

Program EVALPLOT  
(Version 2018-1)

by

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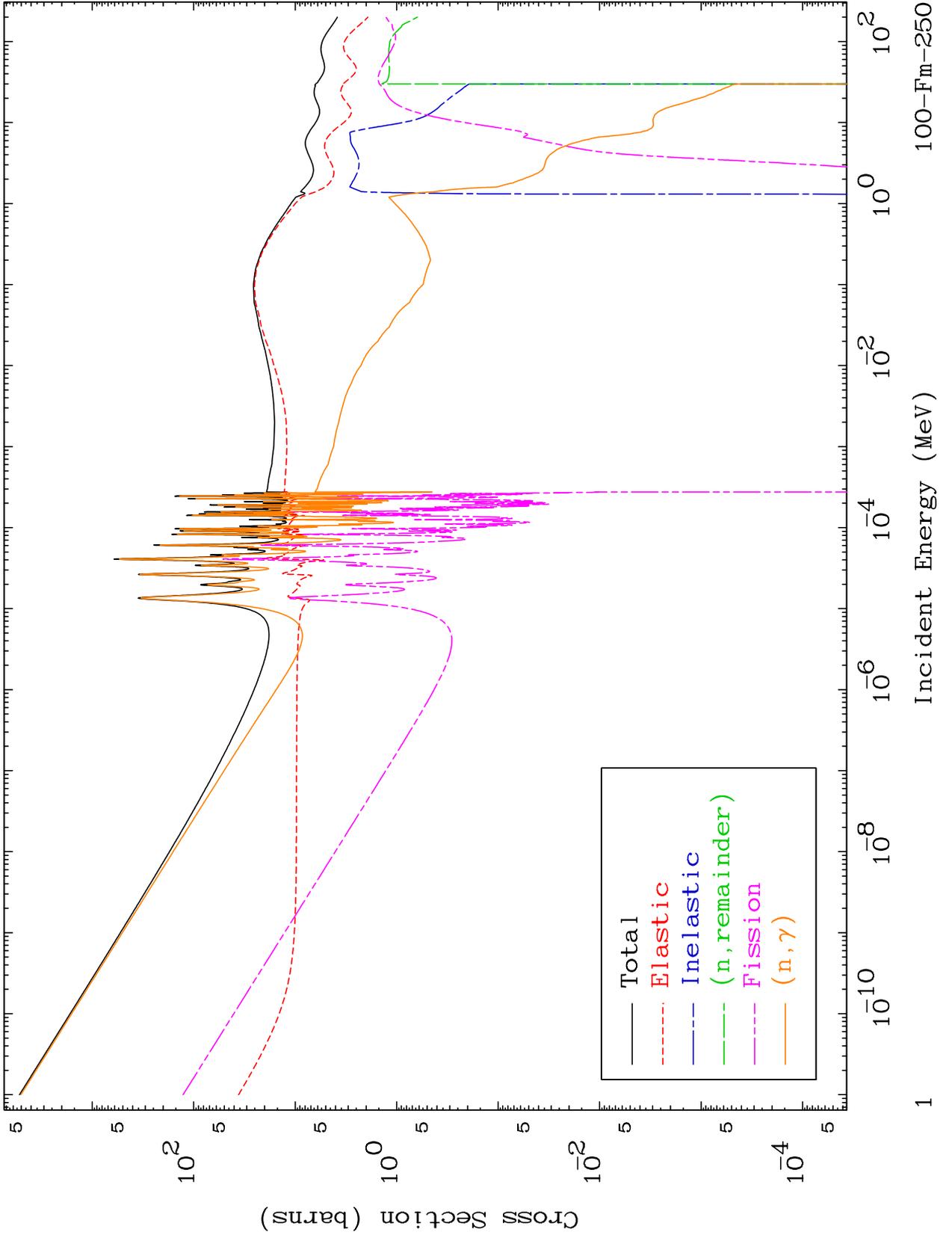
Press Mouse Button to Start

MAT 9931

Major

293 Kelvin Cross Sections

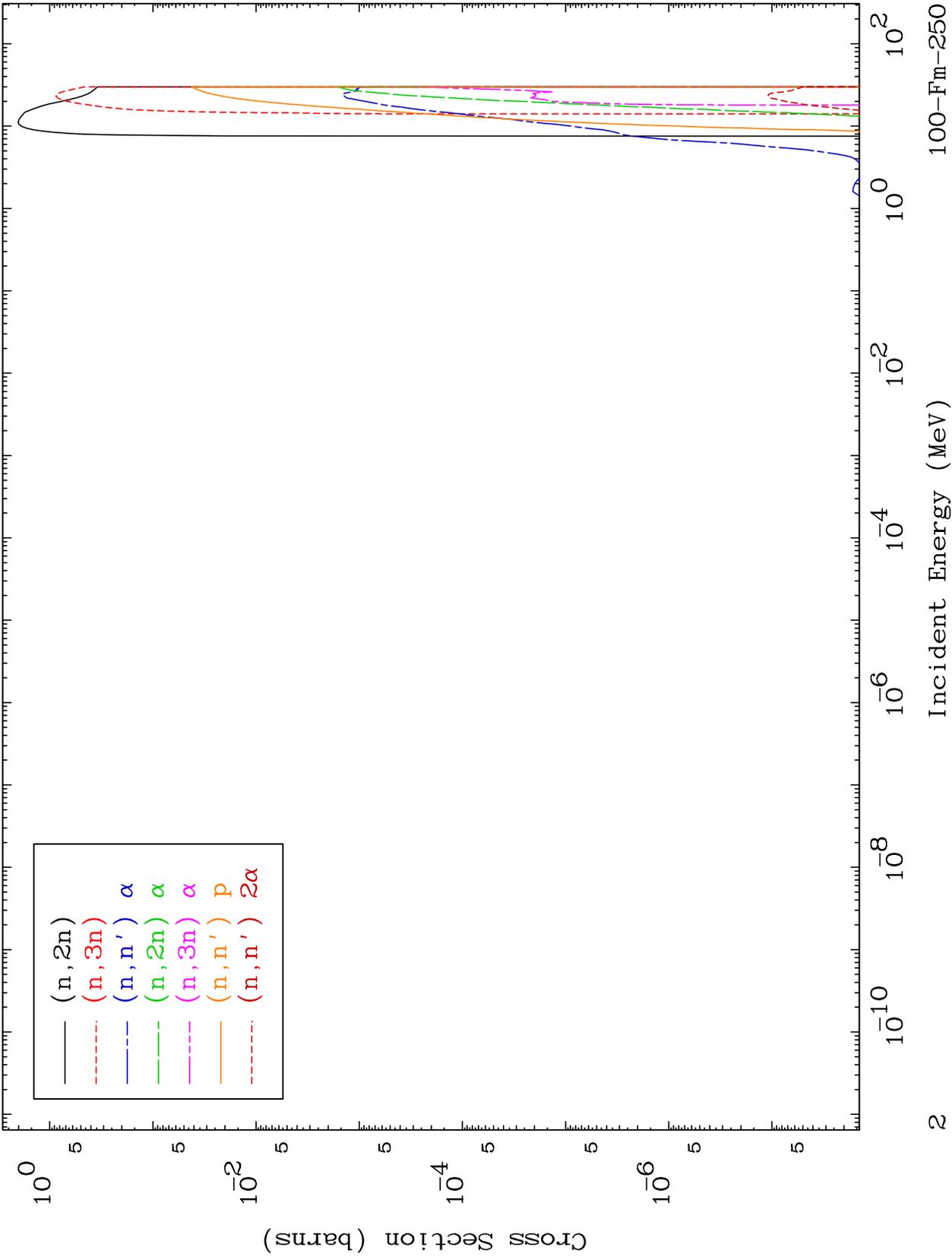
100-Fm-250



MAT 9931

Neutron Production  
293 Kelvin Cross Sections

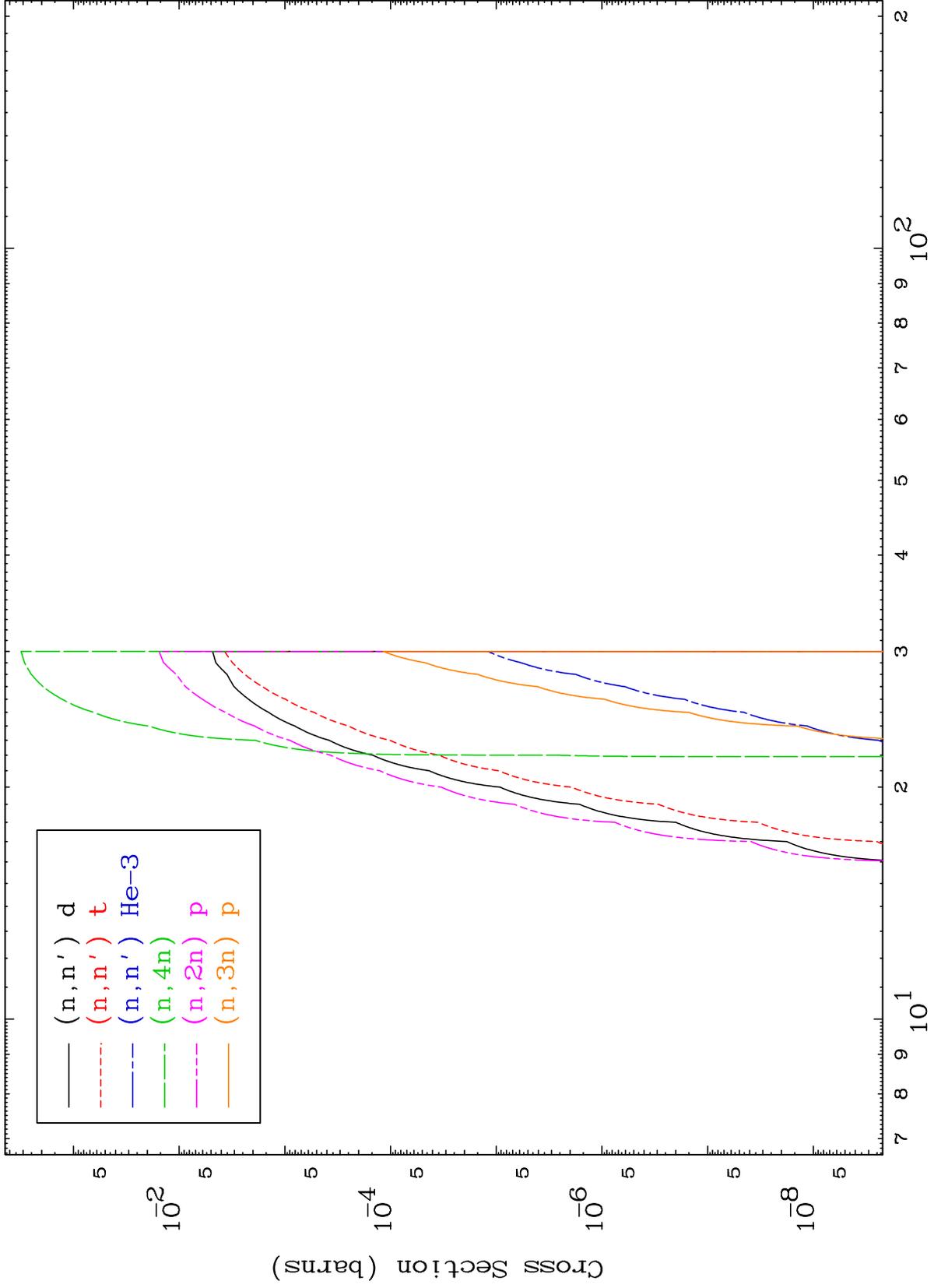
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Neutron Production  
293 Kelvin Cross Sections

