

Program EVALPLOT  
(Version 2018-1)

by

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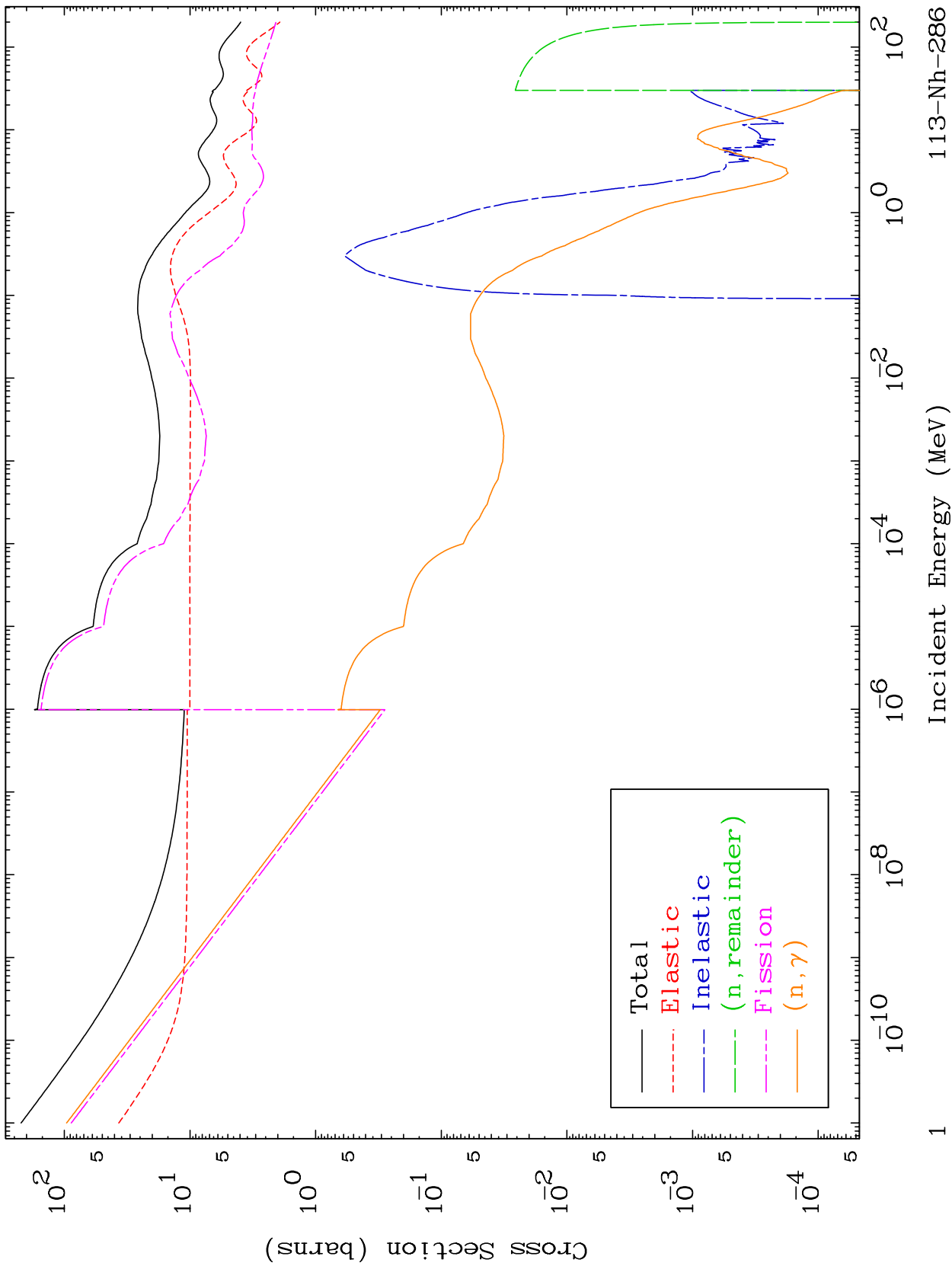
Press Mouse Button to Start

