

Program EVALPLOT
(Version 2018-1)

by

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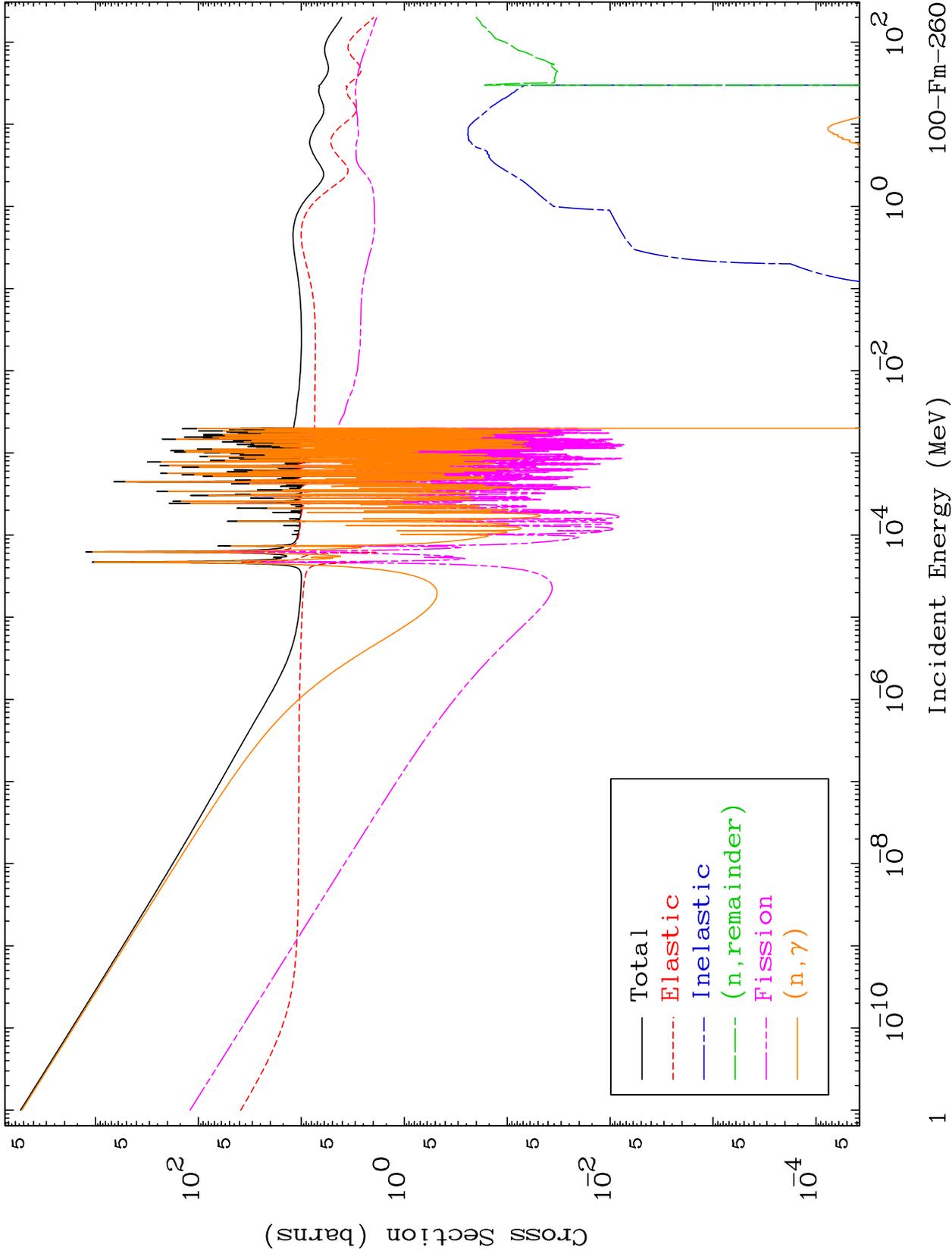
Press Mouse Button to Start

