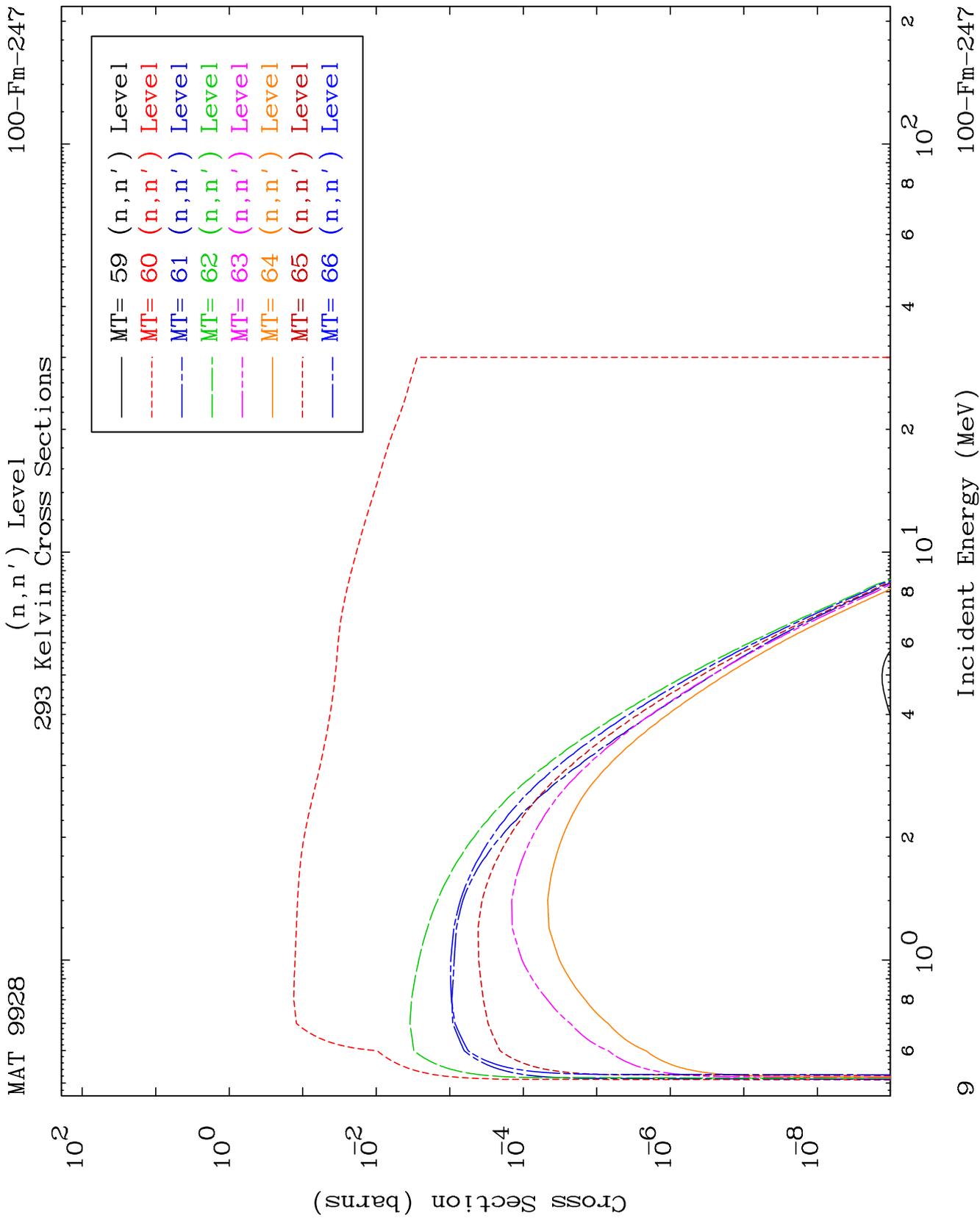
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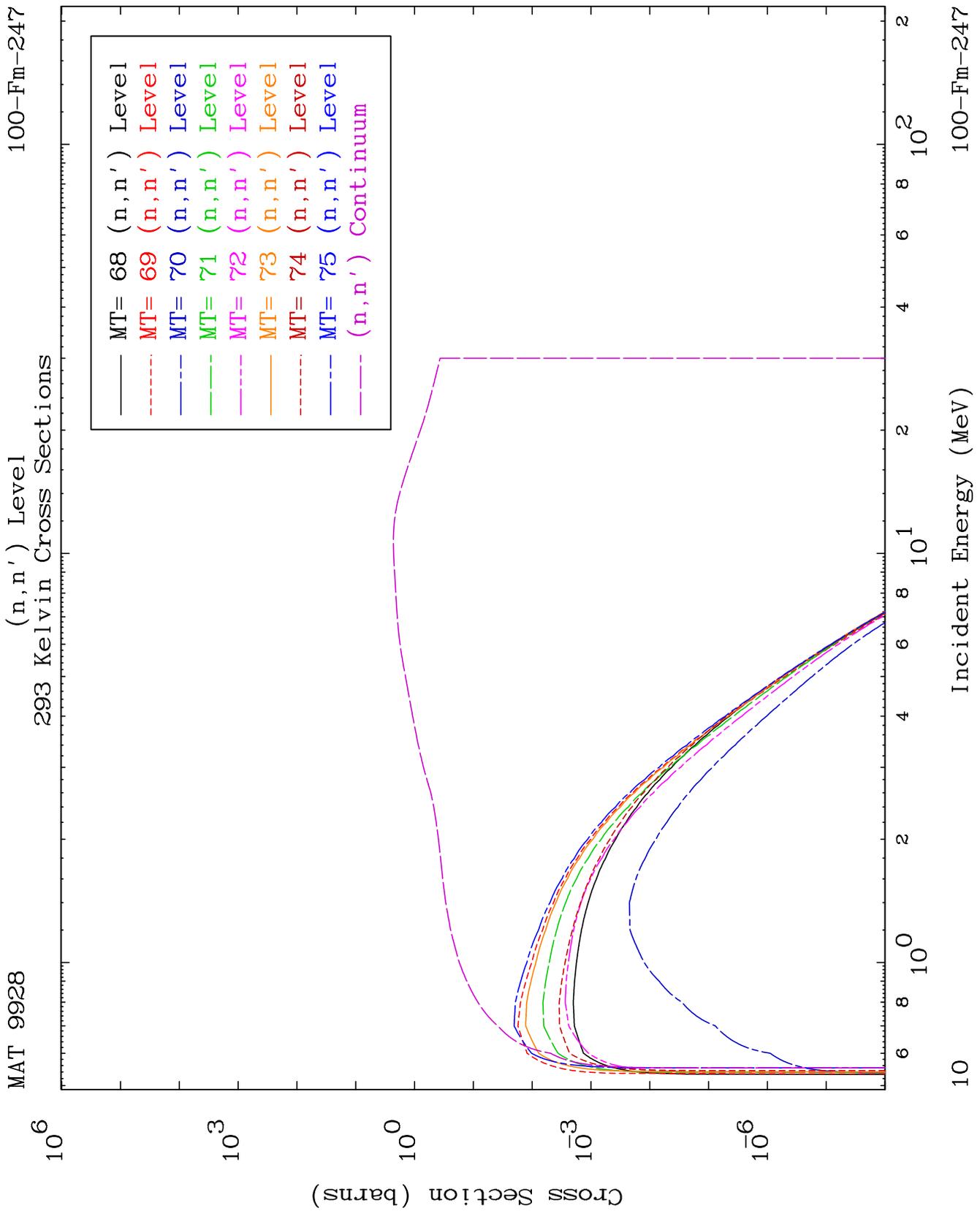


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Incident Energy (MeV)

