

Program EVALPLOT
(Version 2018-1)

by

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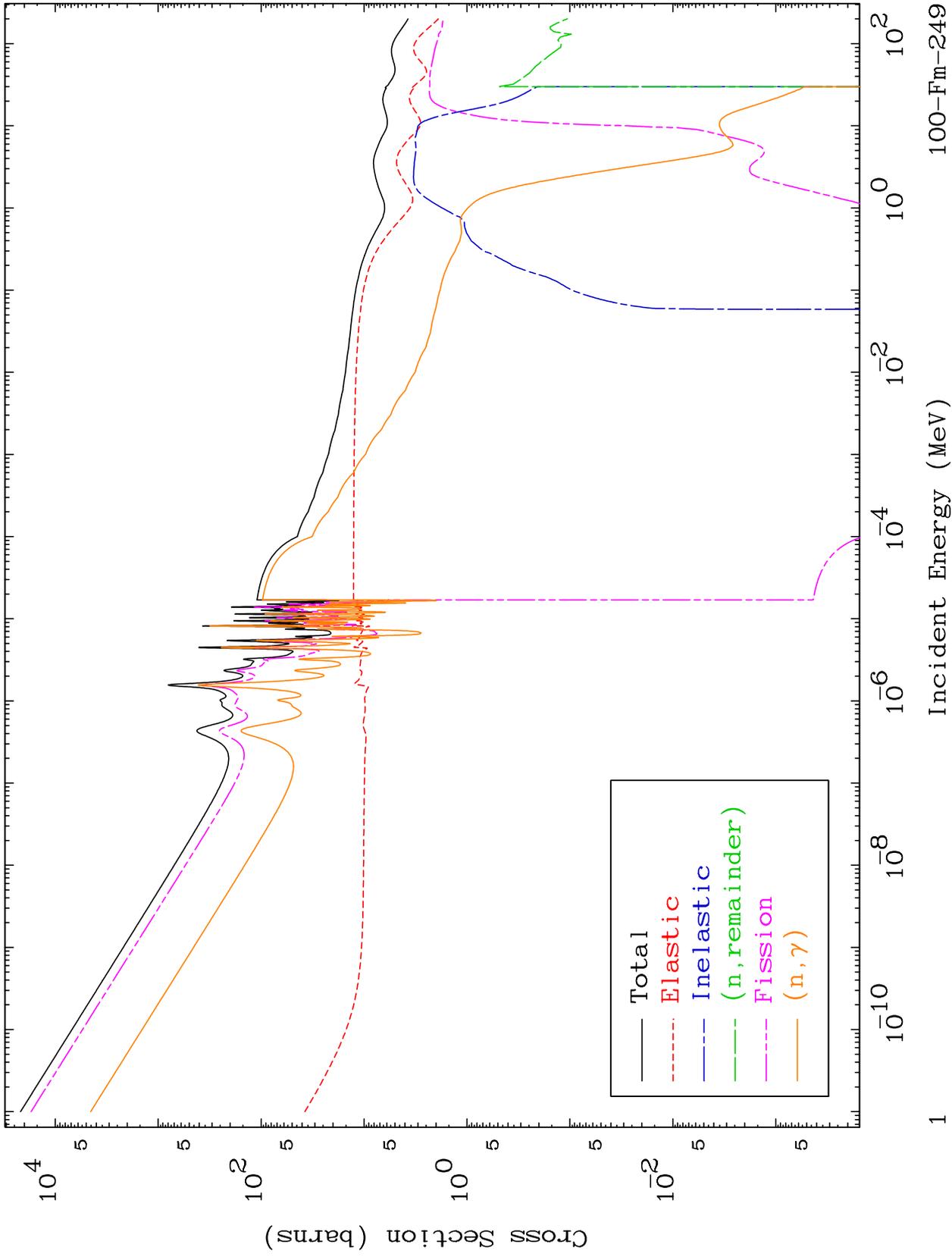
Press Mouse Button to Start

MAT 9930

Major

293 Kelvin Cross Sections

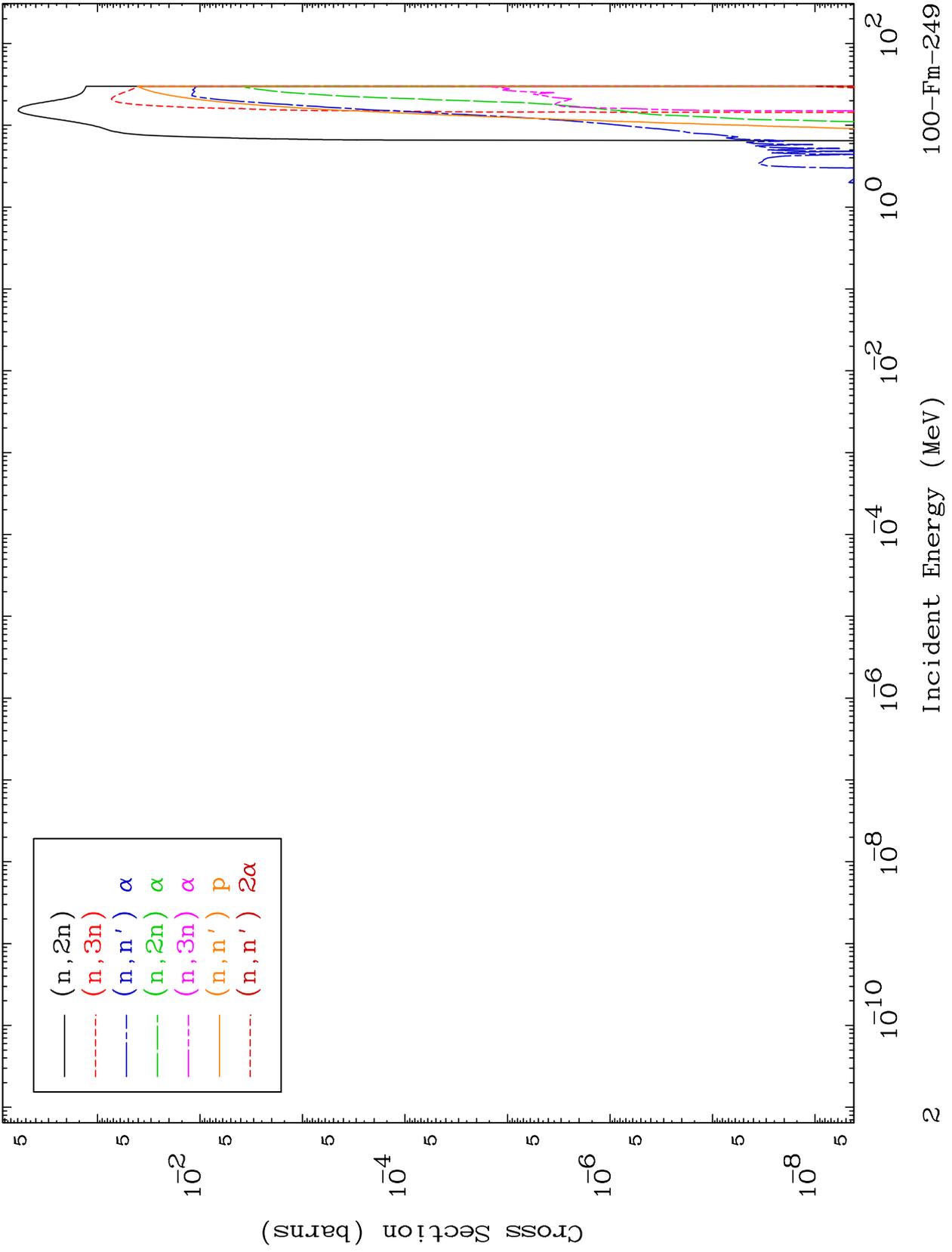
100-Fm-249



MAT 9930

Neutron Production
293 Kelvin Cross Sections

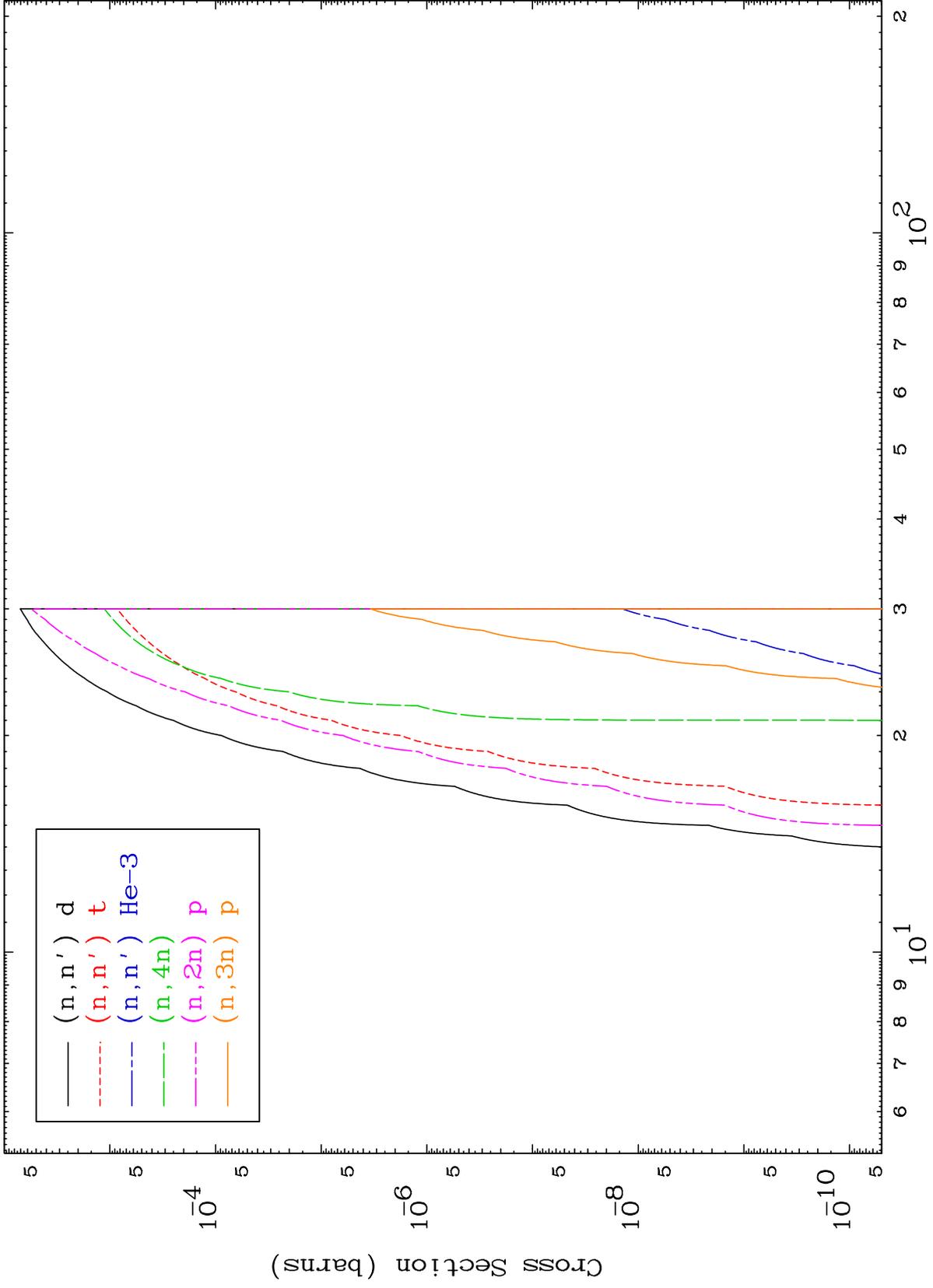
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Neutron Production
293 Kelvin Cross Sections