100-Fm-250



3

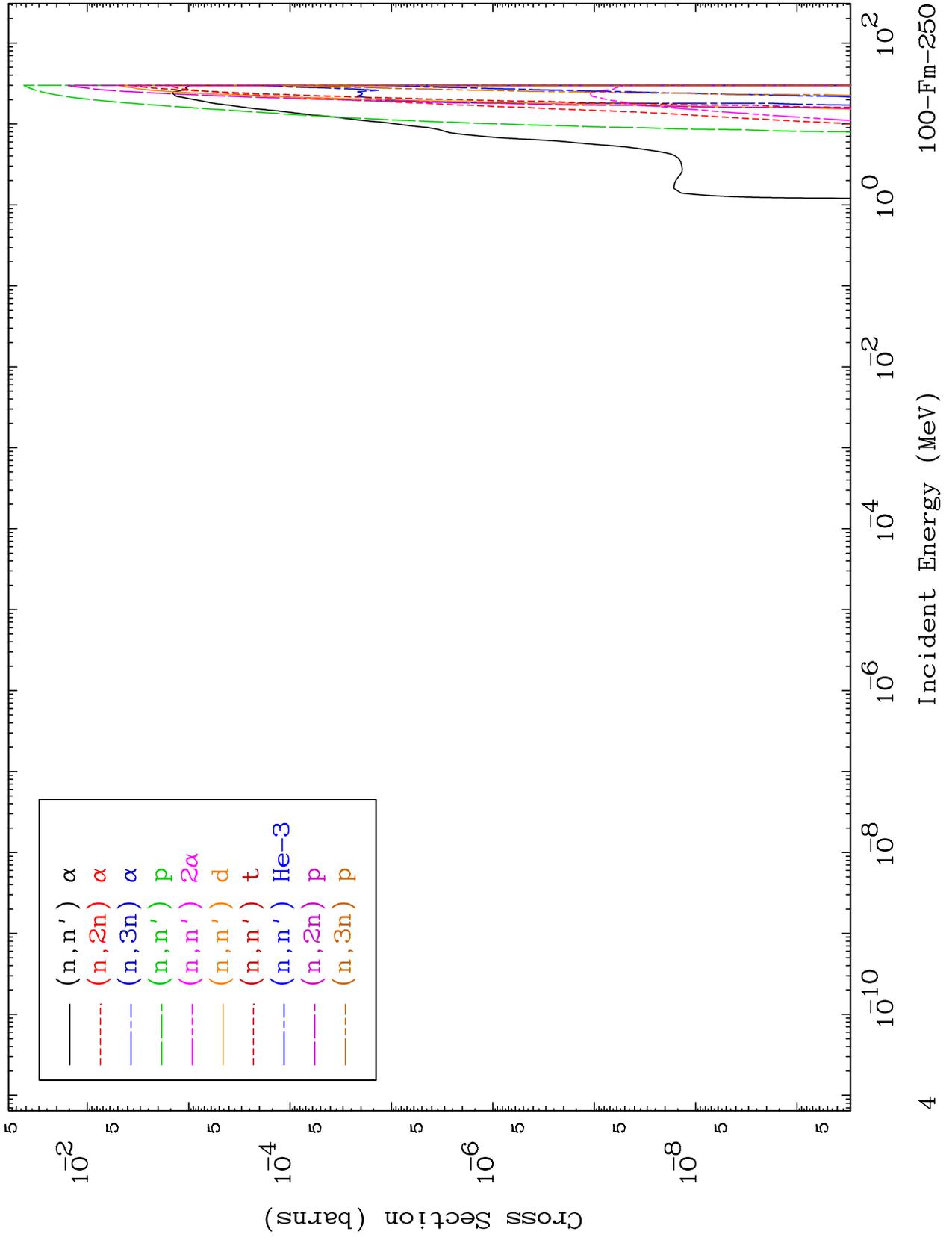
Incident Energy (MeV)

100-Fm-250

MAT 9931

Charged Particle  
293 Kelvin Cross Sections

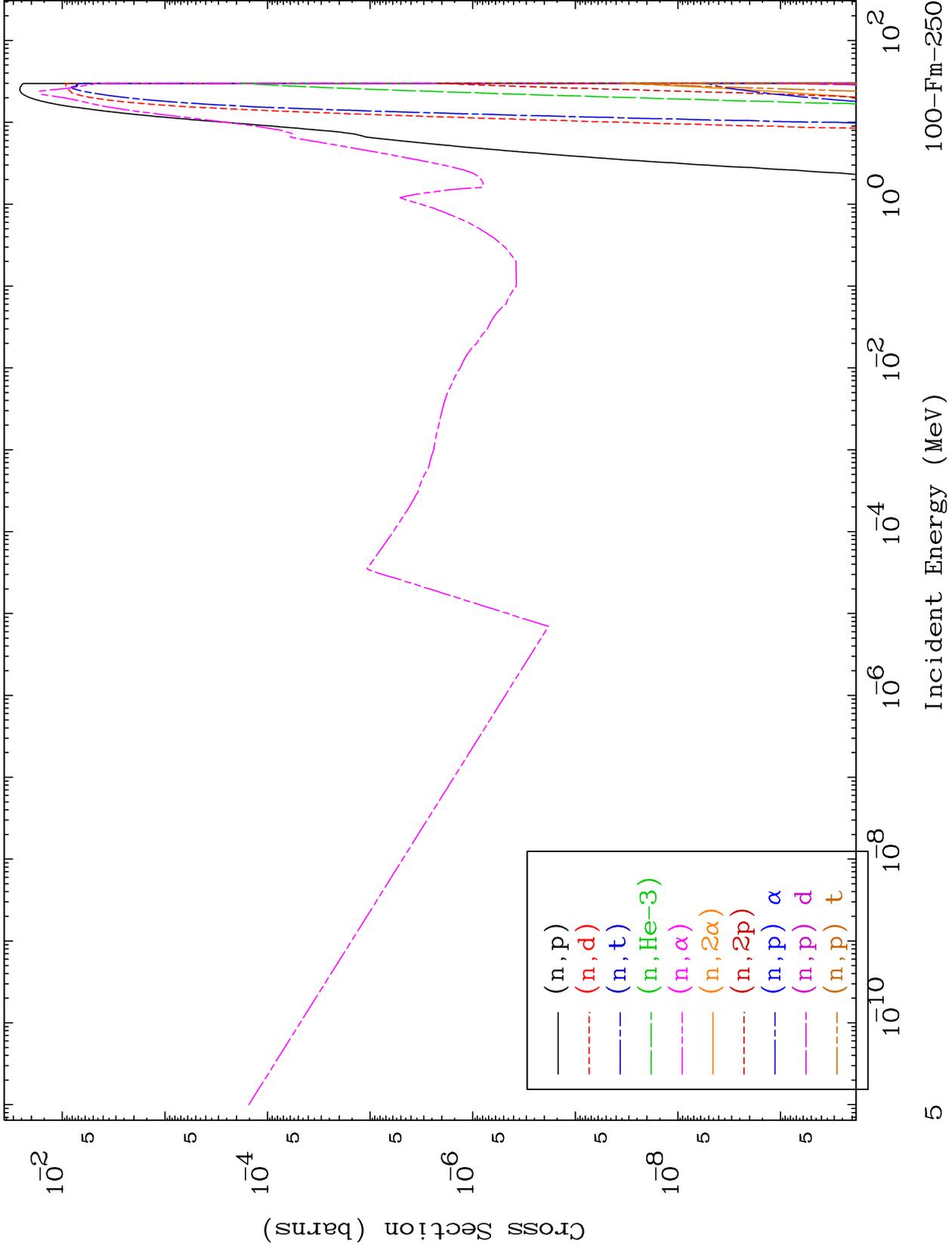
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Charged Particle  
293 Kelvin Cross Sections

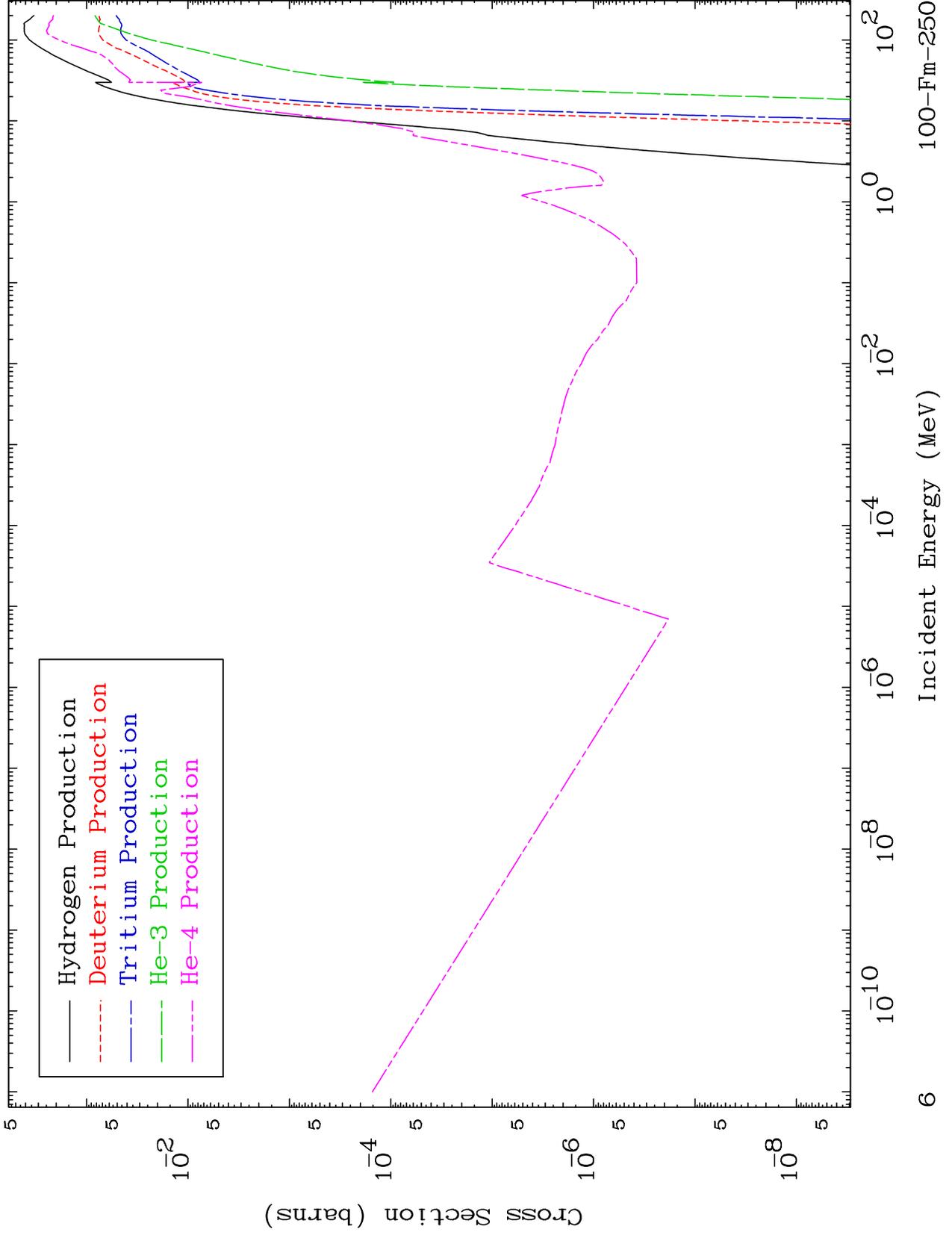
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Particle Production  
293 Kelvin Cross Sections

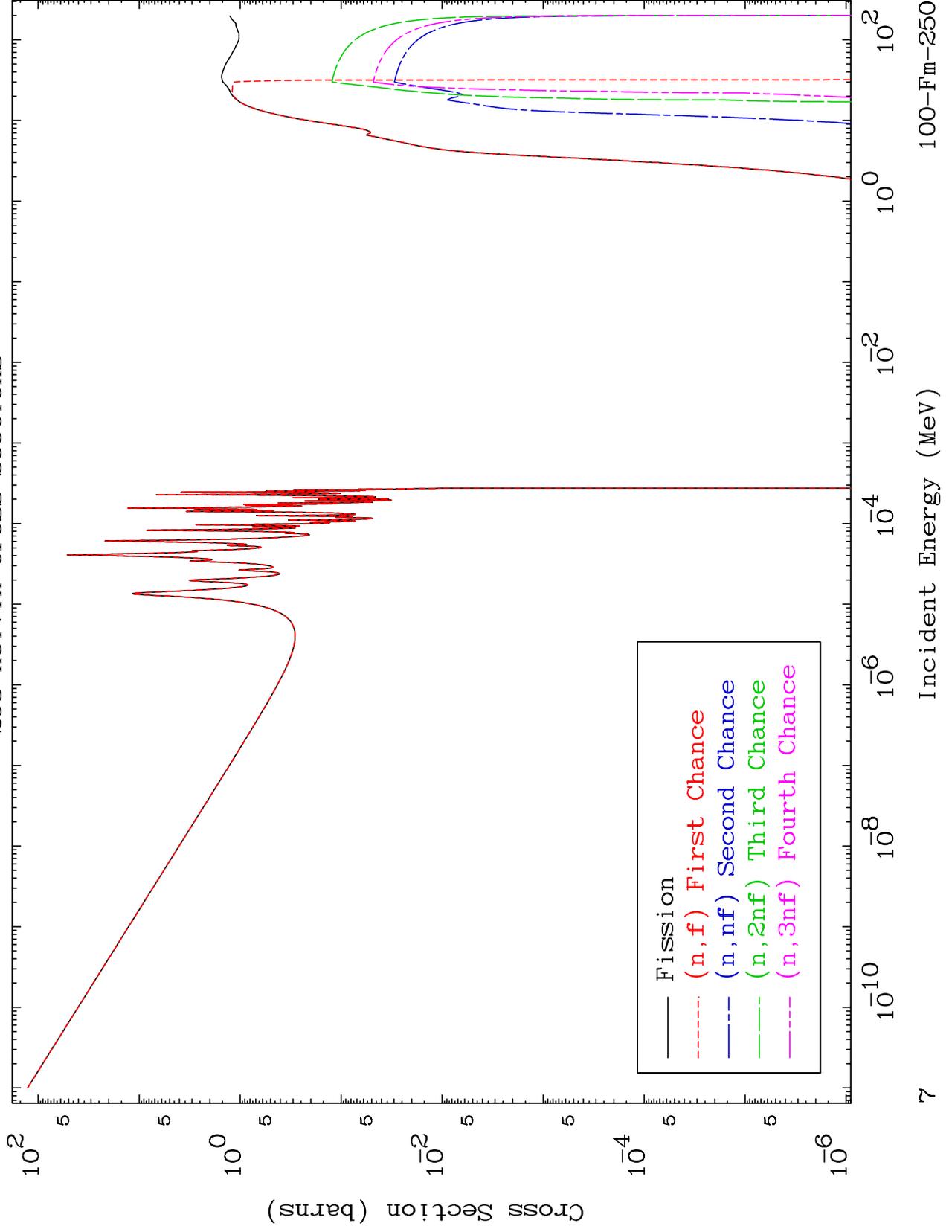
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Fission  
293 Kelvin Cross Sections