100-Fm-247

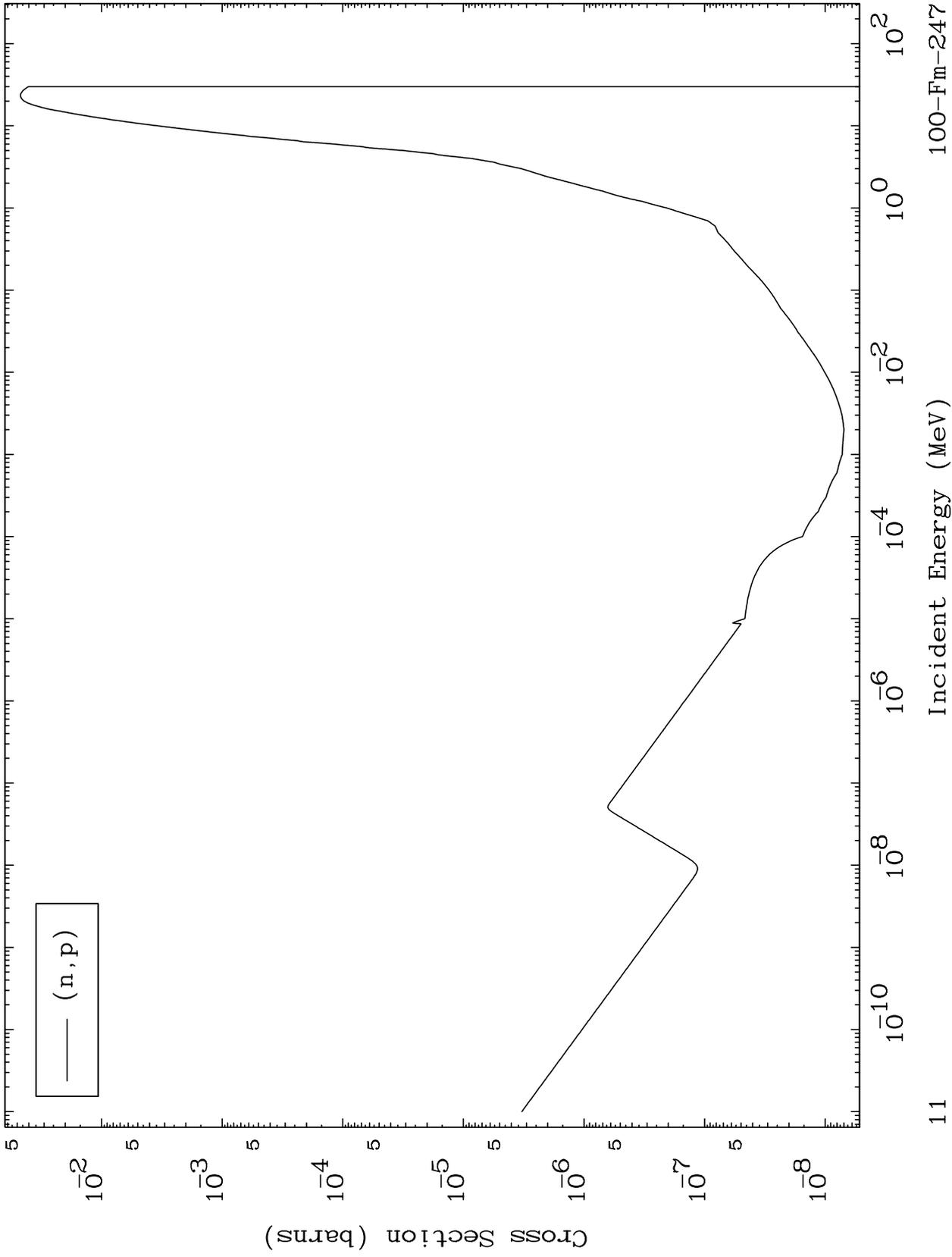




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(n,p) Levels  
293 Kelvin Cross Sections

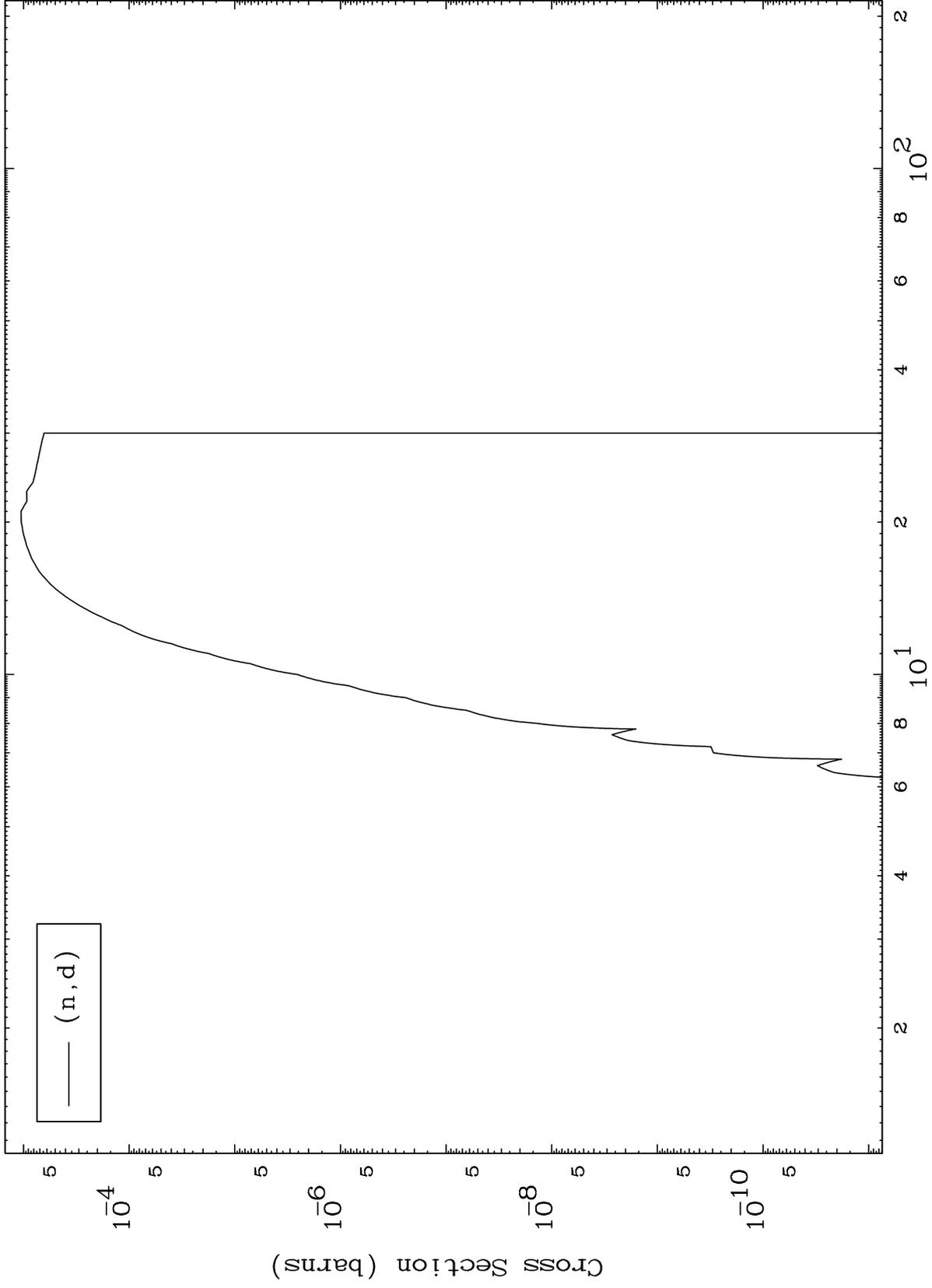
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(n,d) Levels  
293 Kelvin Cross Sections

100-Fm-247



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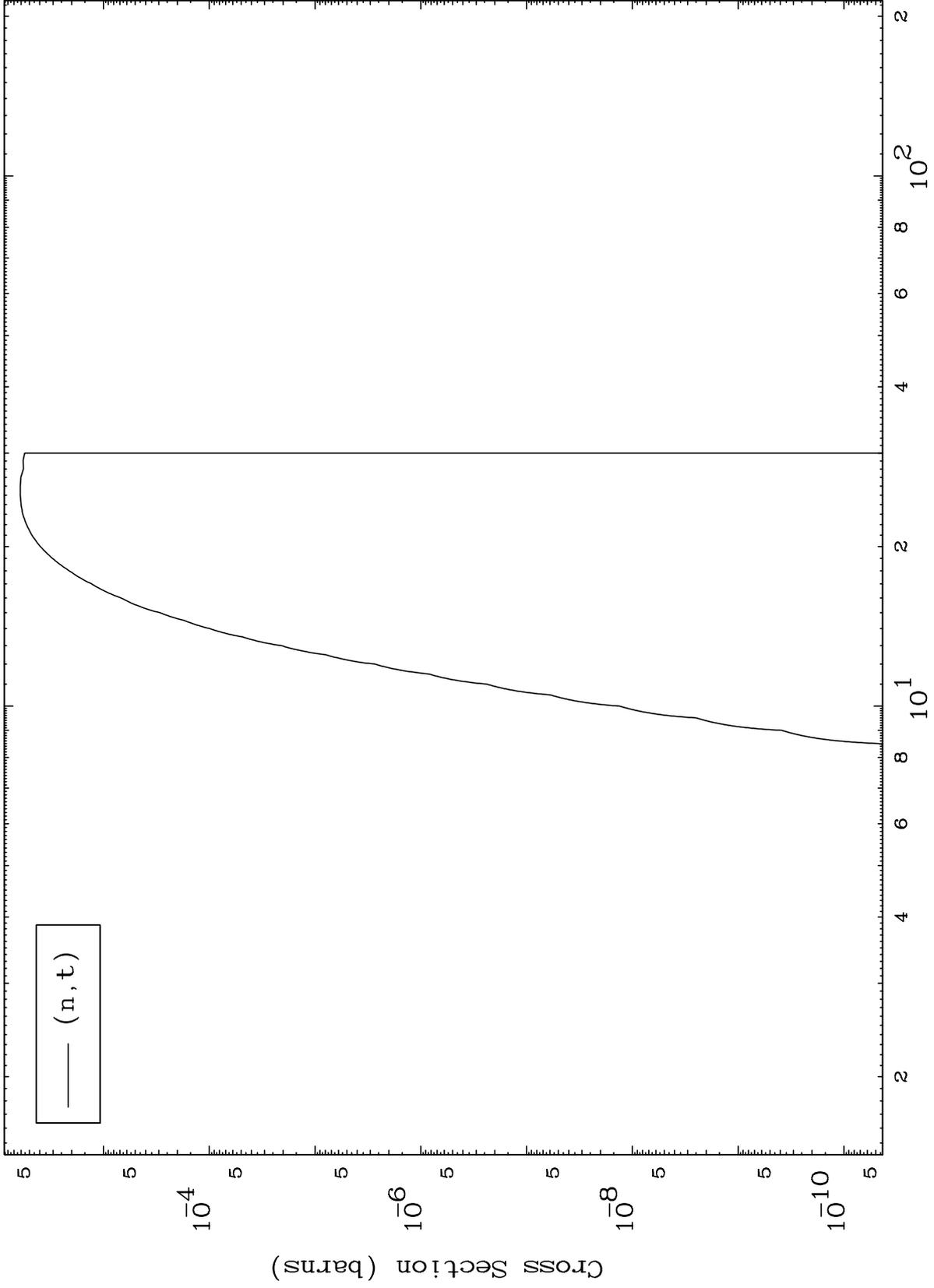
Incident Energy (MeV)