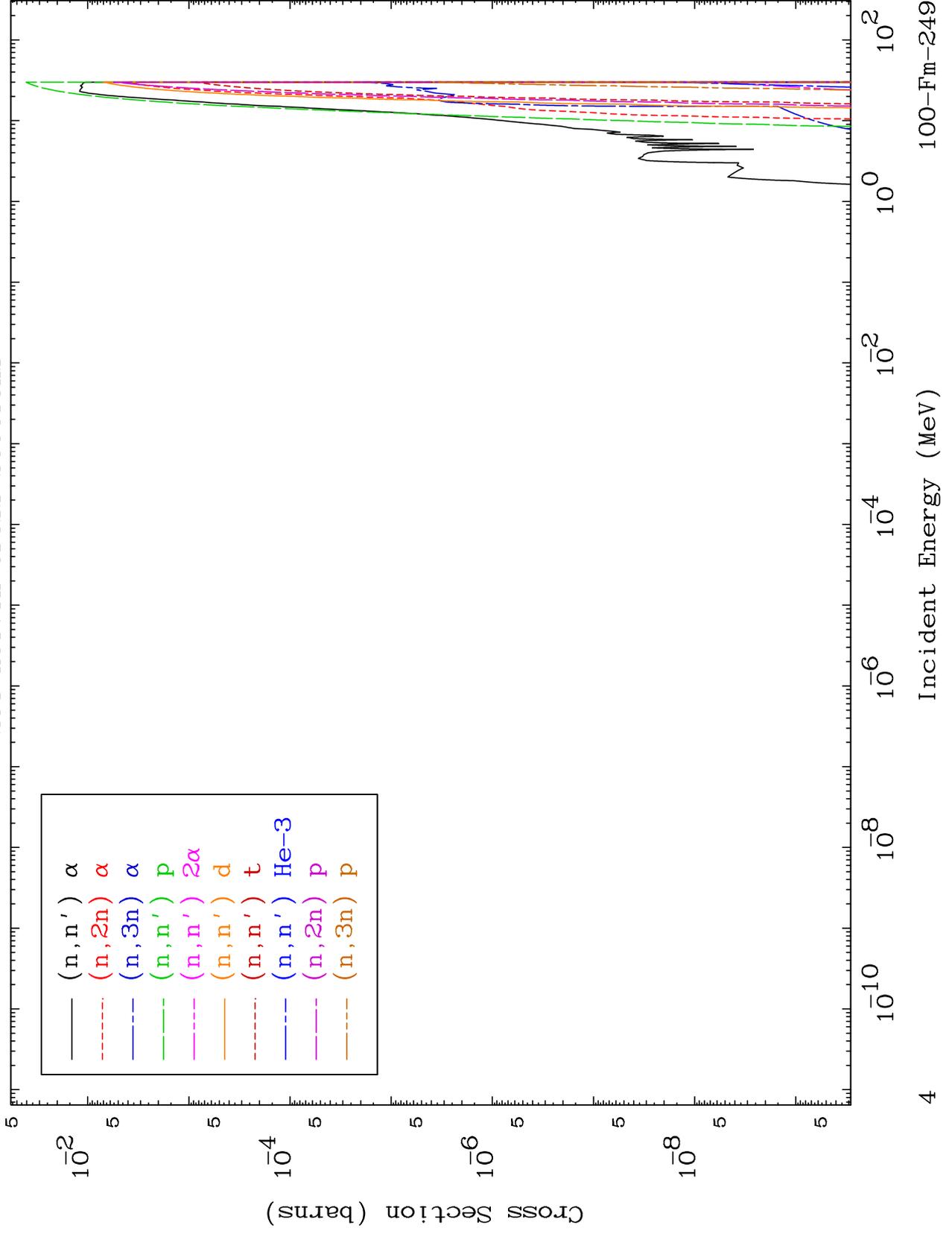
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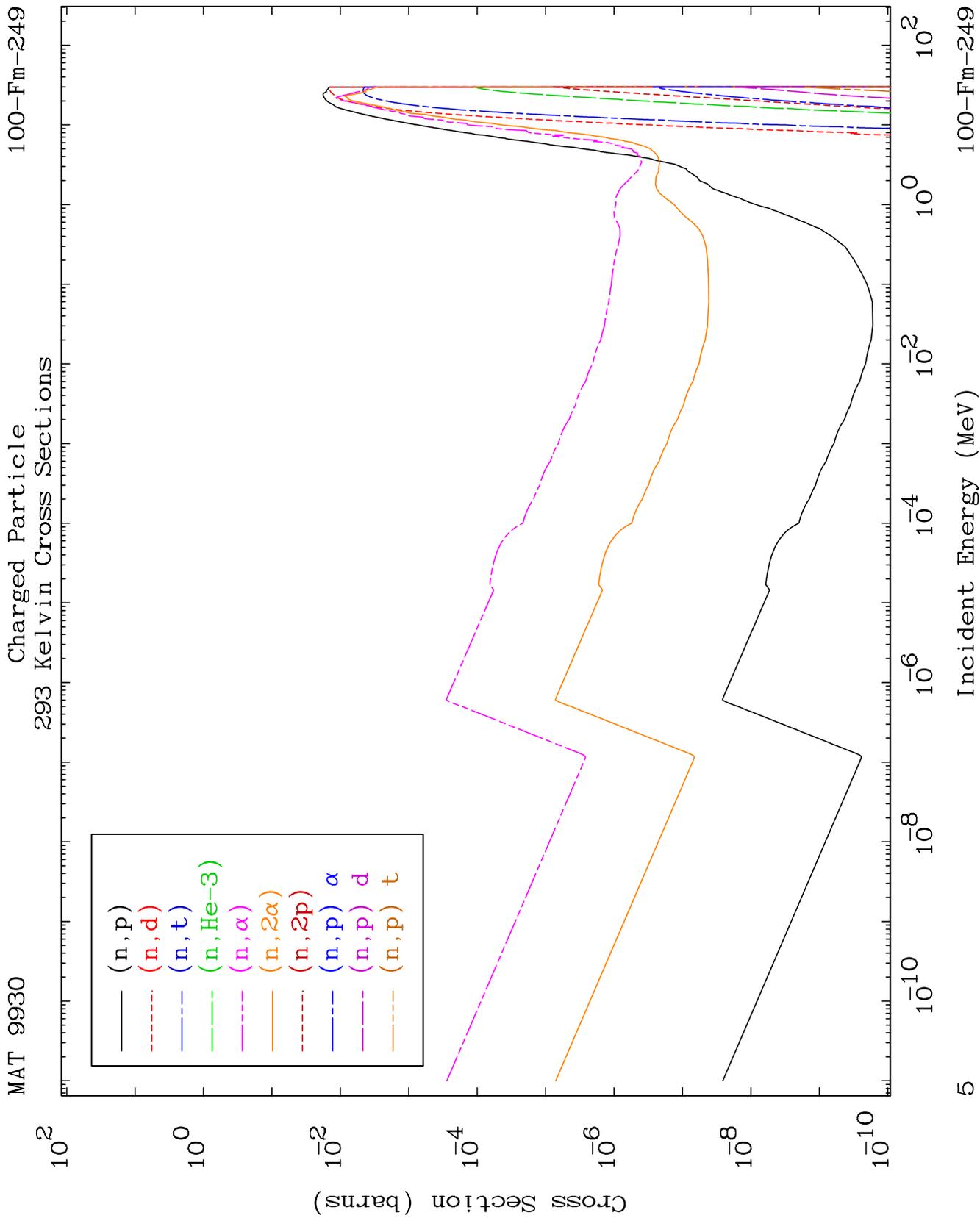


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Charged Particle
293 Kelvin Cross Sections

100-Fm-249

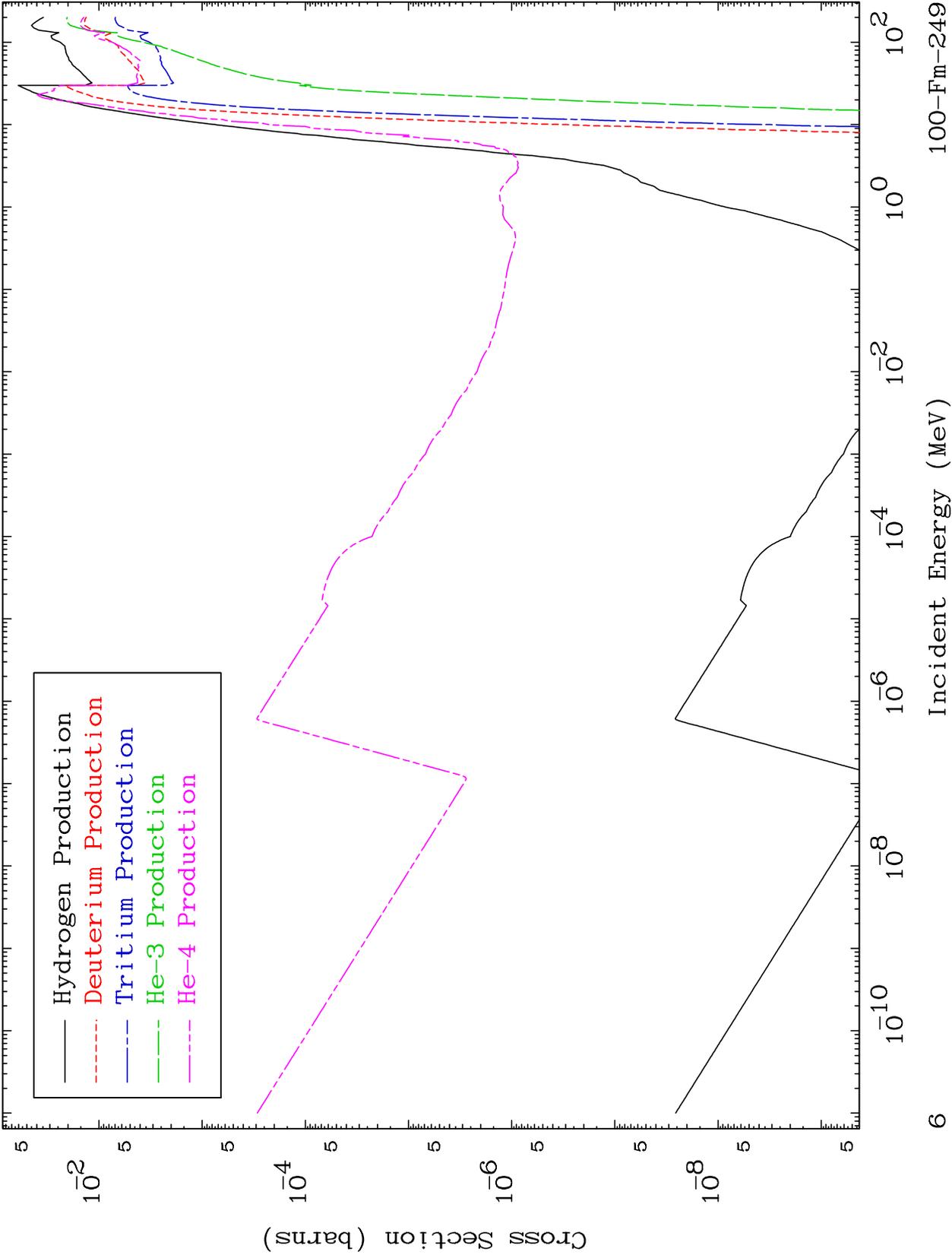




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Particle Production
293 Kelvin Cross Sections

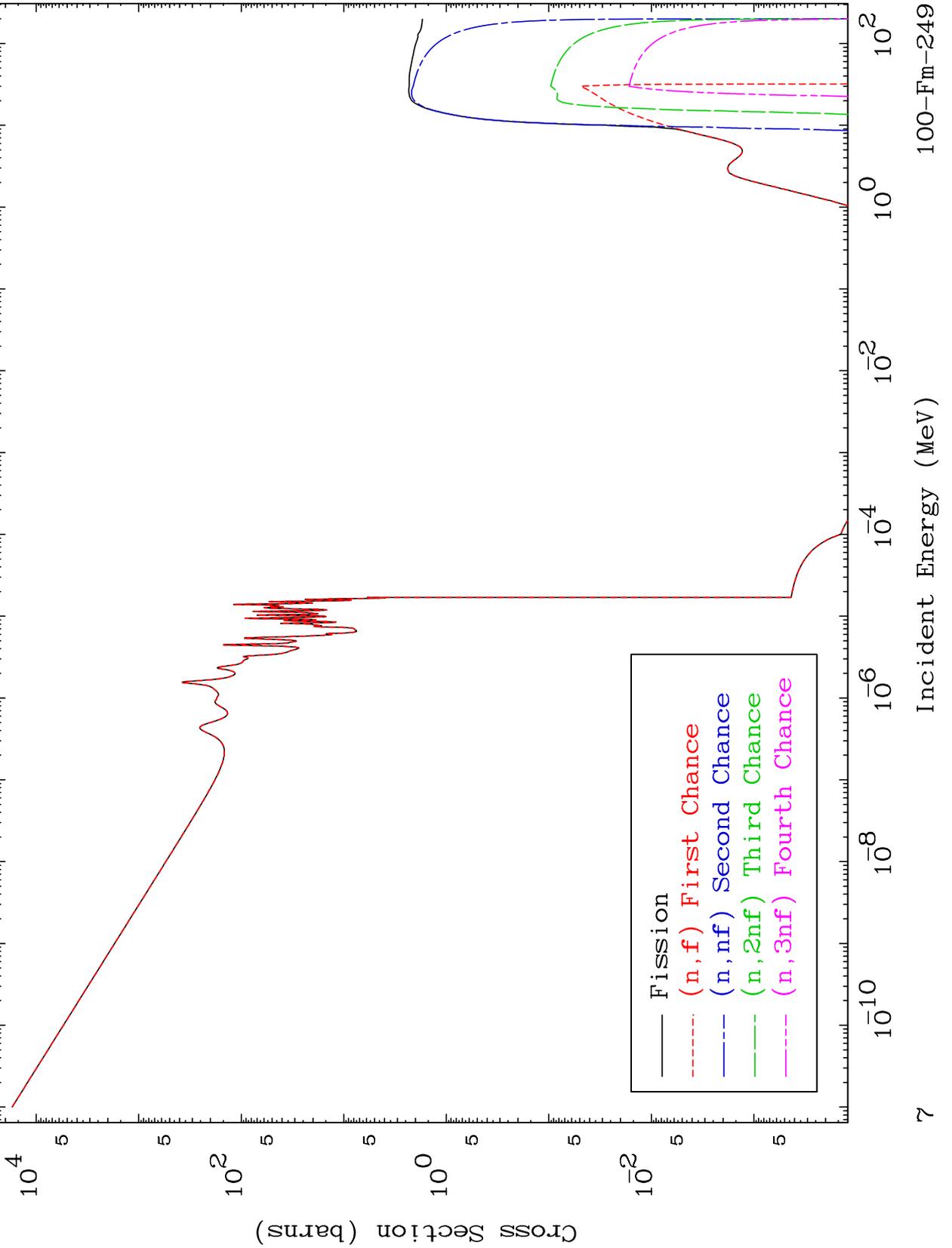
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Fission
293 Kelvin Cross Sections

