

Program EVALPLOT
(Version 2018-1)

by

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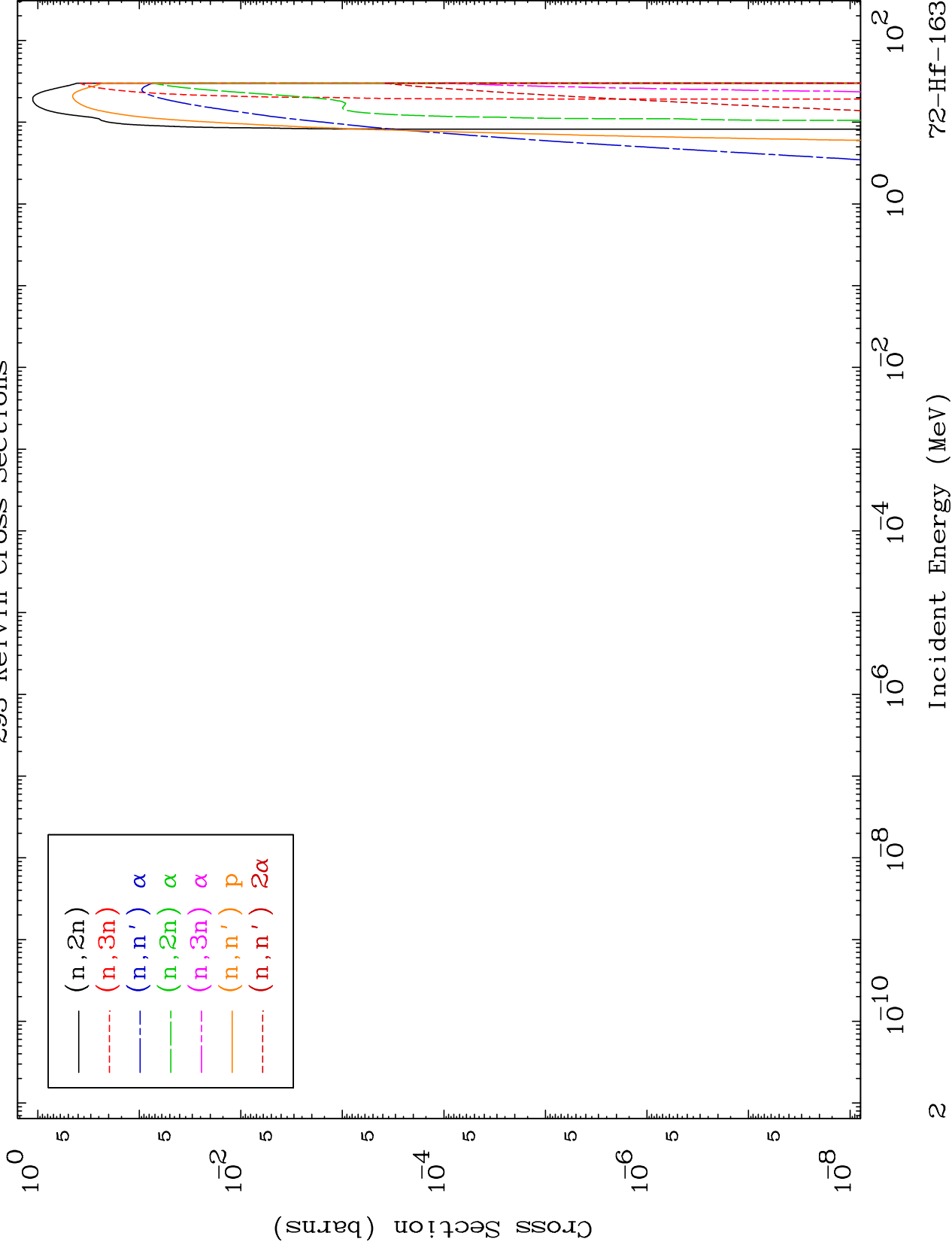
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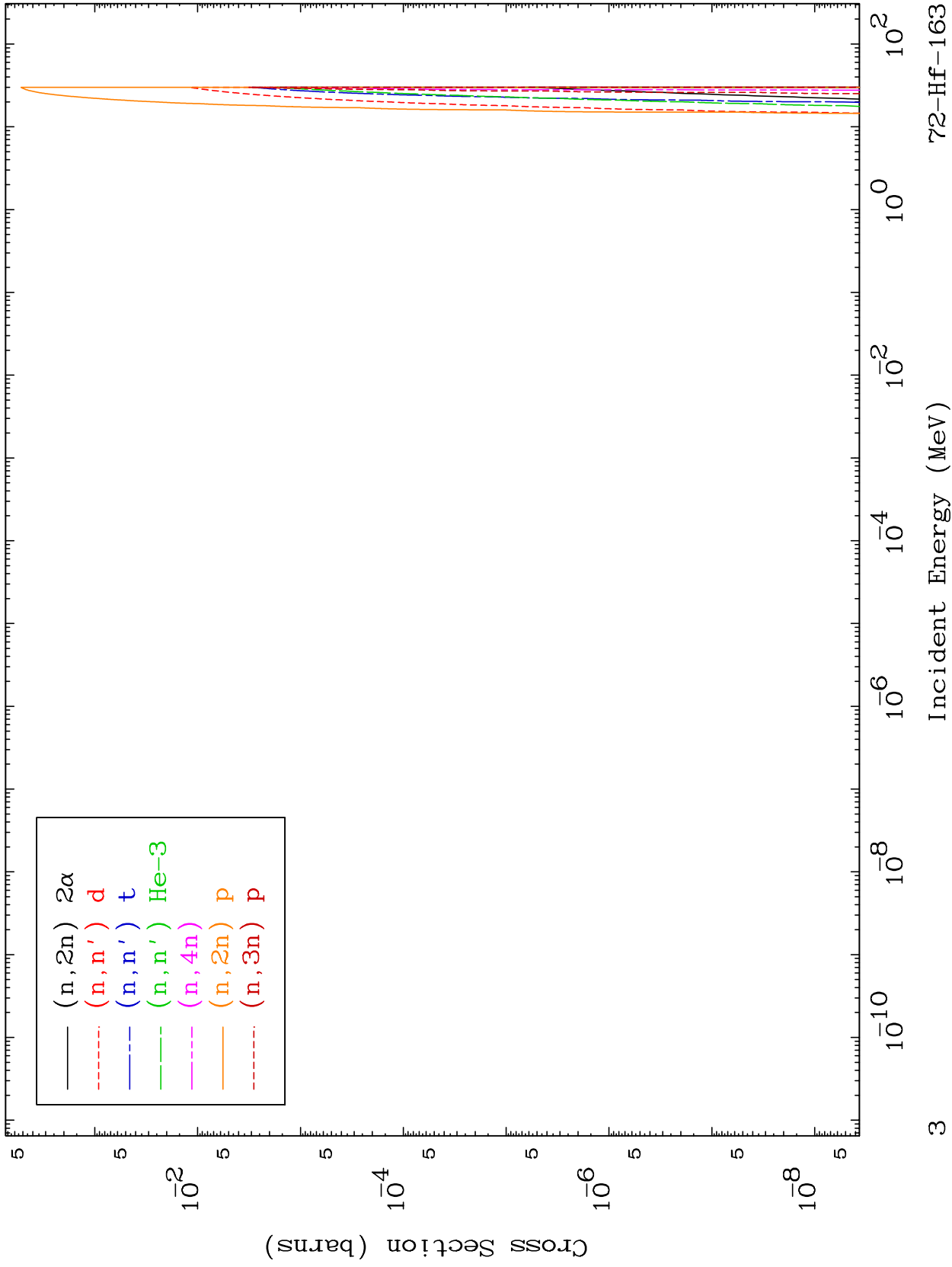
Press Mouse Button to Start