100-Fm-250

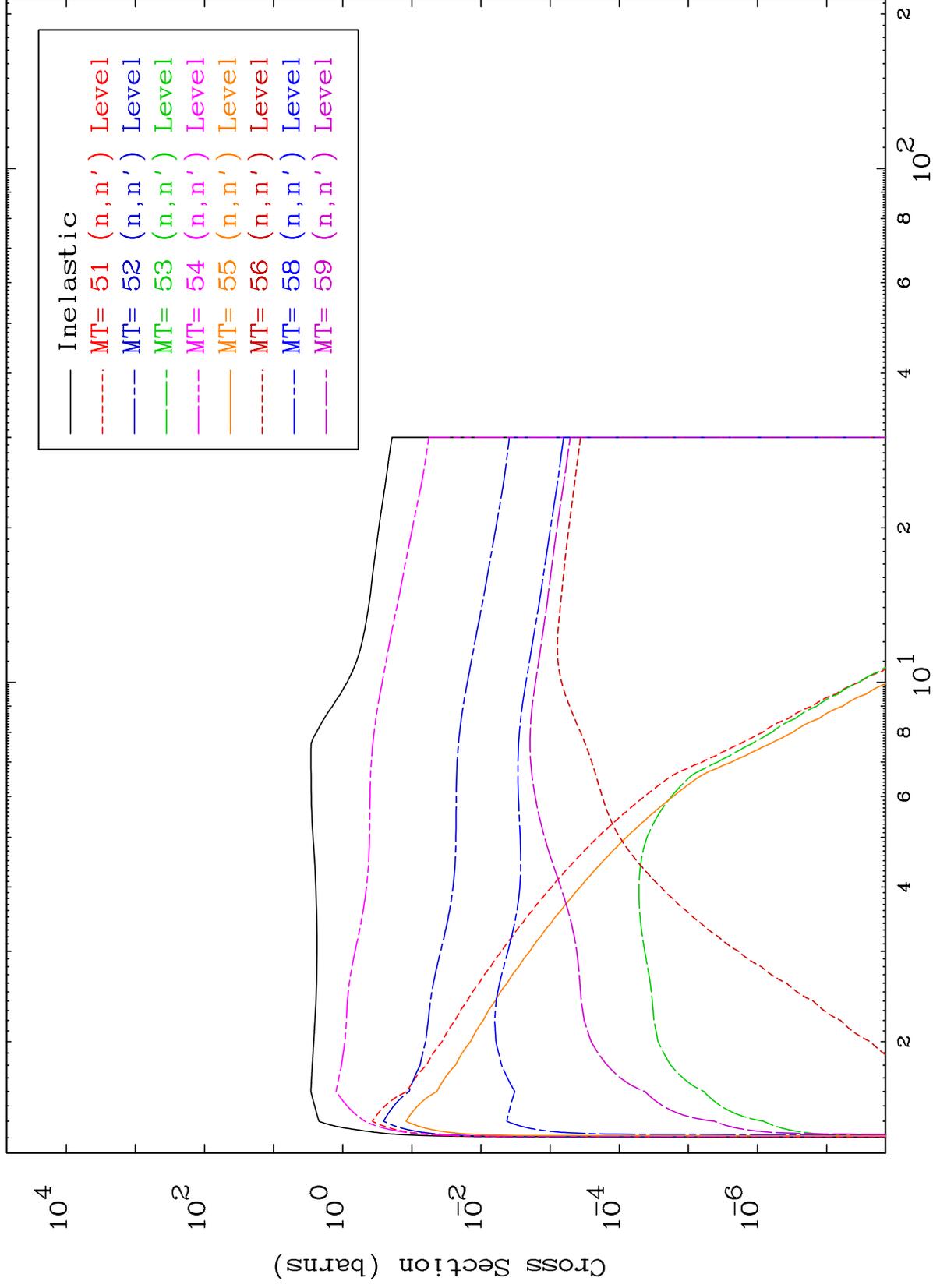


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(n,n') Level

100-Fm-250

293 Kelvin Cross Sections



8

Incident Energy (MeV)

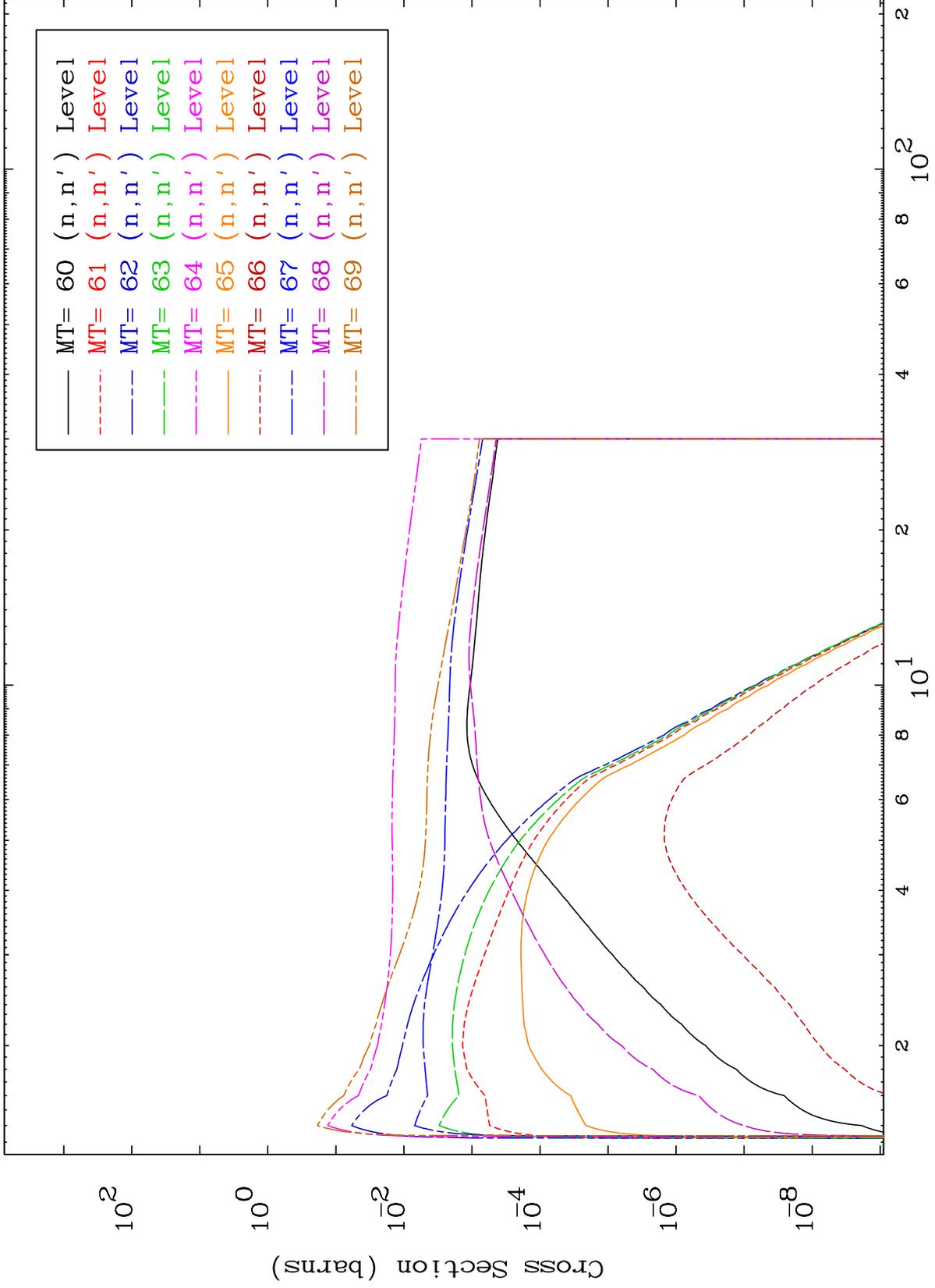
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(n,n') Level