MAT 9941

Major

293 Kelvin Cross Sections

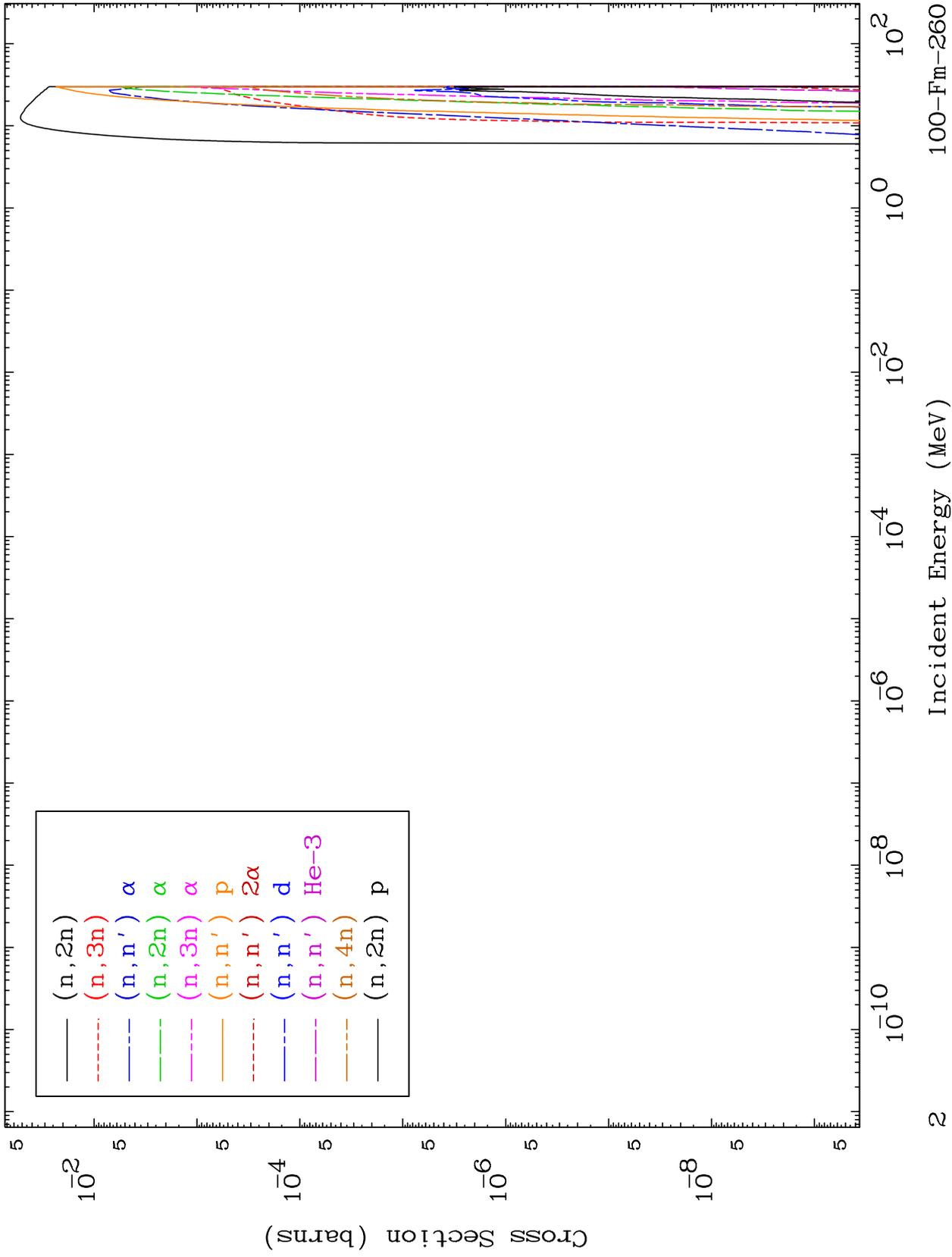
100-Fm-260



MAT 9941

Neutron Production
293 Kelvin Cross Sections

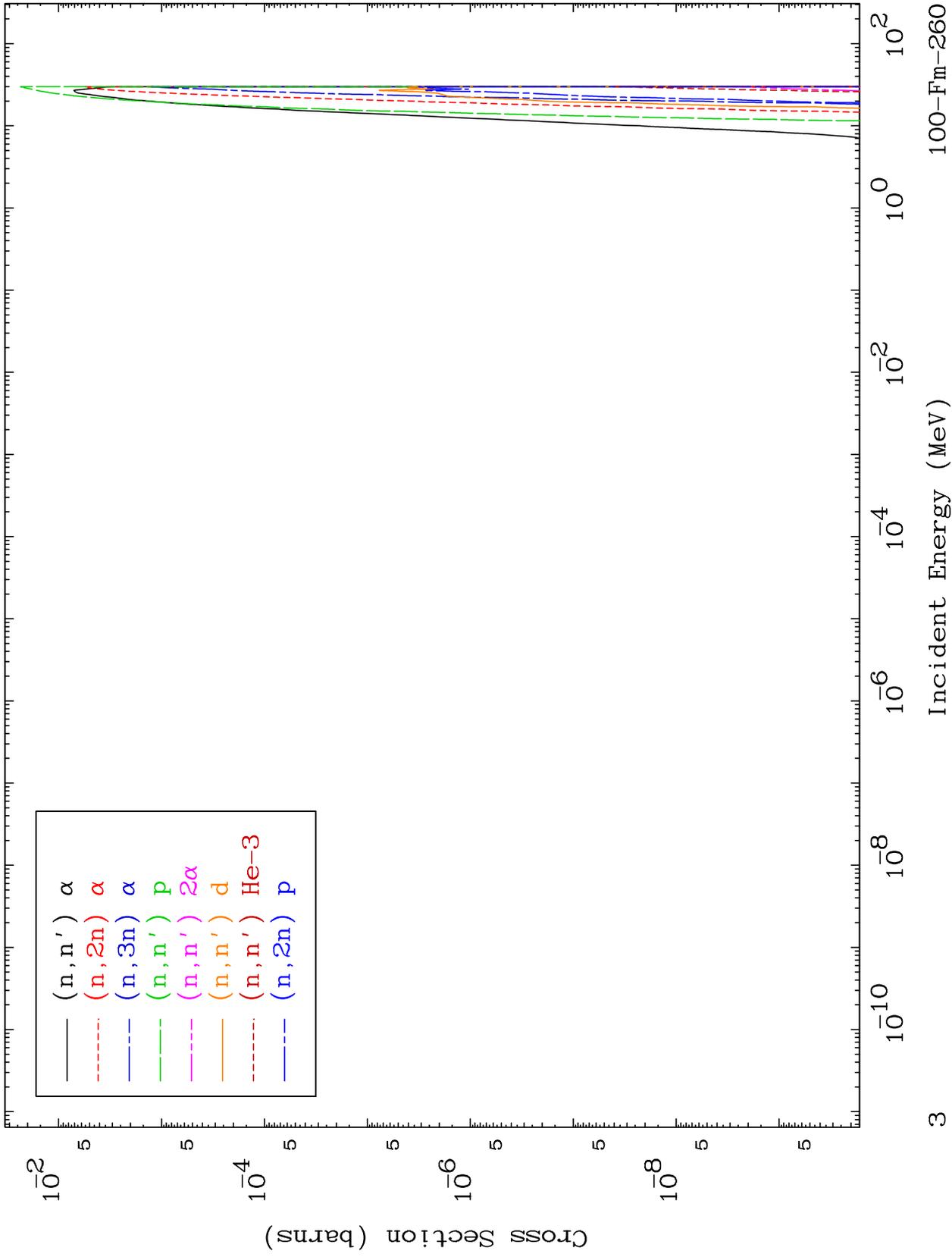
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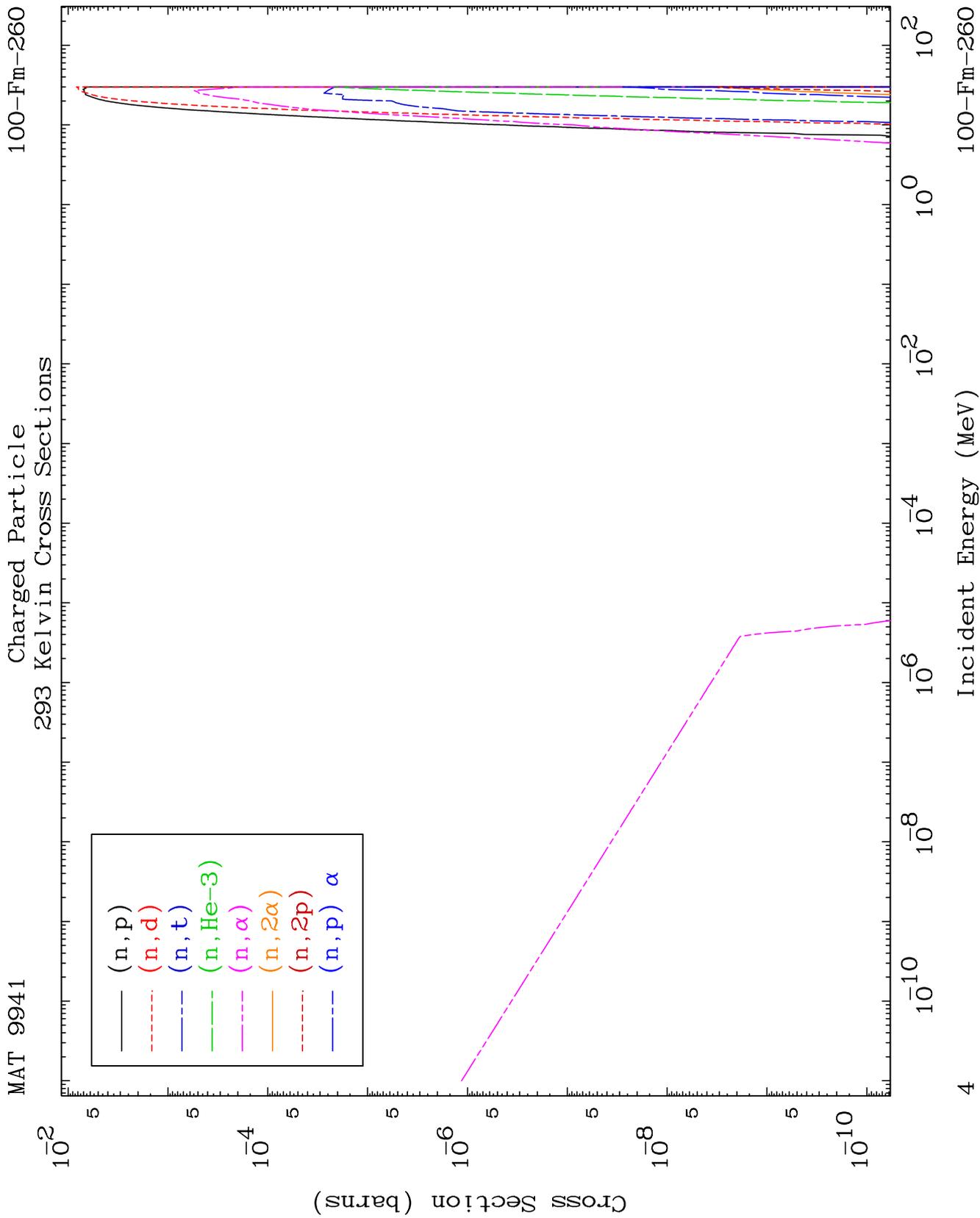


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Charged Particle
293 Kelvin Cross Sections

100-Fm-260

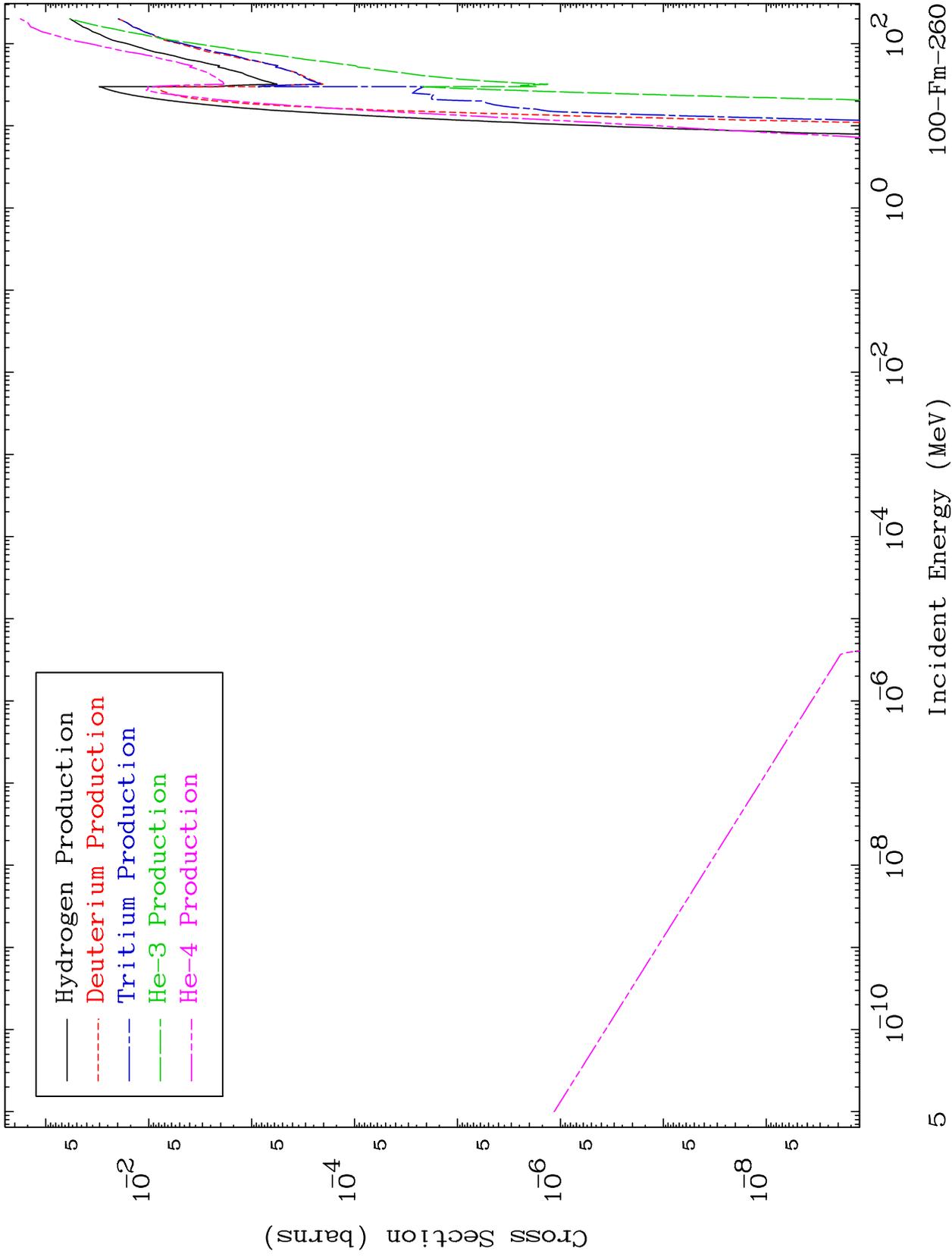




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Particle Production
293 Kelvin Cross Sections

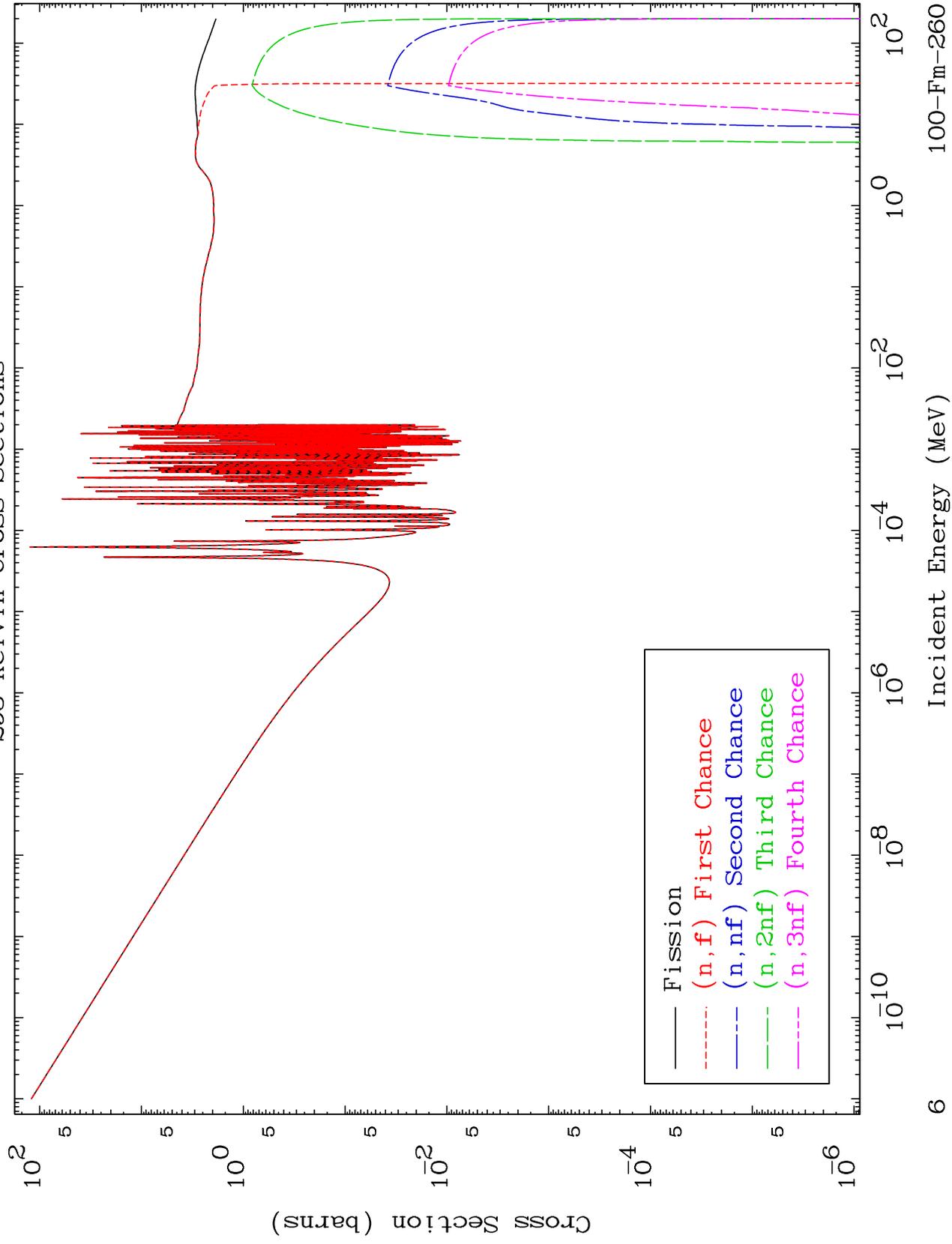
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Fission
293 Kelvin Cross Sections