100-Fm-247

MAT 9928

(n,t) Levels  
293 Kelvin Cross Sections

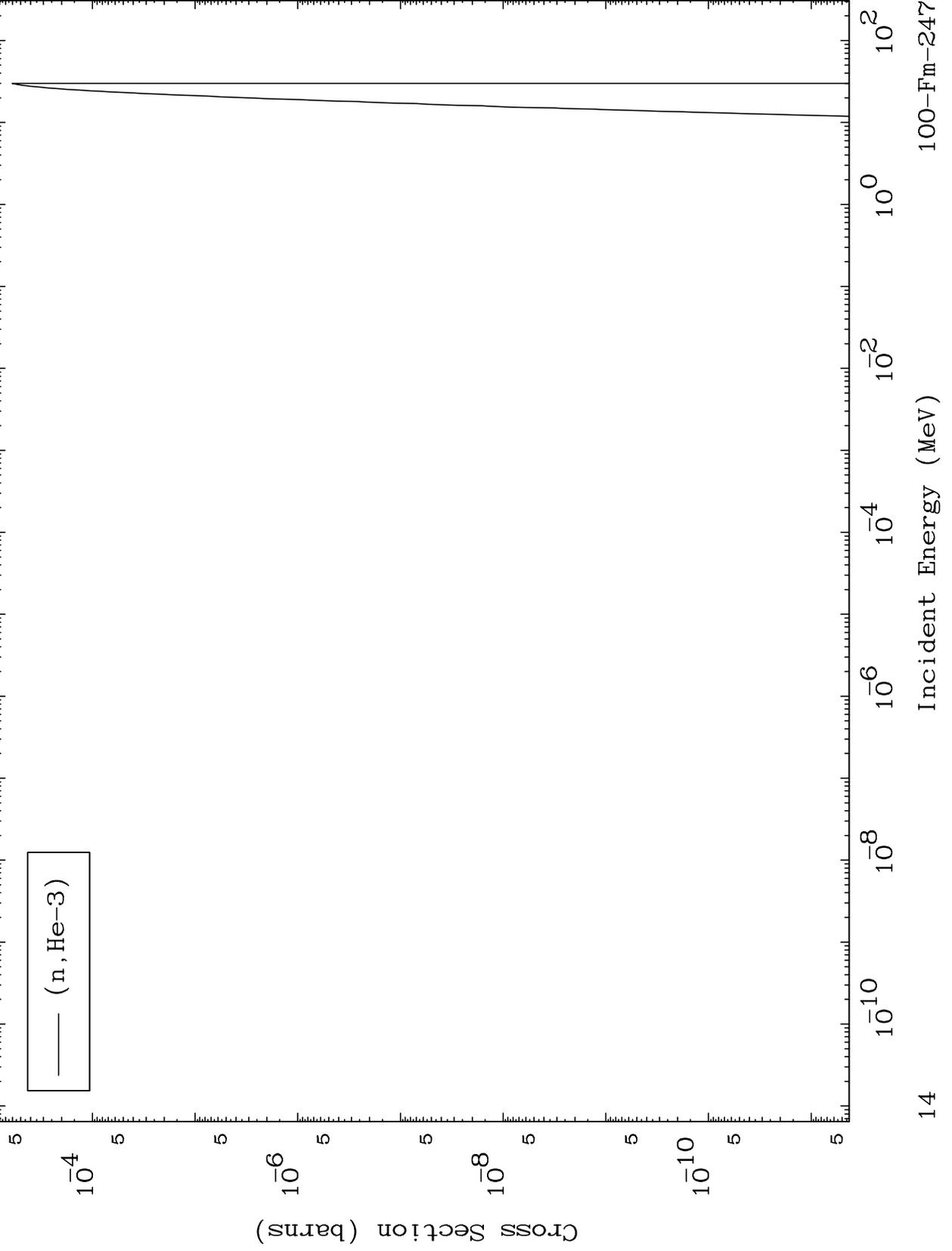
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(n,He3) Levels  
293 Kelvin Cross Sections