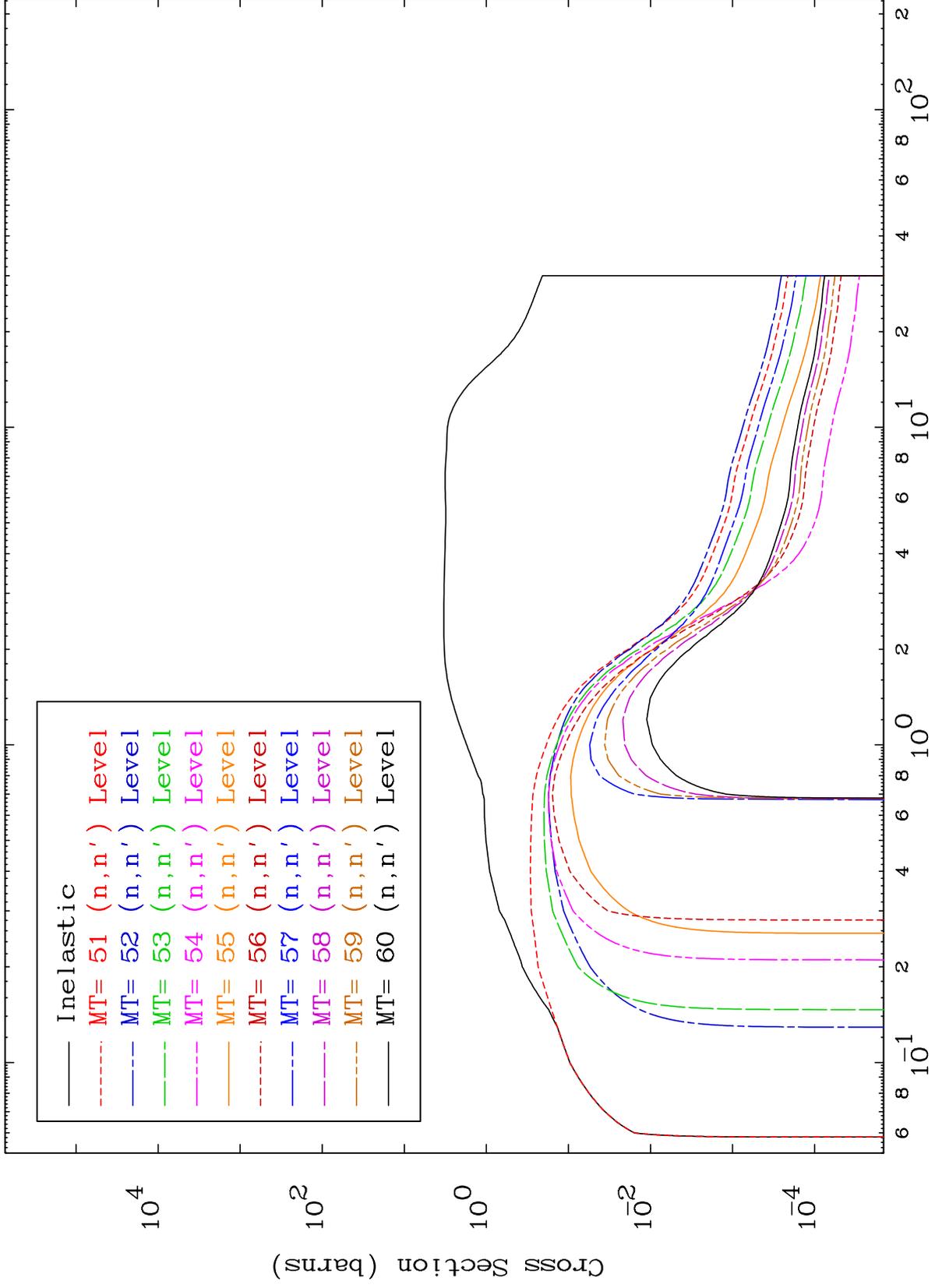
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(n,n') Level
293 Kelvin Cross Sections

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8

Incident Energy (MeV)

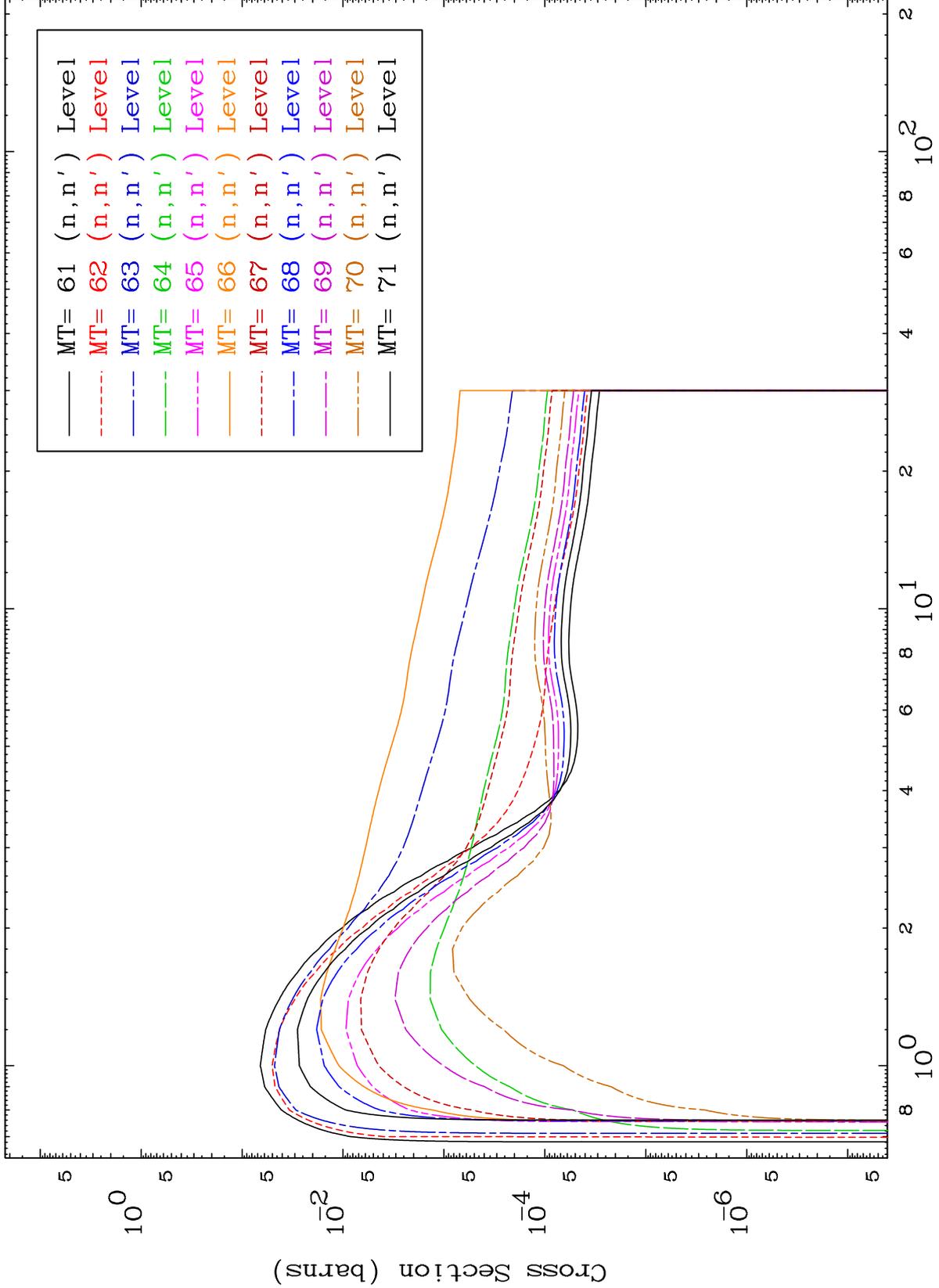
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(n,n') Level

100-Fm-249

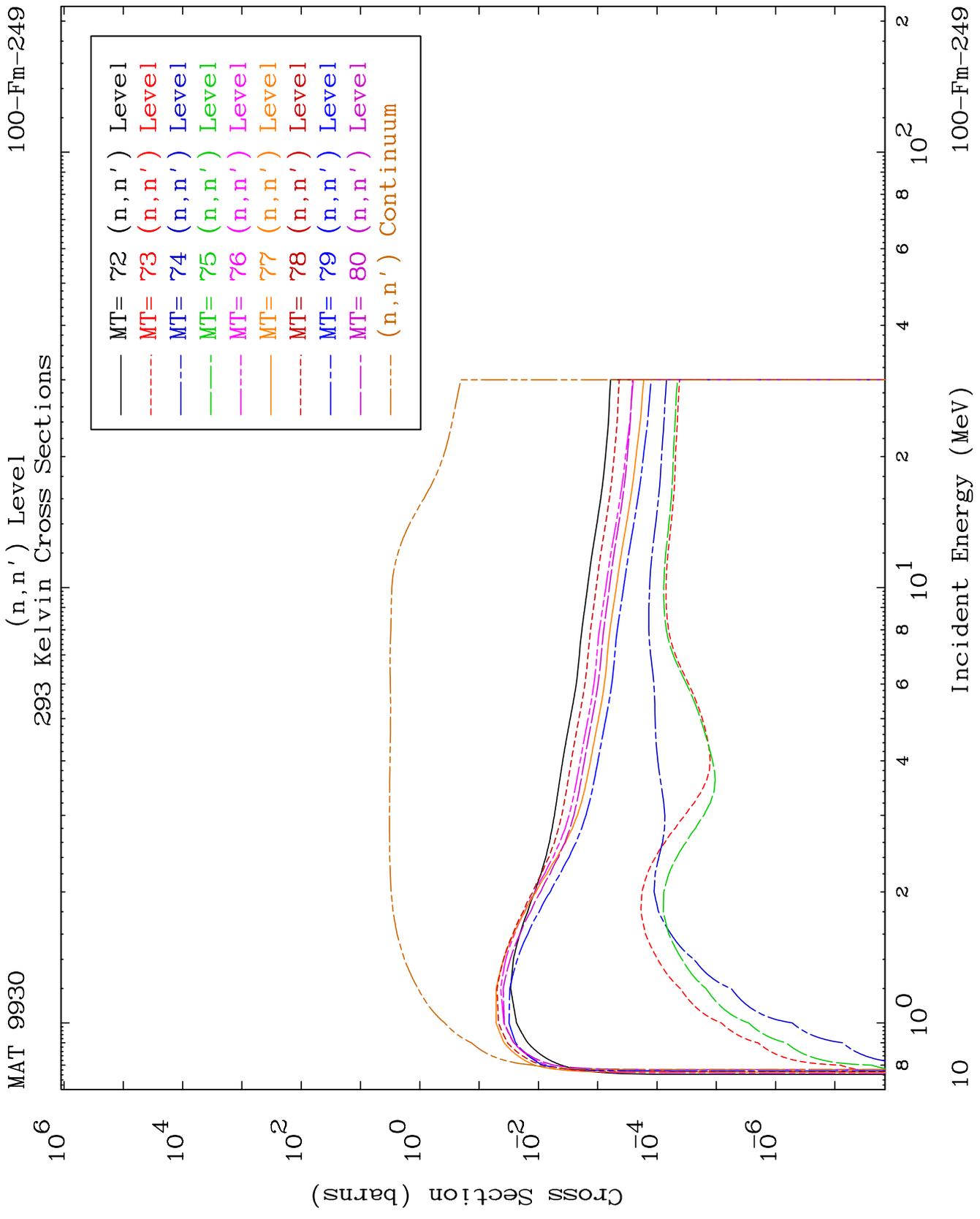
293 Kelvin Cross Sections



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Incident Energy (MeV)