MAT 1386

Major

293 Kelvin Cross Sections

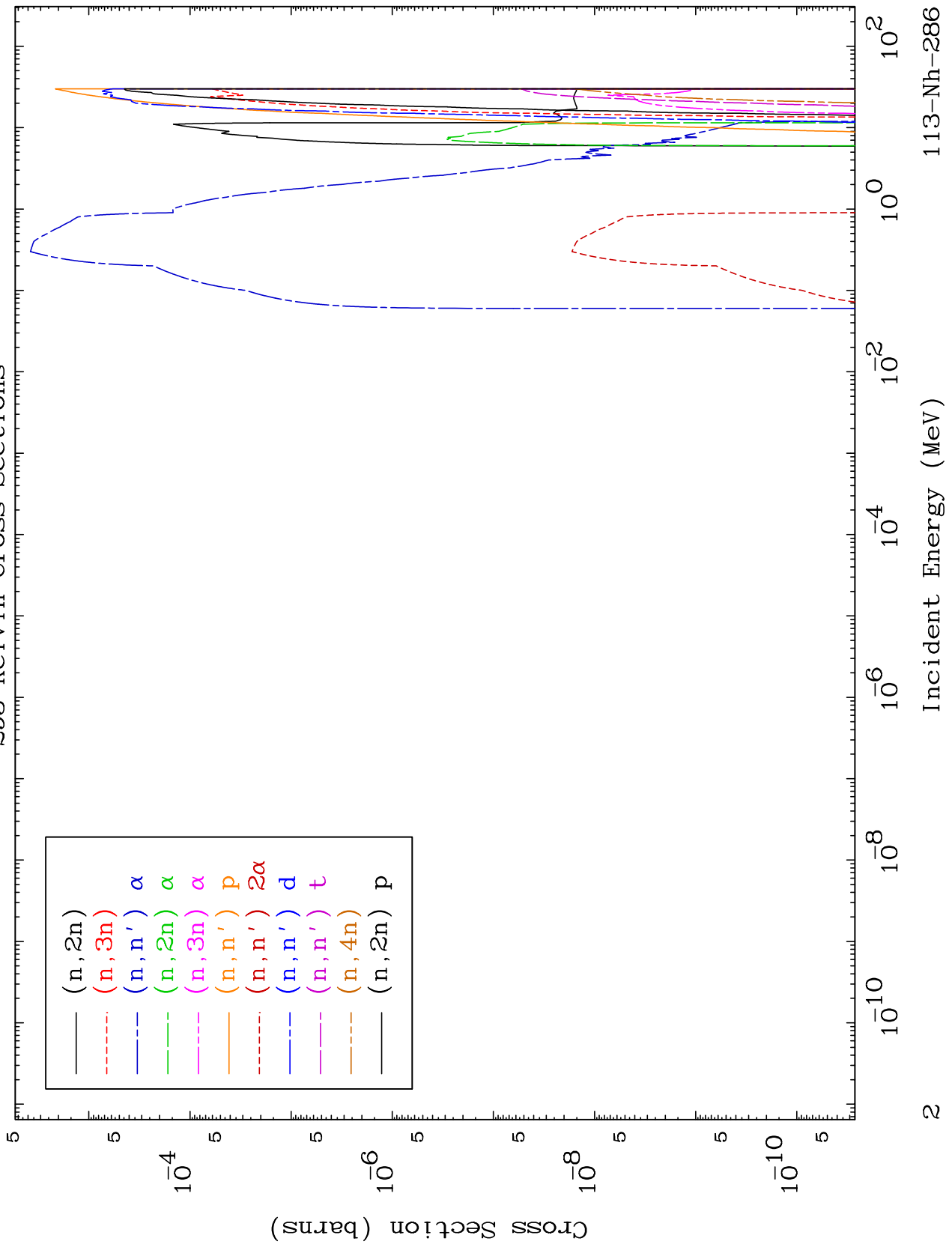
113-Nh-286



MAT 1386

Neutron Production  