100-Fm-250

293 Kelvin Cross Sections

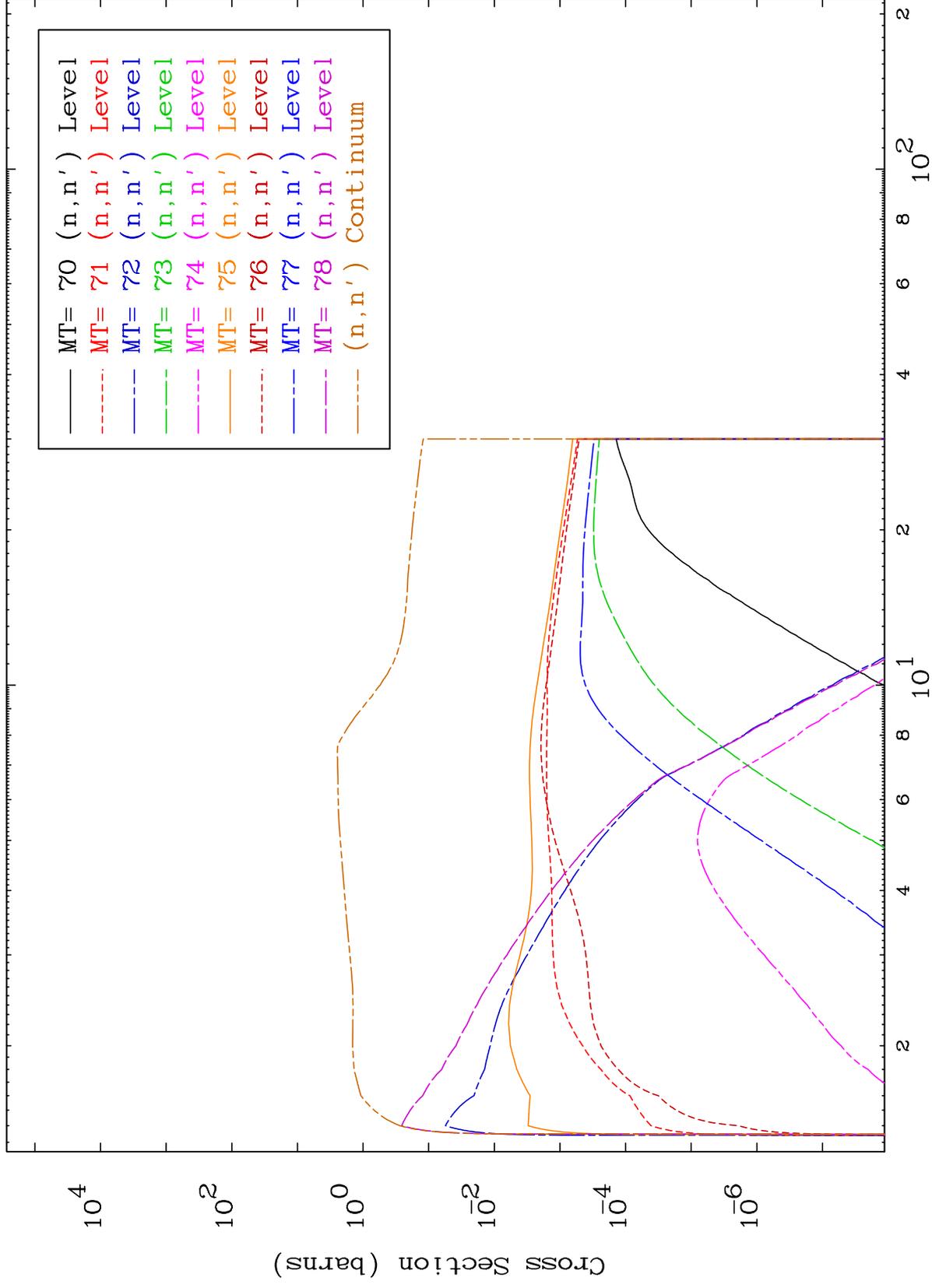


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(n,n') Level

100-Fm-250

293 Kelvin Cross Sections



10

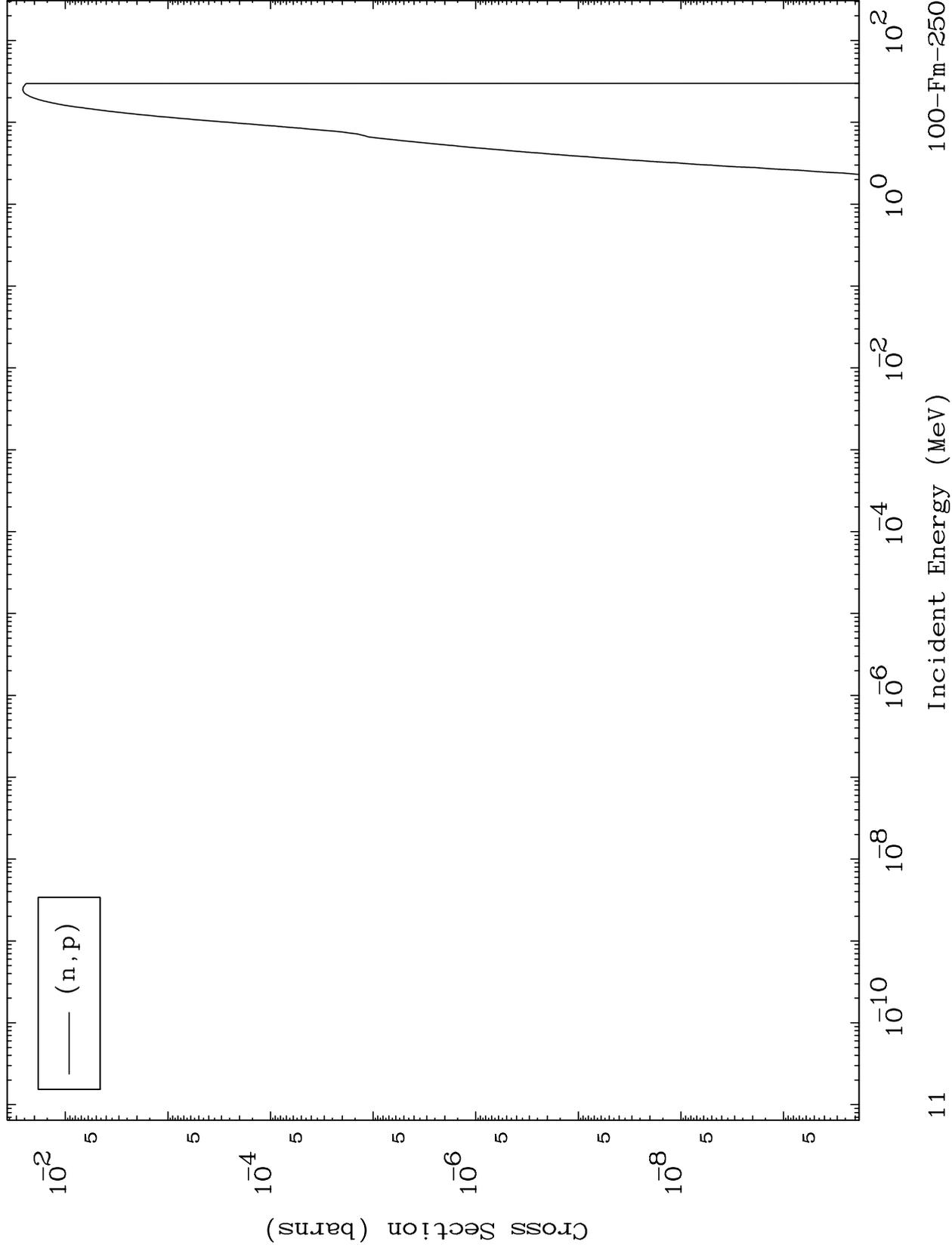
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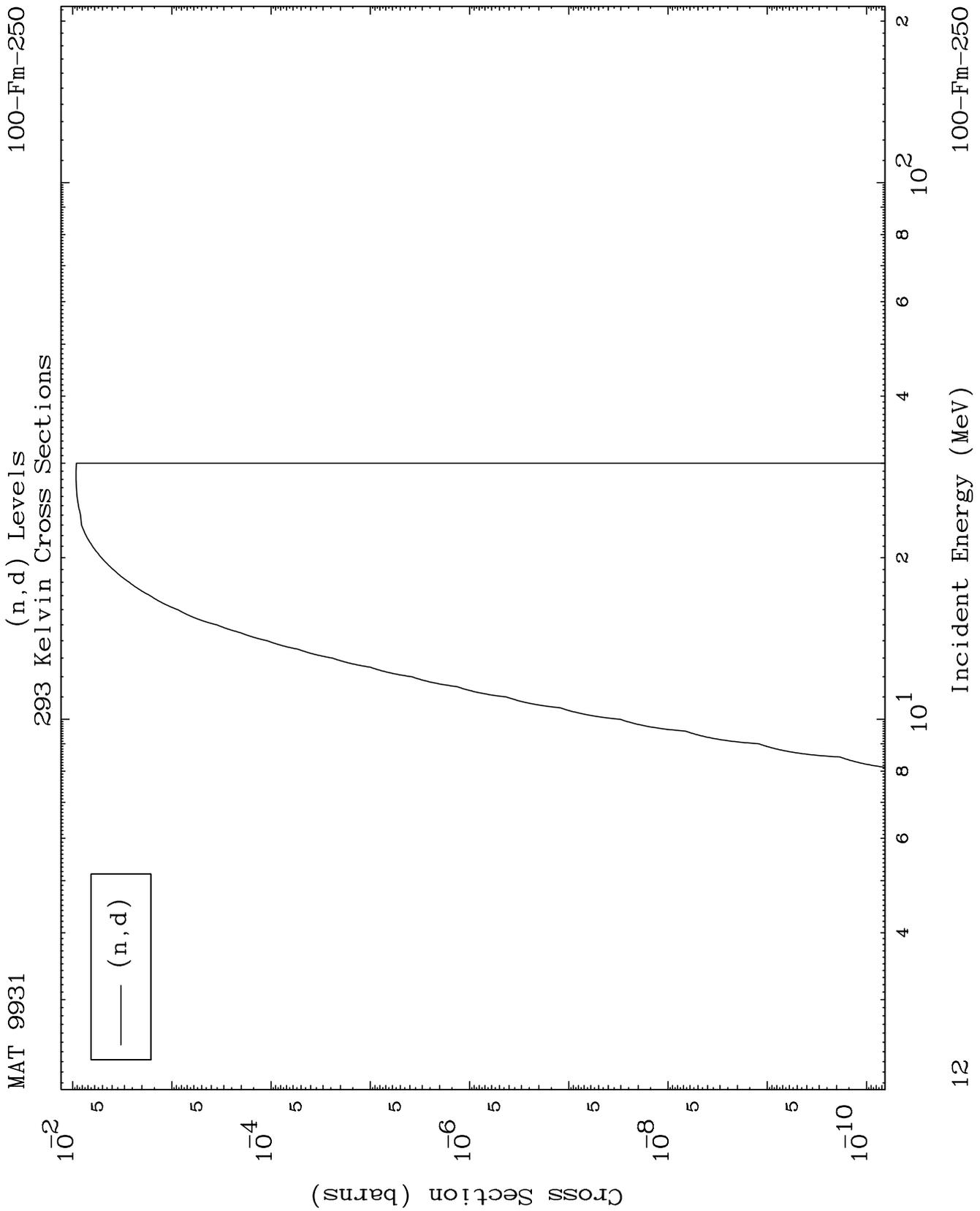
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MAT 9931

(n,p) Levels  
293 Kelvin Cross Sections

100-Fm-250

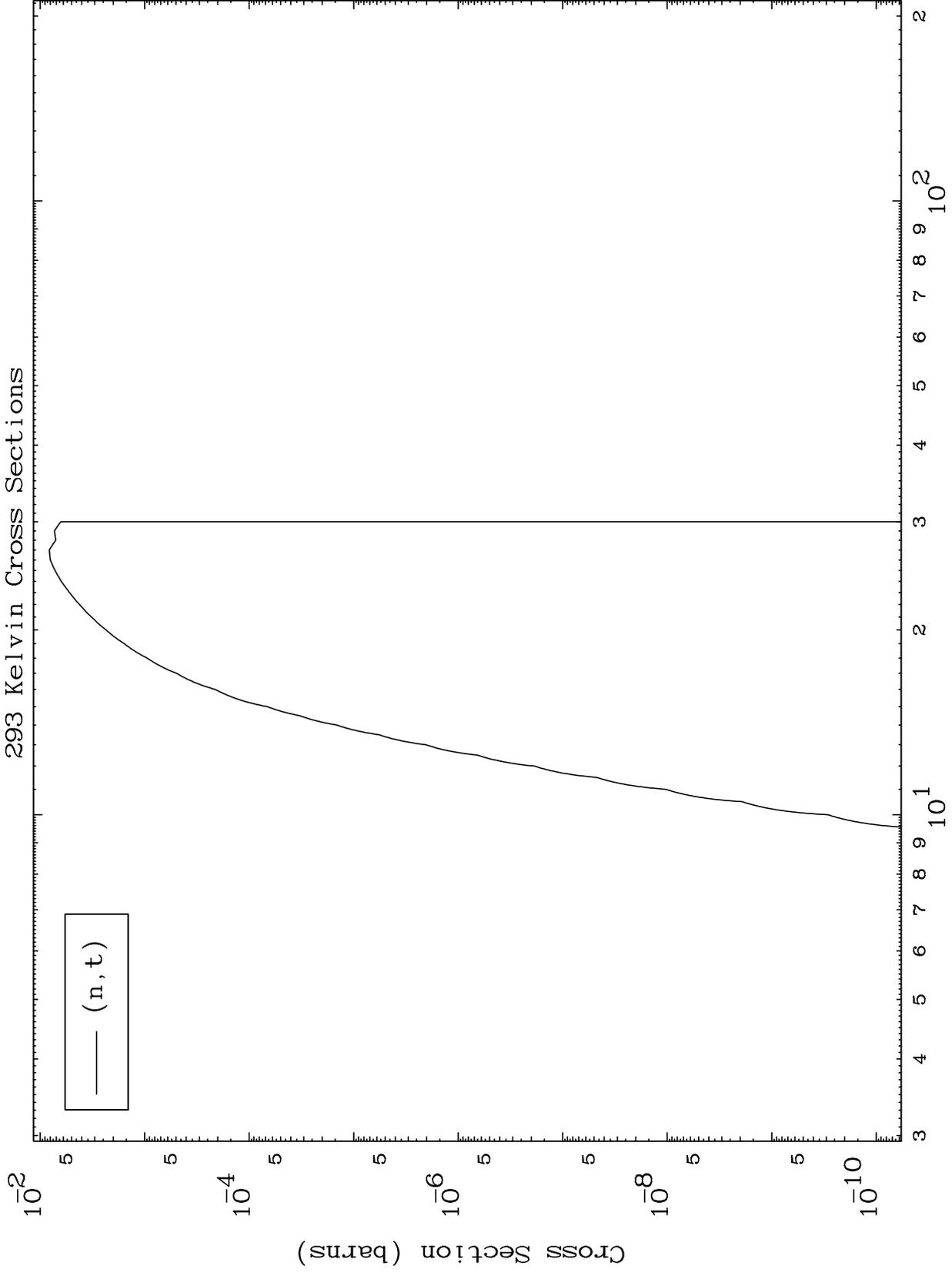




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(n,t) Levels  
293 Kelvin Cross Sections

100-Fm-250



13

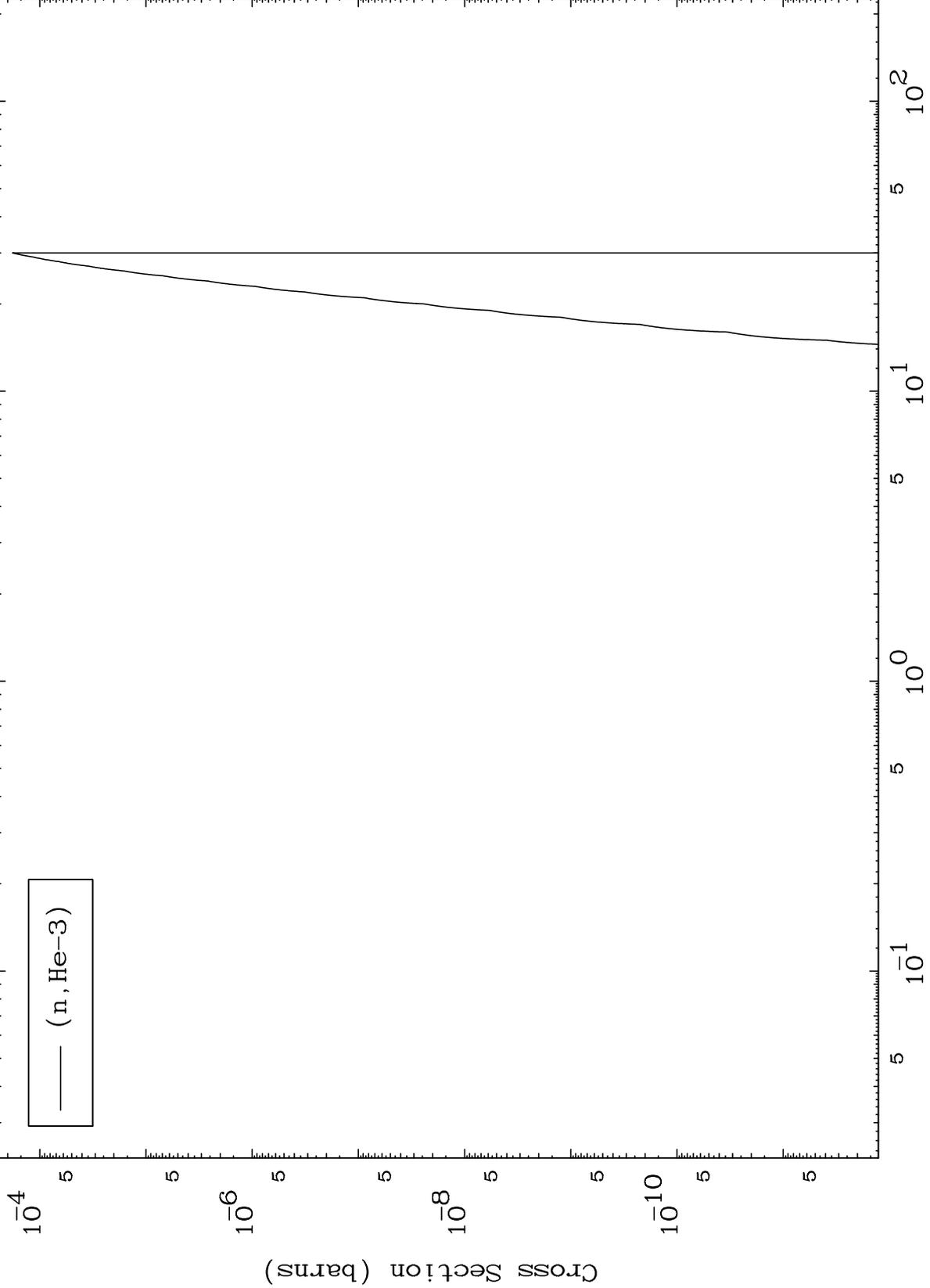
Incident Energy (MeV)