MAT 7192

Neutron Production
293 Kelvin Cross Sections

72-Hf-163

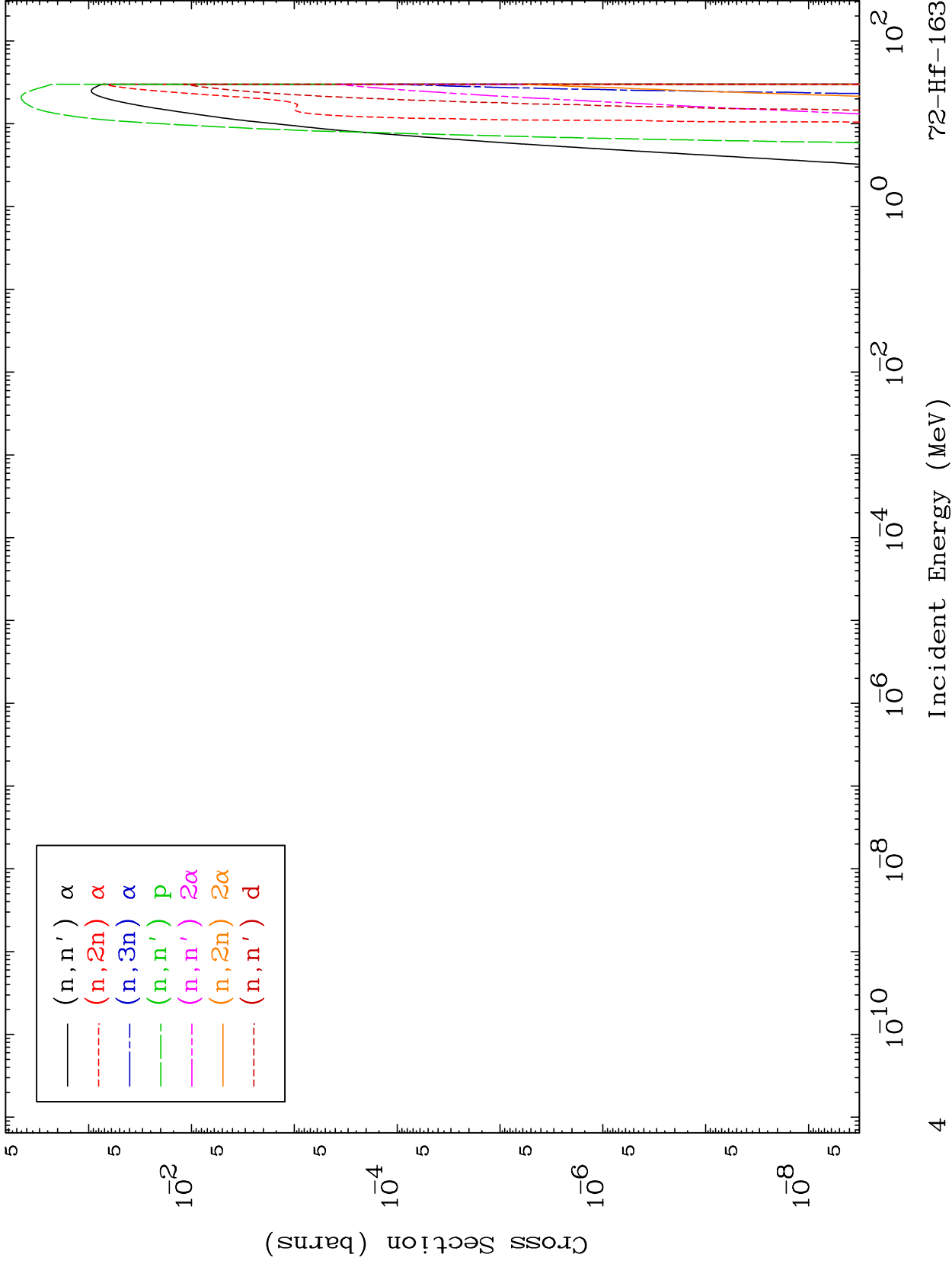


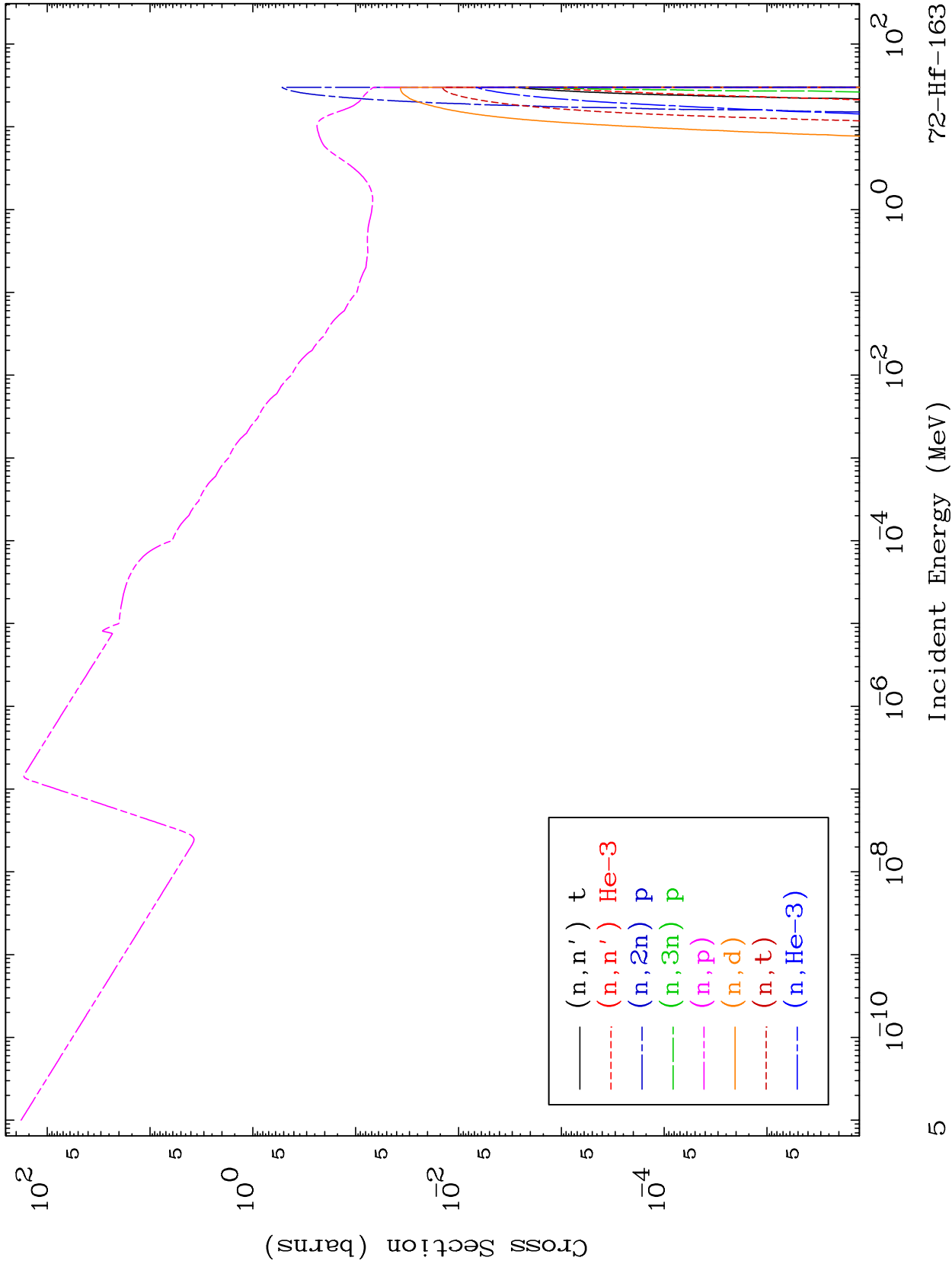


MAT 7192

Charged Particle
293 Kelvin Cross Sections

72-Hf-163

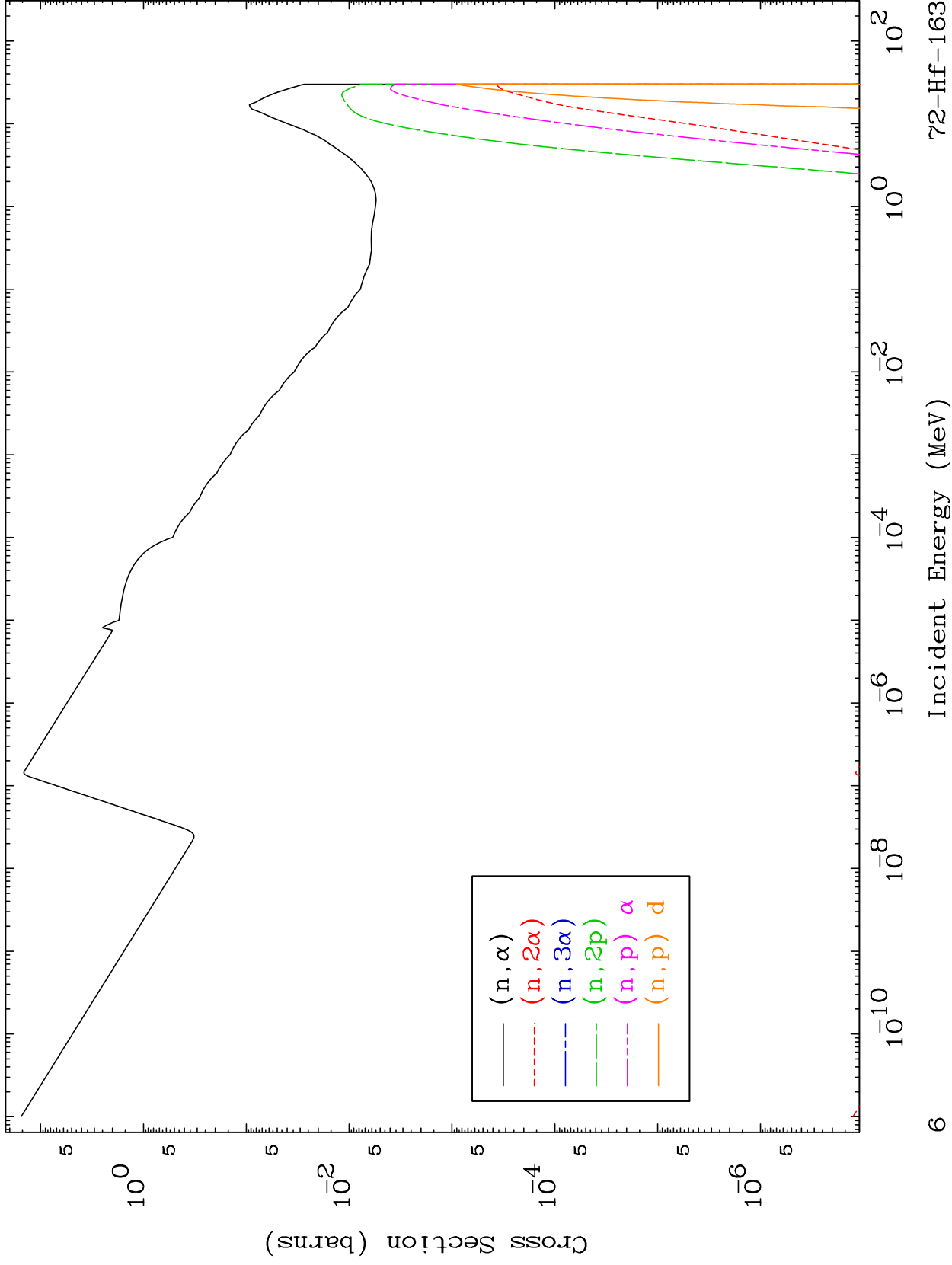


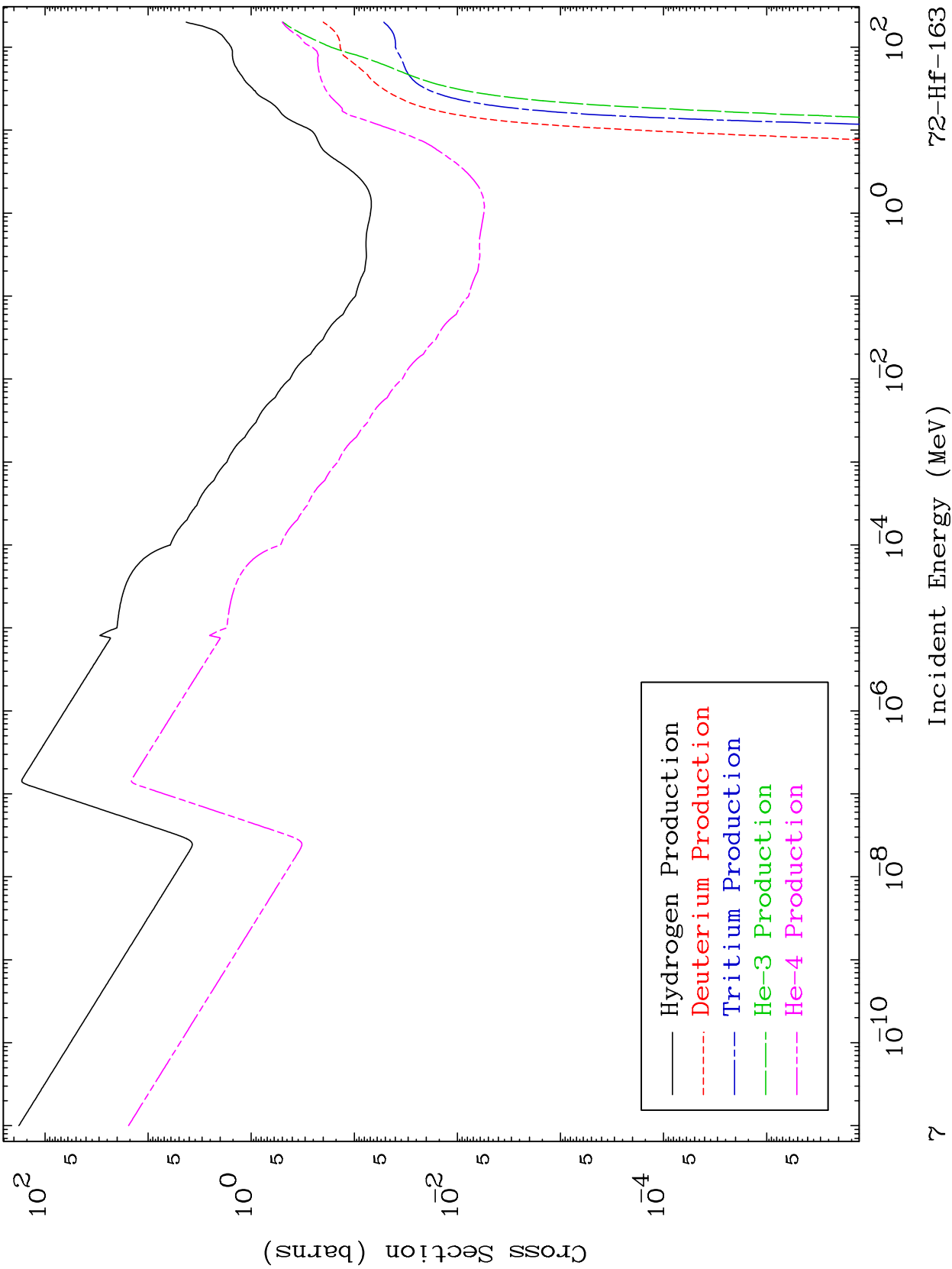


MAT 7192

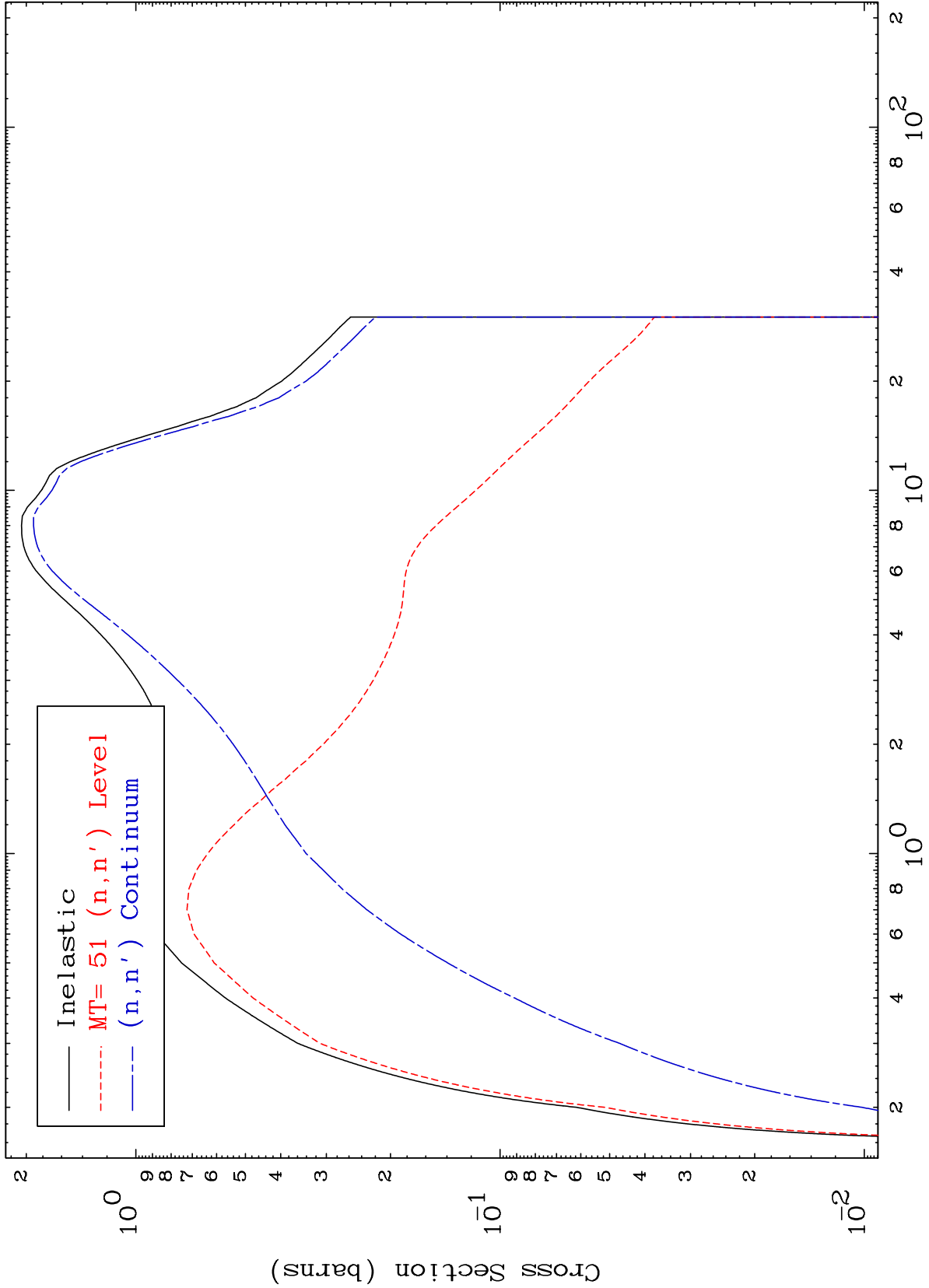
Charged Particle
293 Kelvin Cross Sections