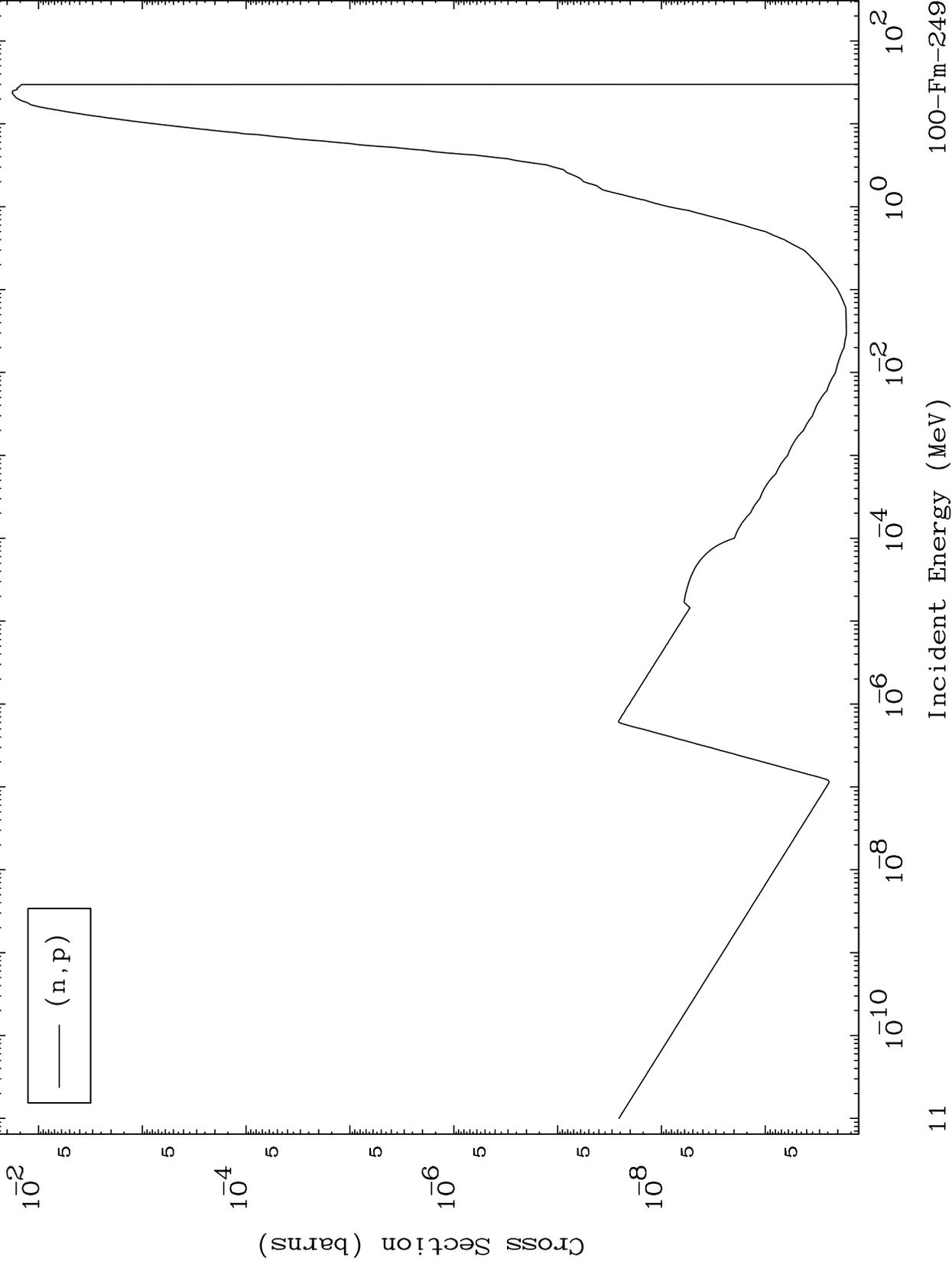
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(n,p) Levels
293 Kelvin Cross Sections

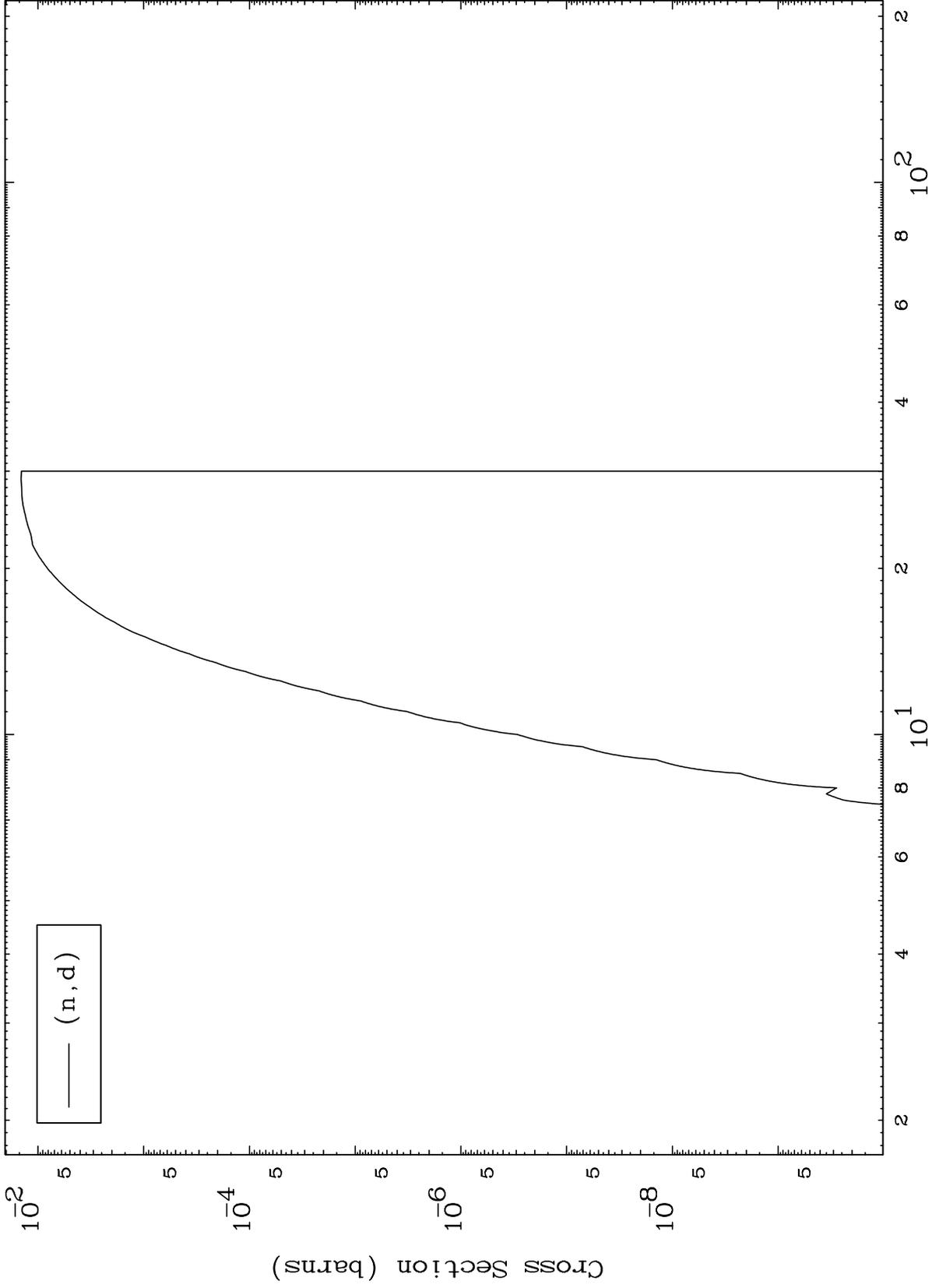
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(n,d) Levels
293 Kelvin Cross Sections

100-Fm-249



12

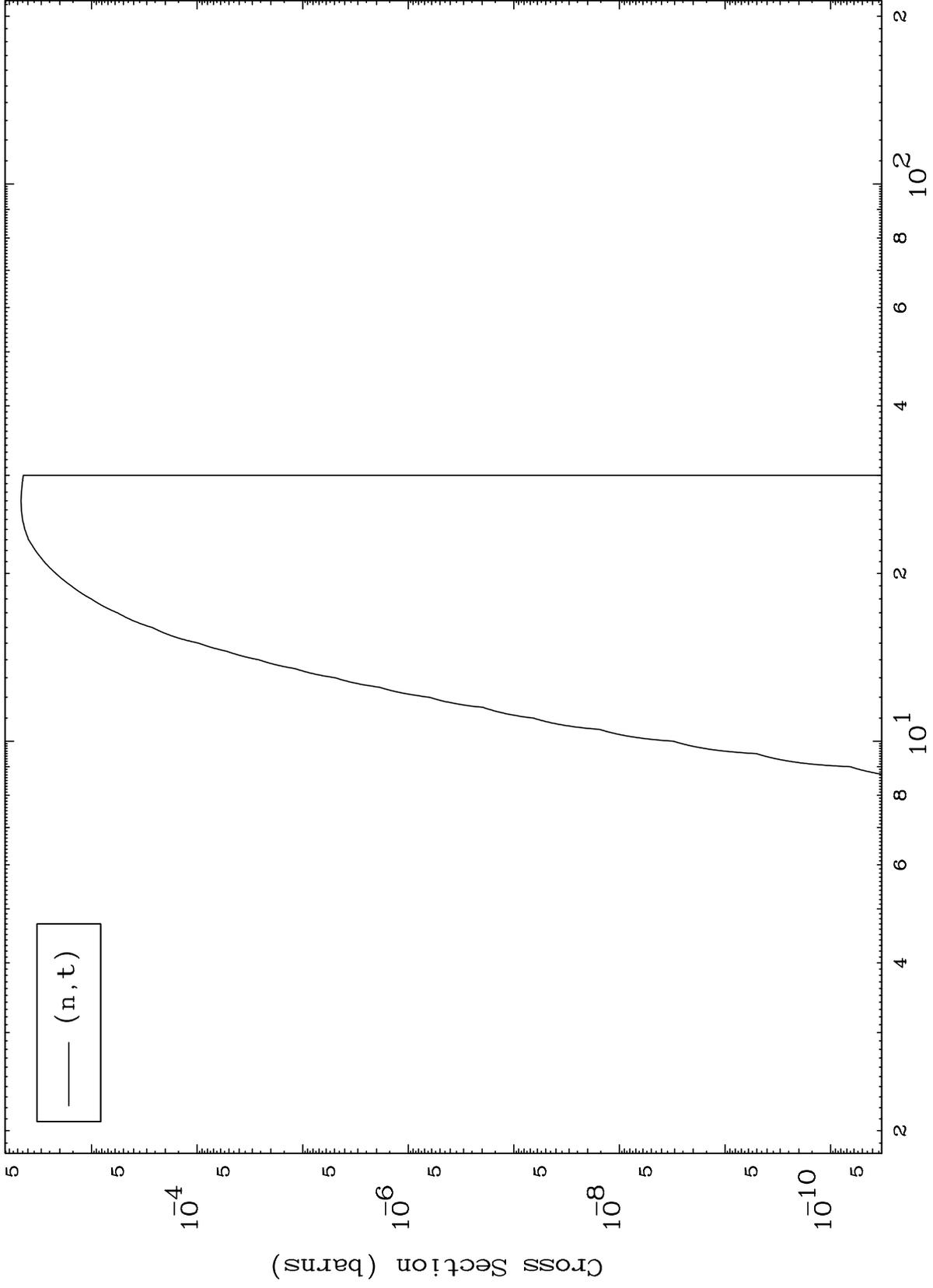
Incident Energy (MeV)

100-Fm-249

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(n,t) Levels
293 Kelvin Cross Sections