100-Fm-250

MAT 9931

(n,He3) Levels  
293 Kelvin Cross Sections

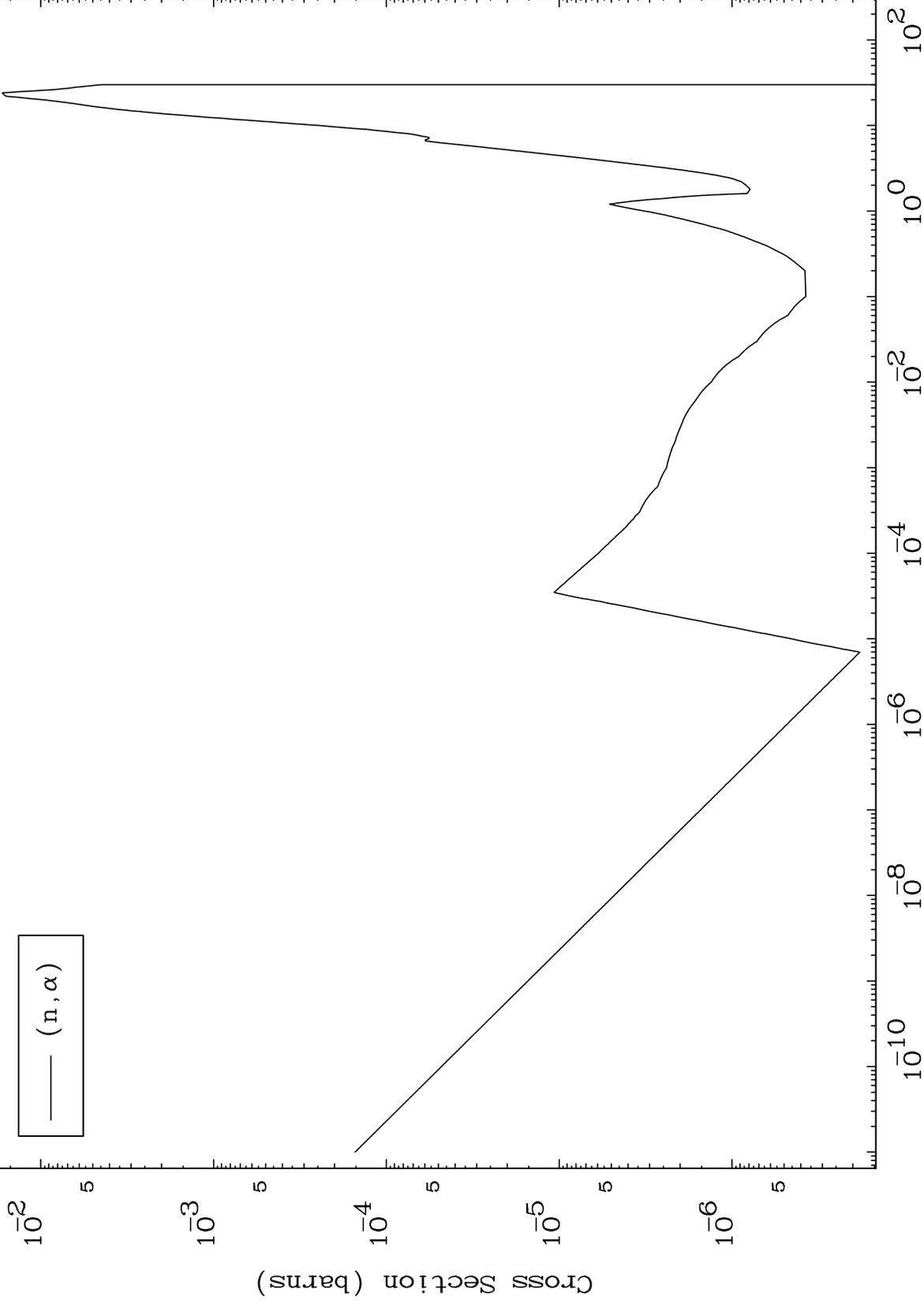
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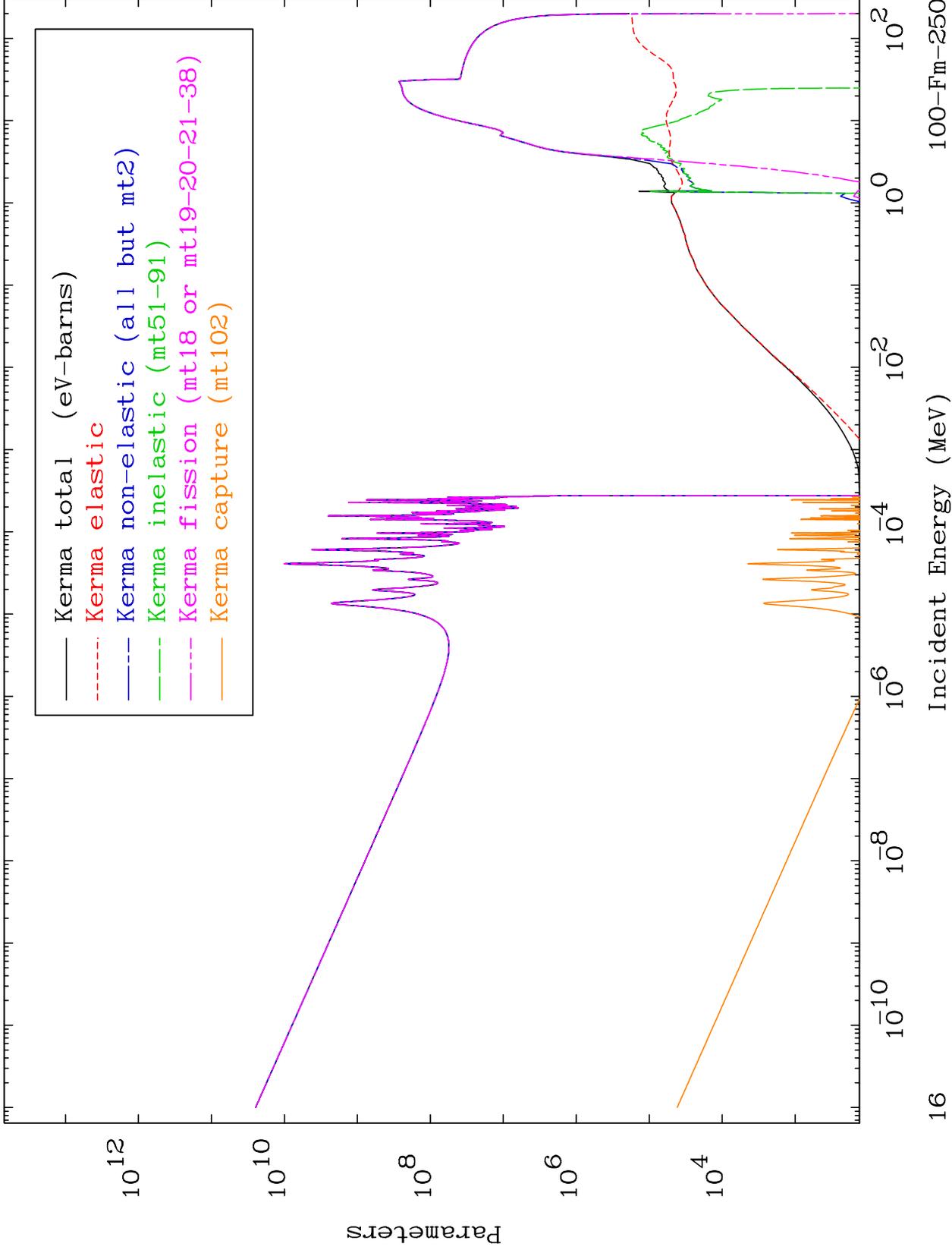


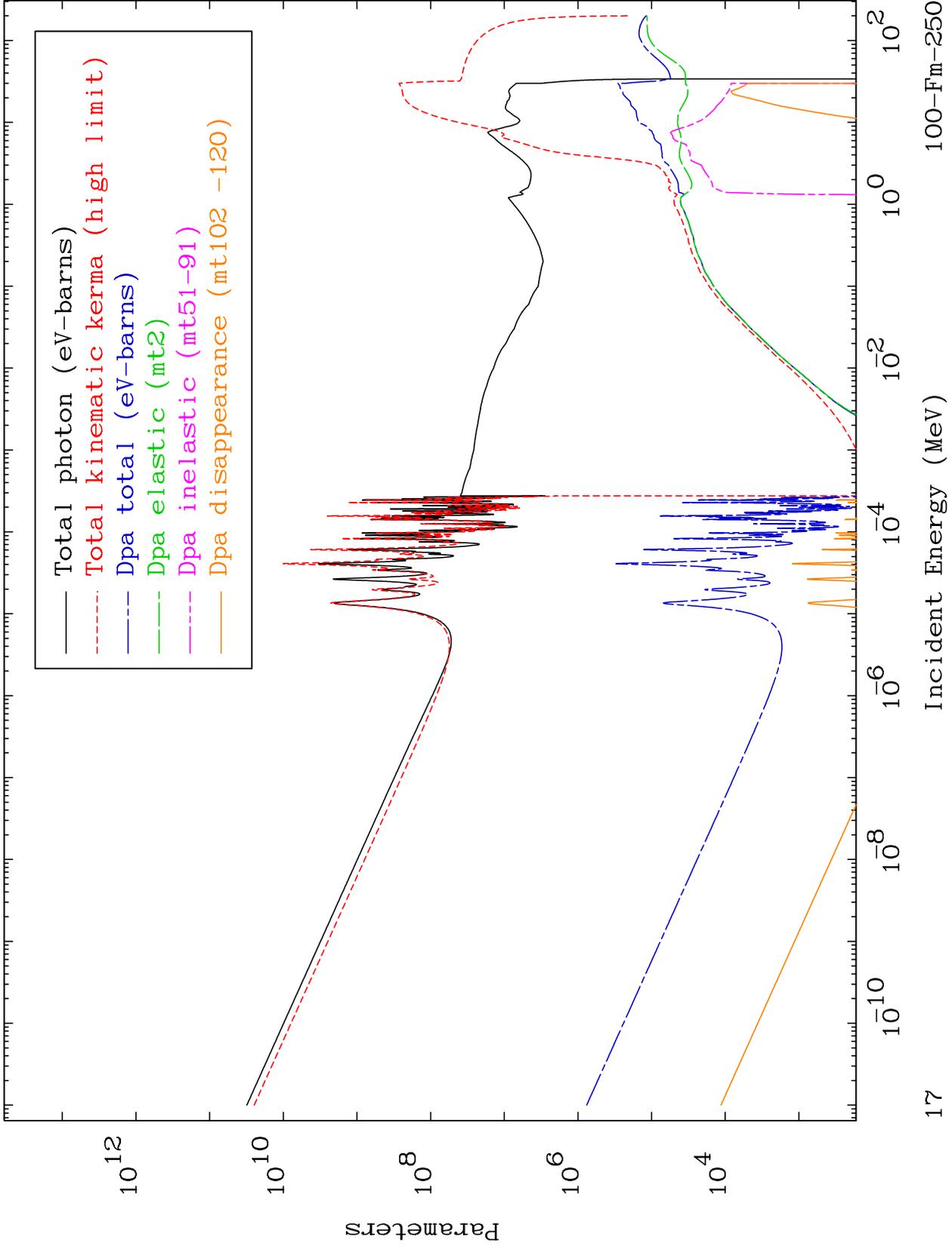
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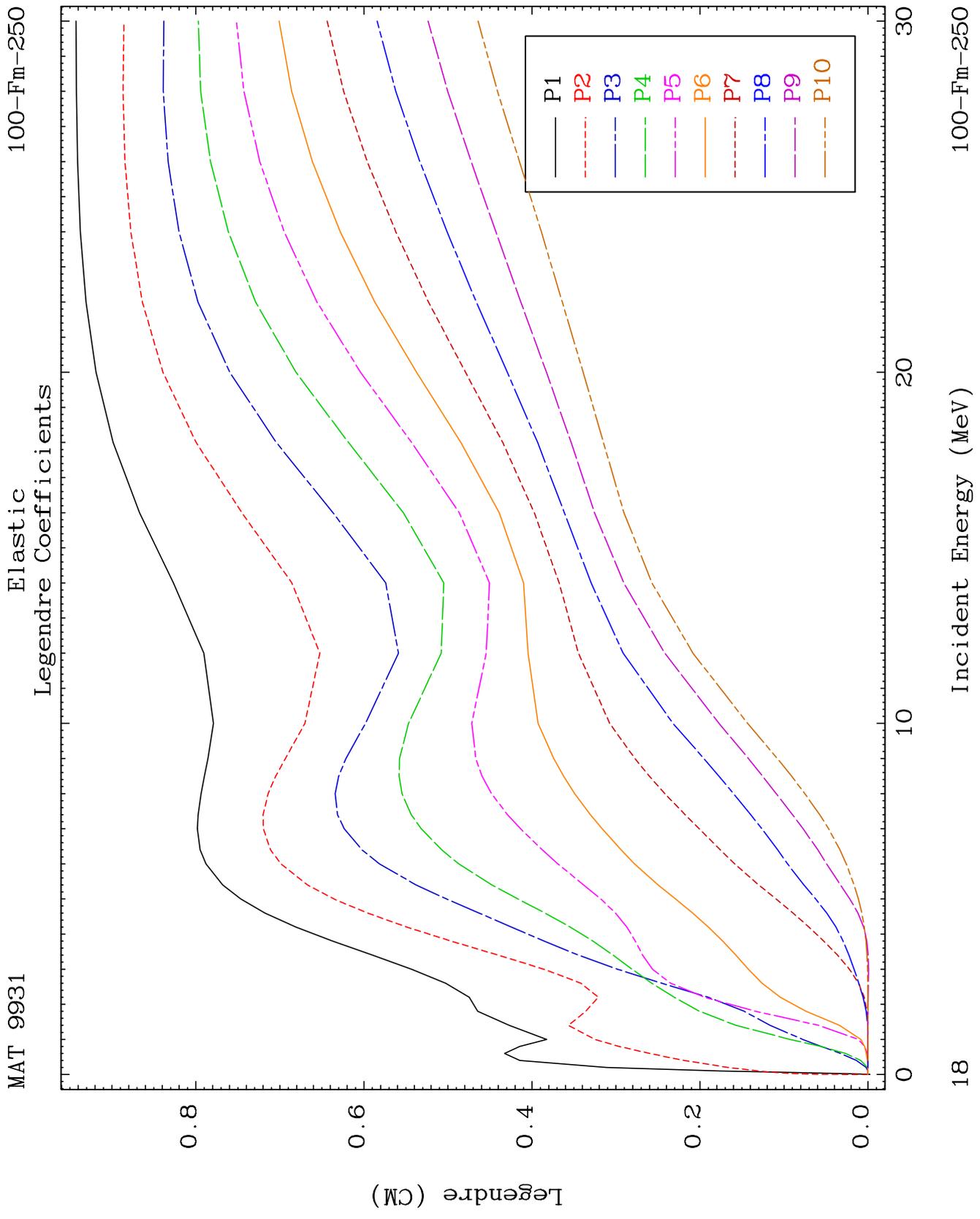
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