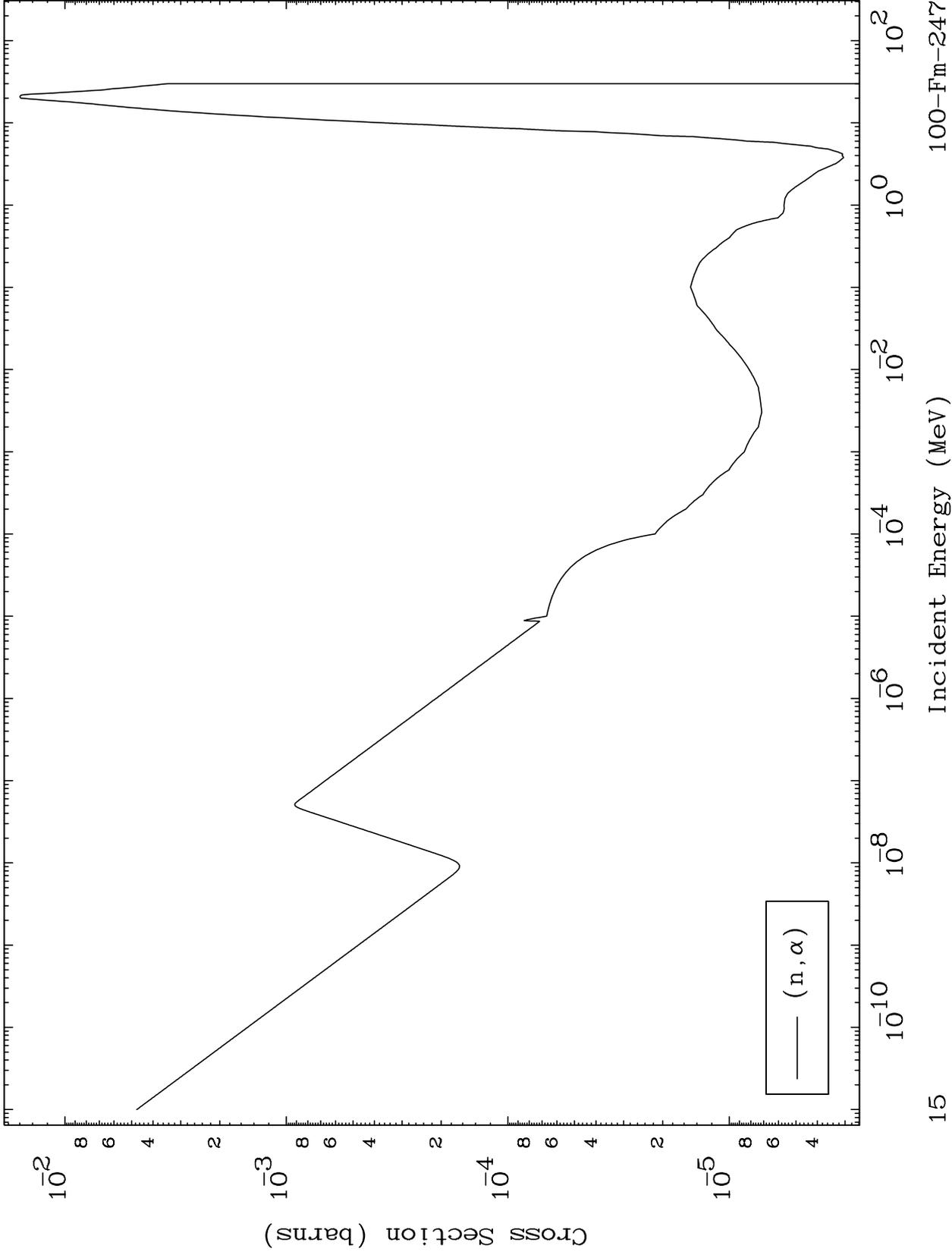
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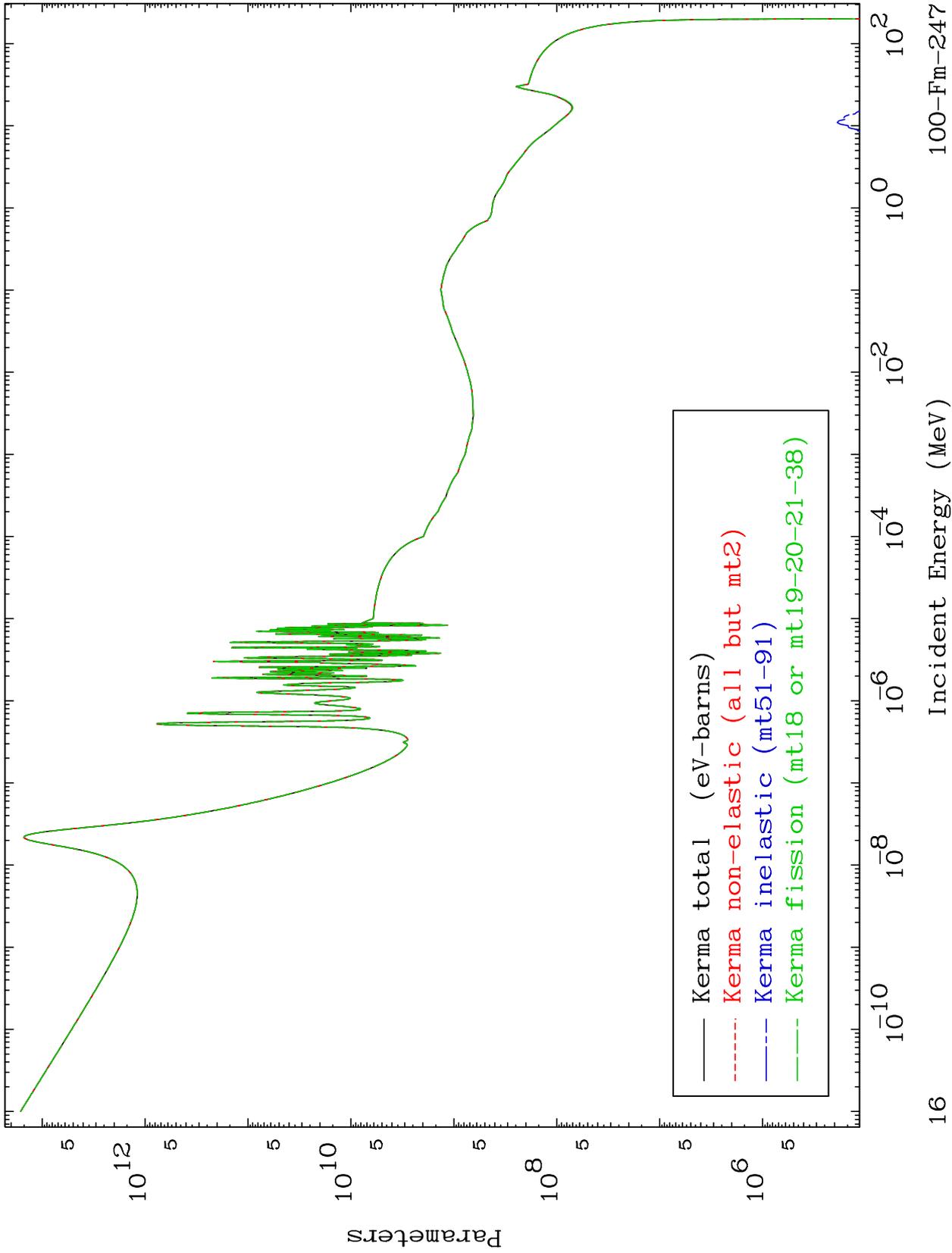


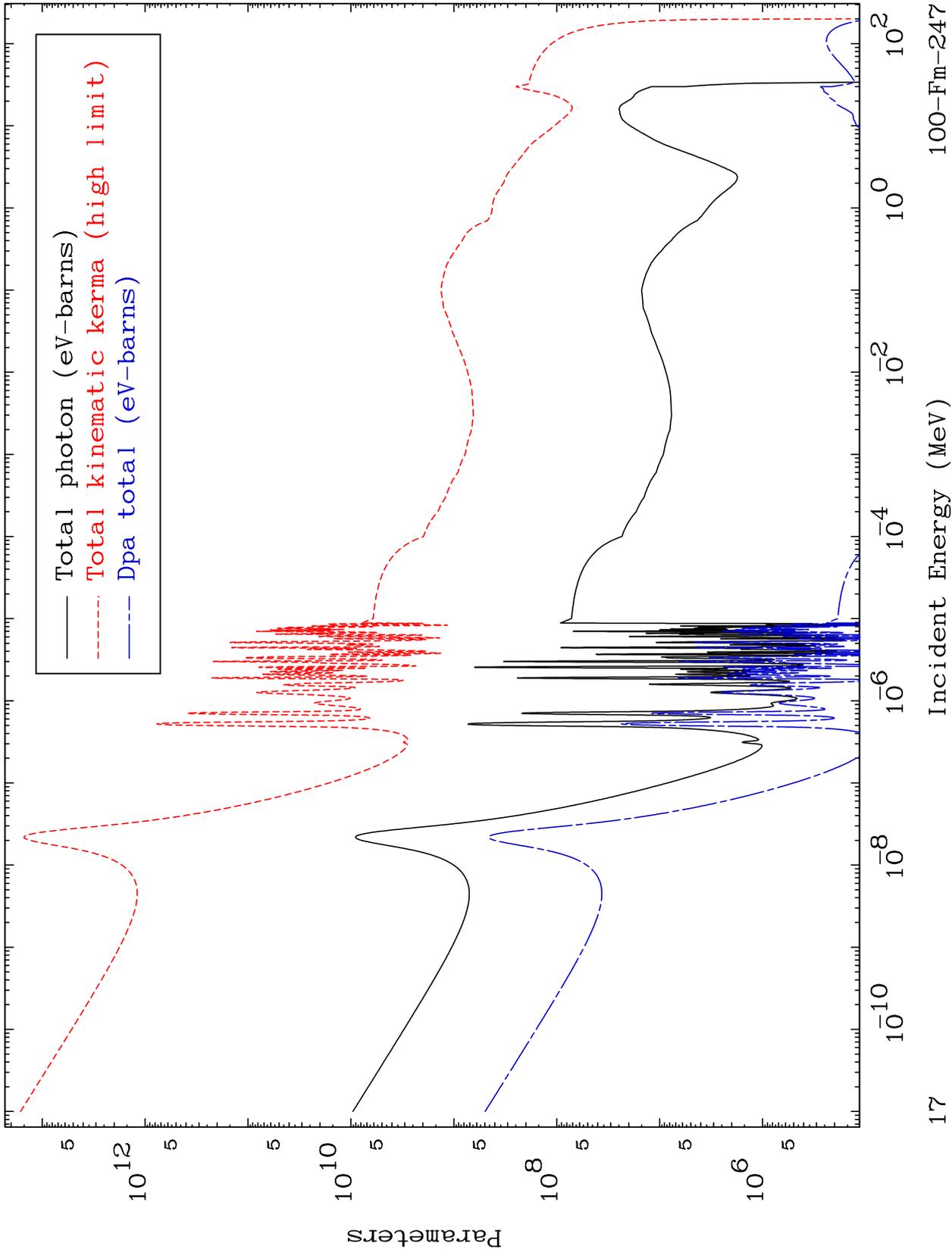
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(n,α) Levels  
293 Kelvin Cross Sections