72-Hf-163





(n,n') Level
293 Kelvin Cross Sections

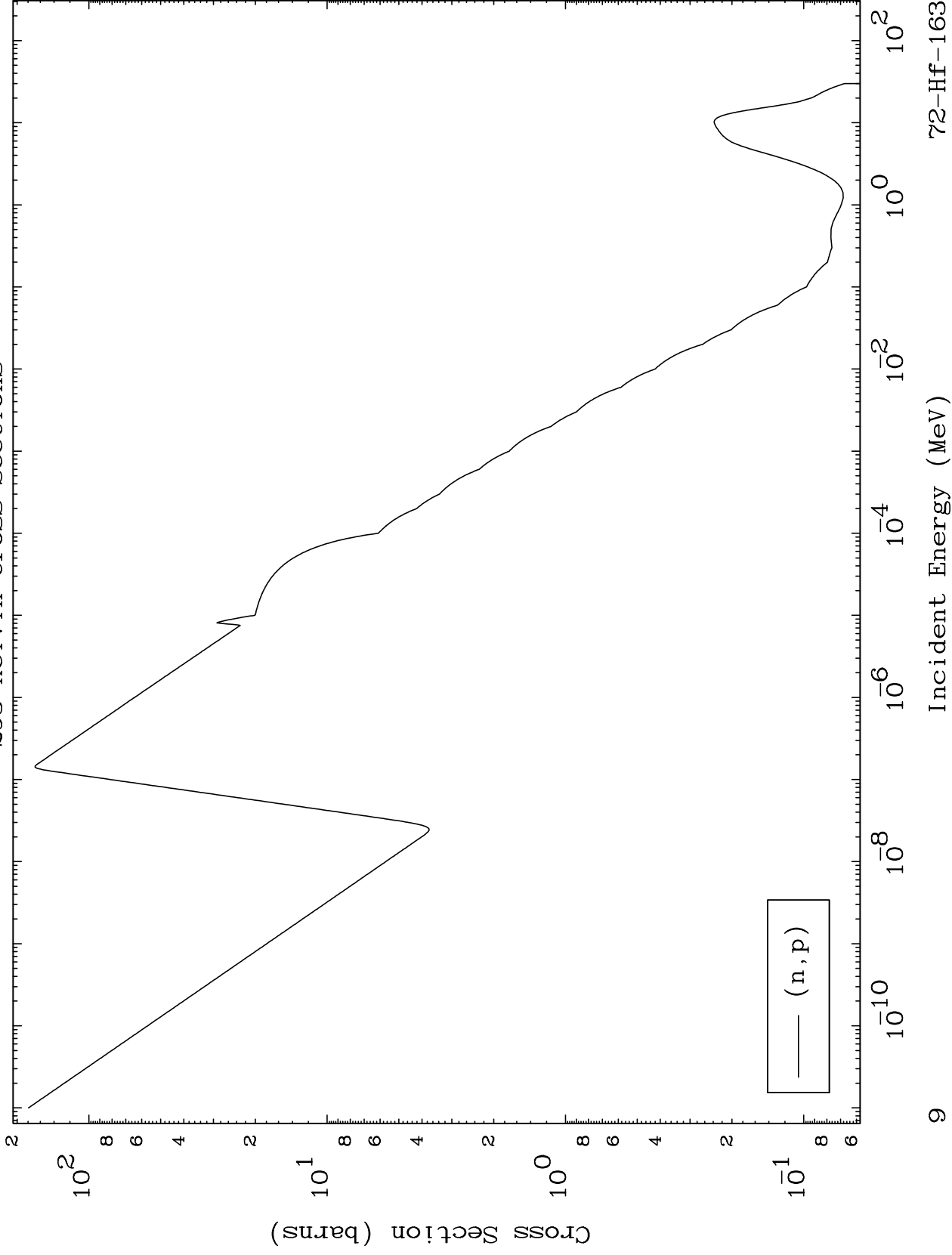


— Inelastic
- - - MT= 51 (n,n') Level
- · - (n,n') Continuum

MAT 7192

(n,p) Levels
293 Kelvin Cross Sections

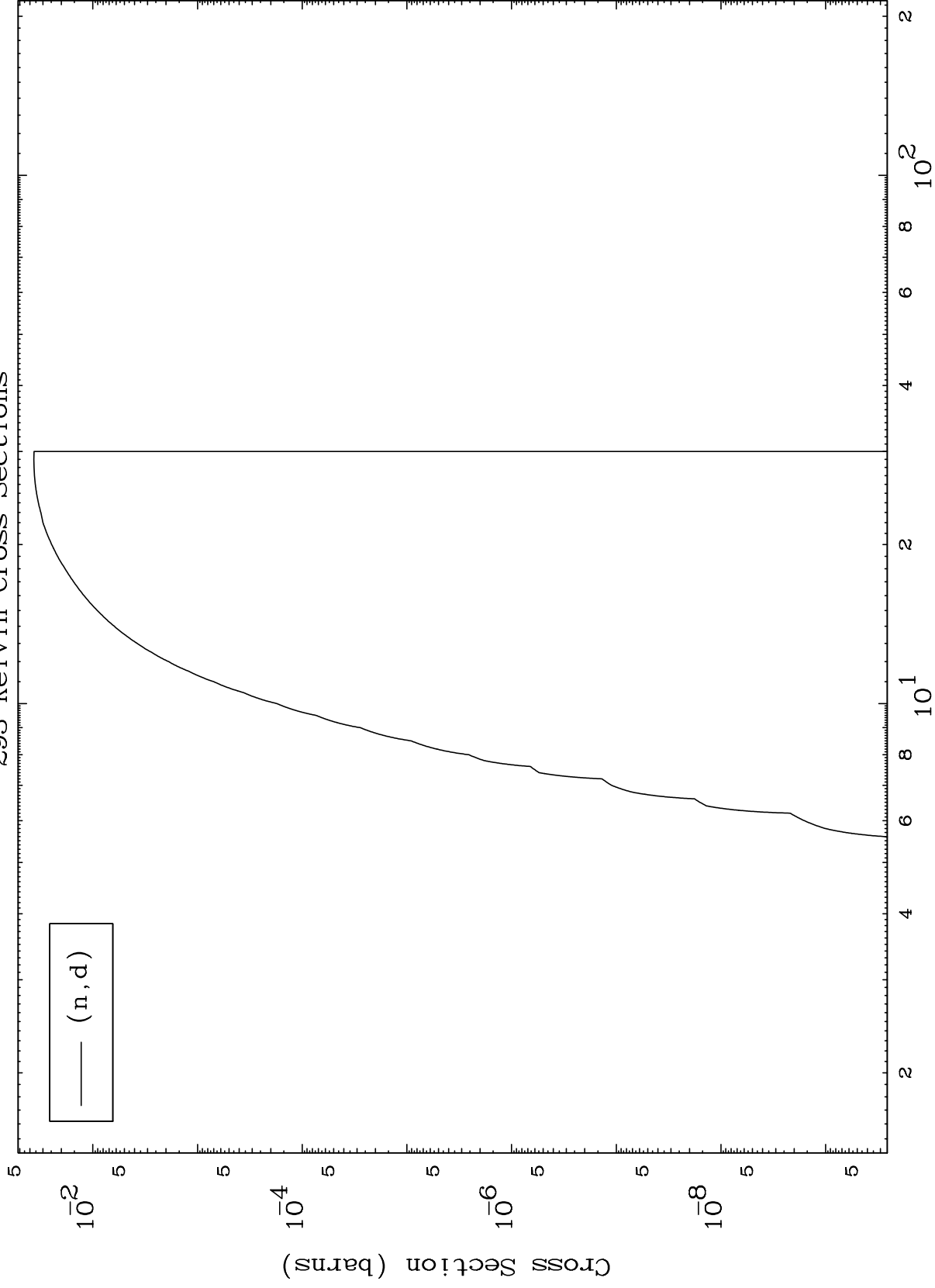
72-Hf-163



MAT 7192

(n,d) Levels
293 Kelvin Cross Sections

72-Hf-163



10

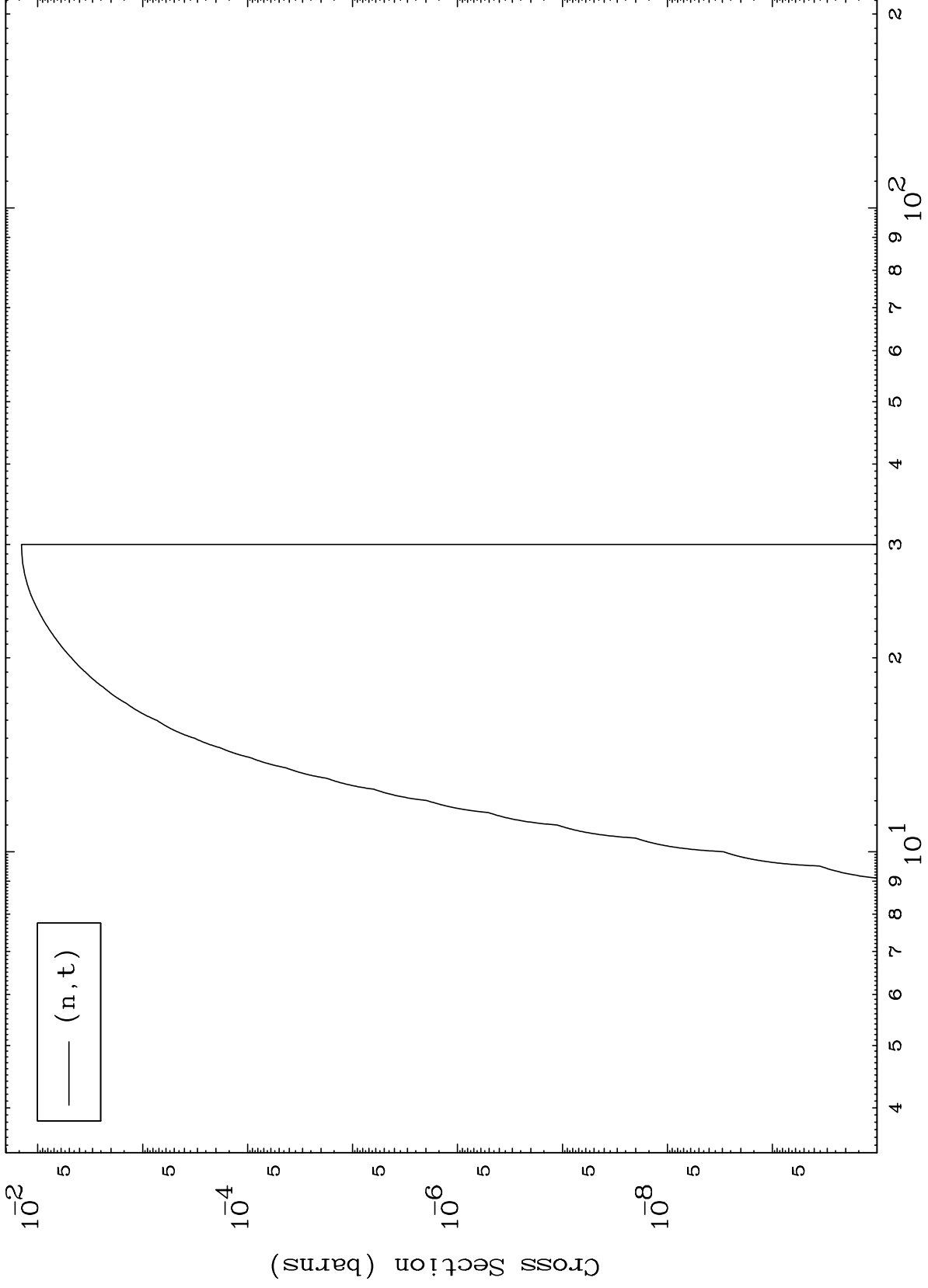
Incident Energy (MeV)