100-Fm-249



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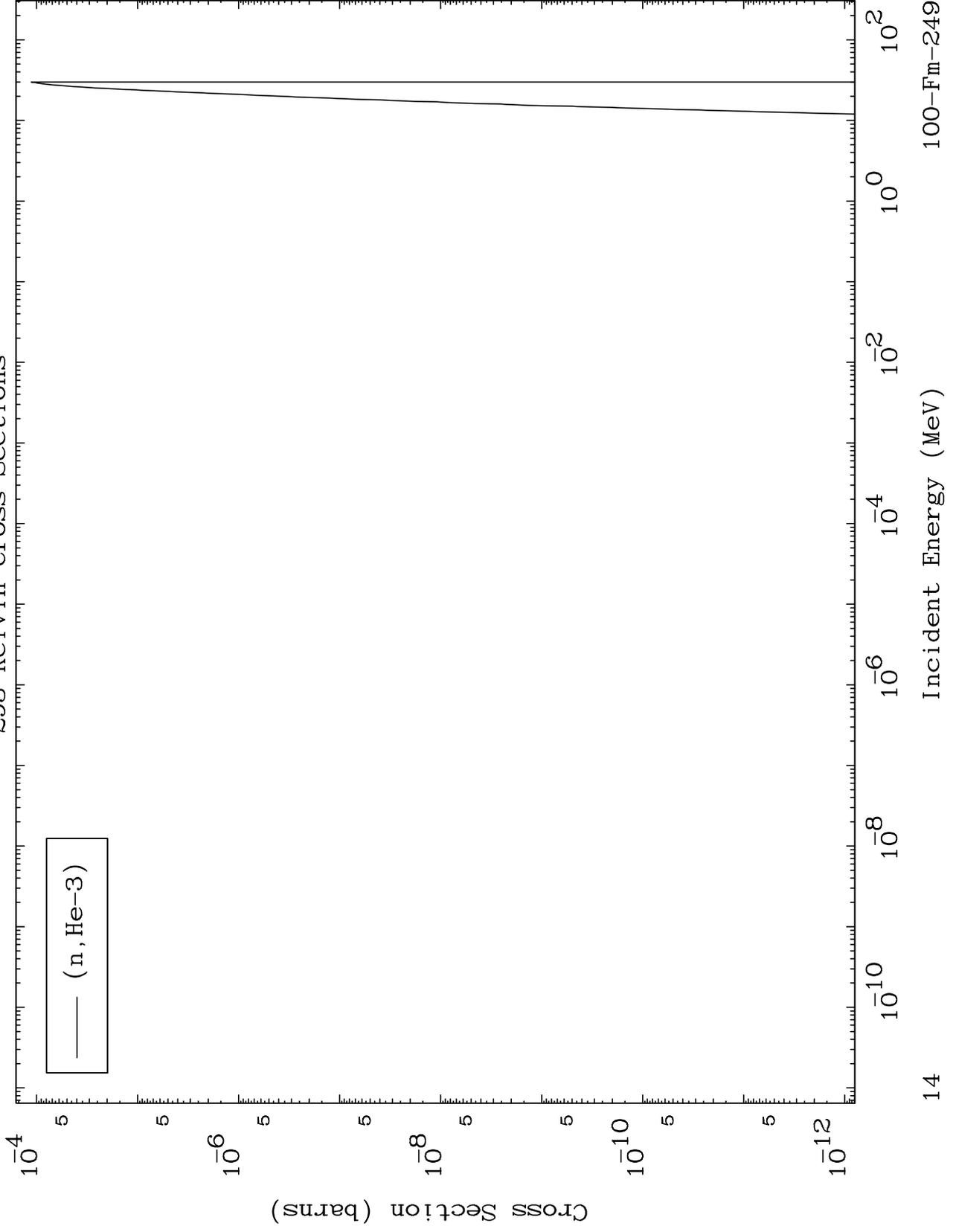
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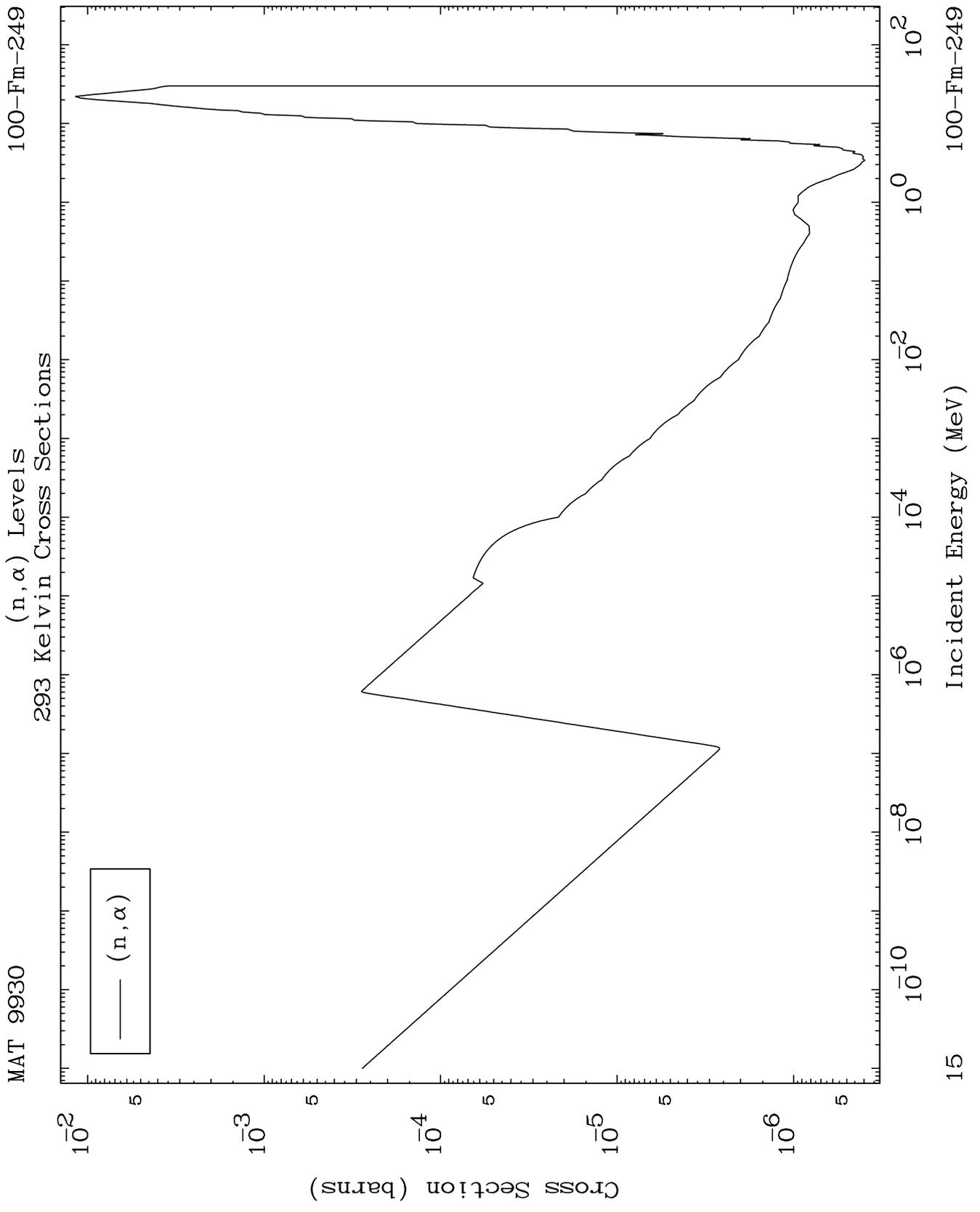
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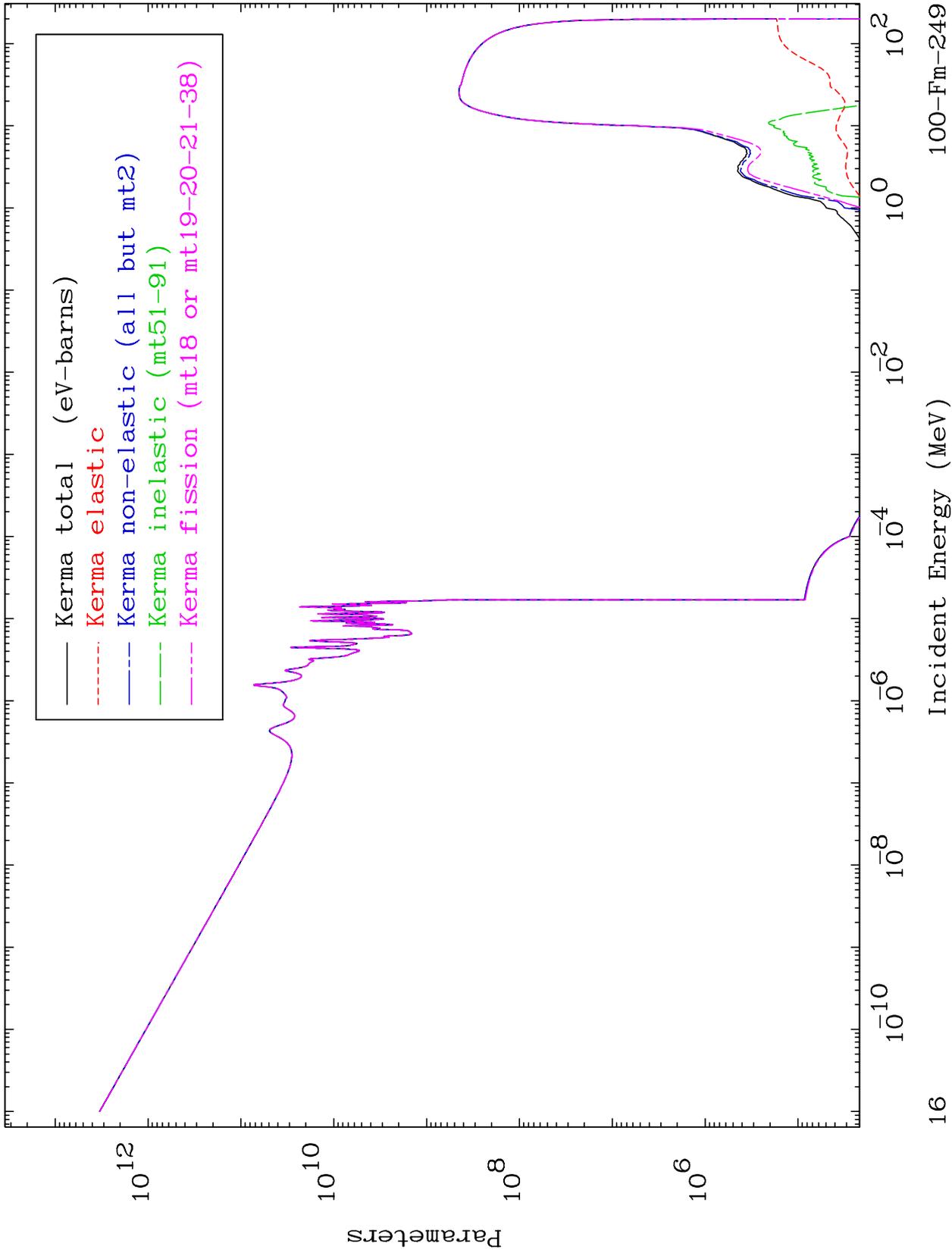
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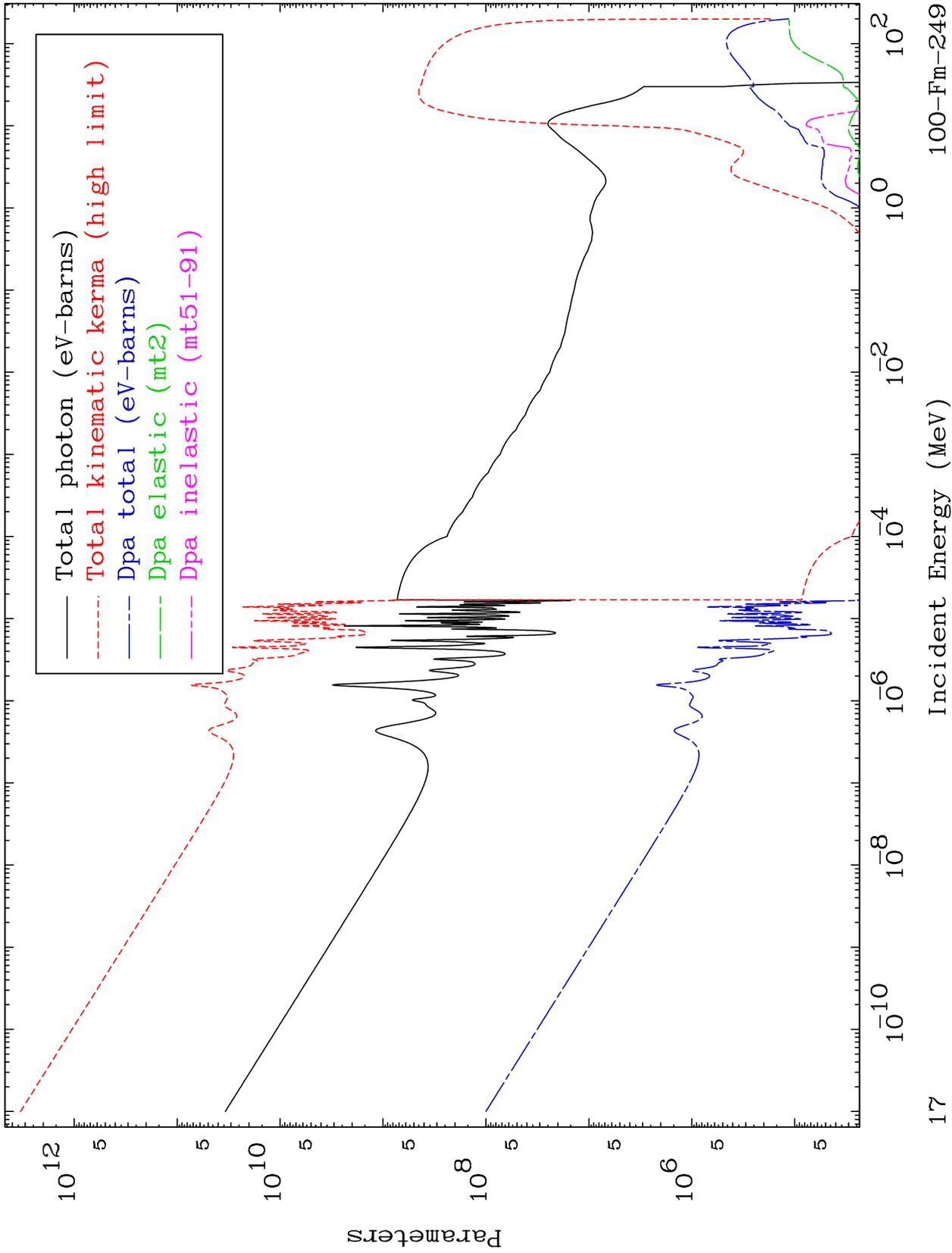
(n,He3) Levels
293 Kelvin Cross Sections