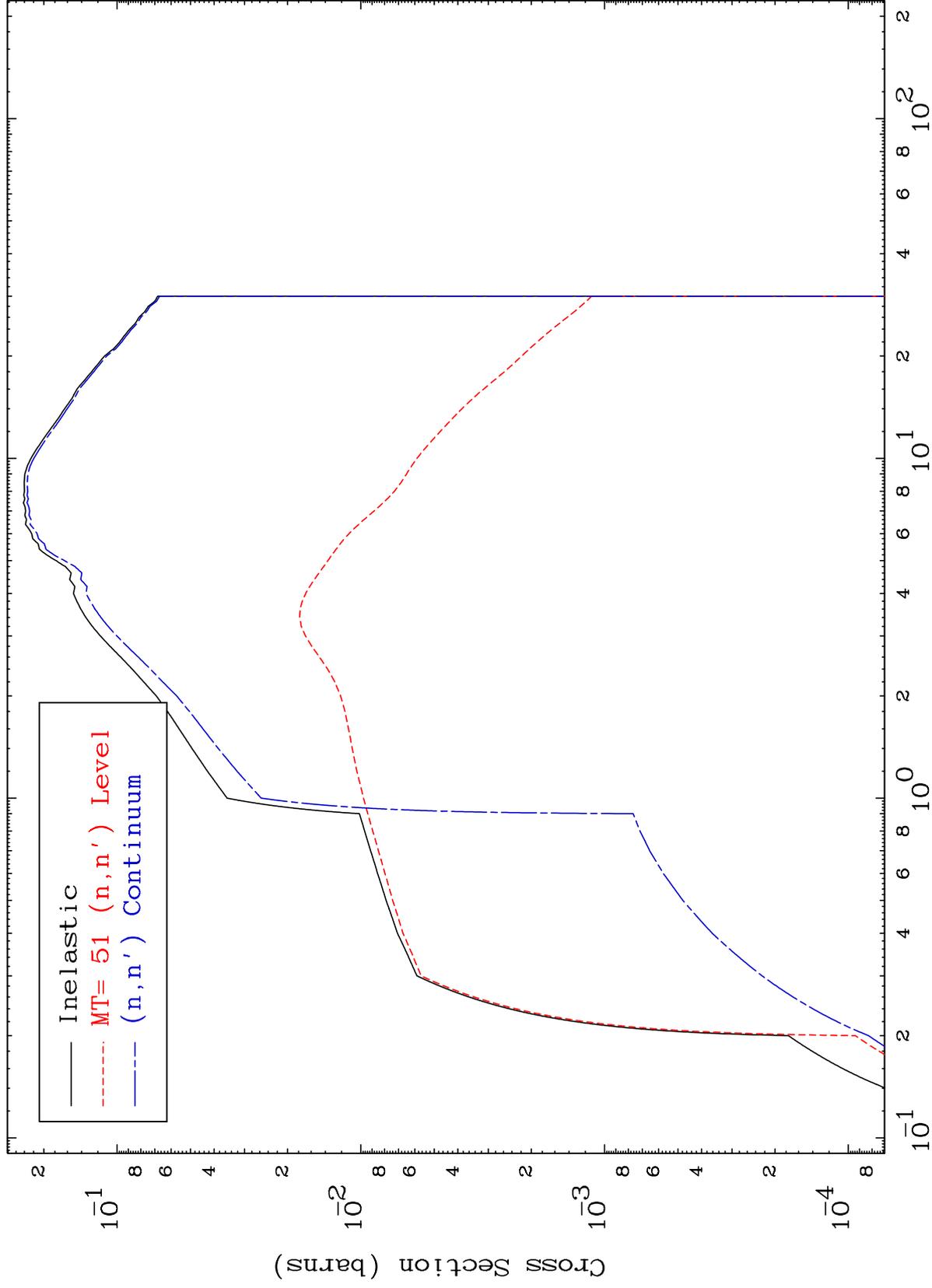
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293 Kelvin Cross Sections
(n,n') Level

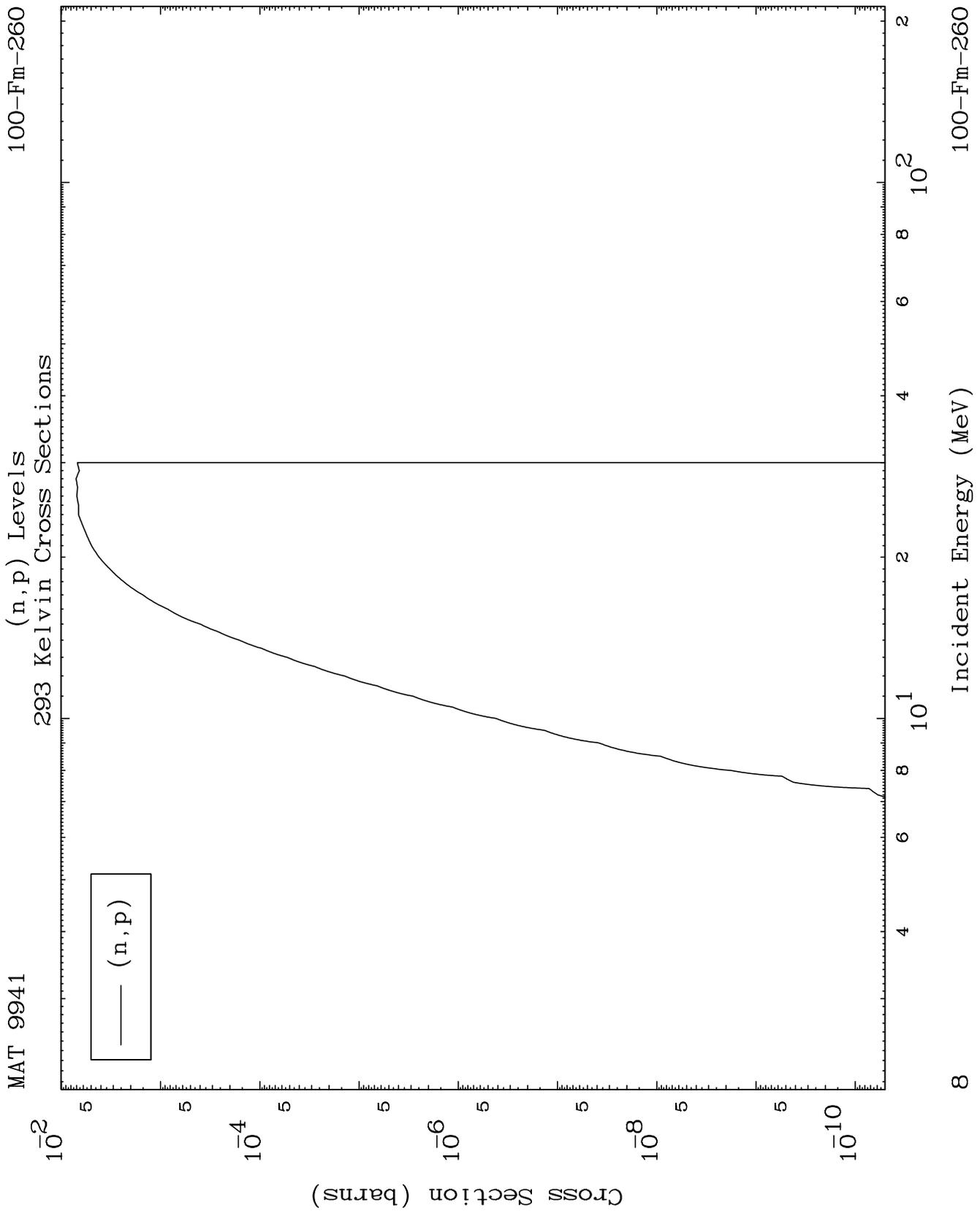
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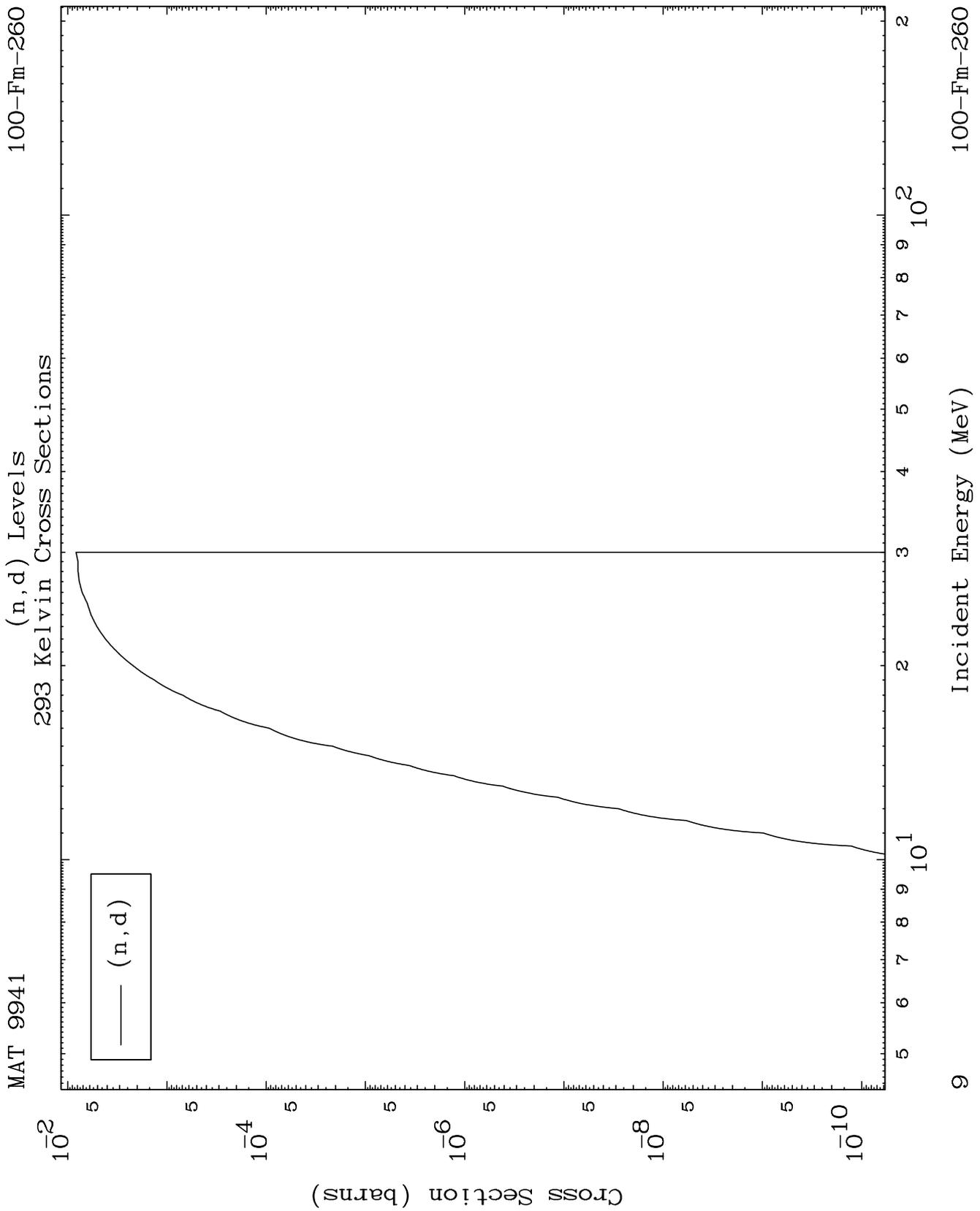