72-Hf-163

MAT 7192

(n,t) Levels
293 Kelvin Cross Sections

72-Hf-163



11

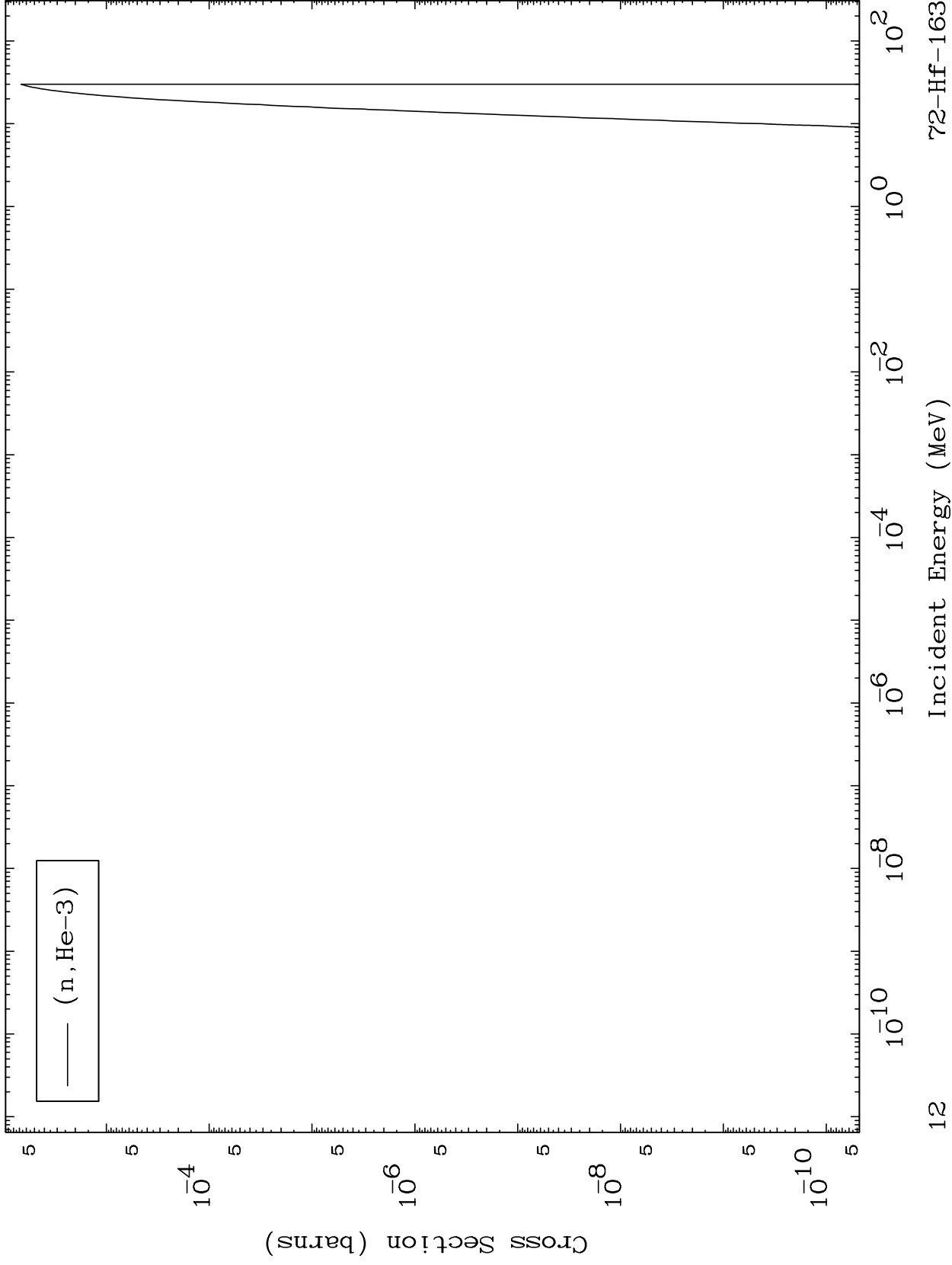
Incident Energy (MeV)

72-Hf-163

MAT 7192

(n,He3) Levels
293 Kelvin Cross Sections

72-Hf-163



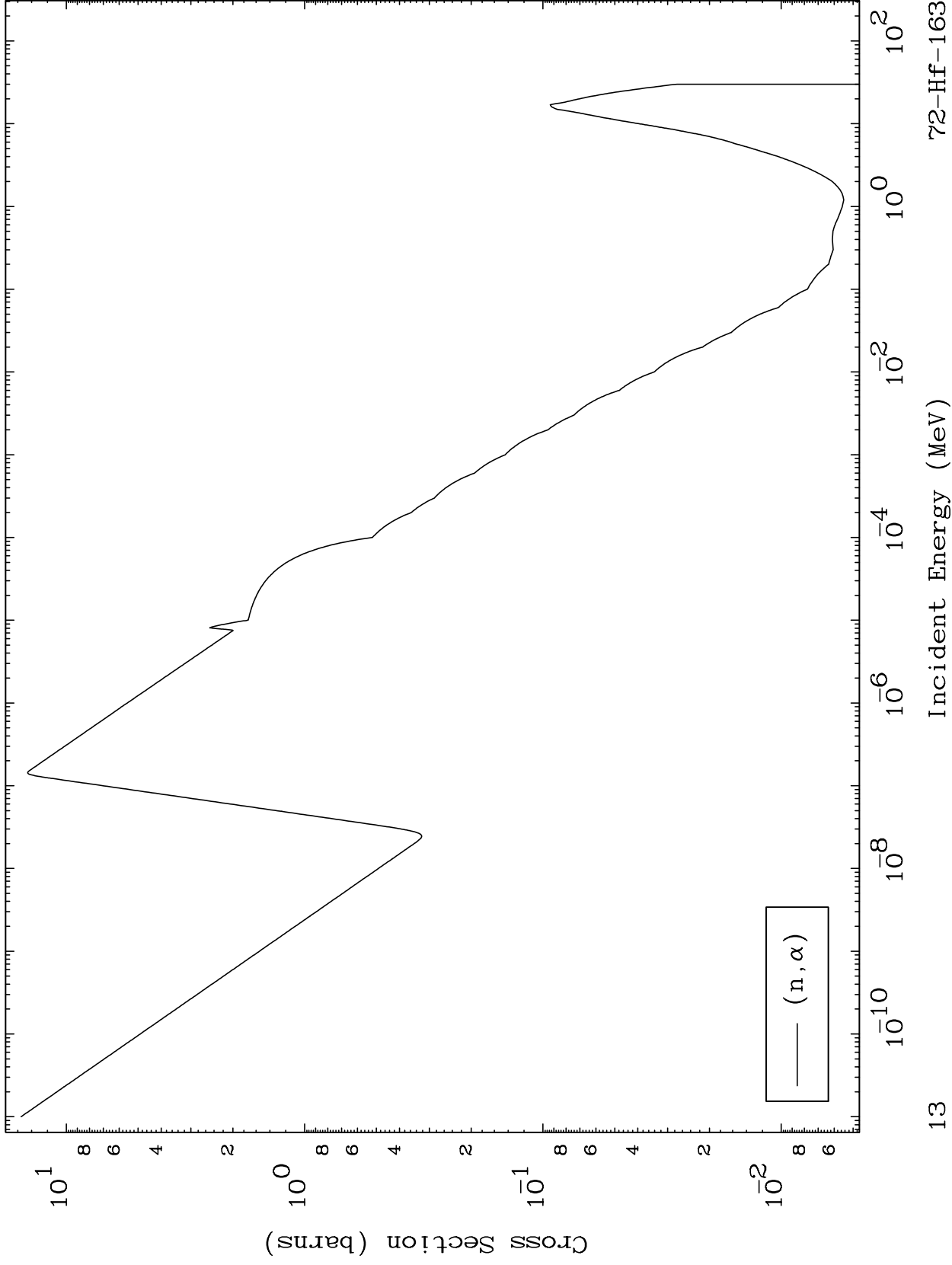
12

72-Hf-163

MAT 7192

(n,α) Levels
293 Kelvin Cross Sections

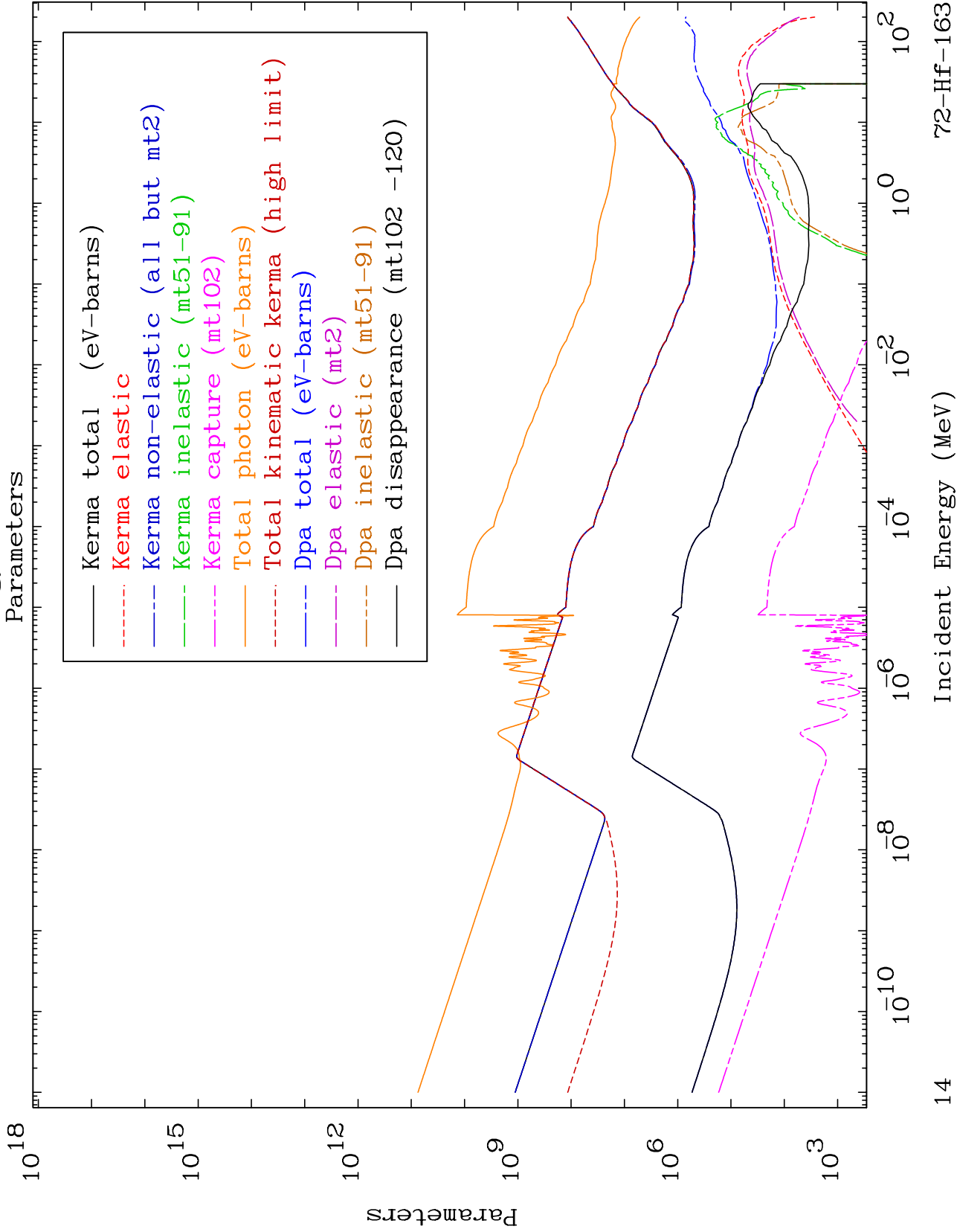
72-Hf-163



13

Incident Energy (MeV)