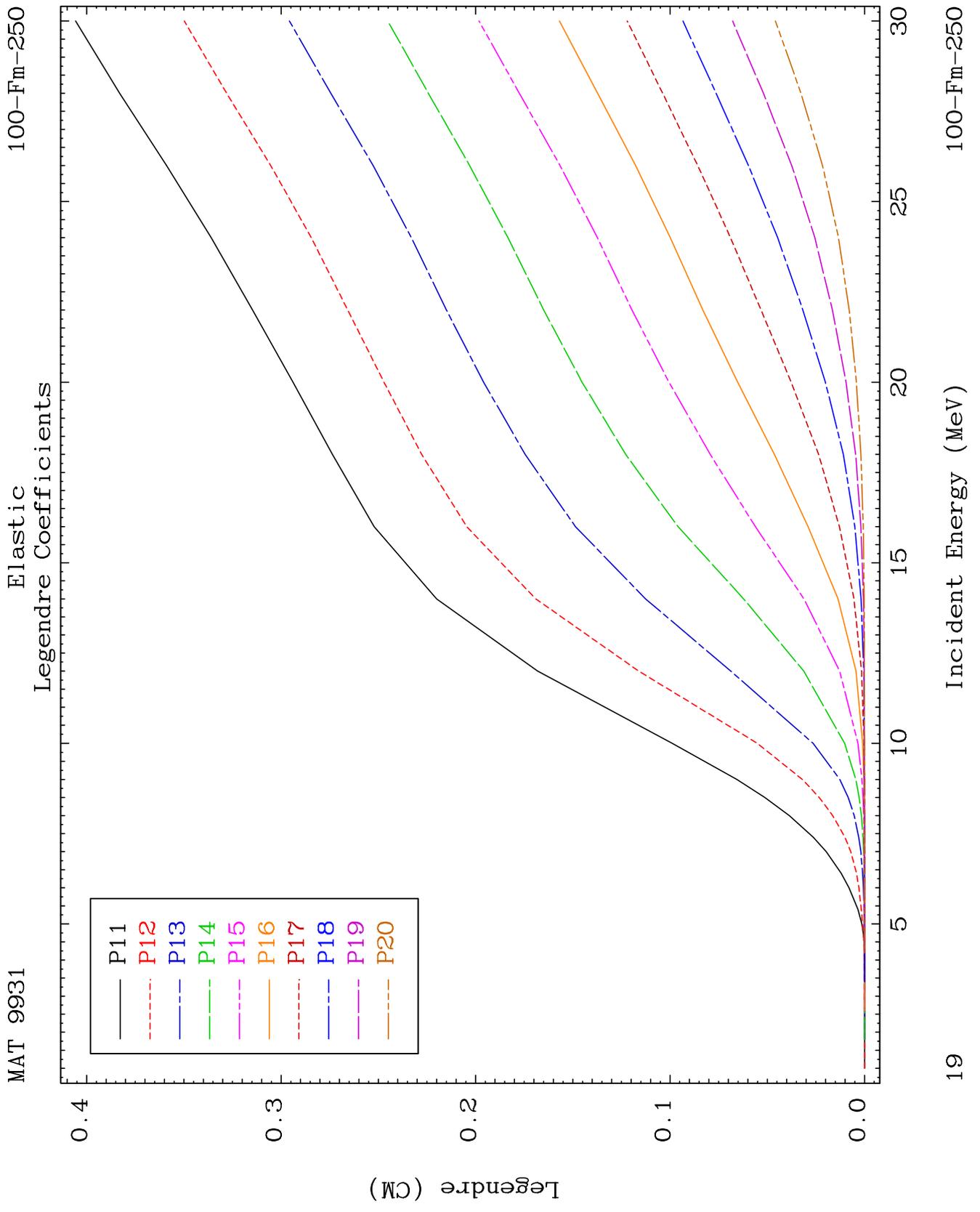
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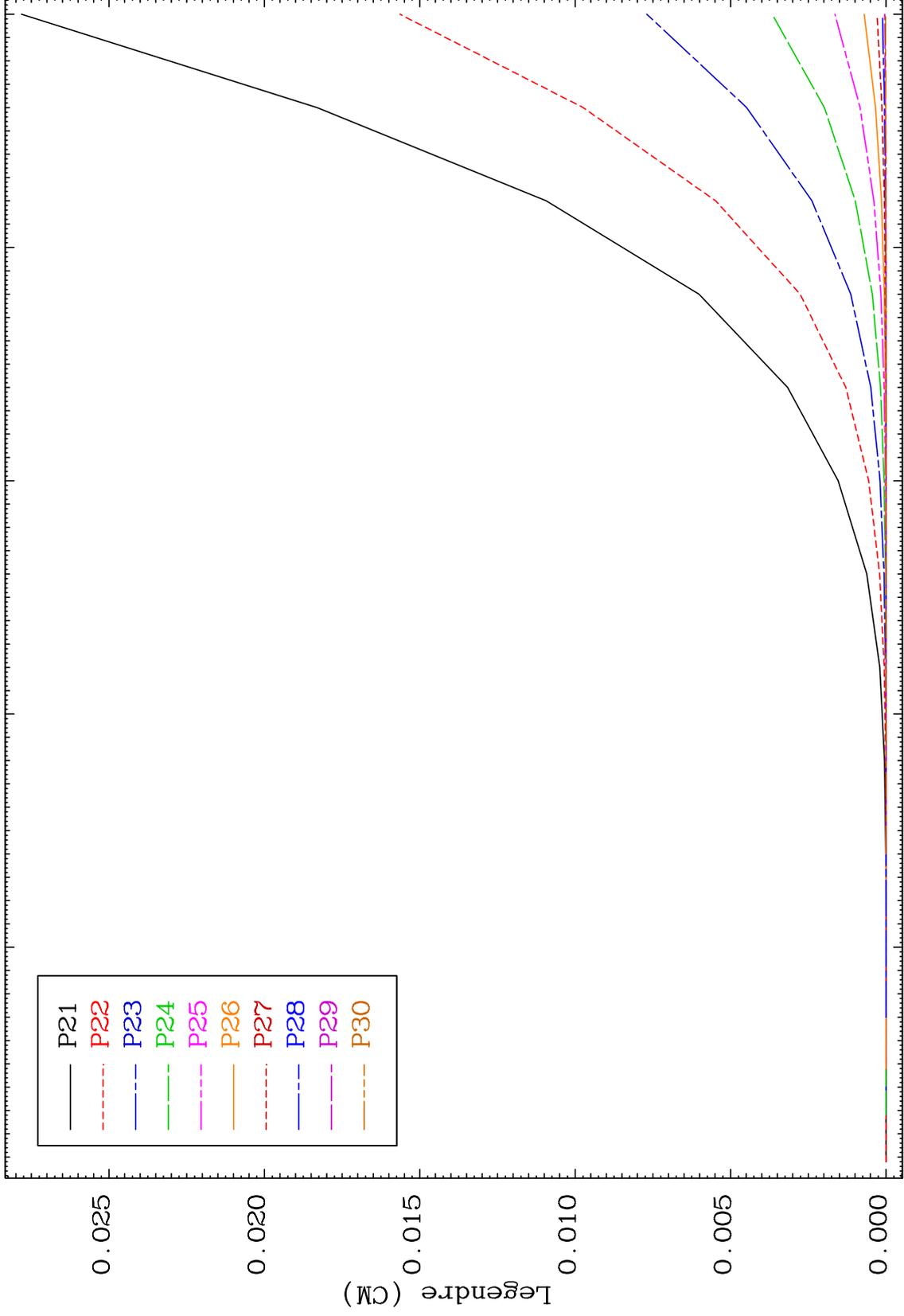




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Elastic  
Legendre Coefficients

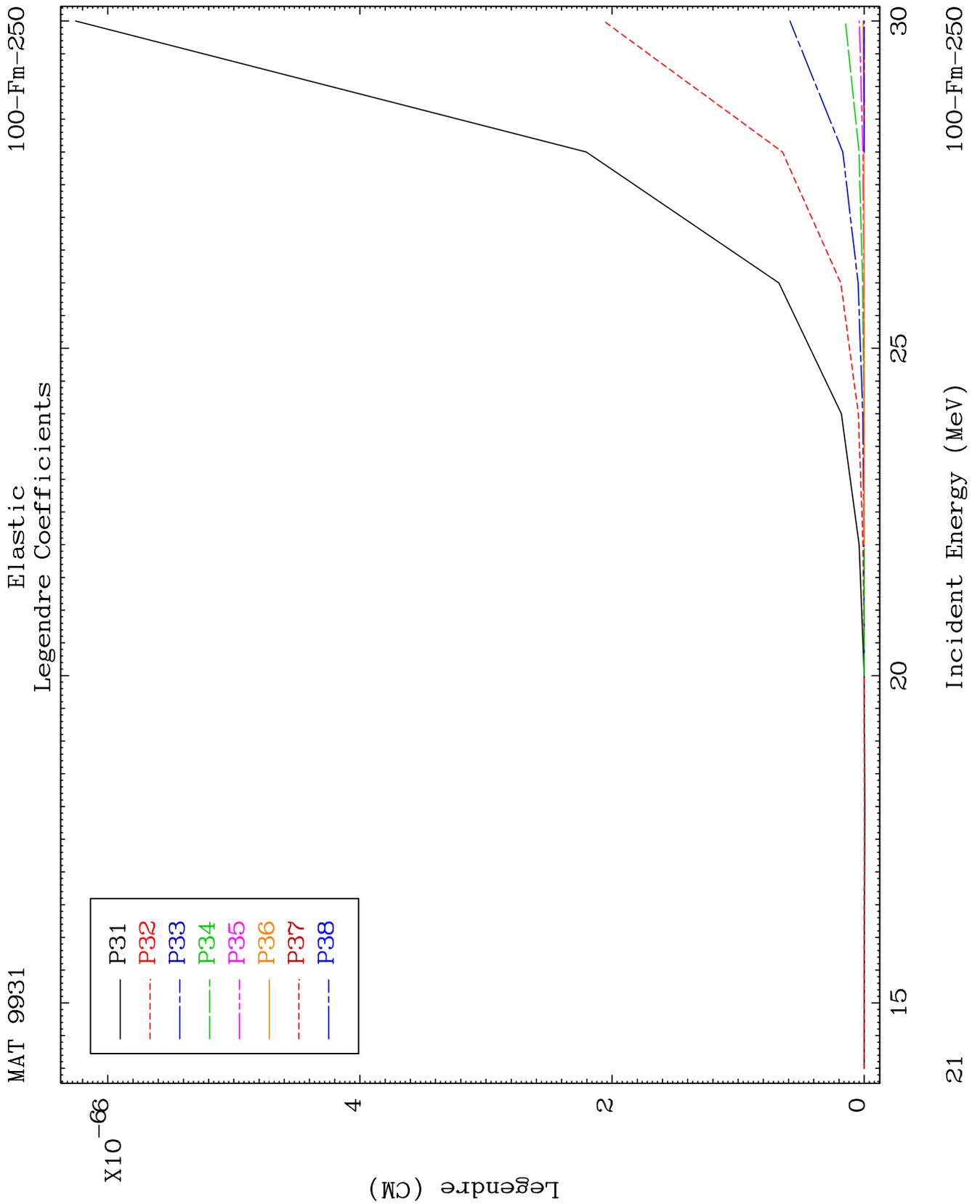
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20

Incident Energy (MeV)