100-Fm-247





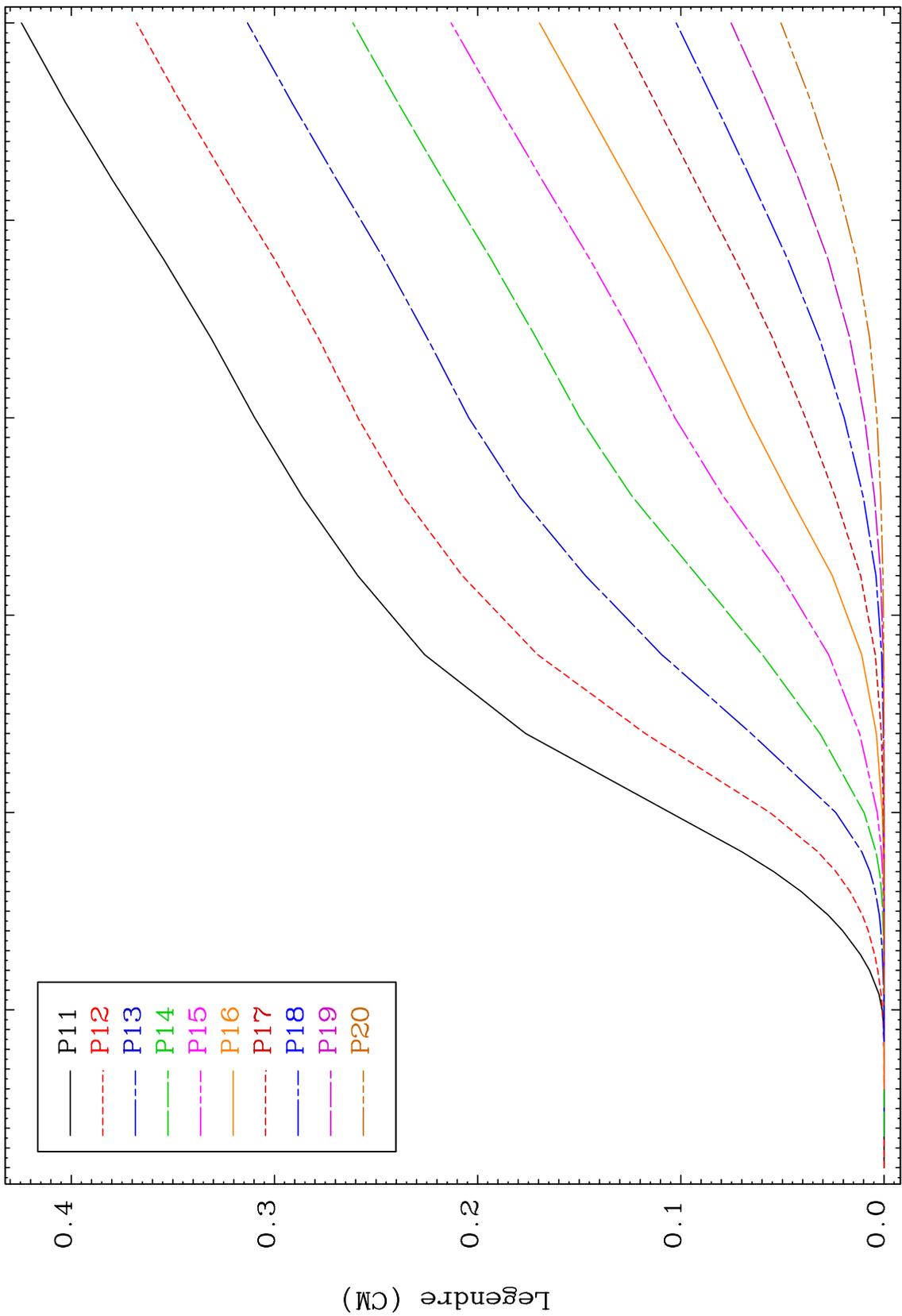




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Elastic Legendre Coefficients

100-Fm-247



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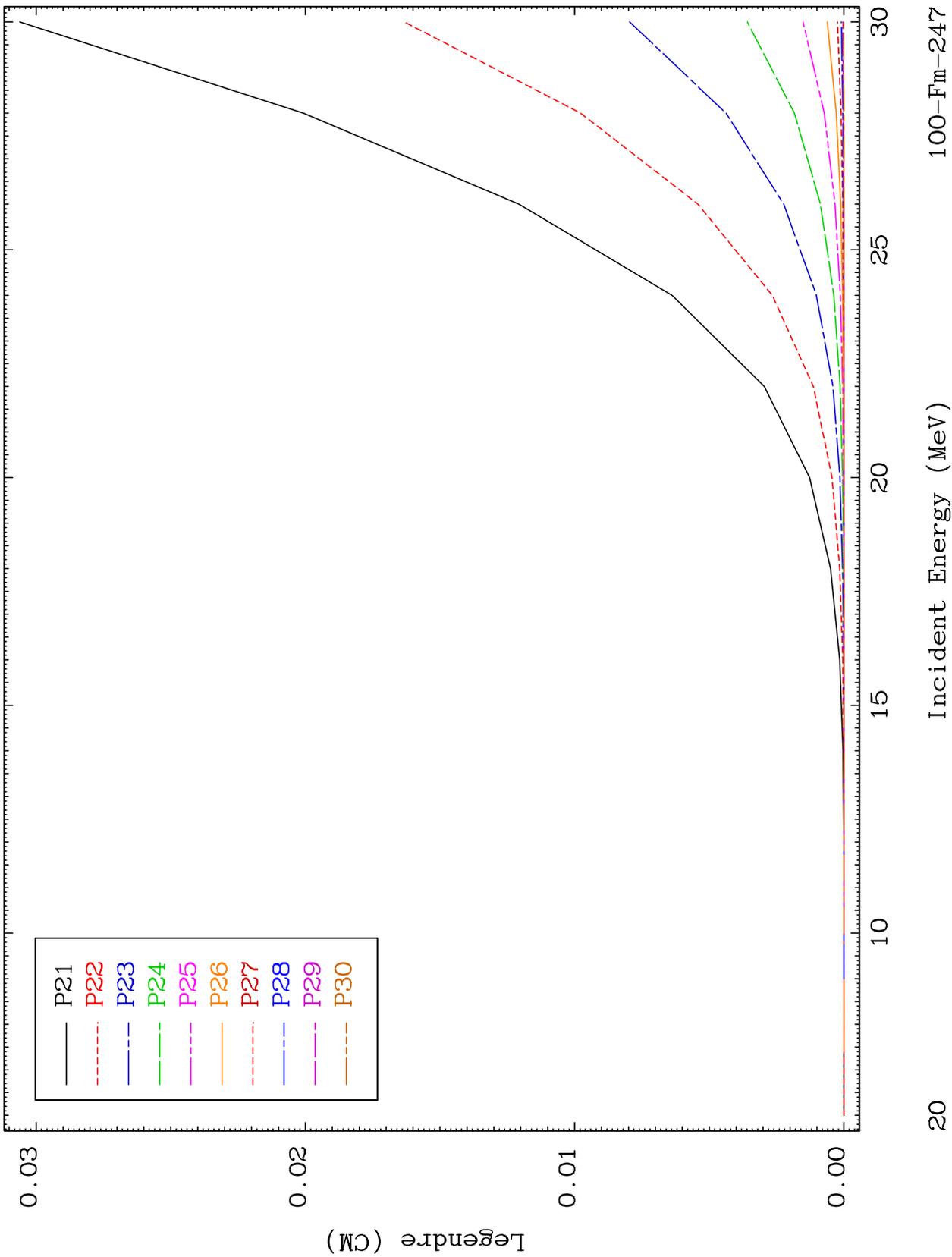
Incident Energy (MeV)

100-Fm-247

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Elastic  
Legendre Coefficients