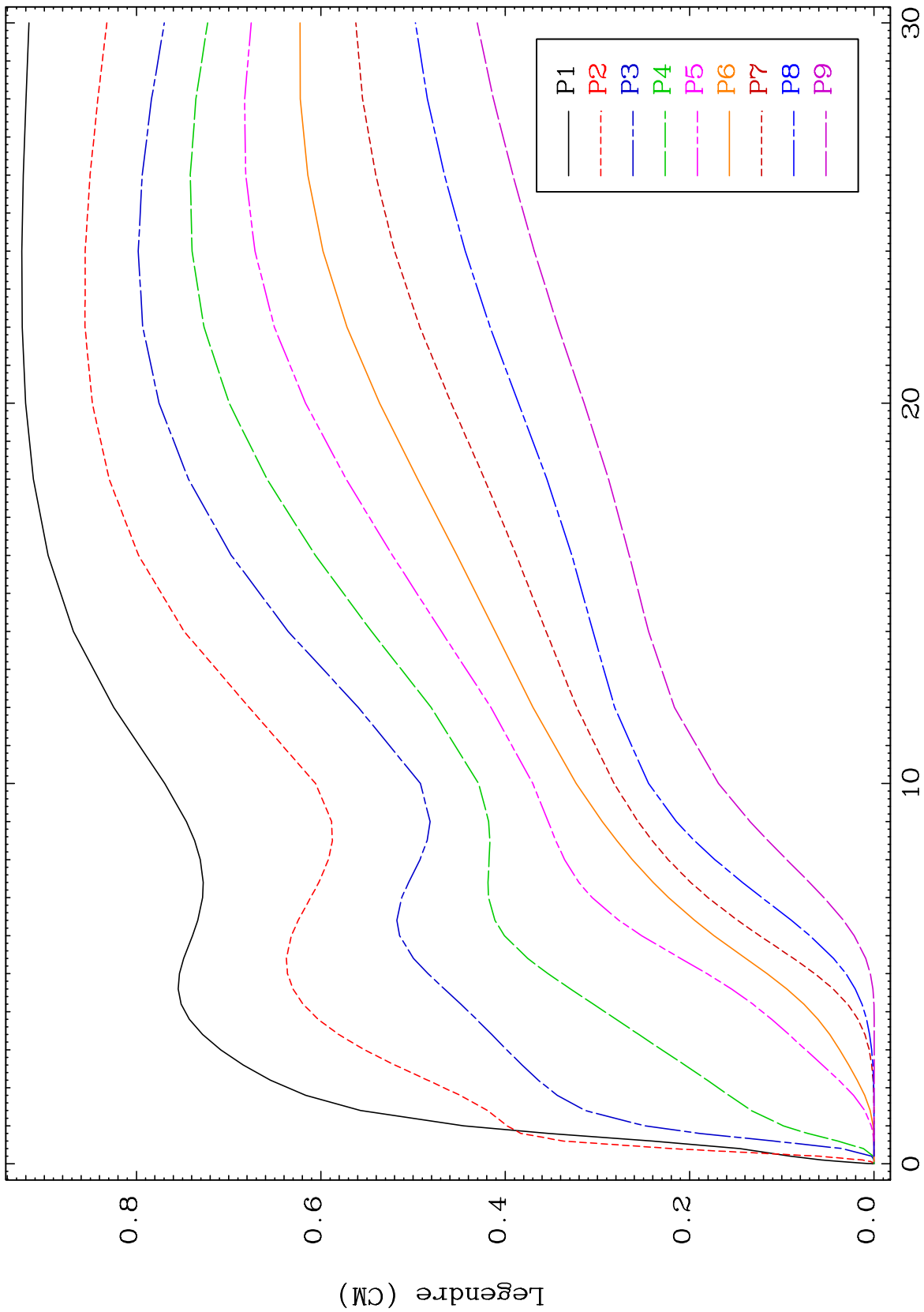
72-Hf-163



MAT 7192

72-Hf-163

Elastic Legendre Coefficients



72-Hf-163

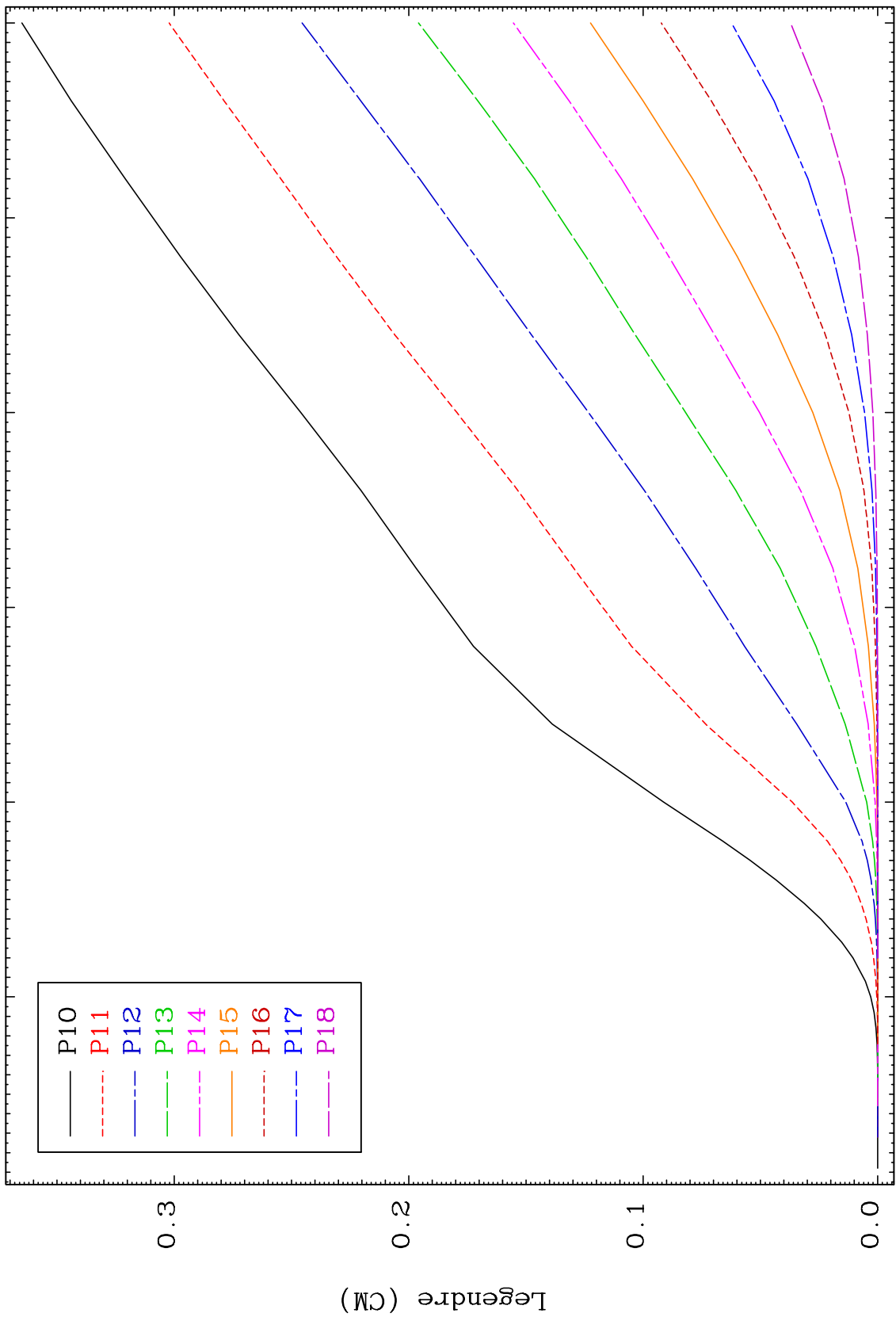
Incident Energy (MeV)

15

MAT 7192

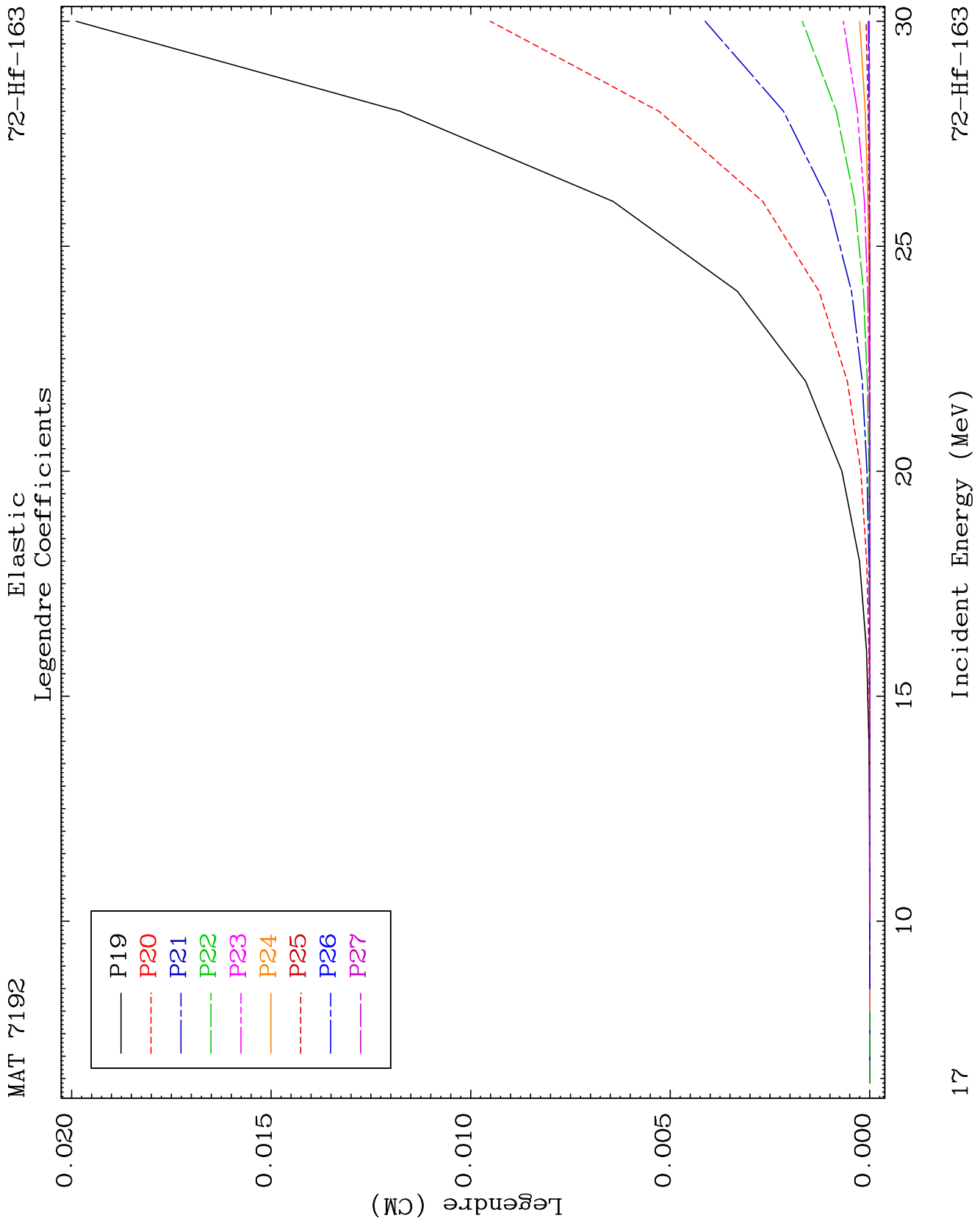
Elastic Legendre Coefficients

72-Hf-163



16

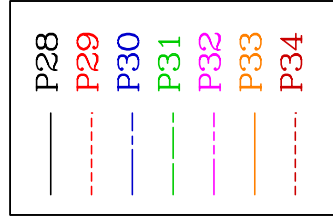
72-Hf-163



MAT 7192

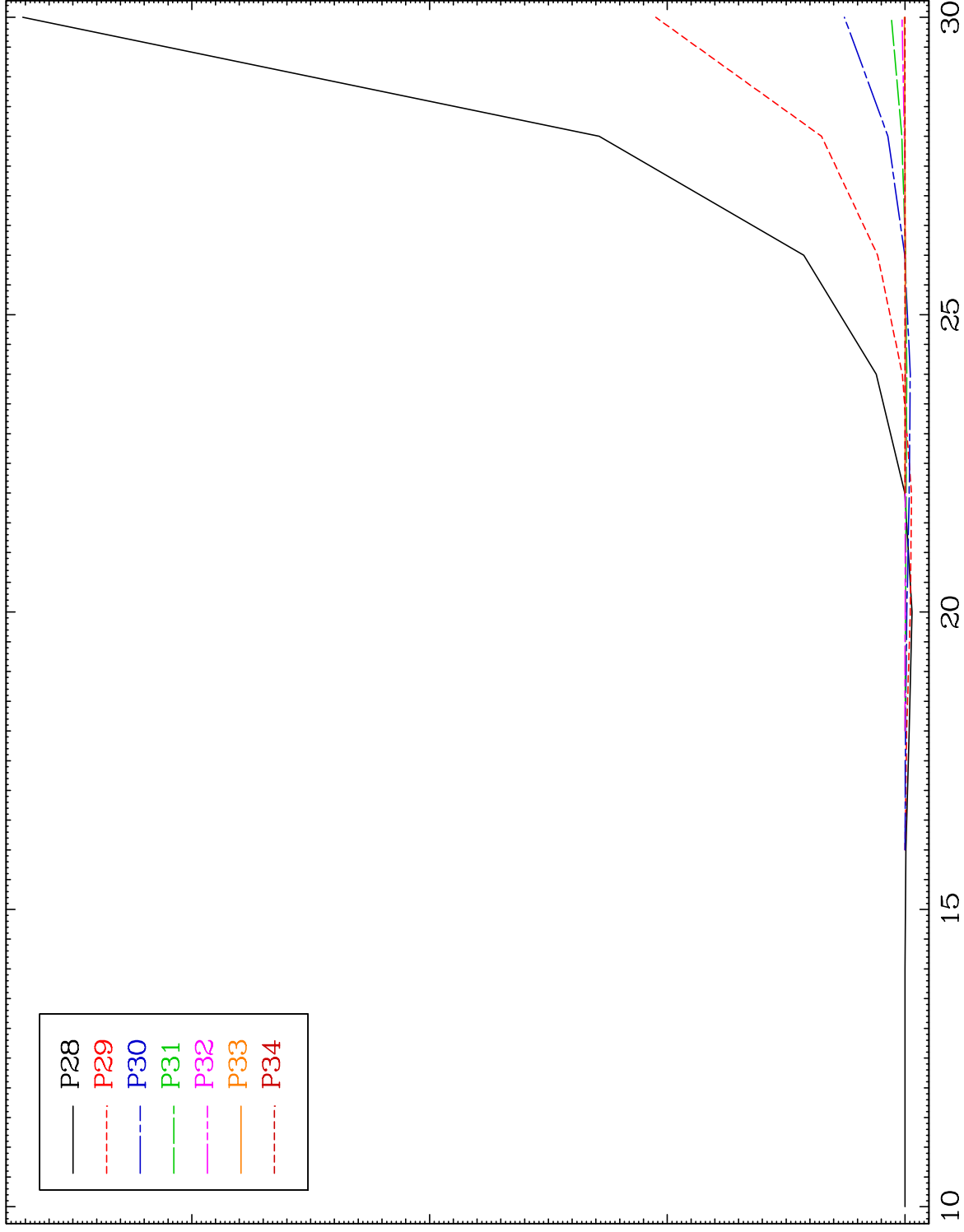
Elastic
Legendre Coefficients

72-Hf-163



$\times 10^{-6}$

Legendre (CM)