Incident Energy (MeV)

100-Fm-260

7

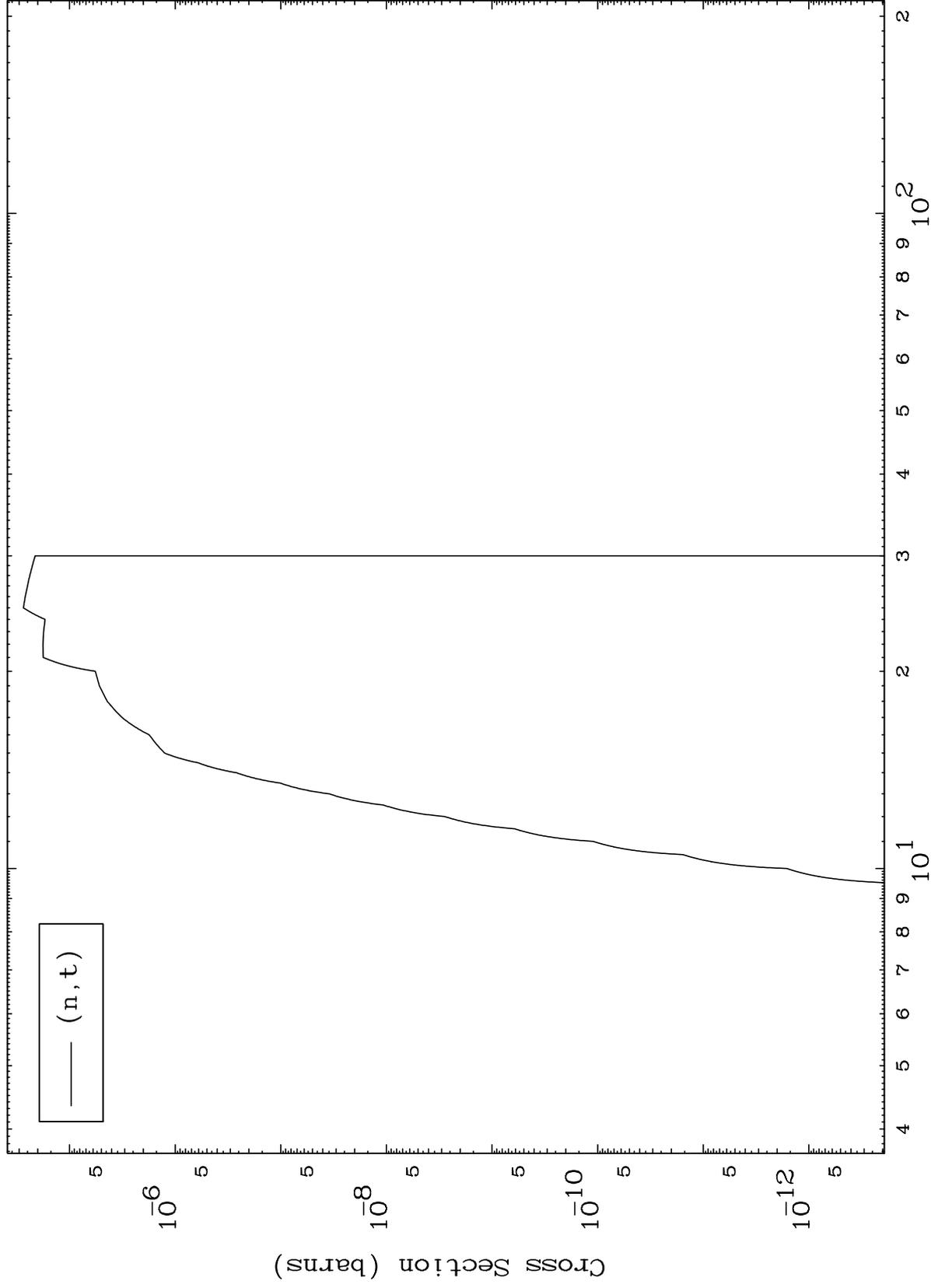




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(n,t) Levels
293 Kelvin Cross Sections

100-Fm-260



10

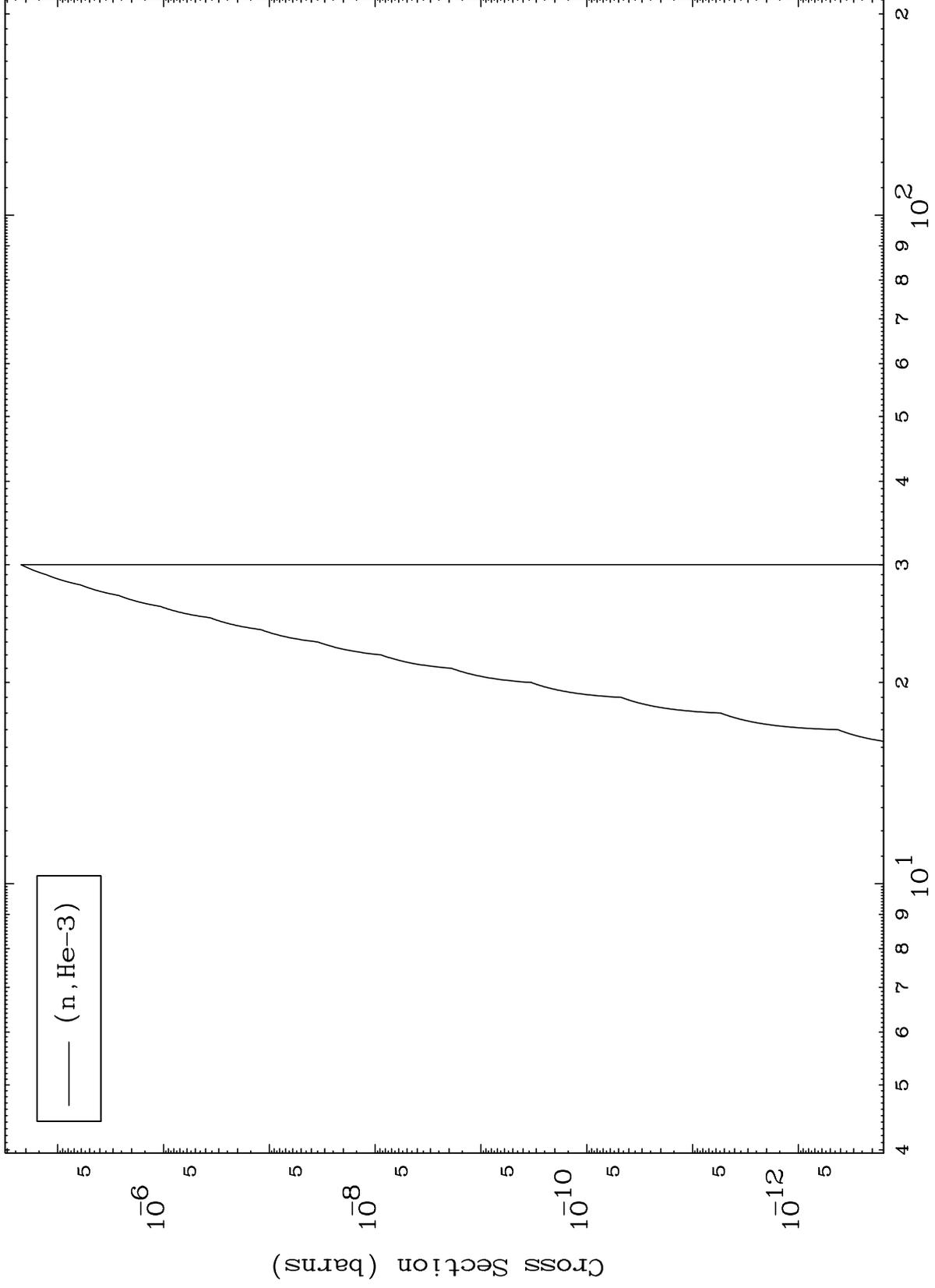
Incident Energy (MeV)

100-Fm-260

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(n,He3) Levels
293 Kelvin Cross Sections