293 Kelvin Cross Sections

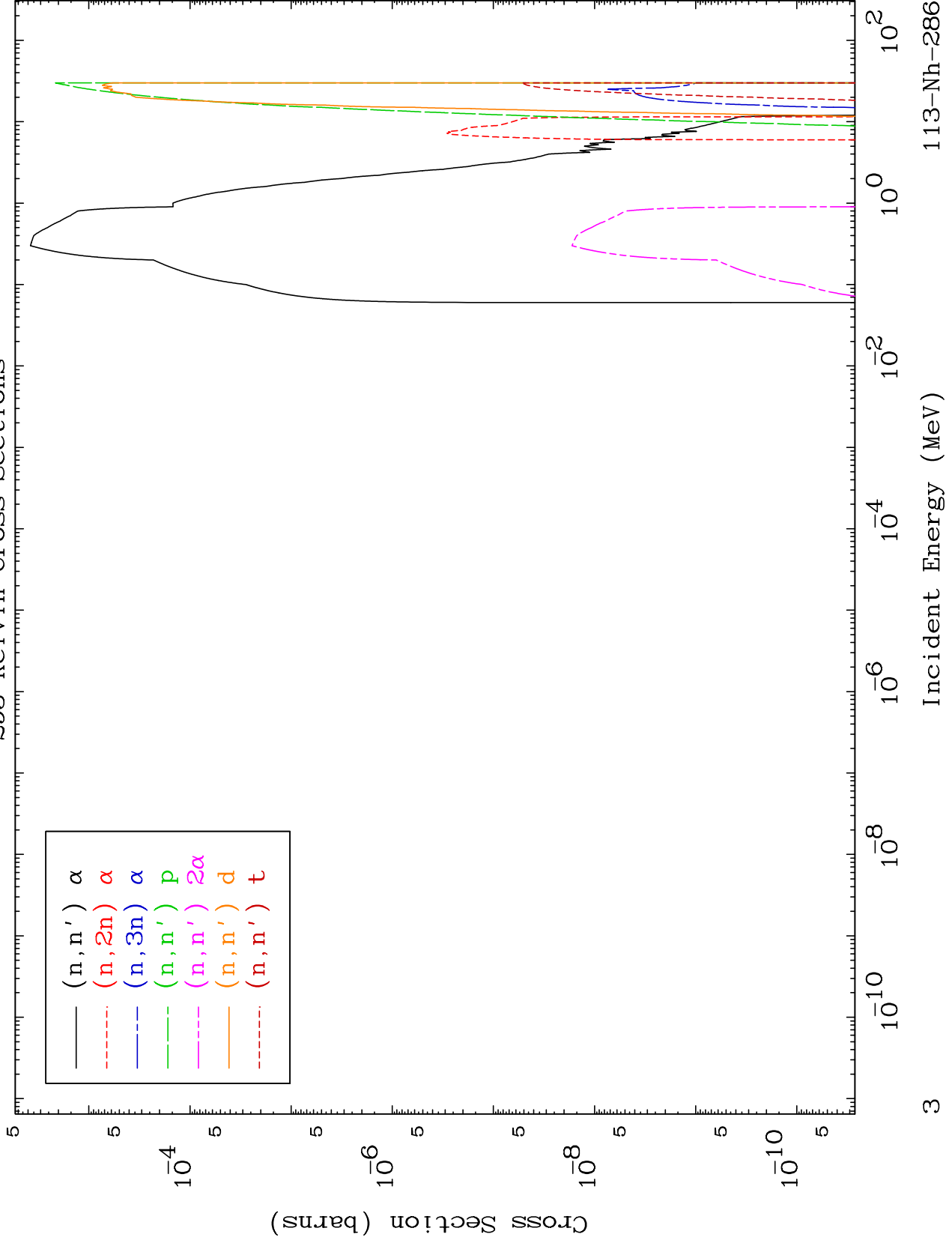
113-Nh-286



MAT 1386

Charged Particle  
293 Kelvin Cross Sections