10

15

20

25

30

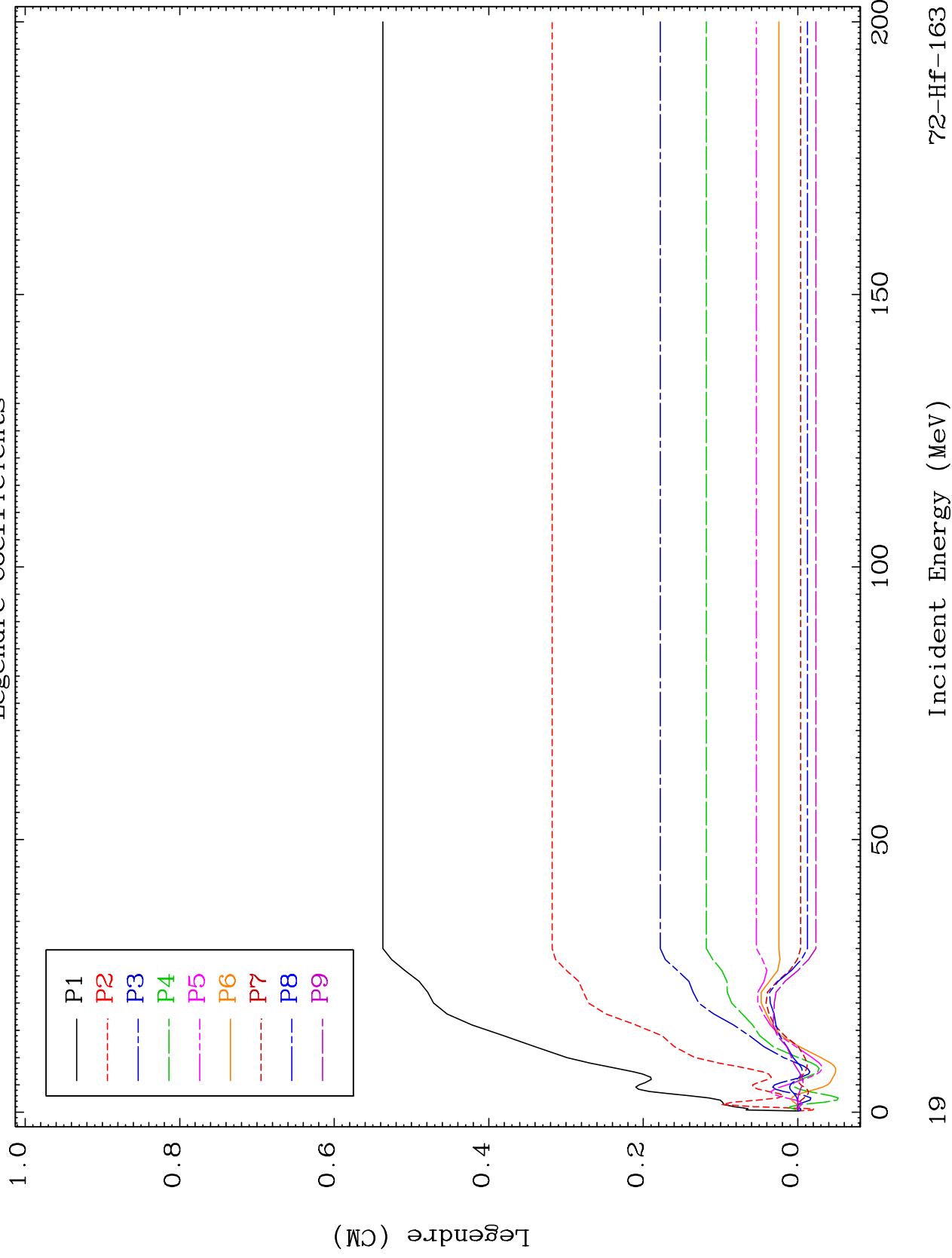
Incident Energy (MeV)

72-Hf-163

MAT 7192

MT= 51 (n,n') Level
Legendre Coefficients

72-Hf-163



72-Hf-163

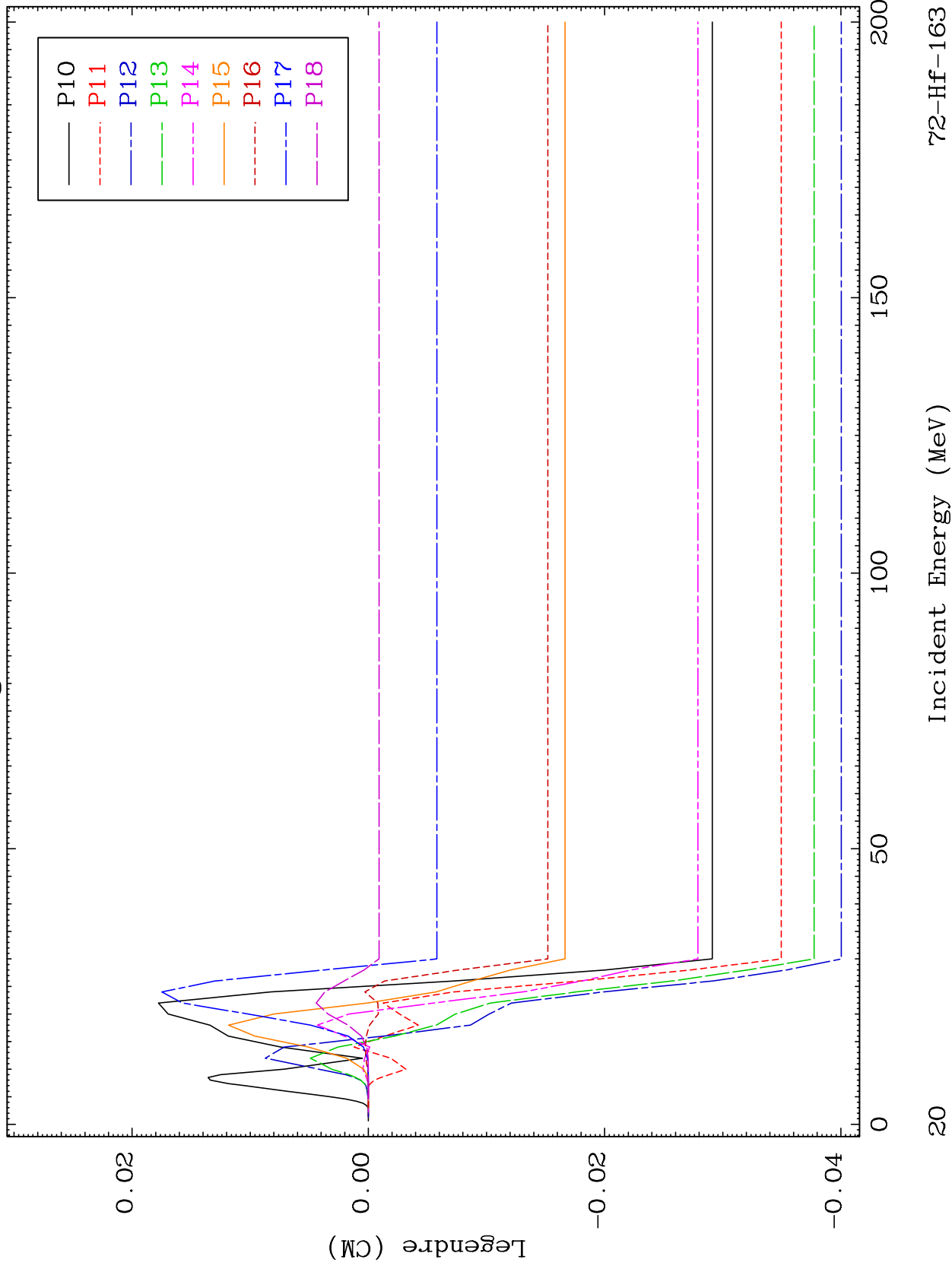
Incident Energy (MeV)

19

MAT 7192

MT= 51 (n,n') Level
Legendre Coefficients

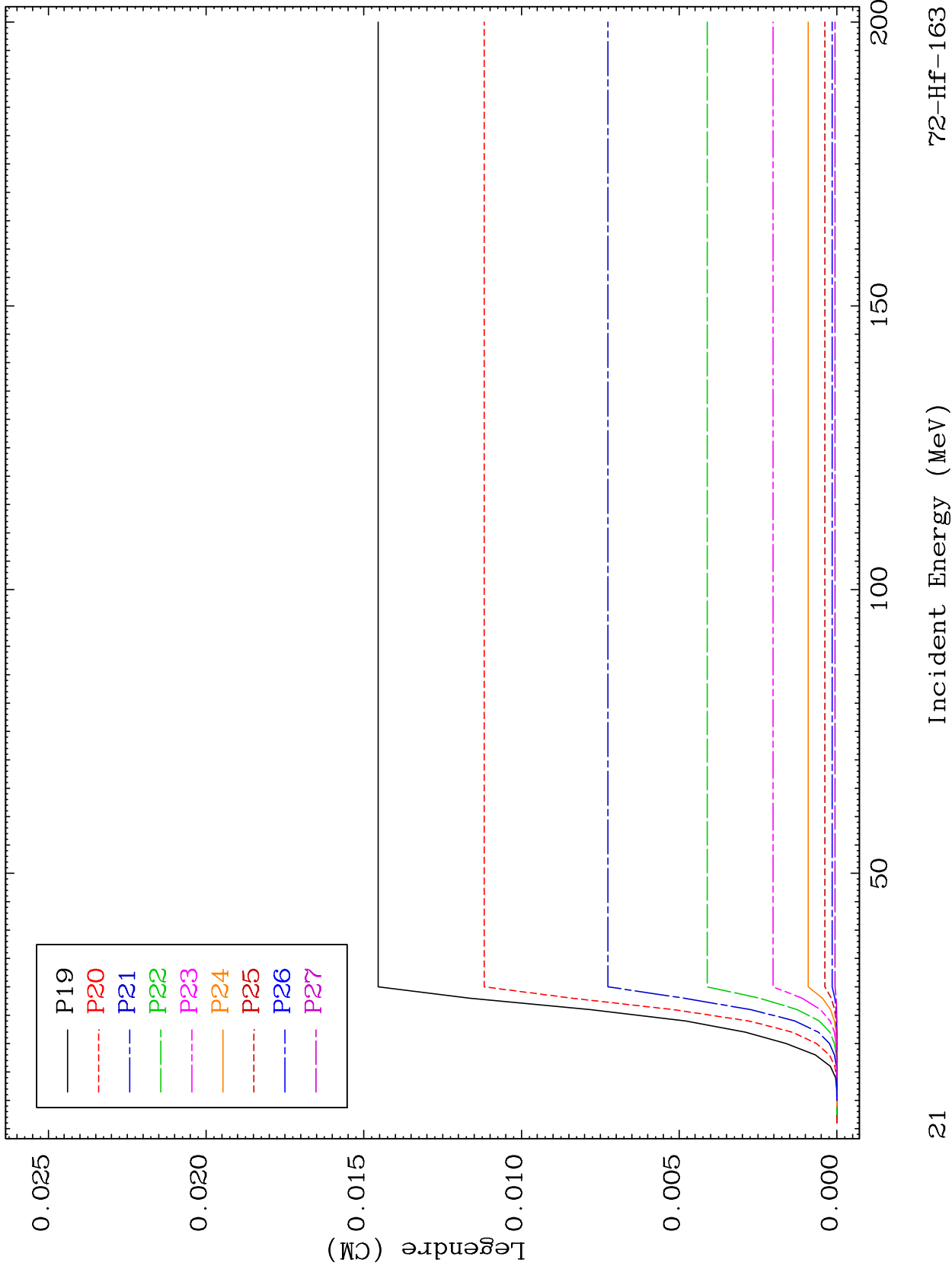
72-Hf-163



72-Hf-163

Incident Energy (MeV)