100-Fm-247



20

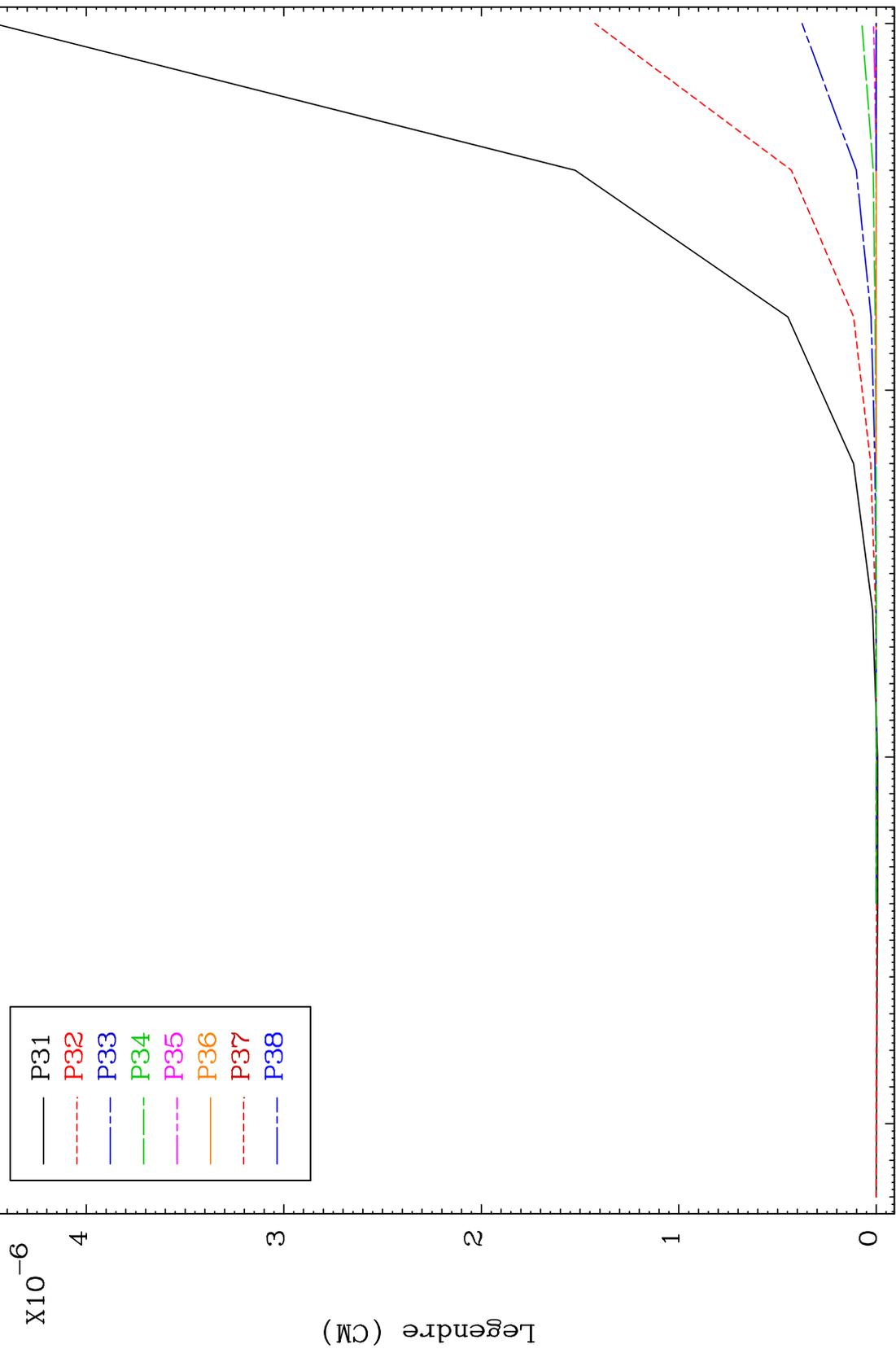
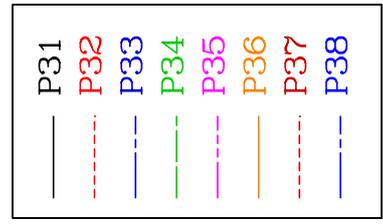
Incident Energy (MeV)

100-Fm-247

MAT 9928

Elastic Legendre Coefficients

100-Fm-247



21

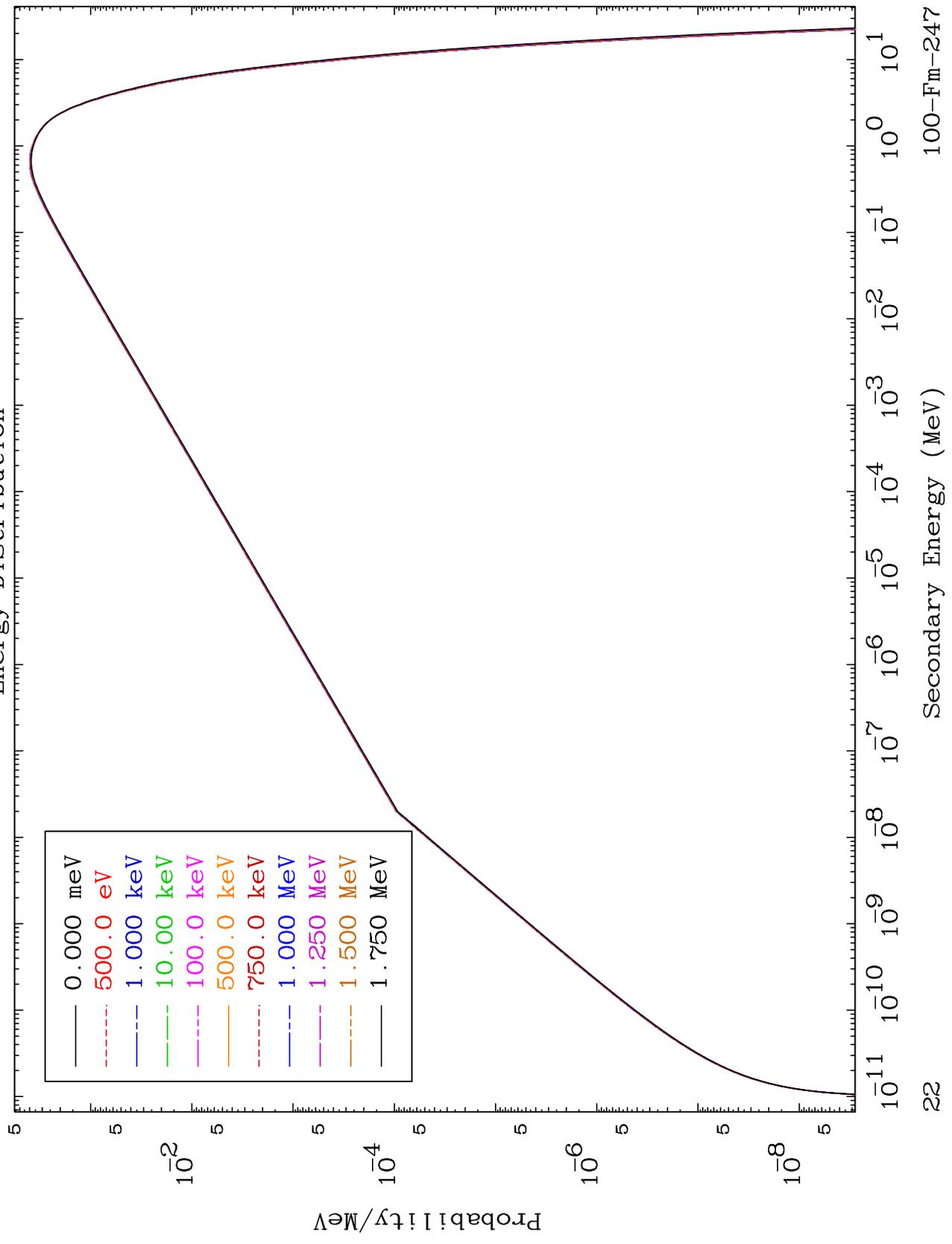
Incident Energy (MeV)

100-Fm-247

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### Fission Energy Distribution