100-Fm-249





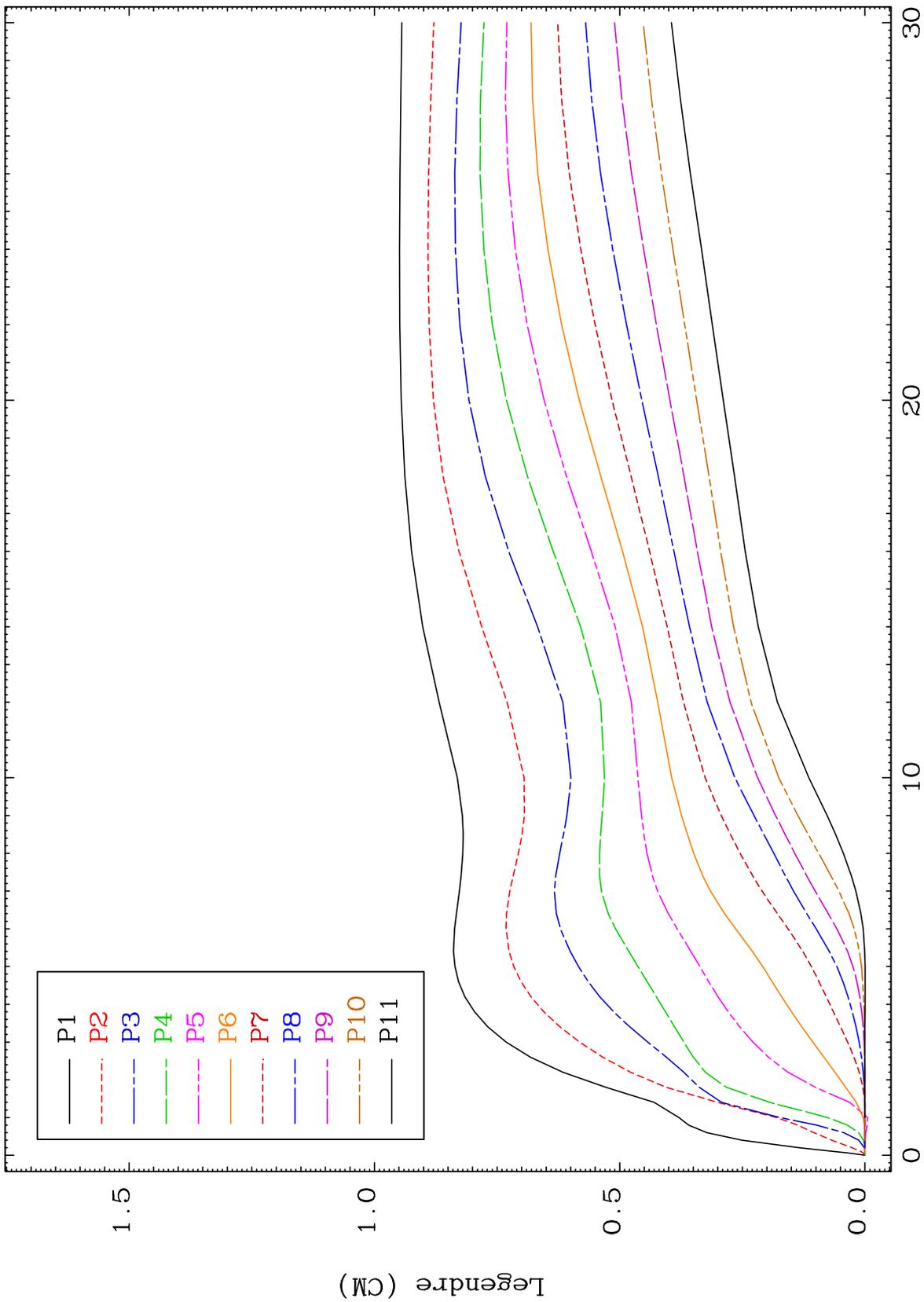




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Elastic Legendre Coefficients

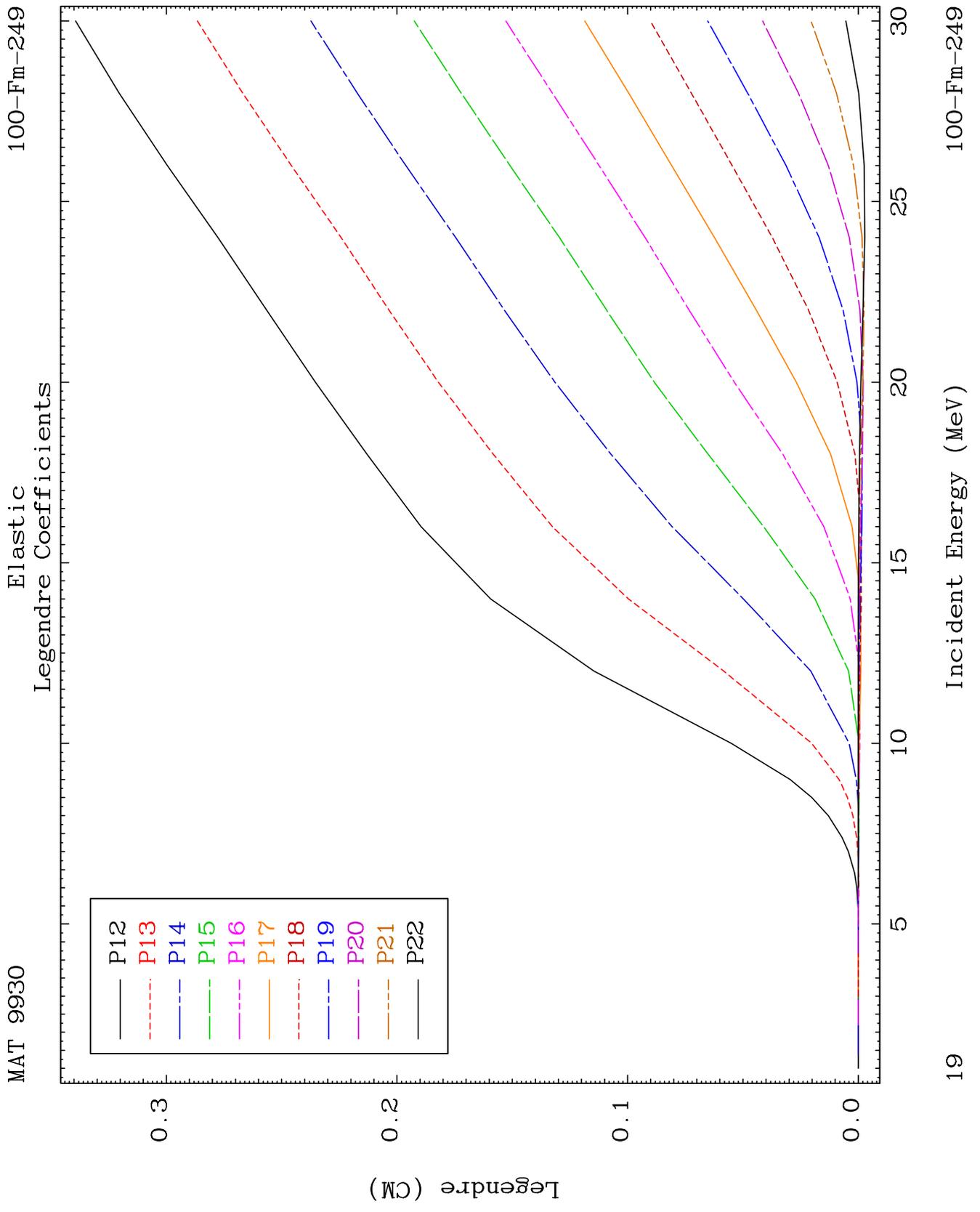
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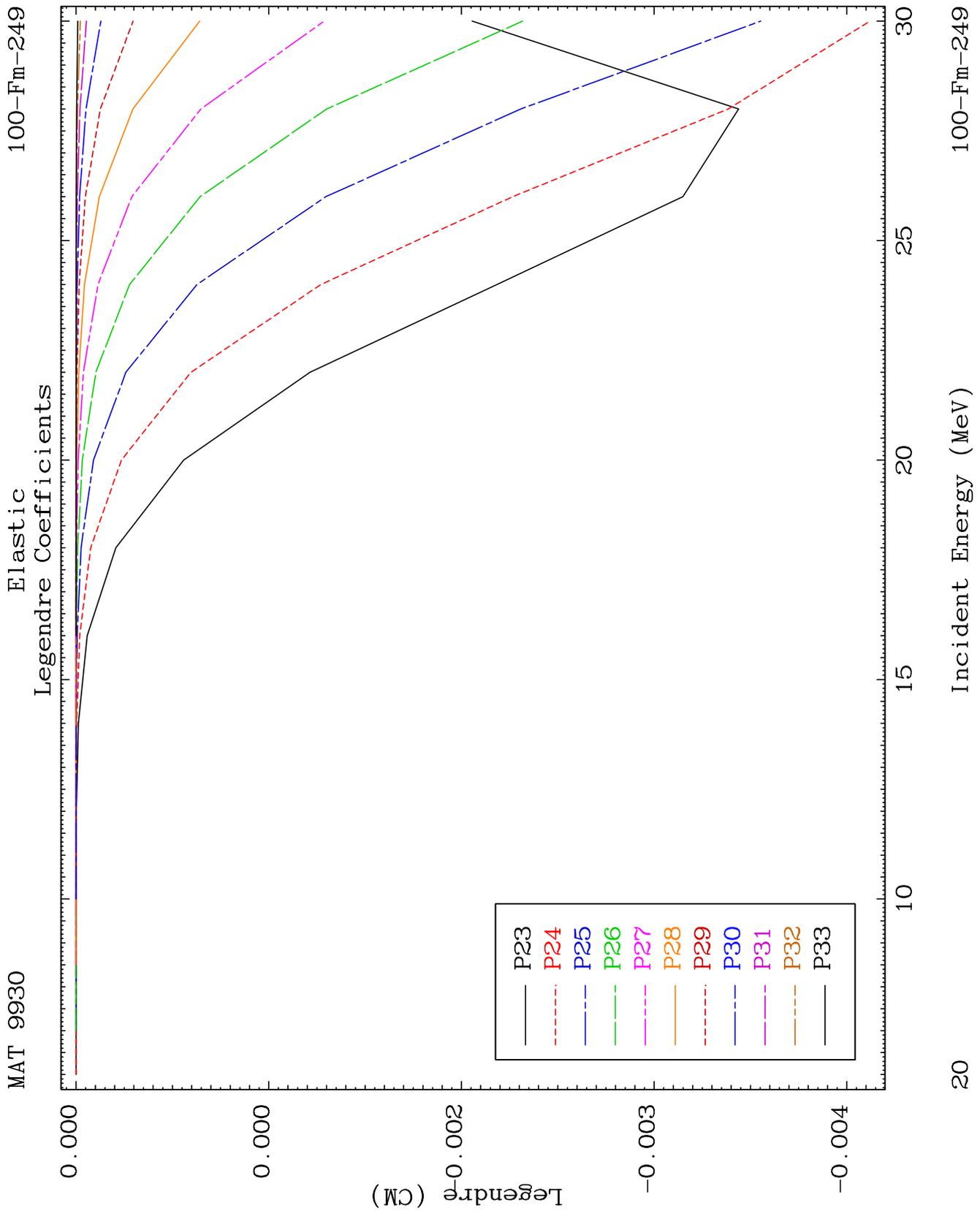