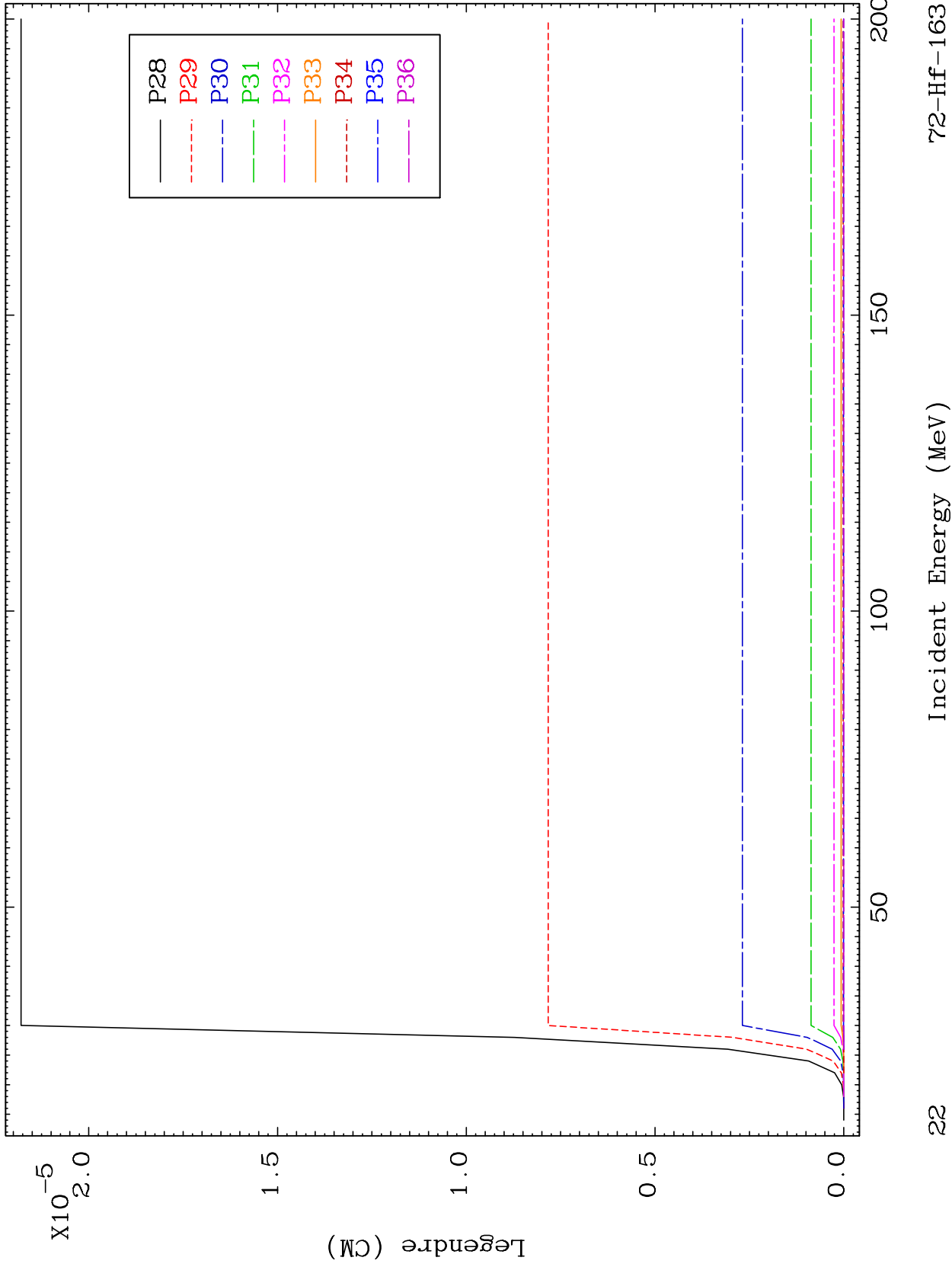
20



MAT 7192

MT= 51 (n,n') Level
Legendre Coefficients

72-Hf-163

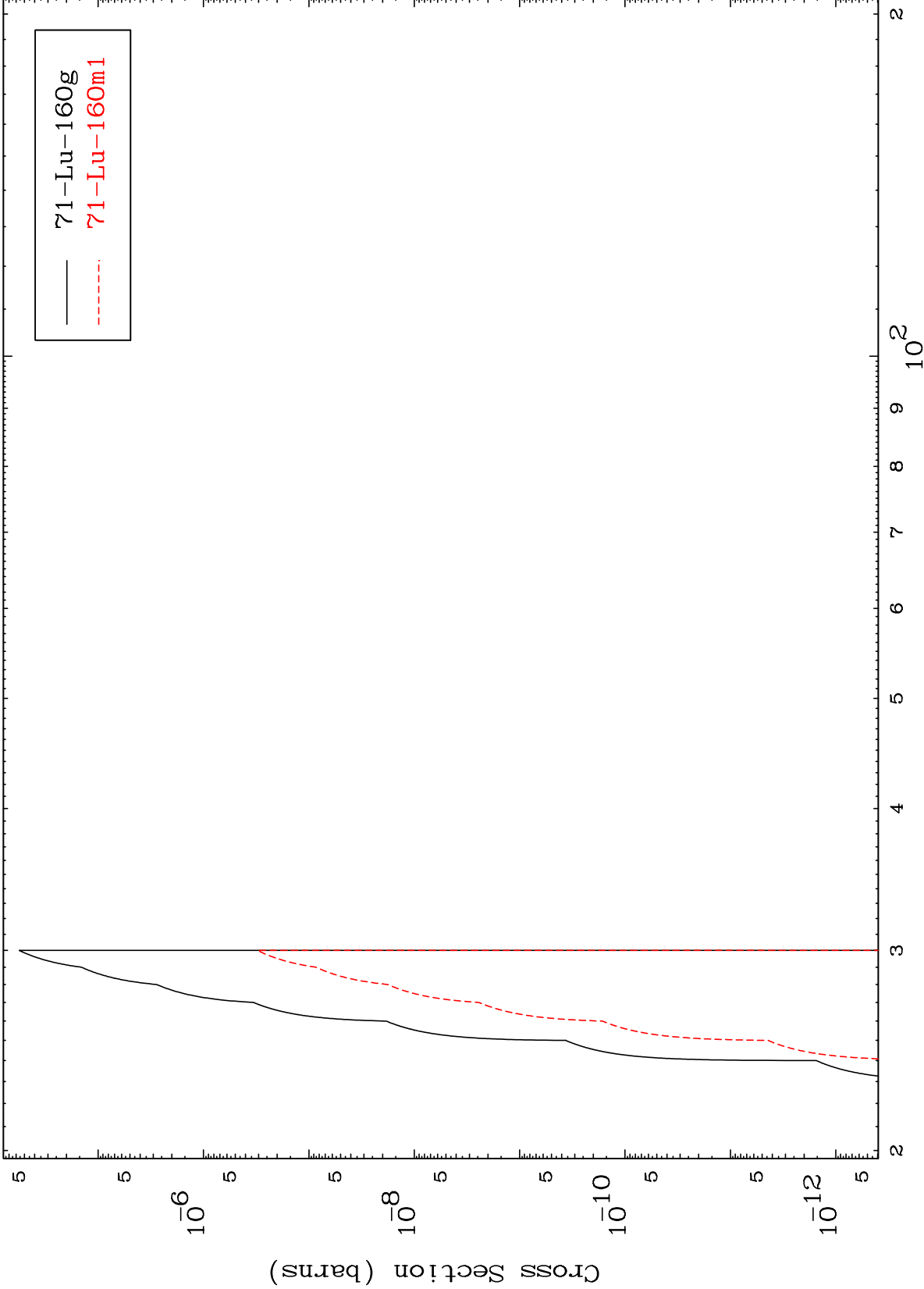


22

Incident Energy (MeV)

72-Hf-163

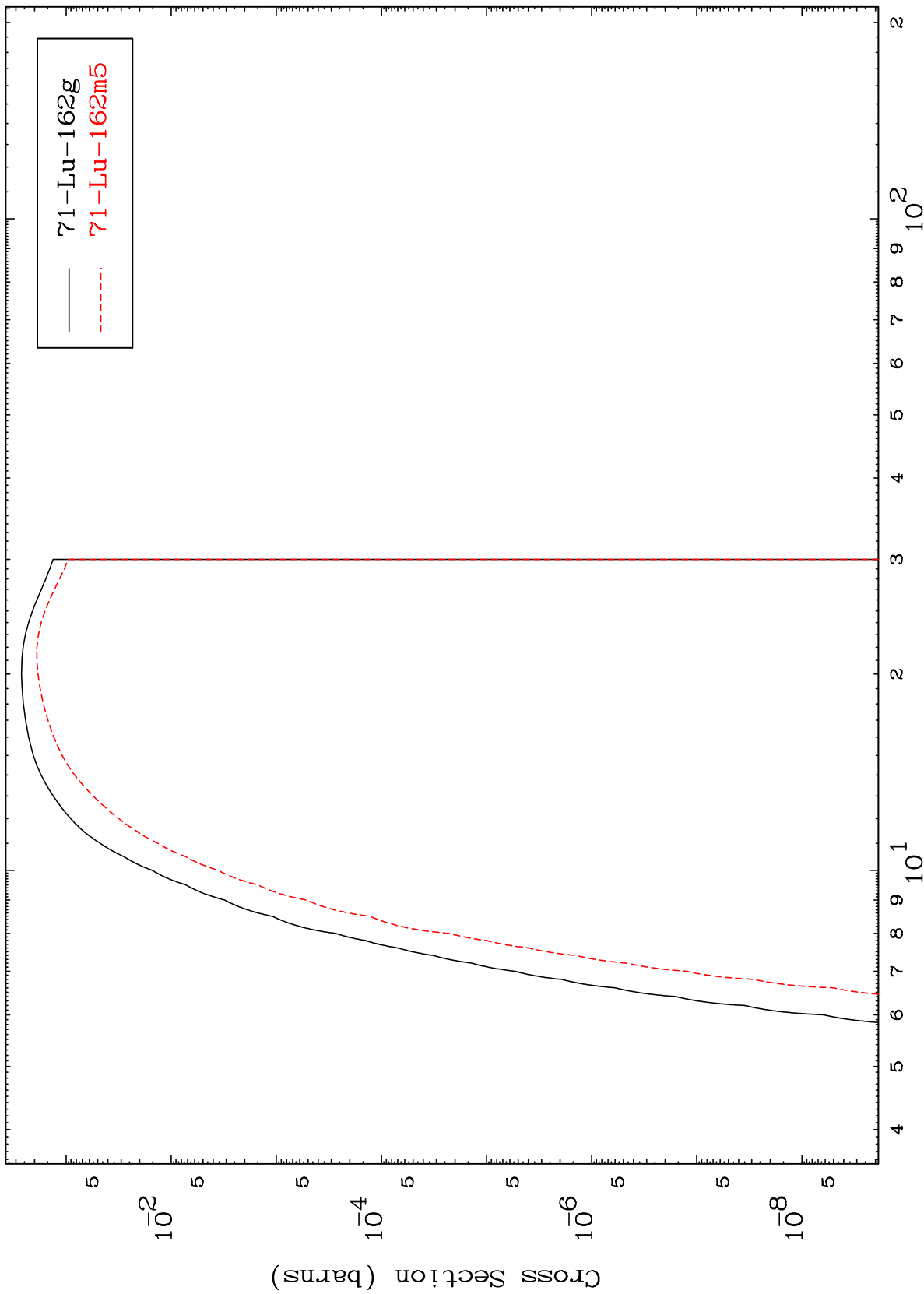
Radionuclide Production Cross Section



MAT 7192

72-Hf-163

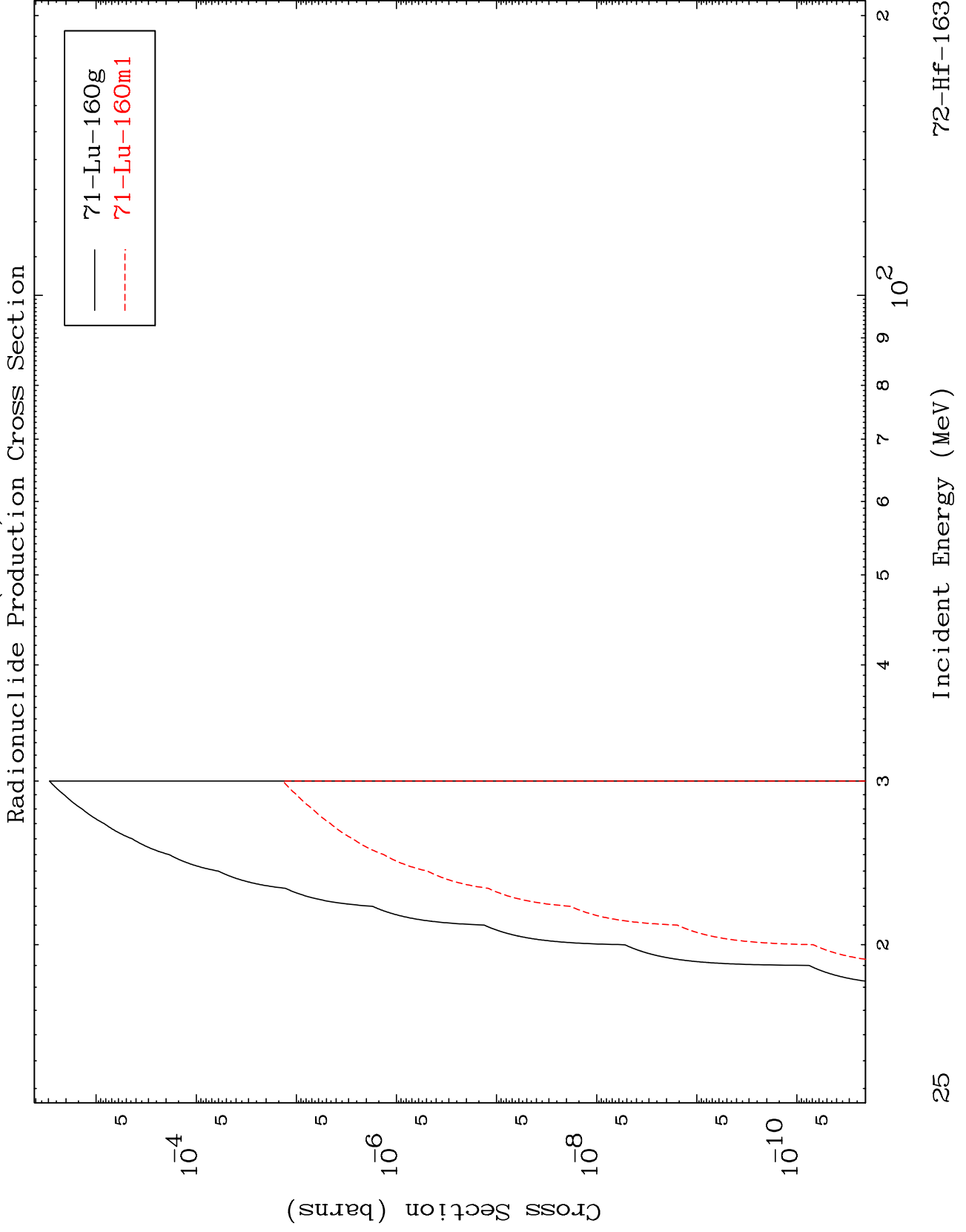
(n, n') p
Radionuclide Production Cross Section