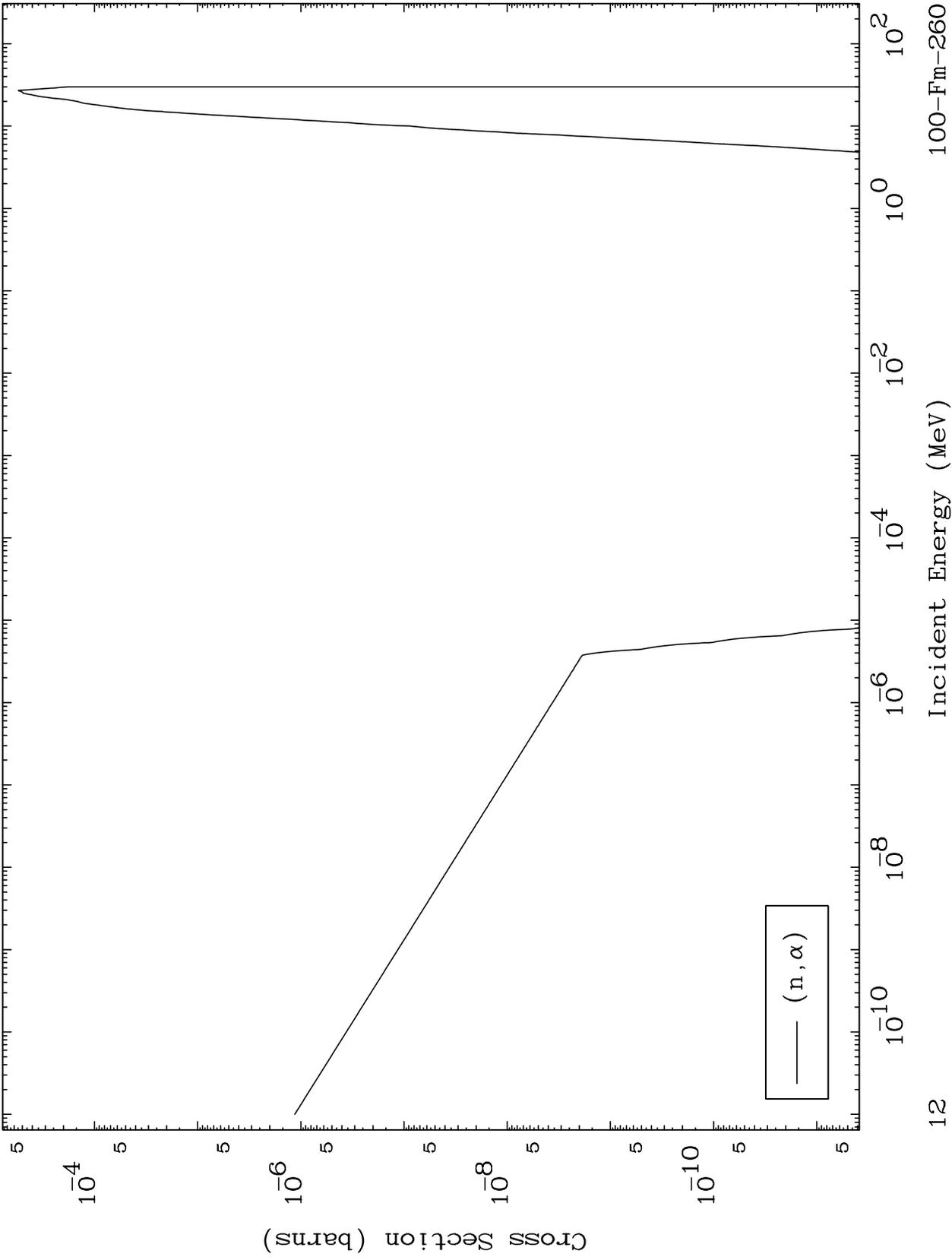
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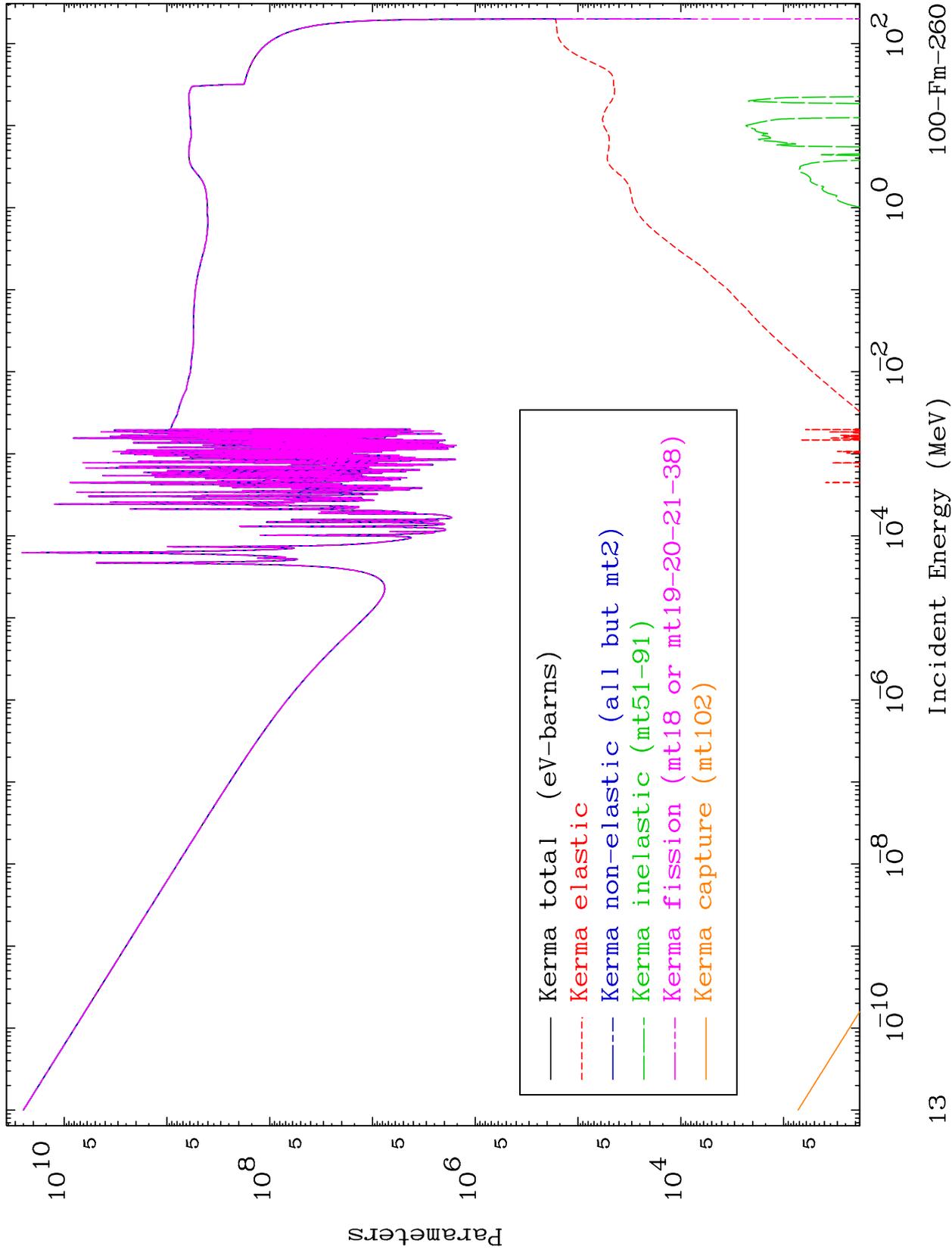