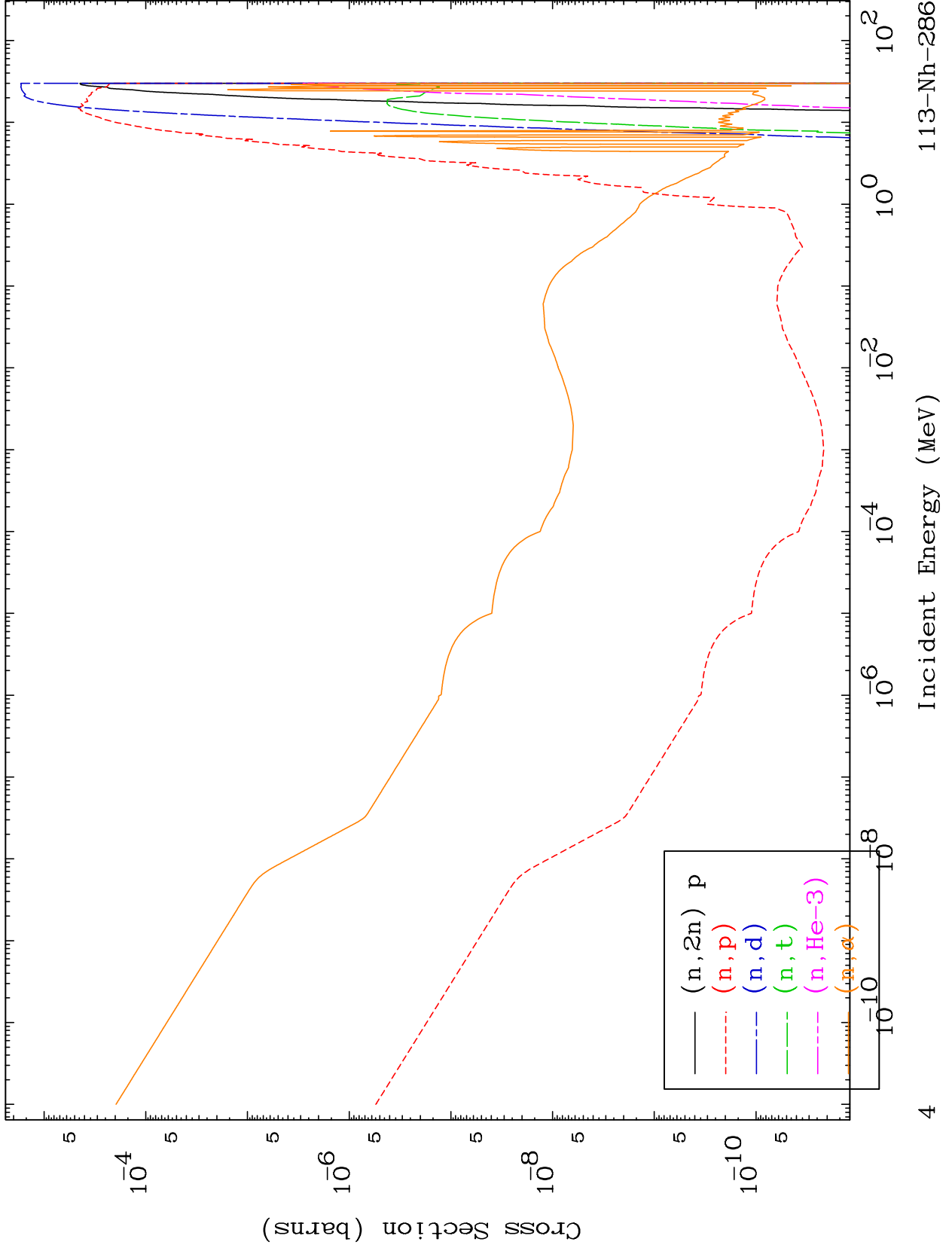
113-Nh-286



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Charged Particle  
293 Kelvin Cross Sections