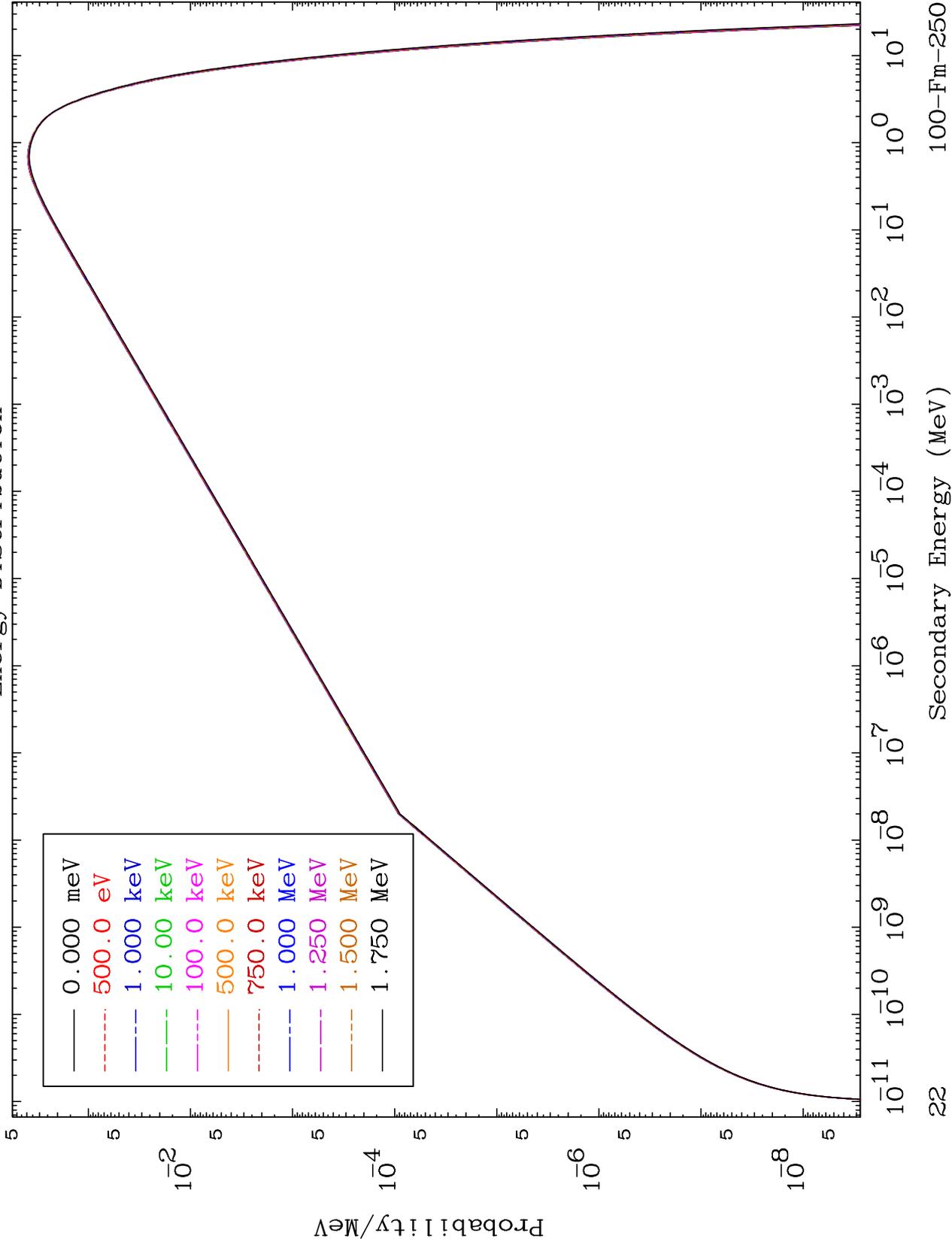
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### Fission Energy Distribution

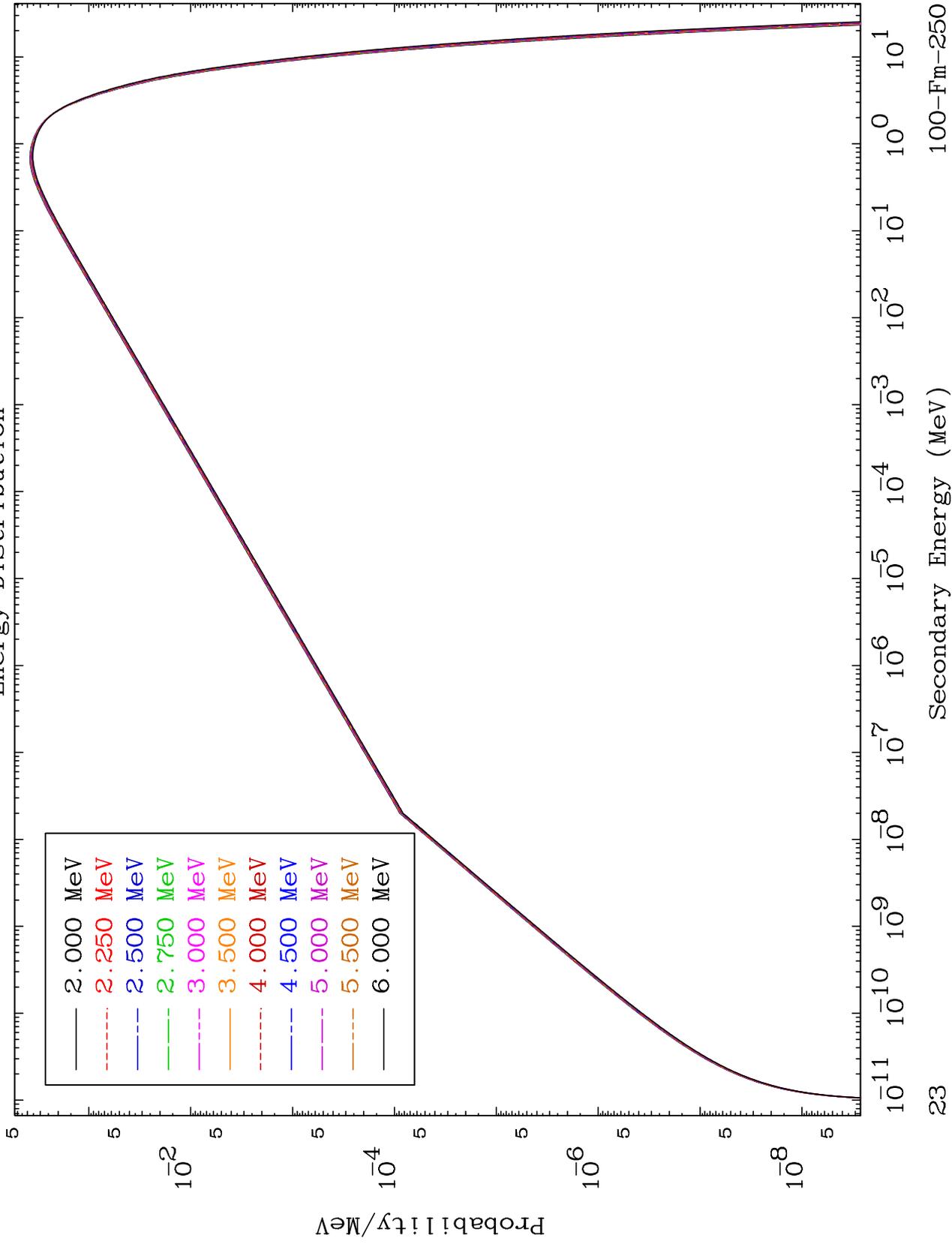
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### Fission Energy Distribution

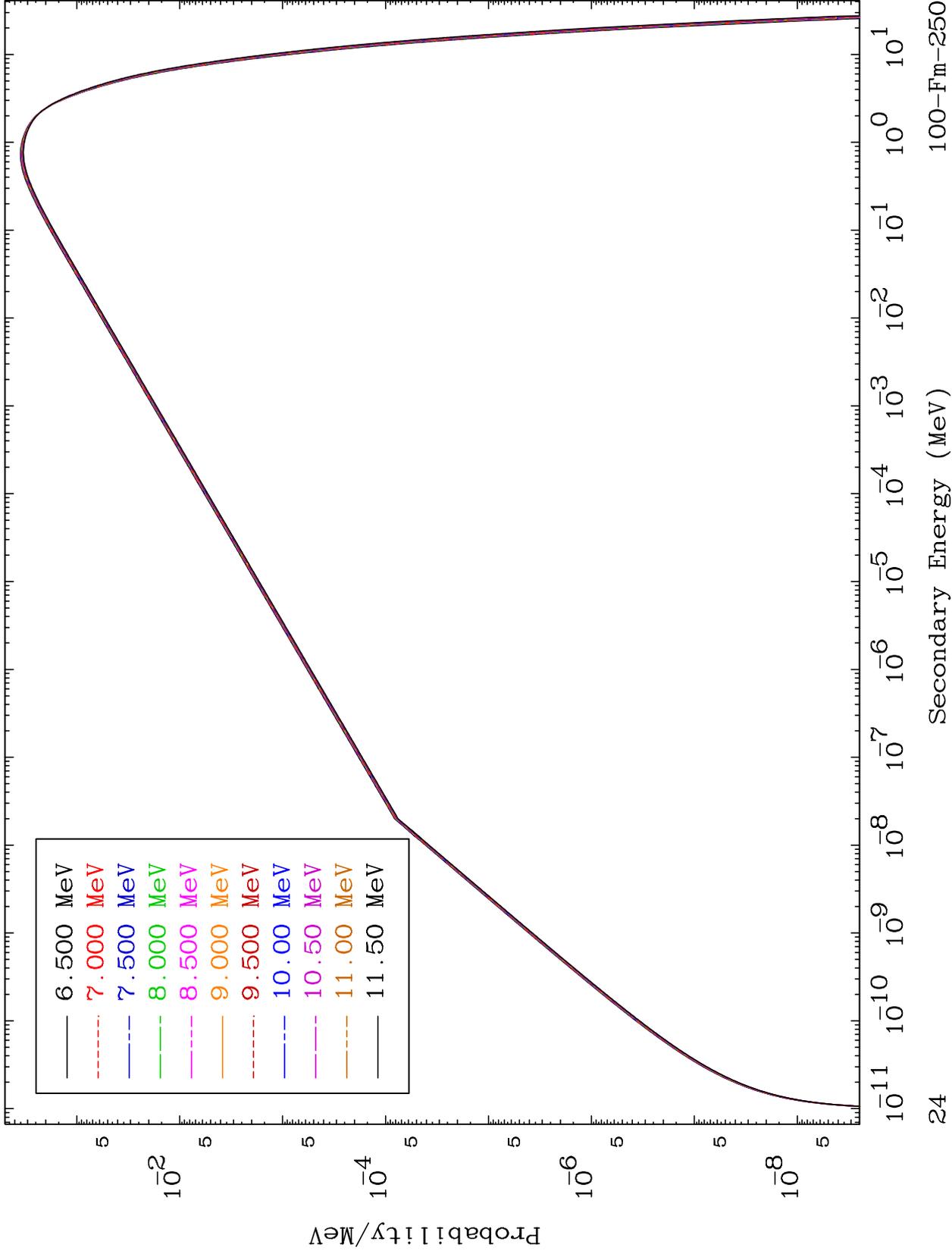
100-Fm-250



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### Fission Energy Distribution

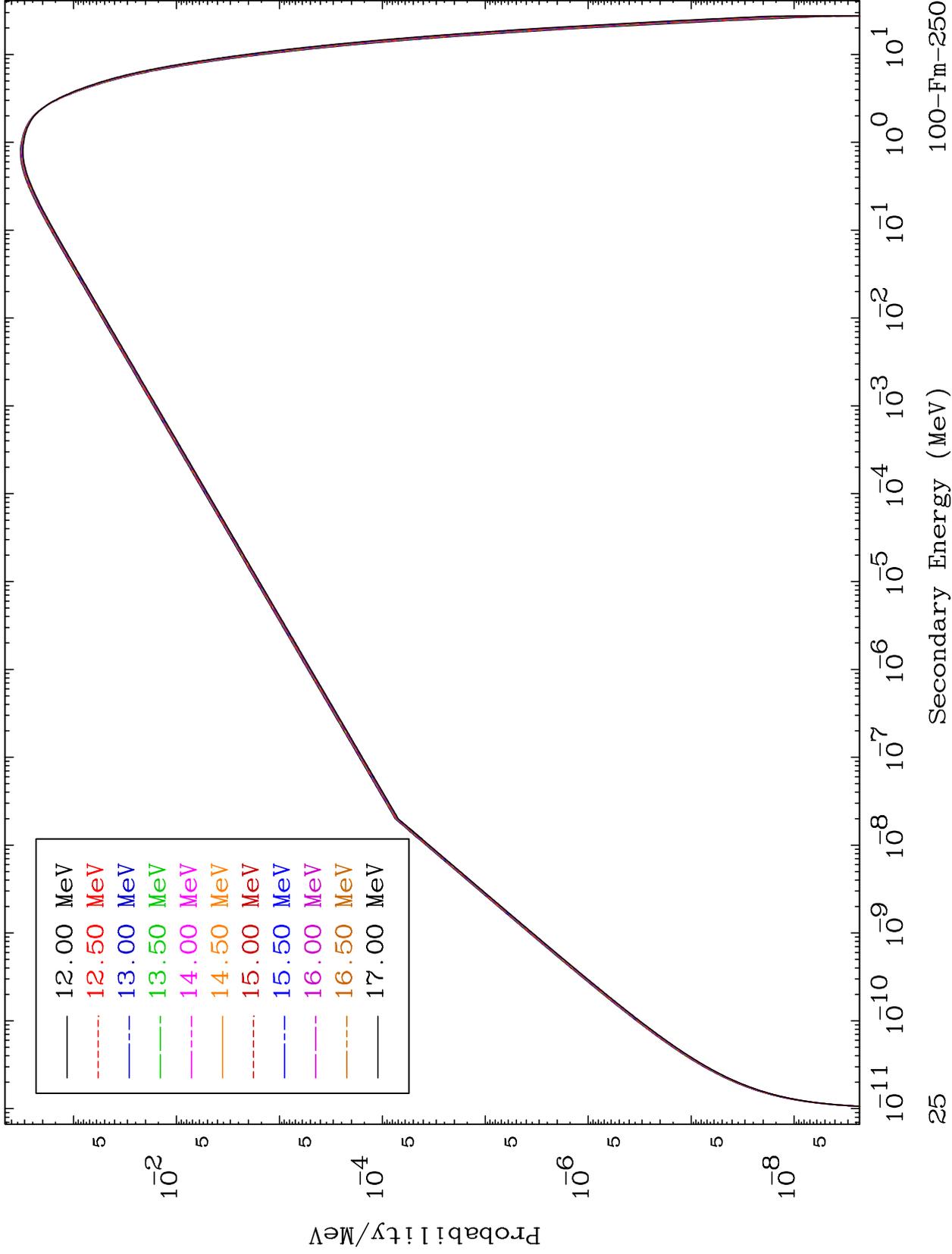
100-Fm-250



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### Fission Energy Distribution