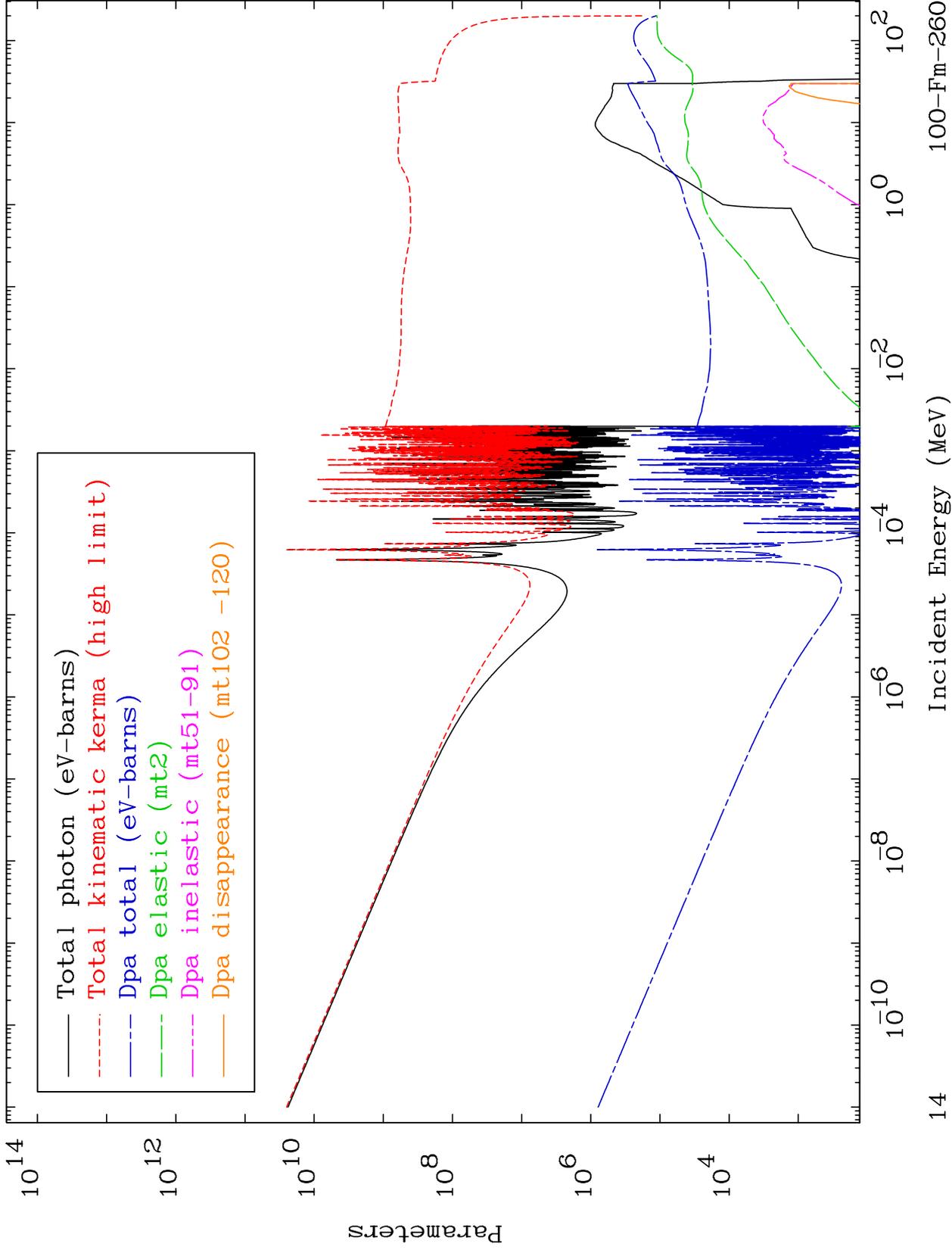
MAT 9941

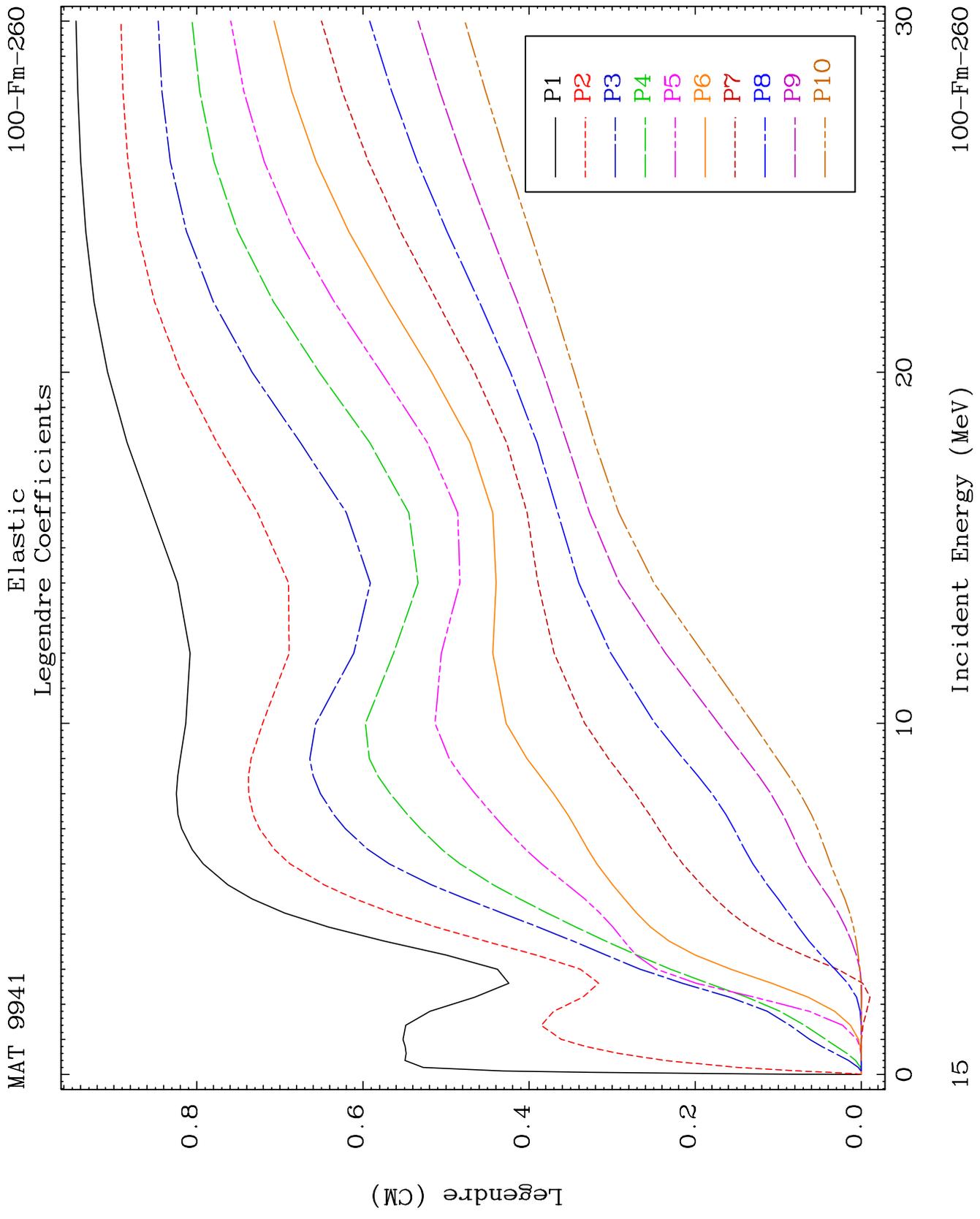
(n,α) Levels
293 Kelvin Cross Sections

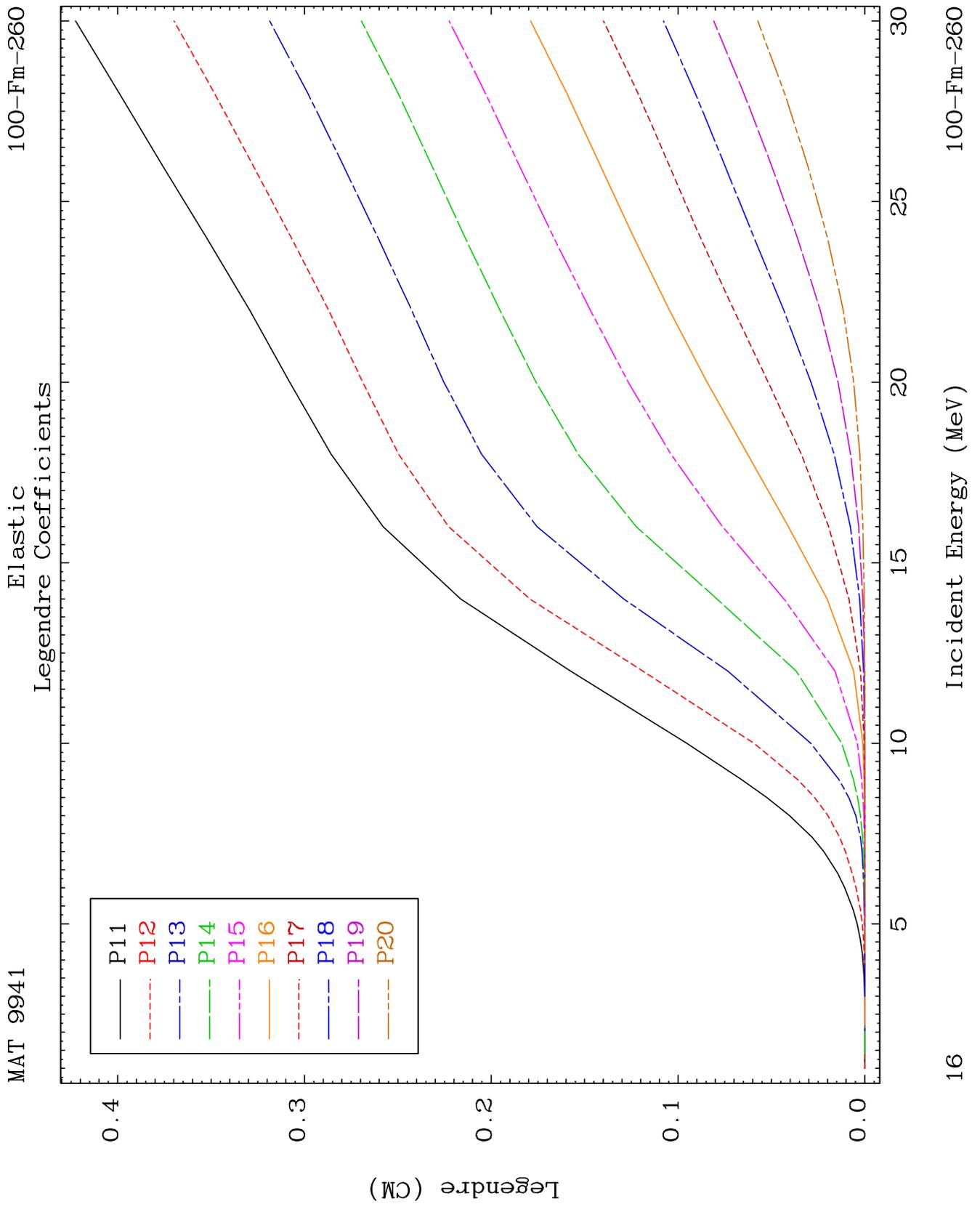
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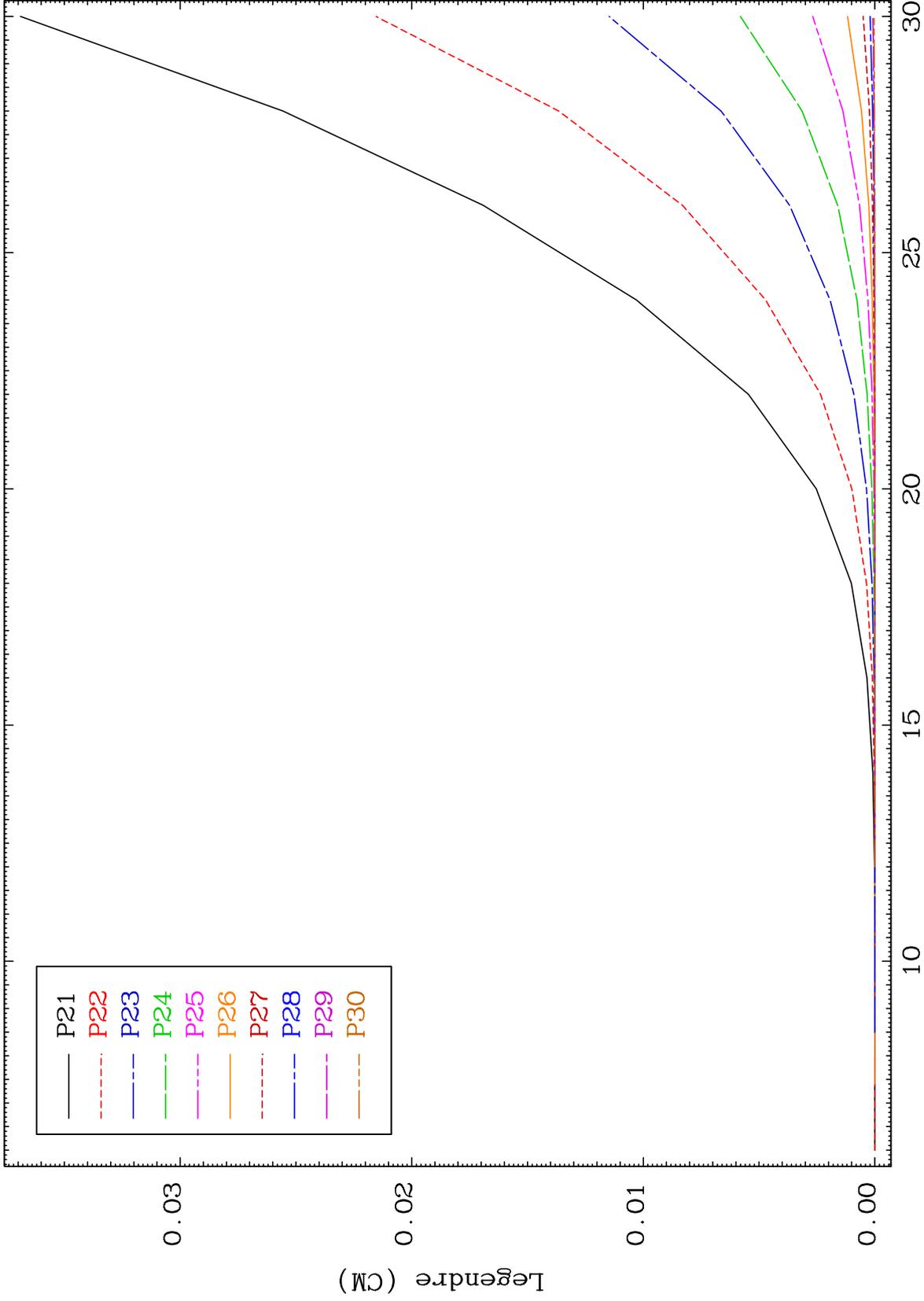