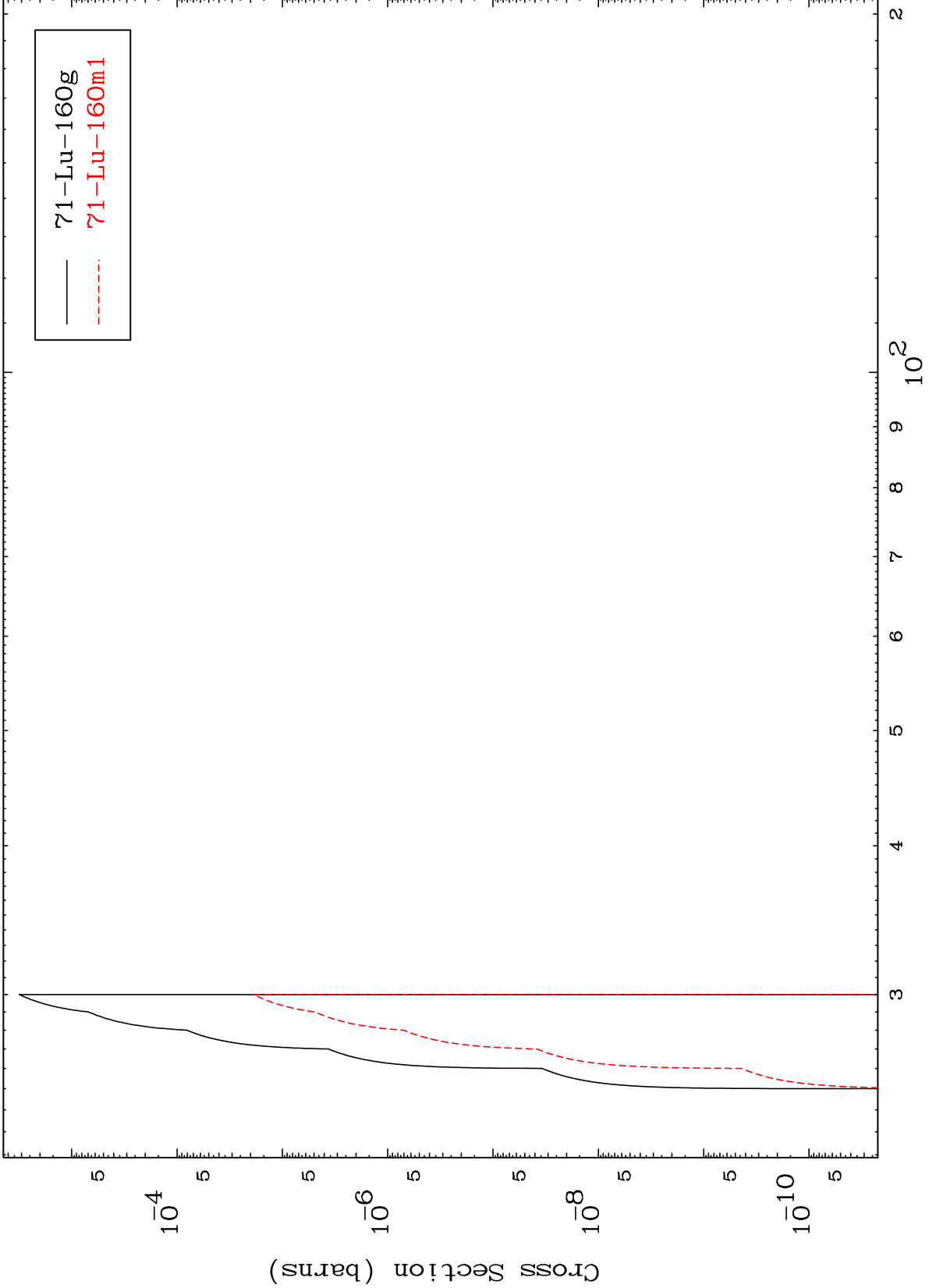
24

Incident Energy (MeV)

72-Hf-163



Radionuclide Production Cross Section

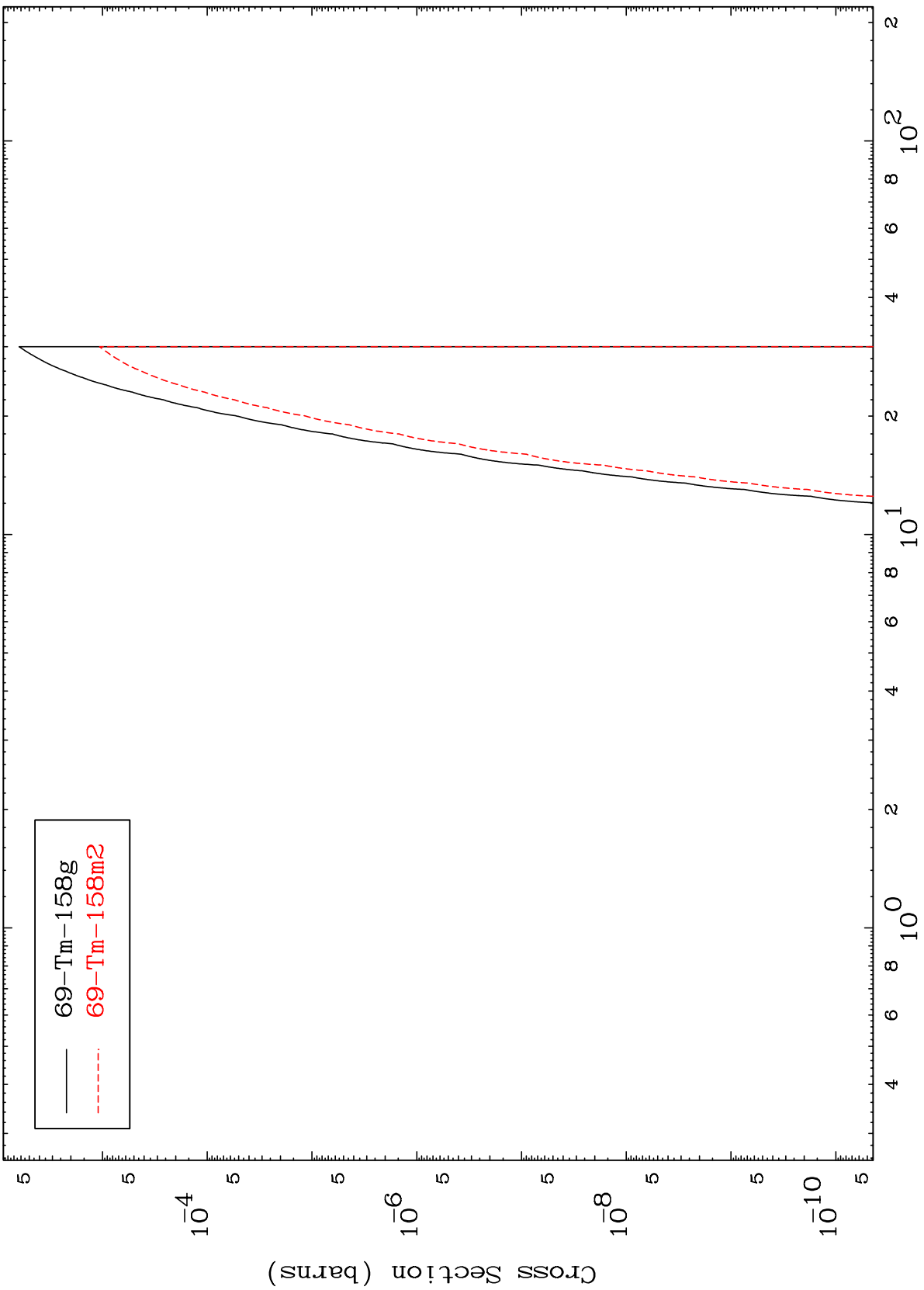


MAT 7192

(n,n') p α

72-Hf-163

Radionuclide Production Cross Section



27

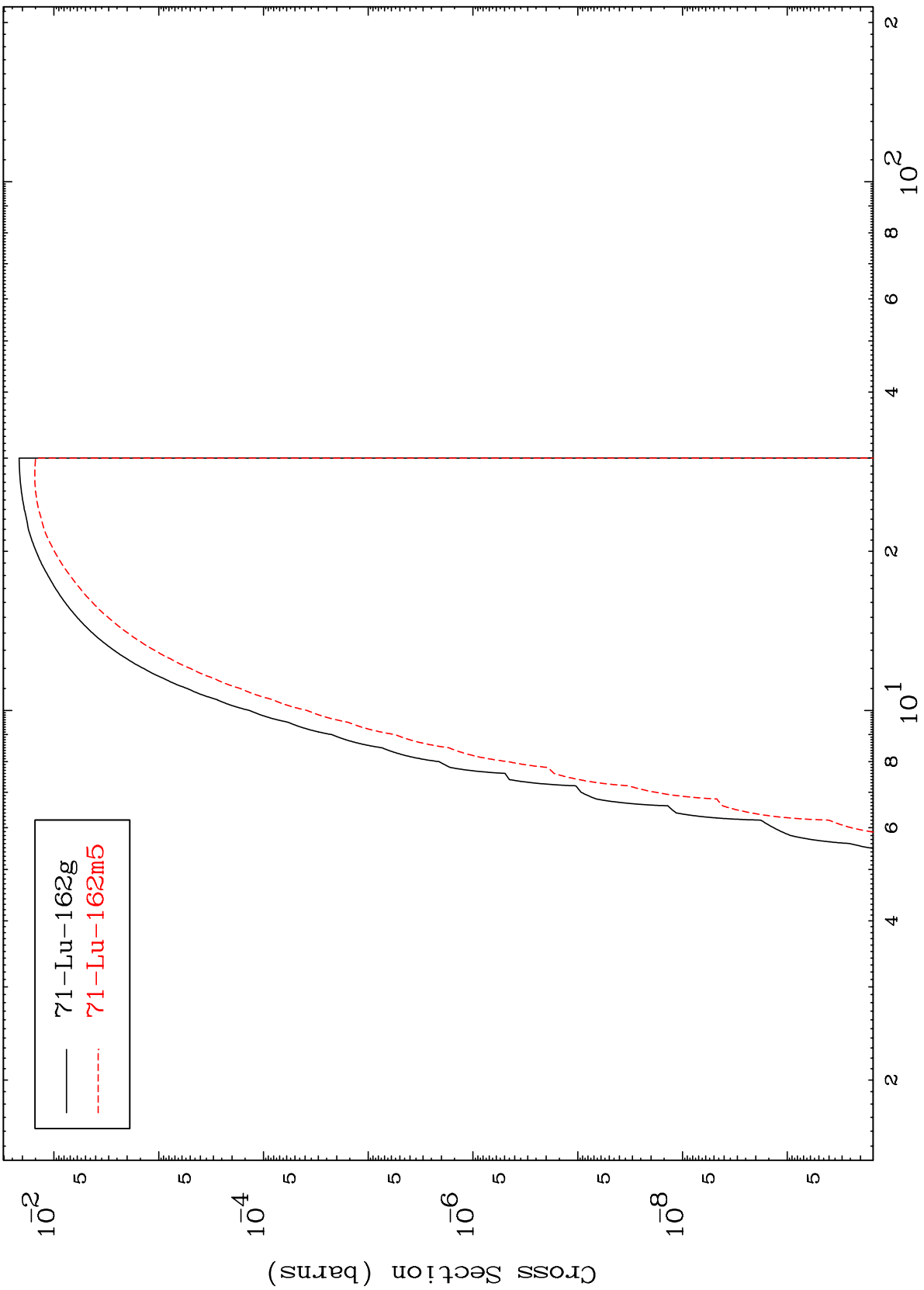
Incident Energy (MeV)

72-Hf-163

MAT 7192

72-Hf-163

(n,d)
Radionuclide Production Cross Section



28

72-Hf-163

Incident Energy (MeV)

Radionuclide Production Cross Section