100-Fm-247

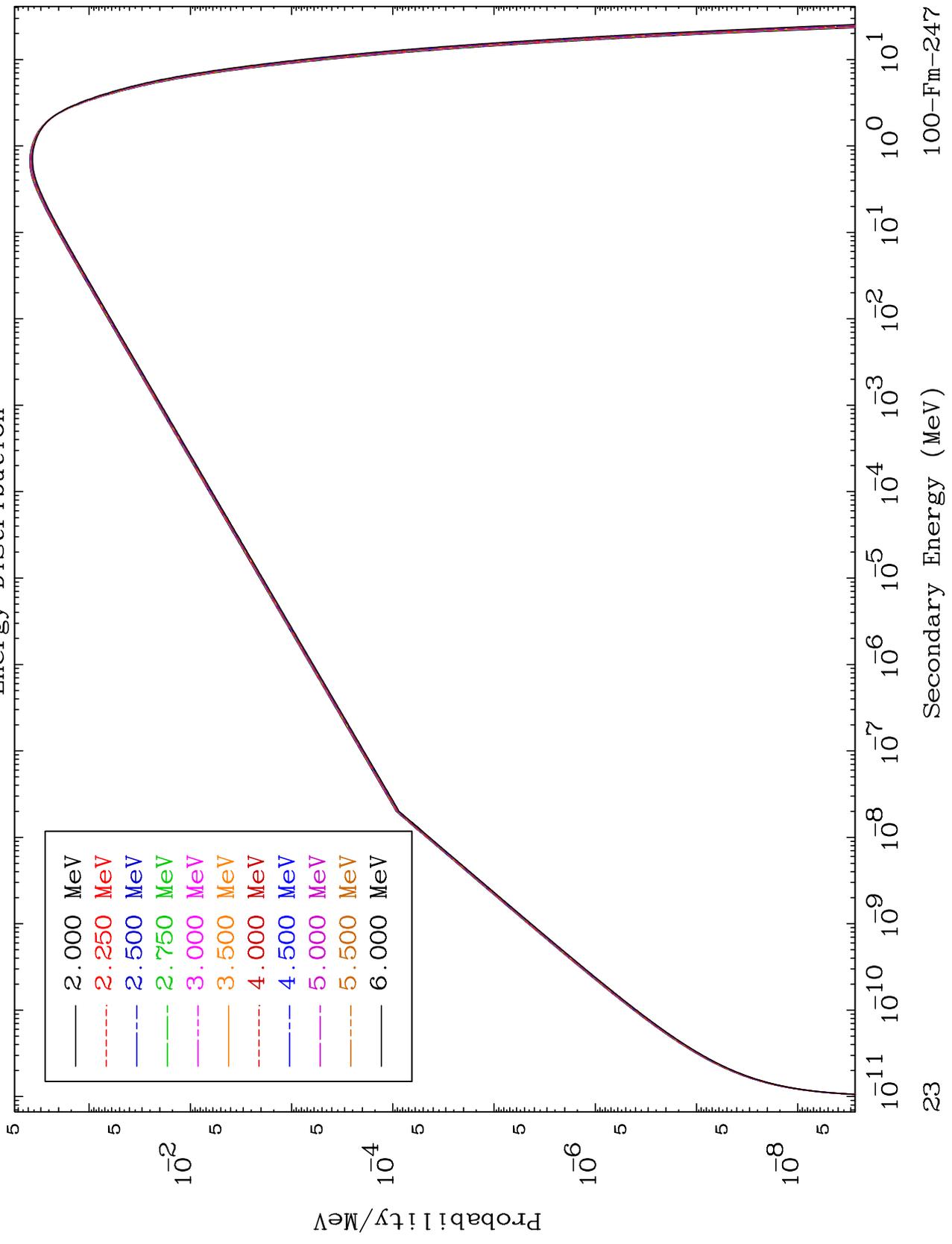


100-Fm-247

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### Fission Energy Distribution

100-Fm-247

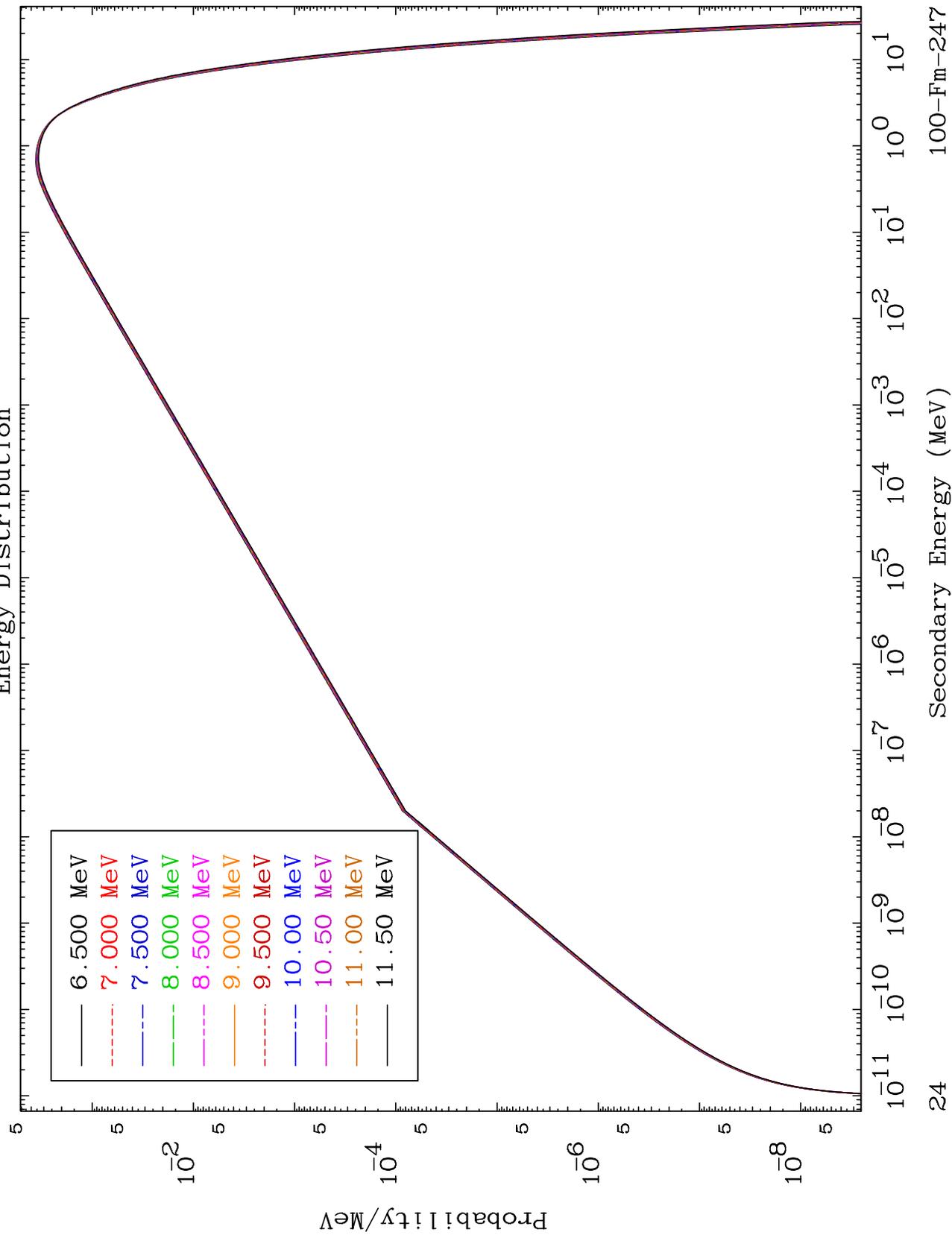


100-Fm-247

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### Fission Energy Distribution

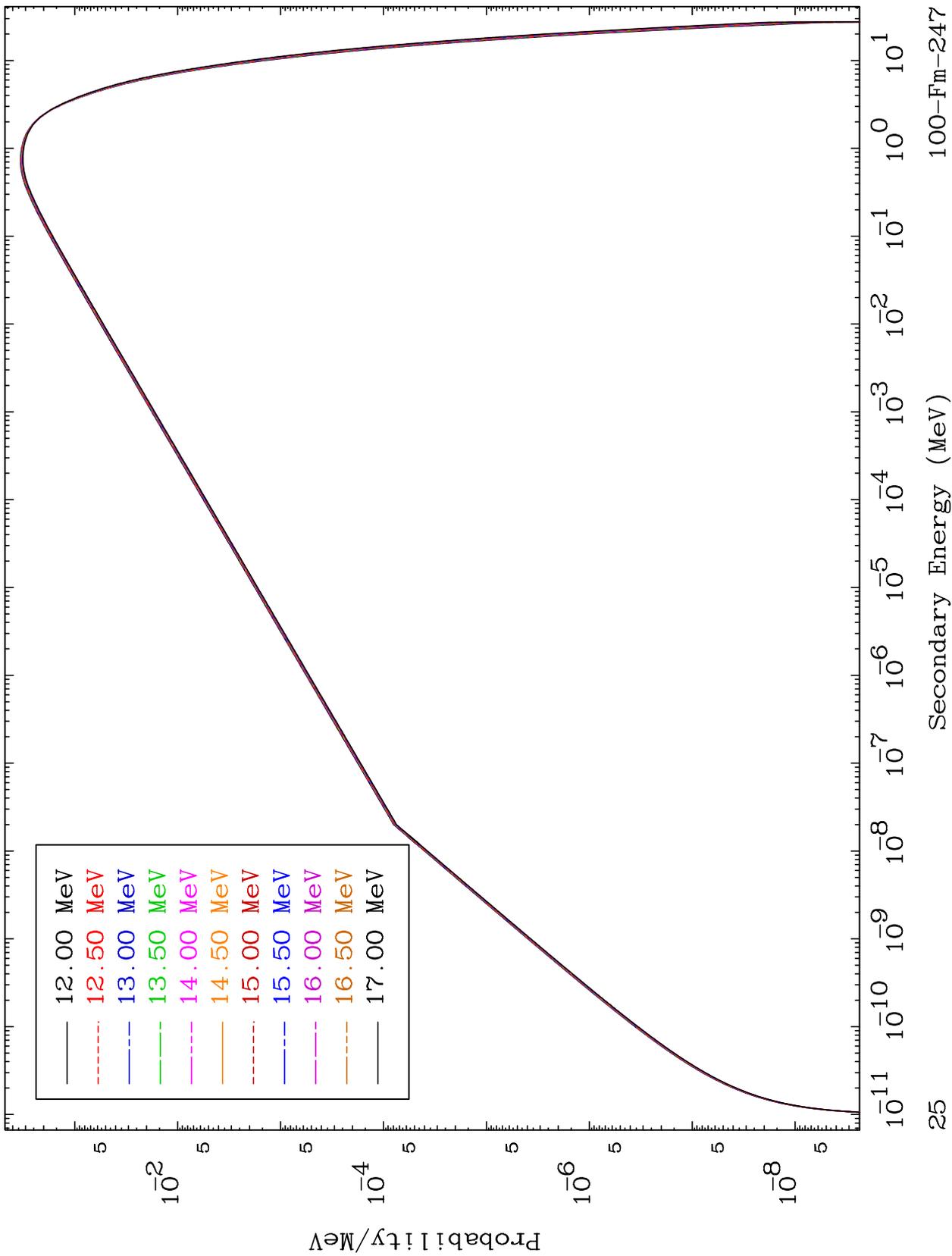
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### Fission Energy Distribution