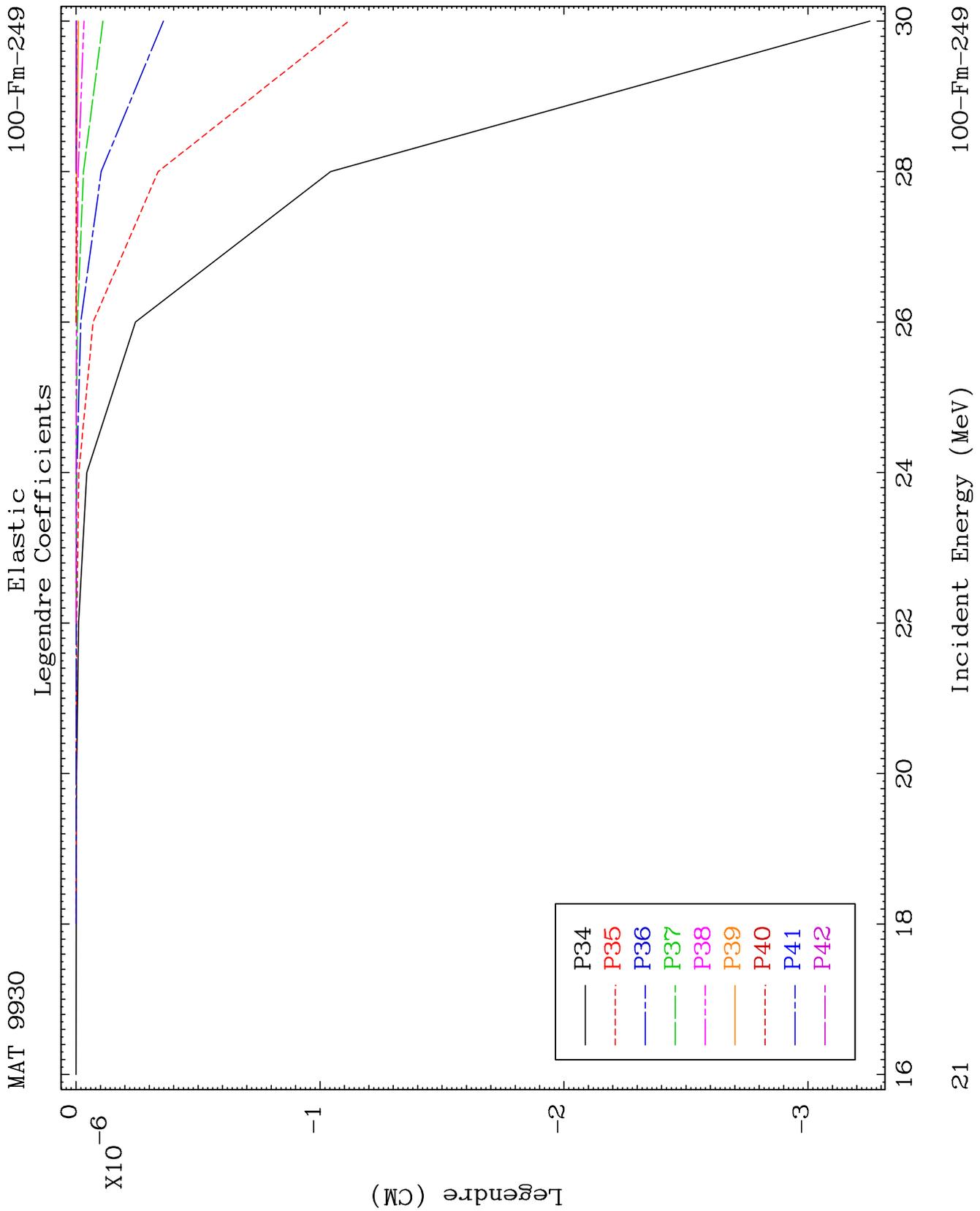
Incident Energy (MeV)

100-Fm-249

18



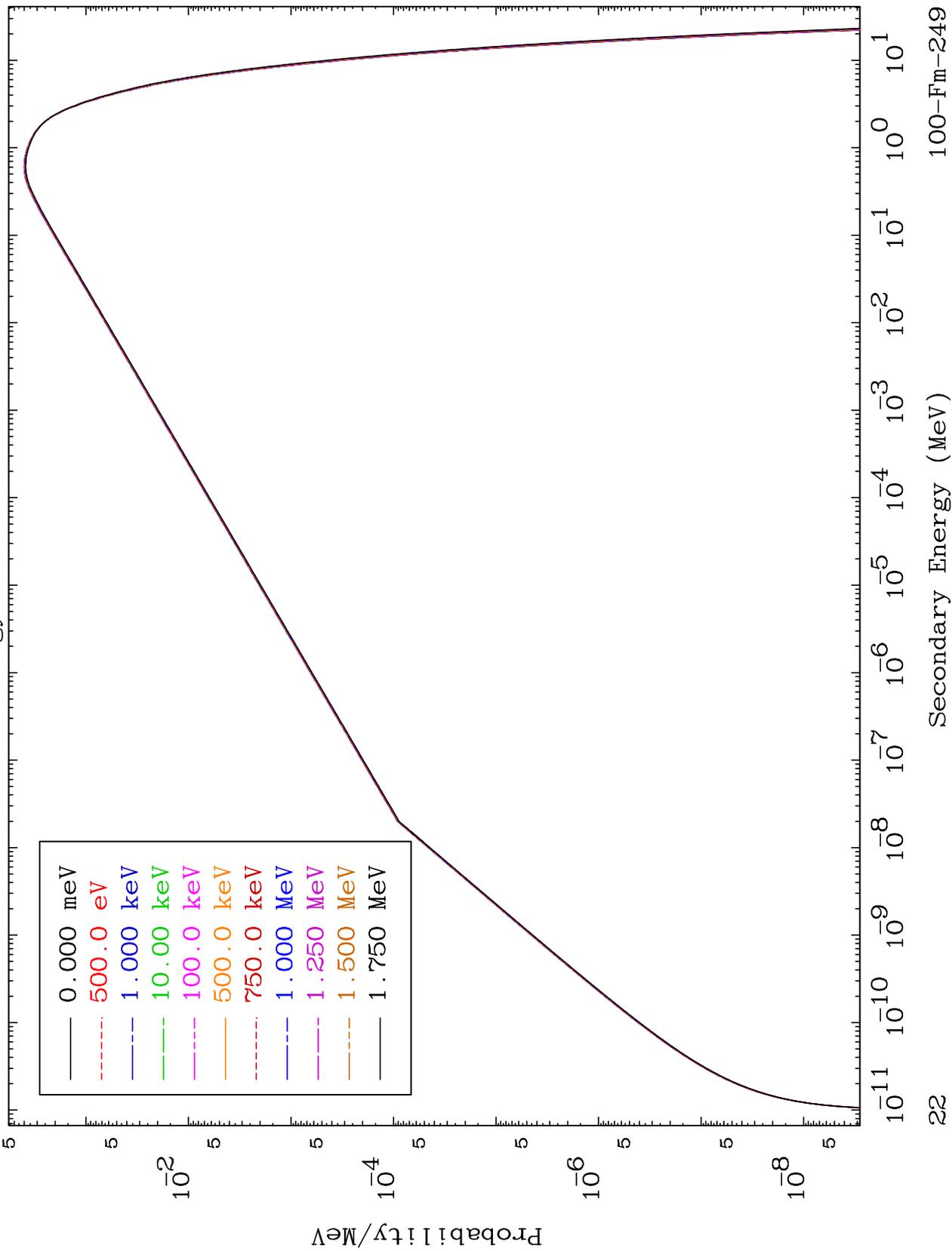




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Fission Energy Distribution

100-Fm-249

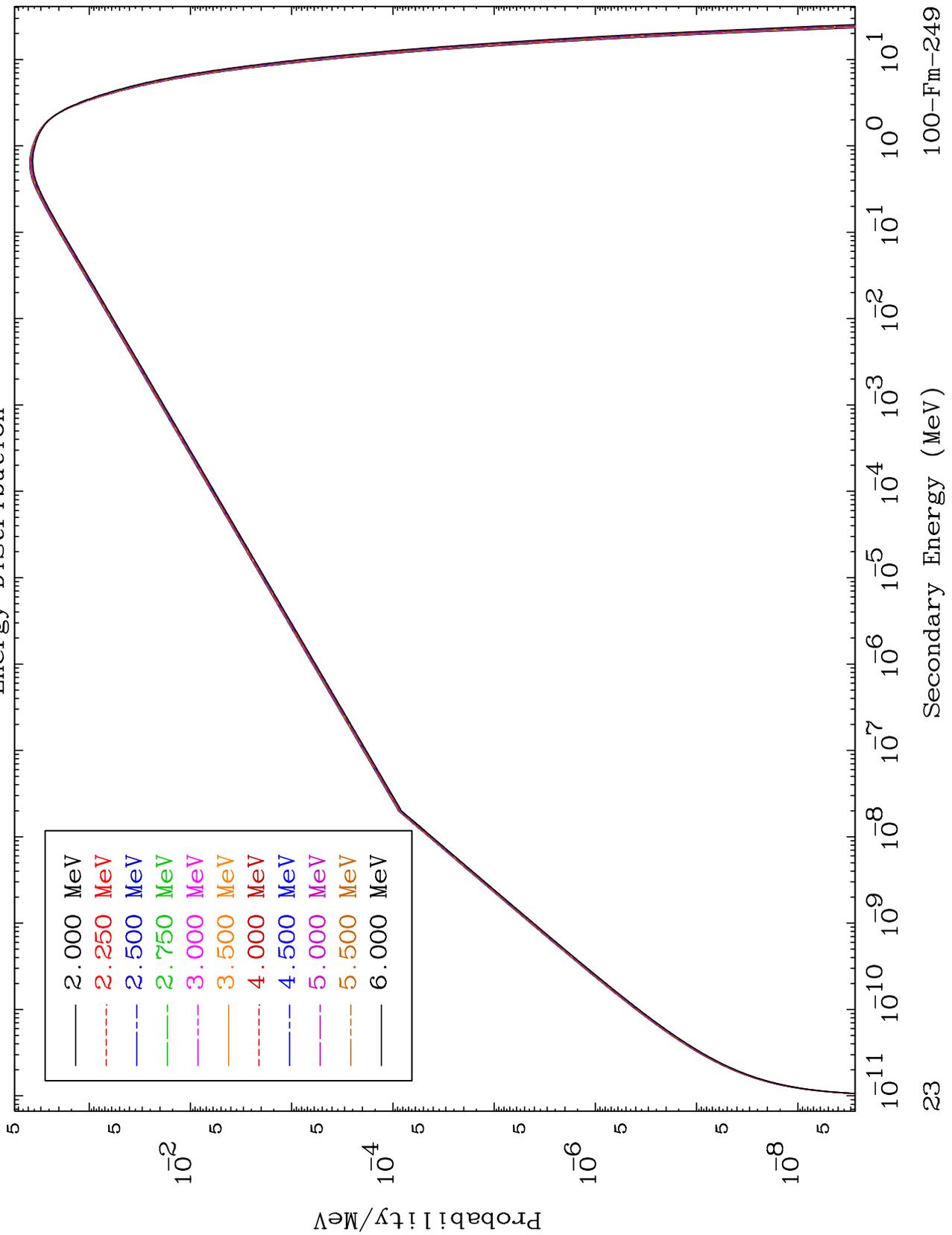


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Fission Energy Distribution

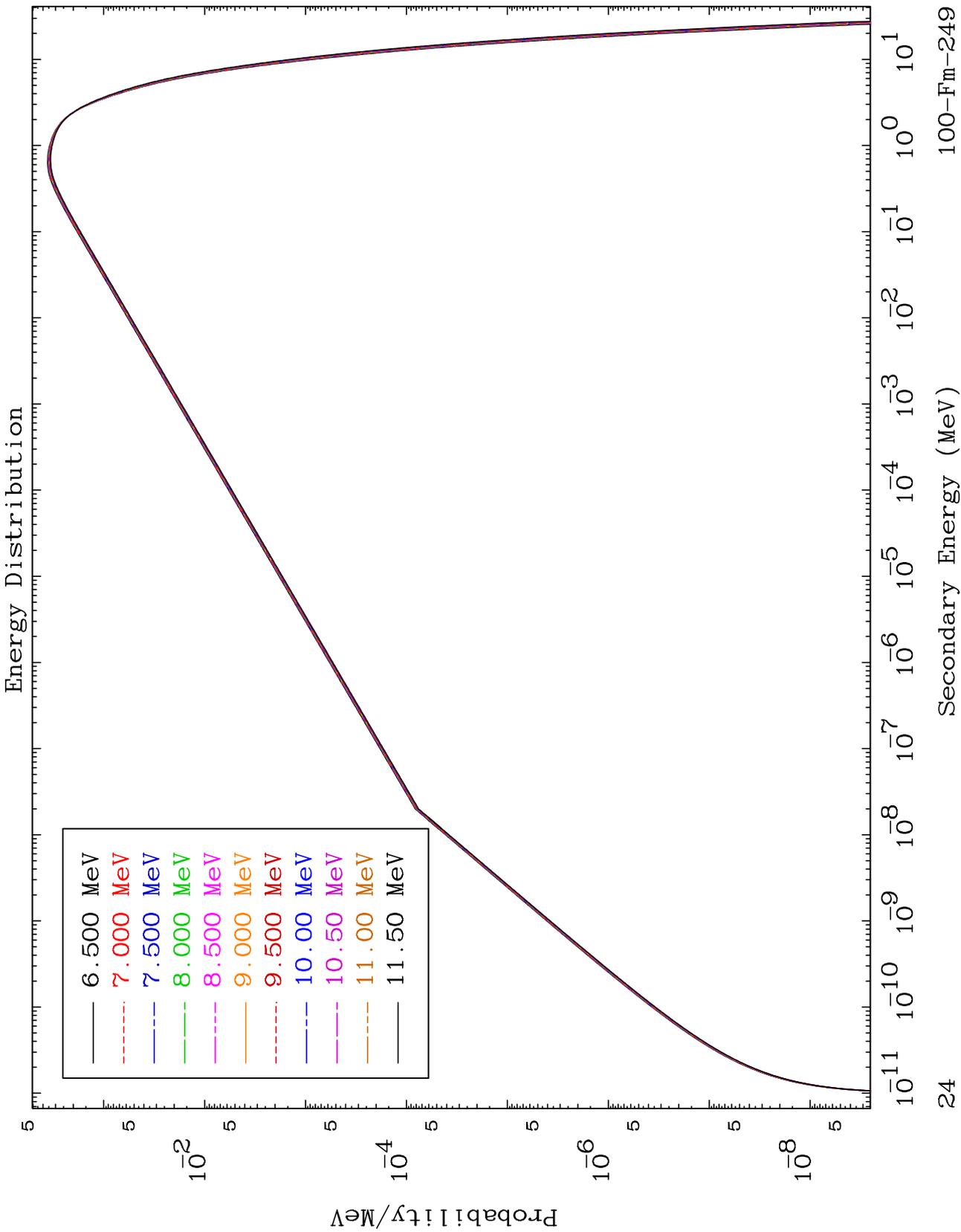
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Fission