100-Fm-250



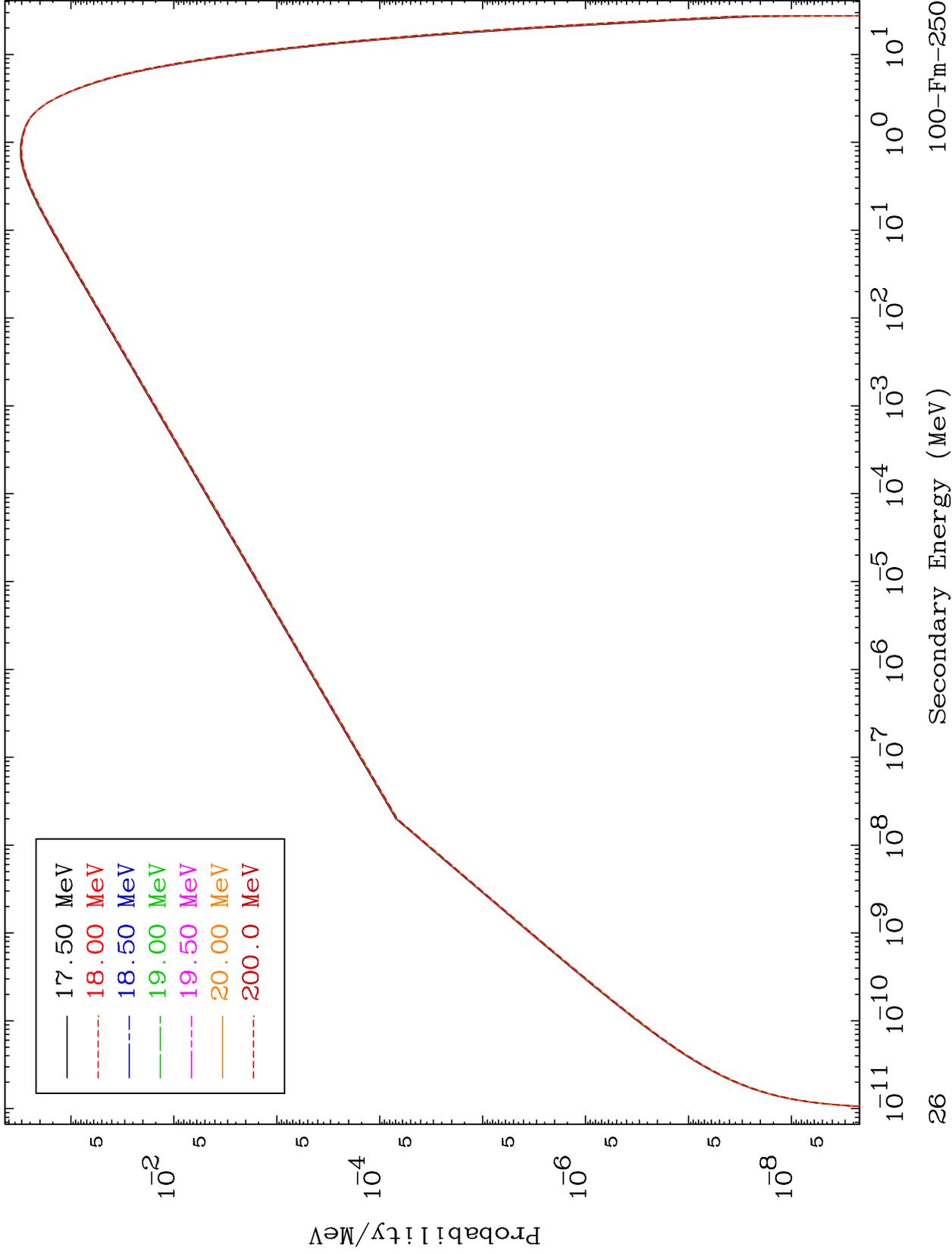
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100-Fm-250

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### Fission Energy Distribution

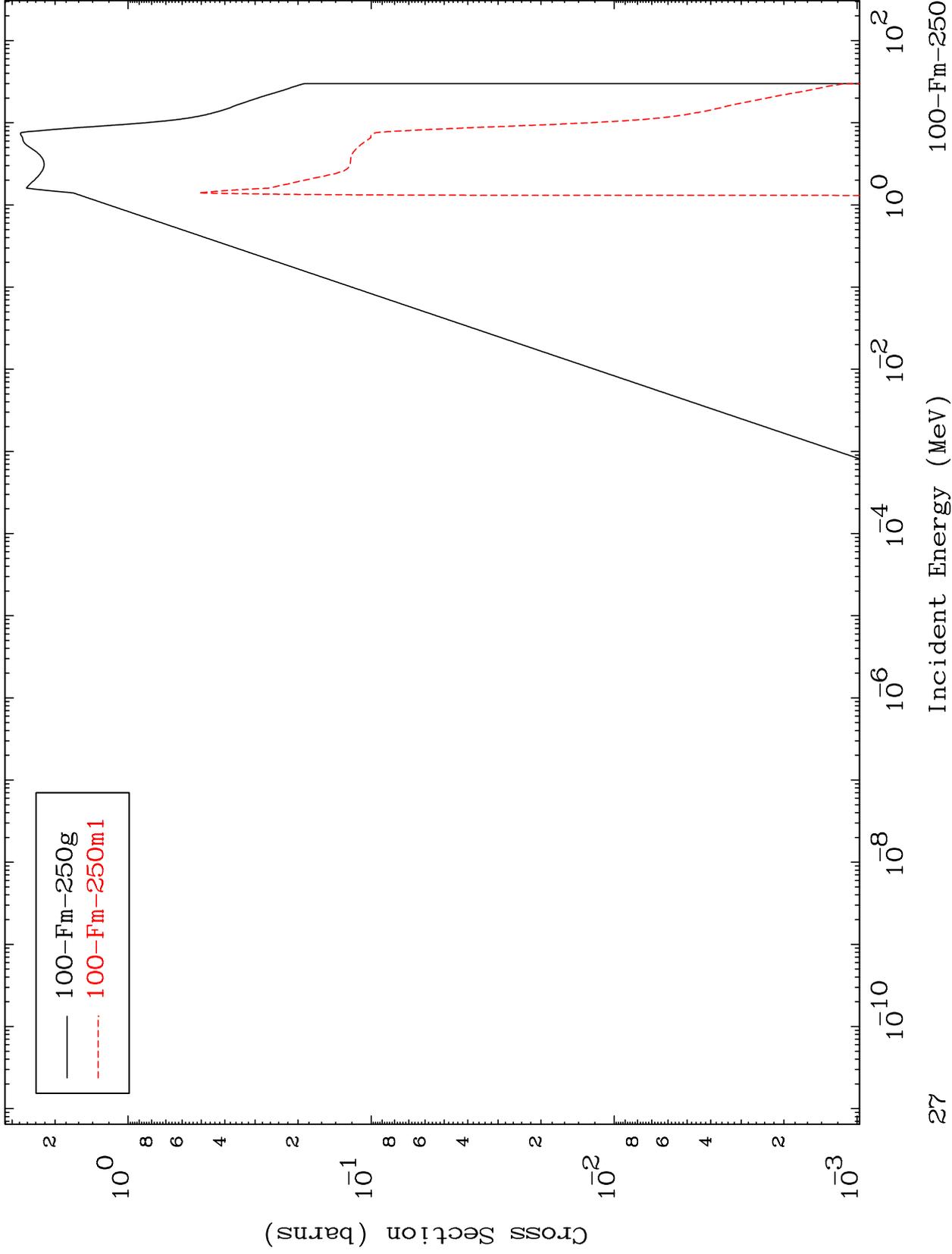
100-Fm-250



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Inelastic  
Radionuclide Production Cross Section

100-Fm-250

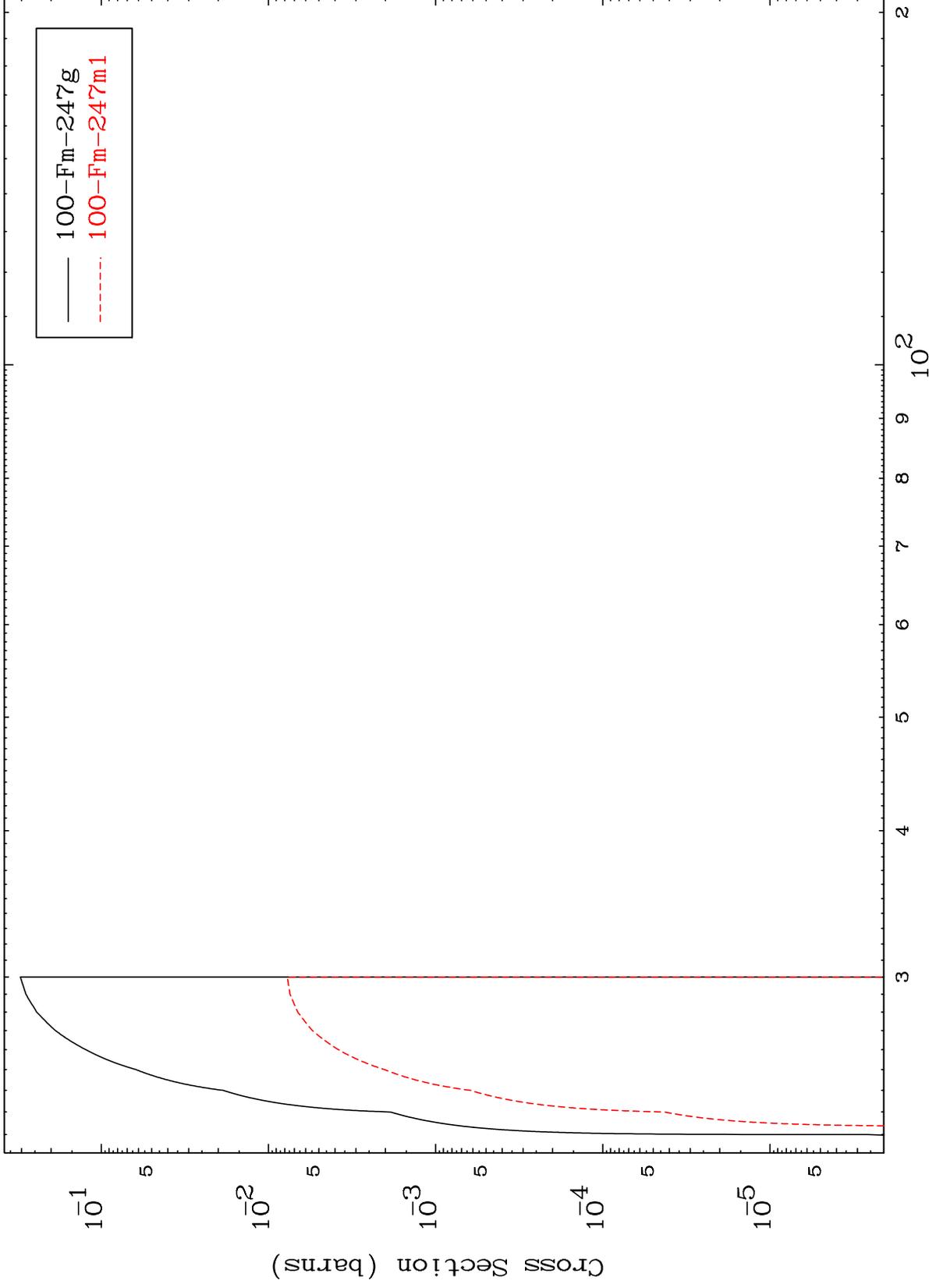


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(n,4n)

100-Fm-250

Radionuclide Production Cross Section



28

Incident Energy (MeV)

100-Fm-250

MAT 9931

100-Fm-250

(n,p)  
Radionuclide Production Cross Section