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Elastic
Legendre Coefficients

100-Fm-260



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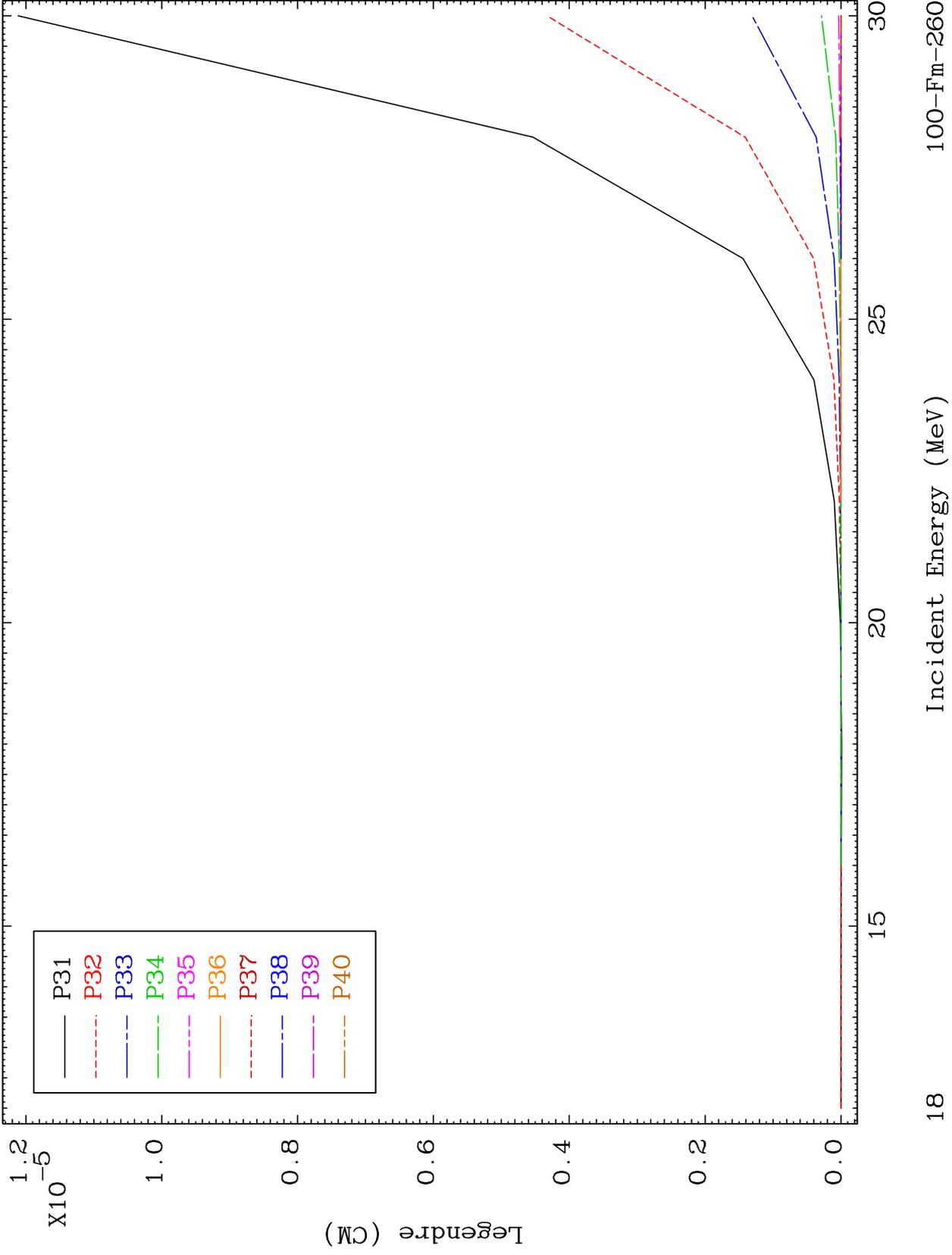
Incident Energy (MeV)

100-Fm-260

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Elastic
Legendre Coefficients

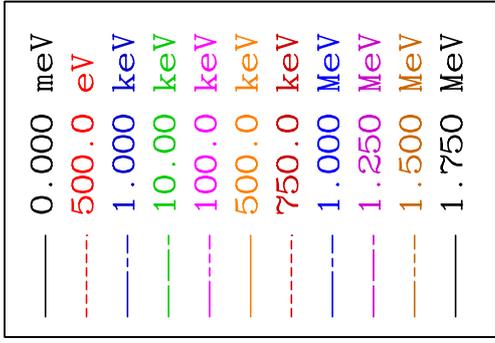
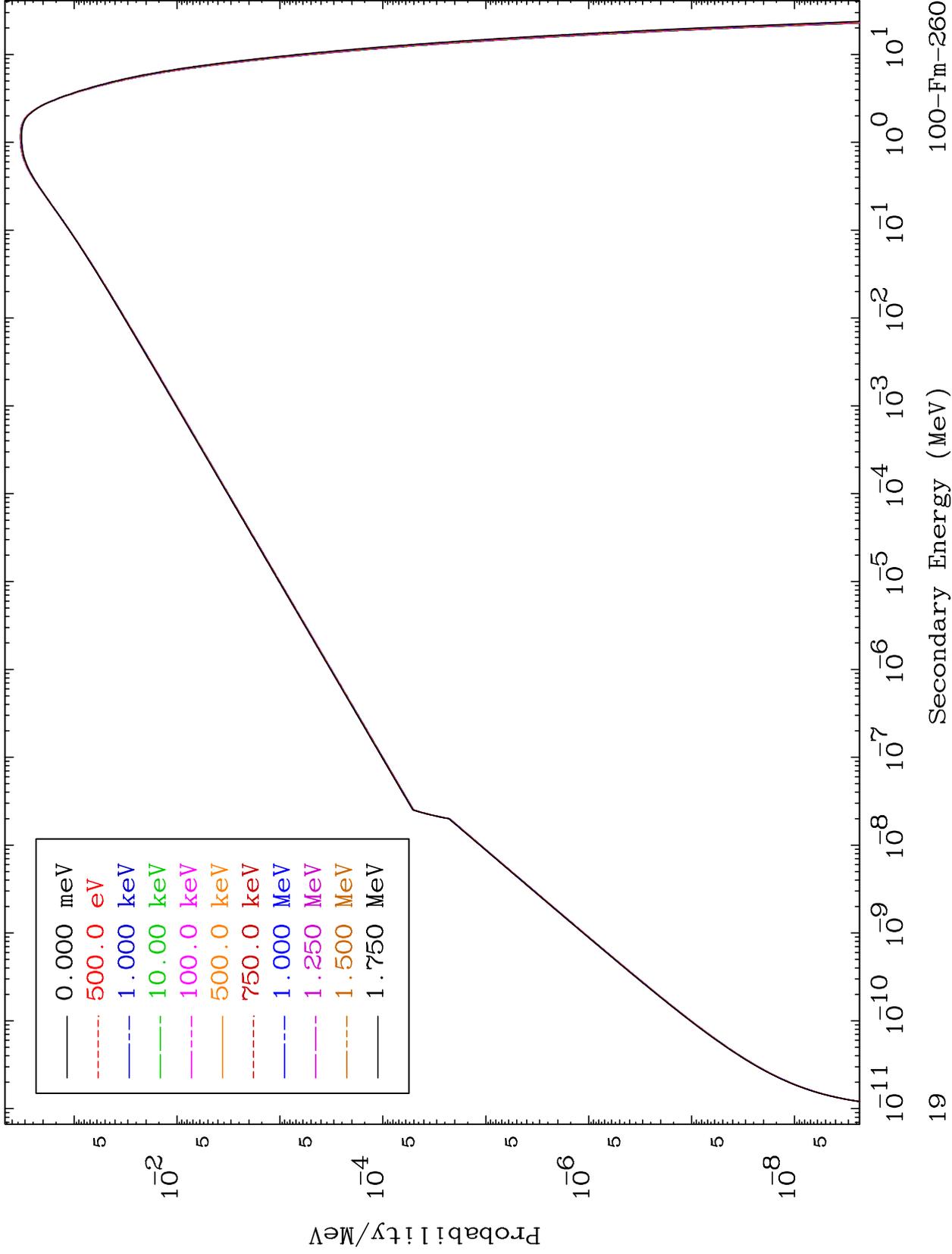
100-Fm-260