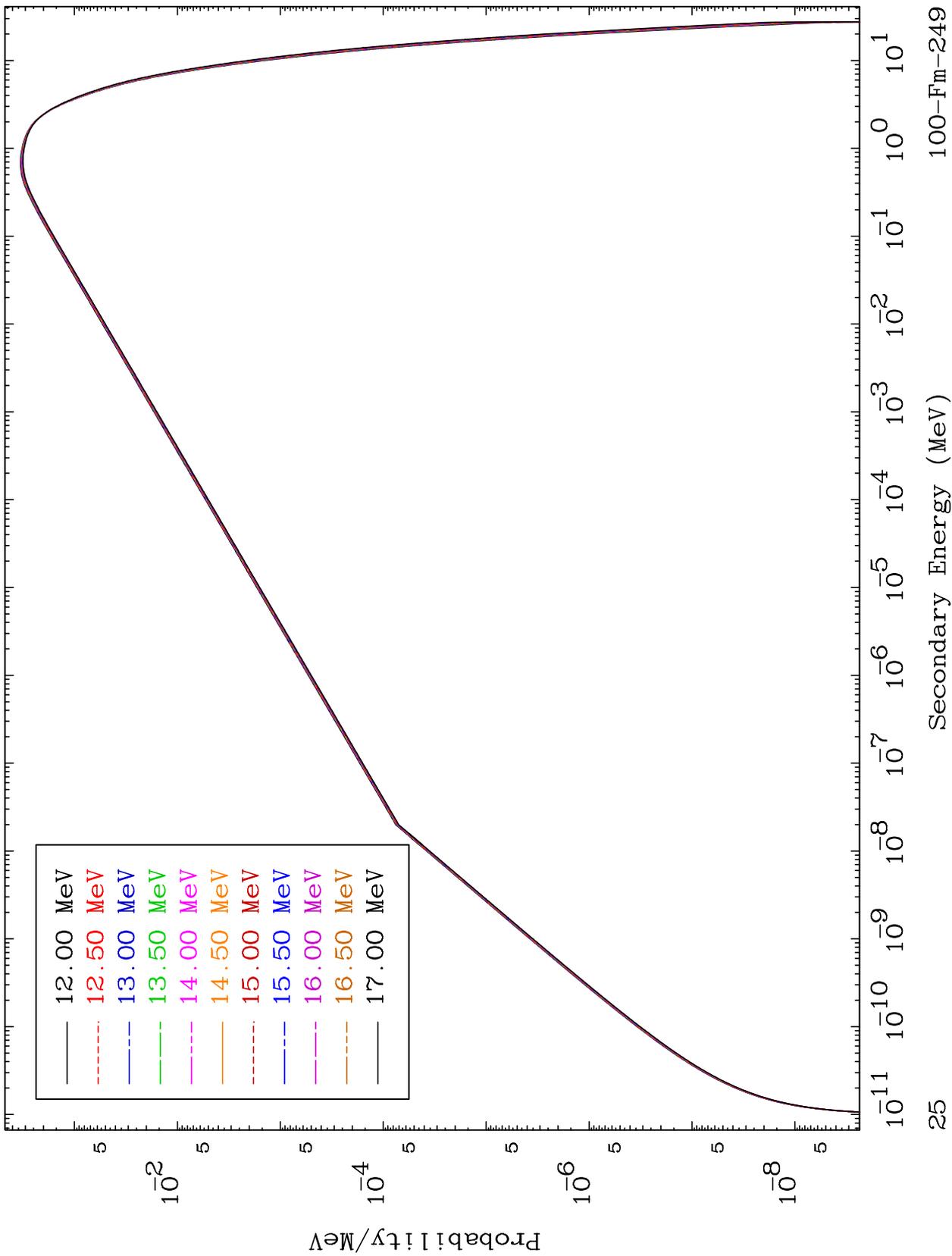
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Fission Energy Distribution

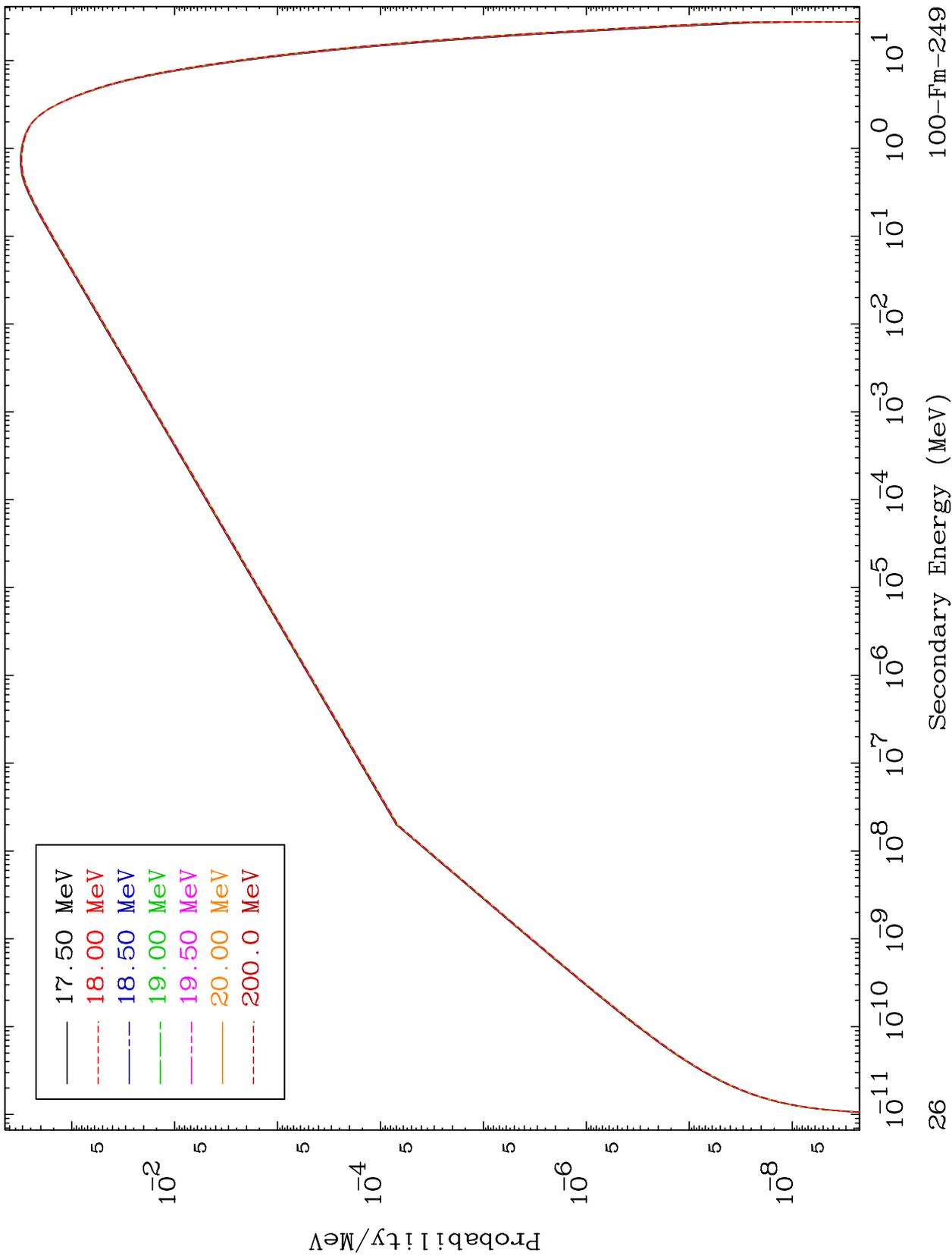
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Fission Energy Distribution

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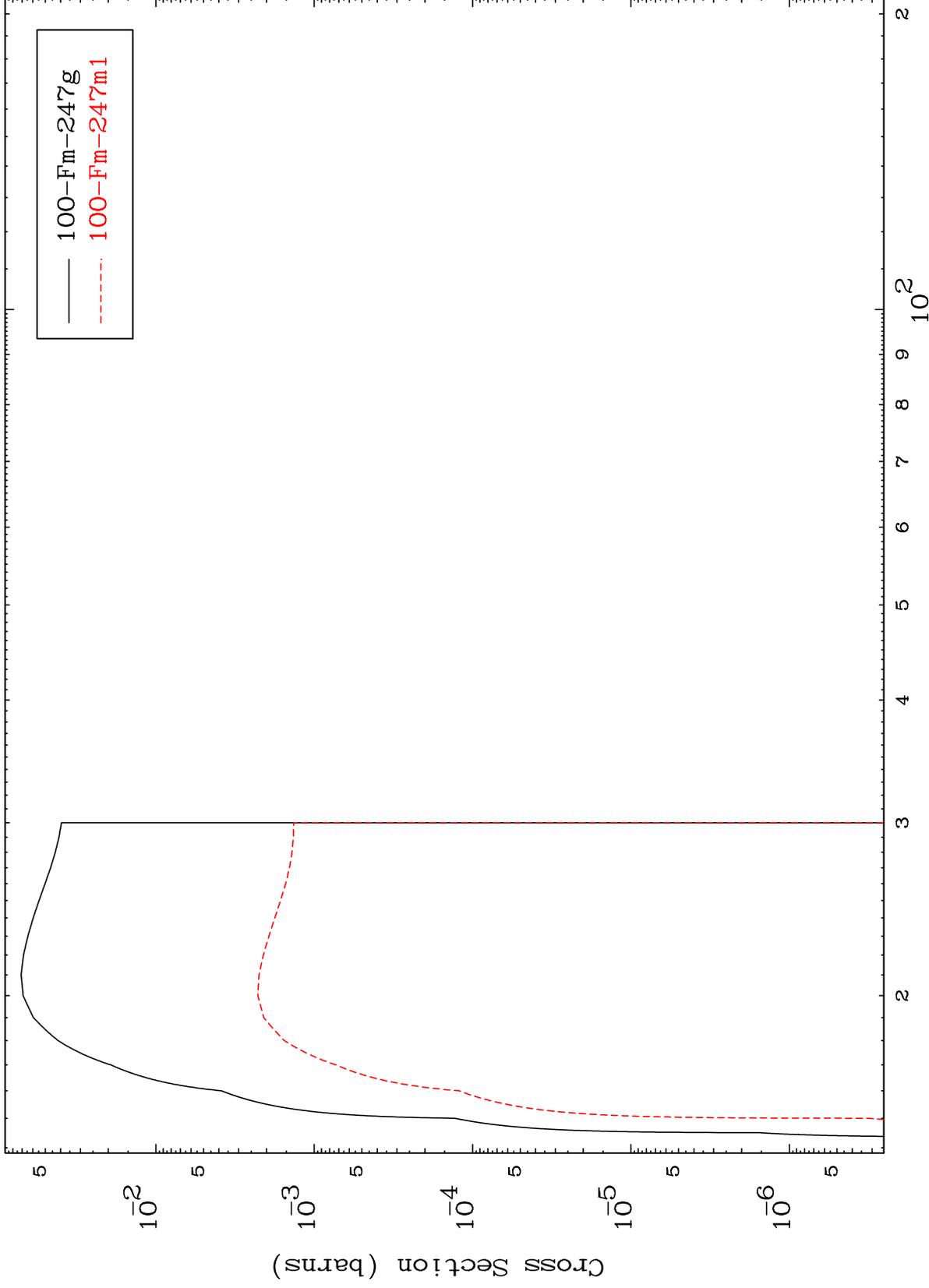


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(n,3n)

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Radionuclide Production Cross Section



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Incident Energy (MeV)

100-Fm-249

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100-Fm-249

Radionuclide Production Cross Section
(n, γ)