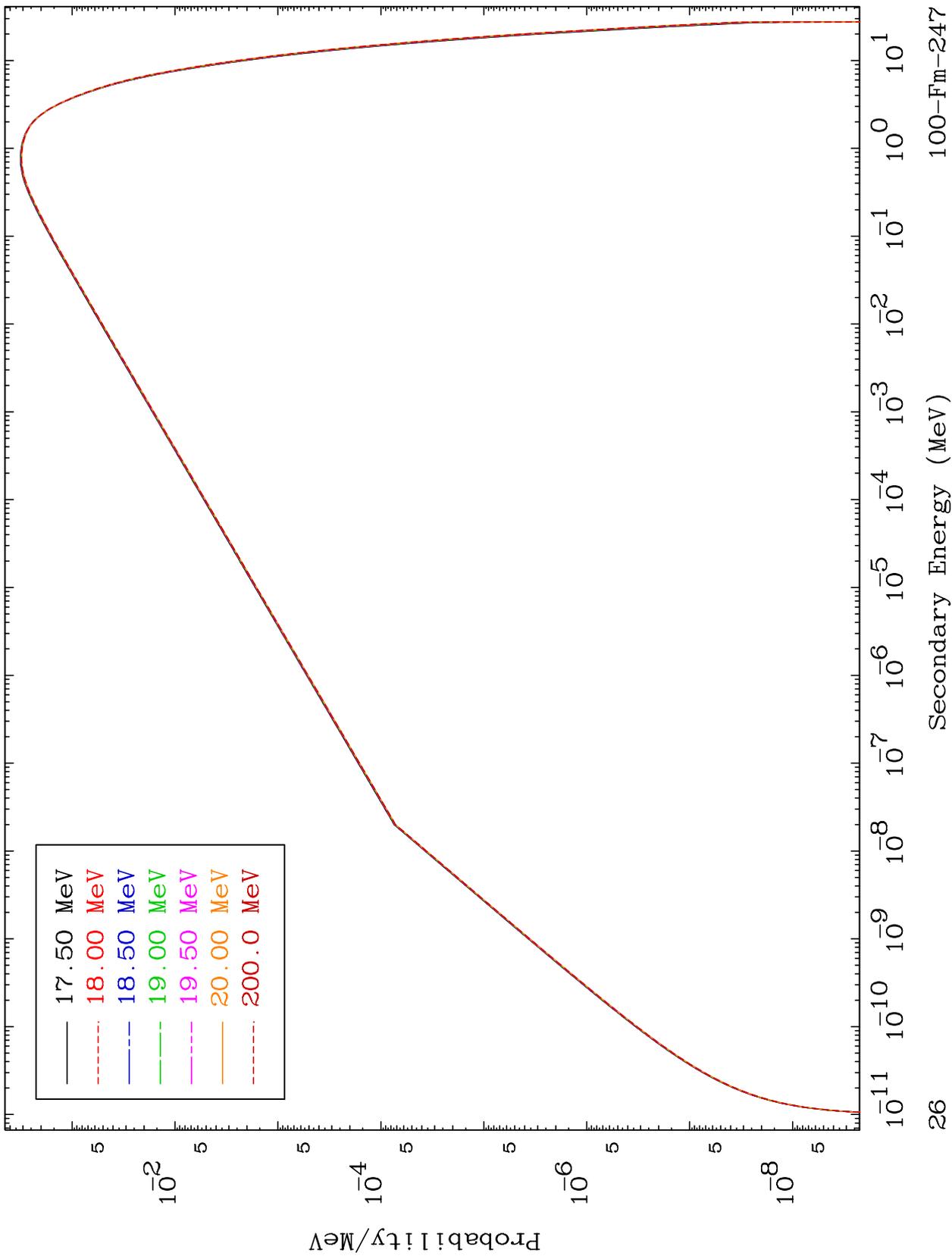
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### Fission Energy Distribution

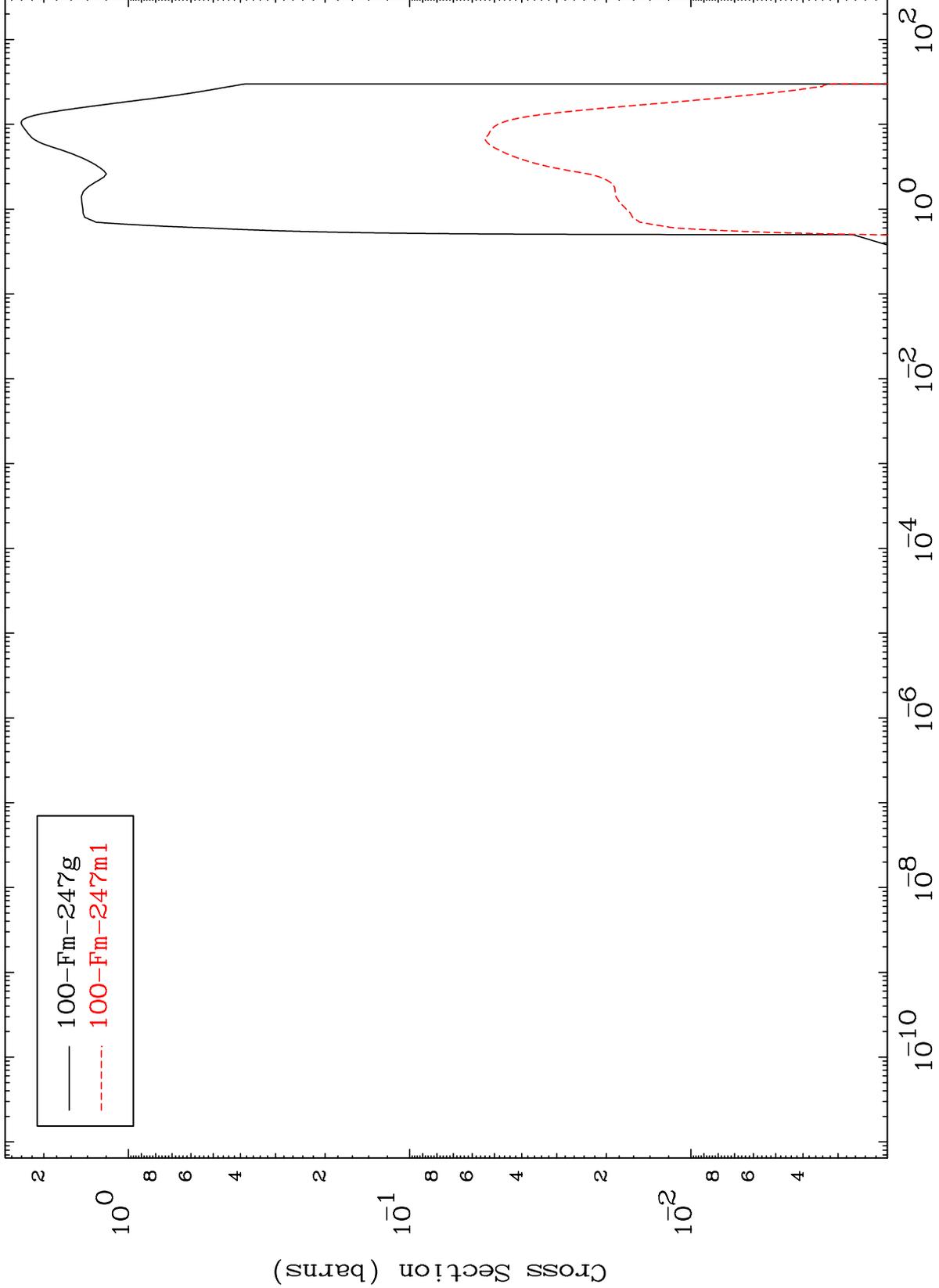
100-Fm-247



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Inelastic  
Radionuclide Production Cross Section

100-Fm-247



— 100-Fm-247g  
- - - 100-Fm-247m1