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Fission Energy Distribution

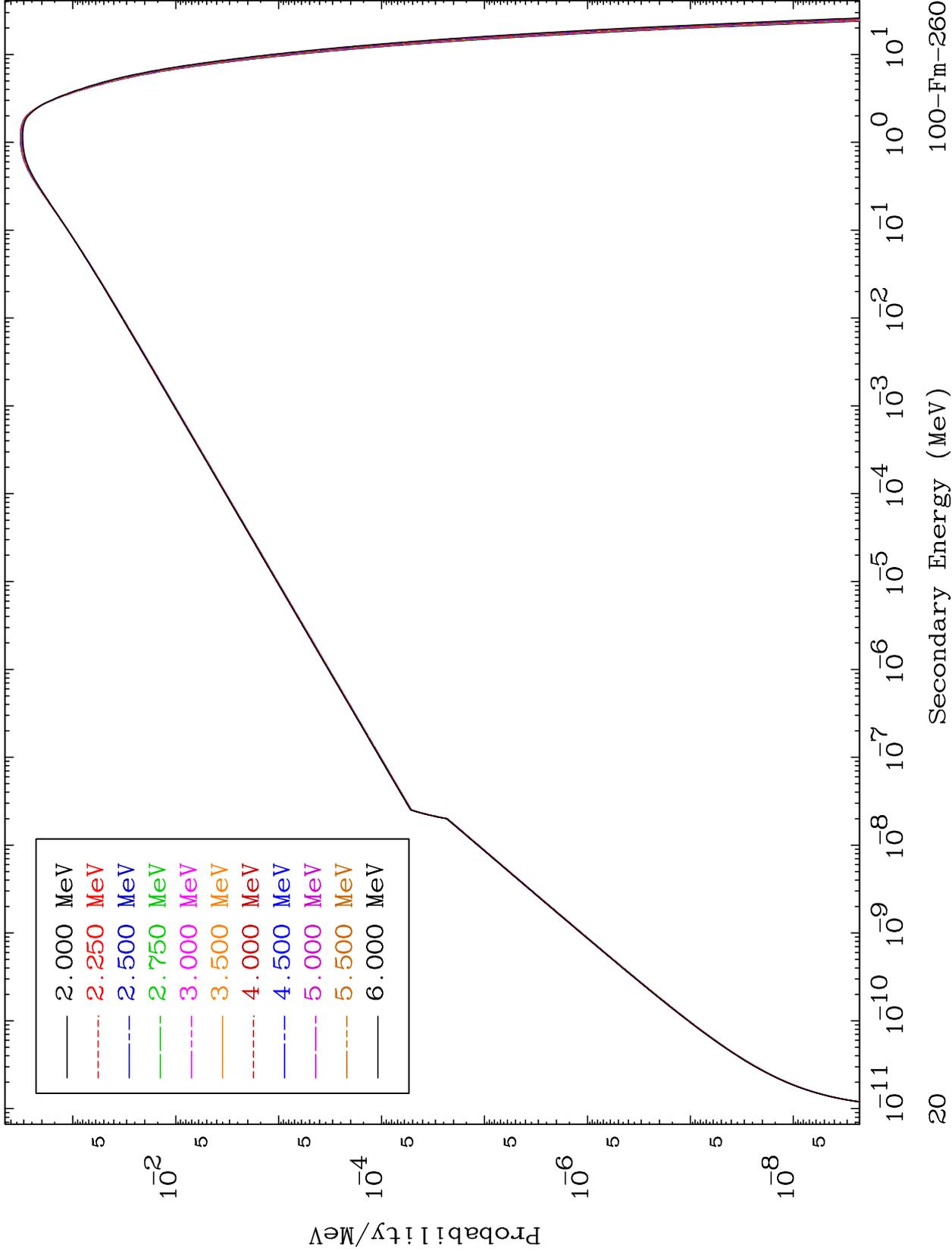
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Fission Energy Distribution

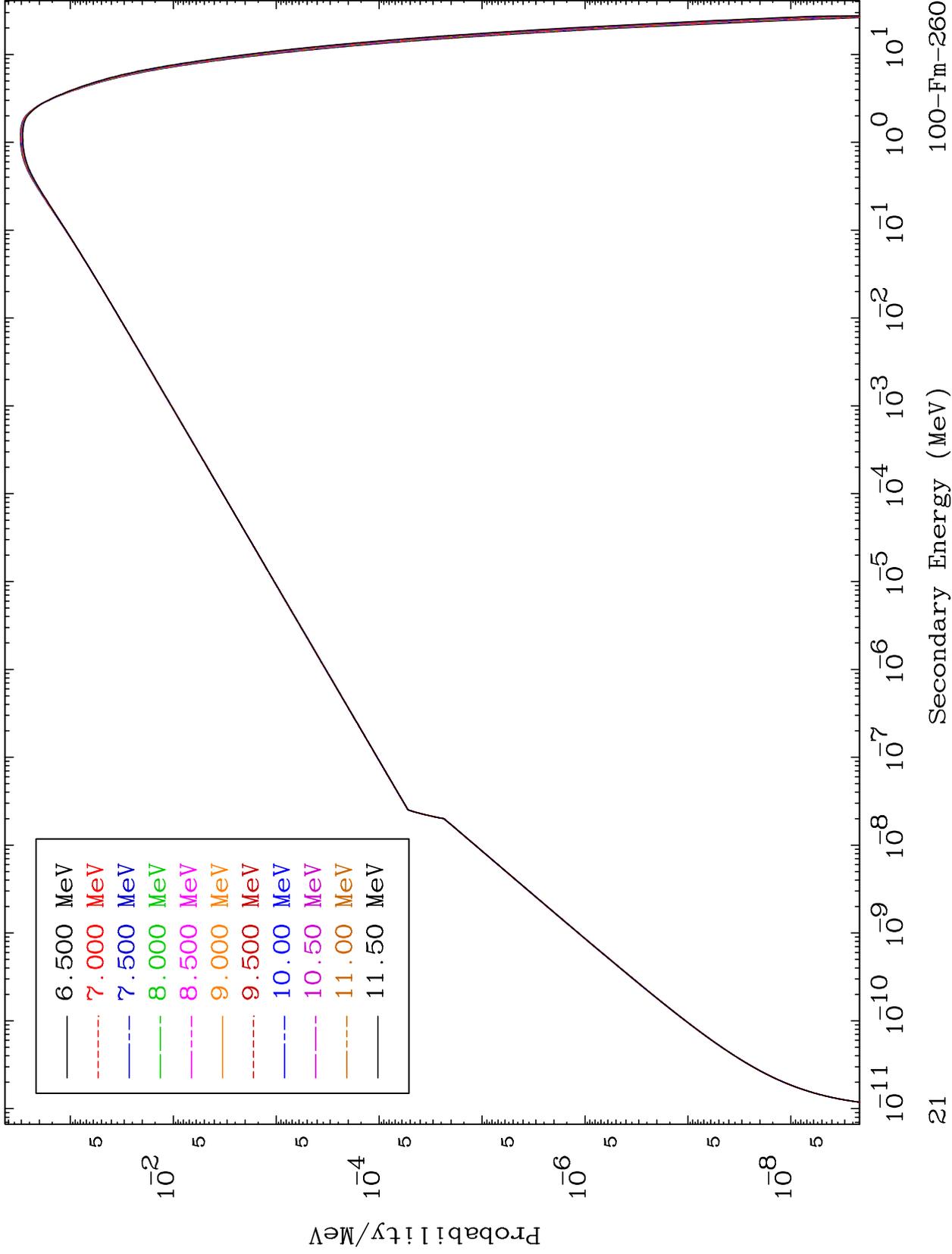
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Fission Energy Distribution

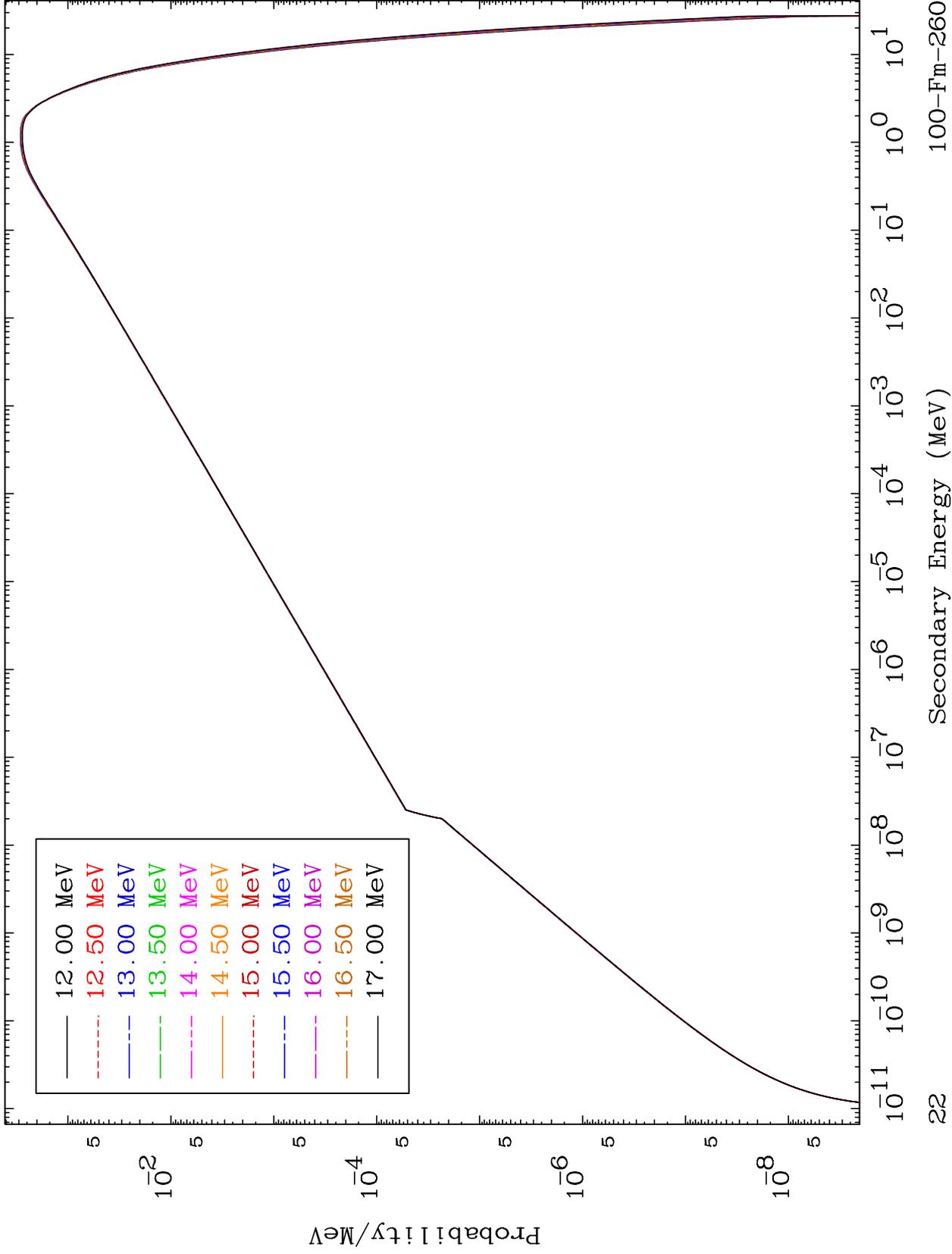
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Fission Energy Distribution

100-Fm-260



MAT 9941

Fission Energy Distribution

100-Fm-260