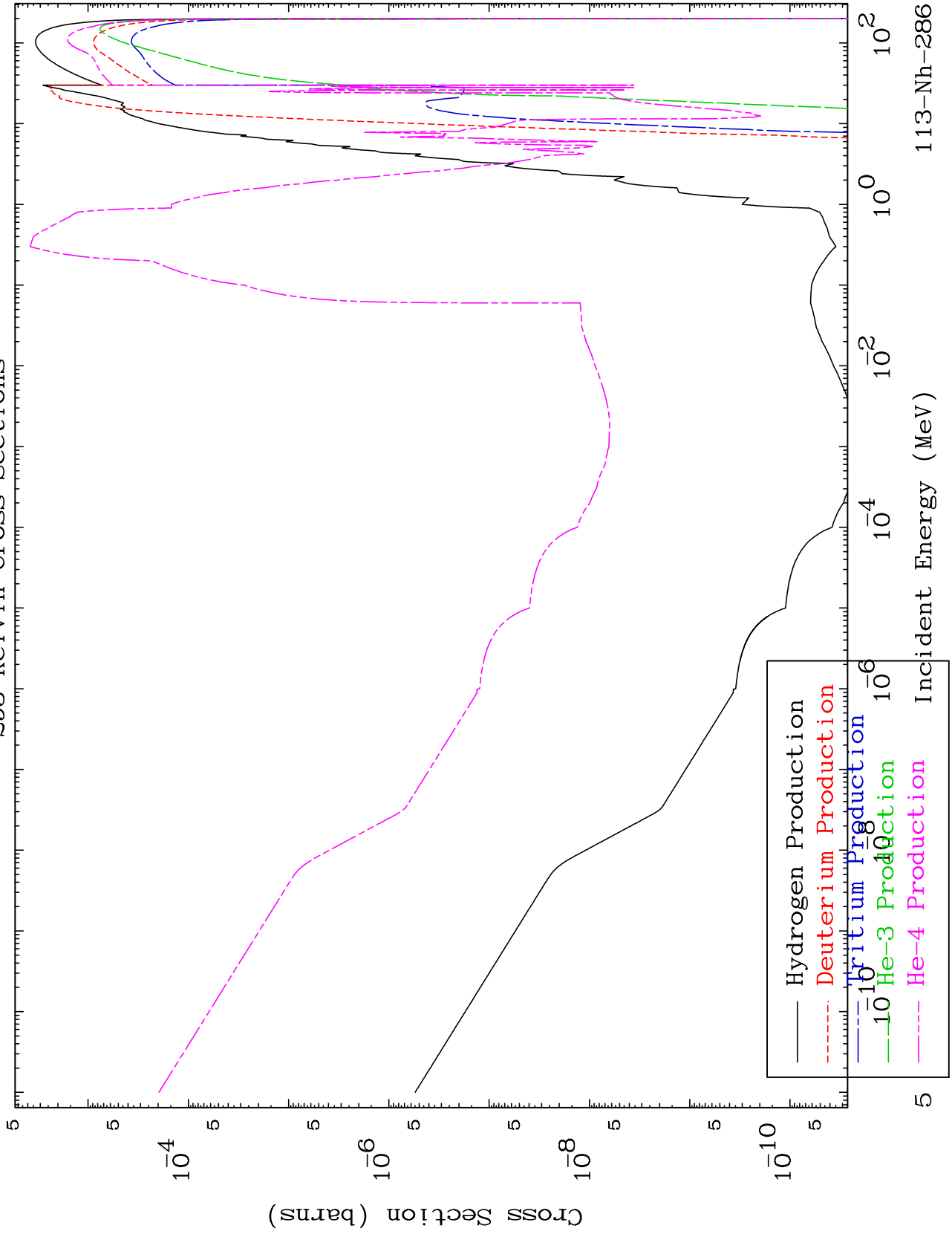
113-Nh-286



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Particle Production  
293 Kelvin Cross Sections

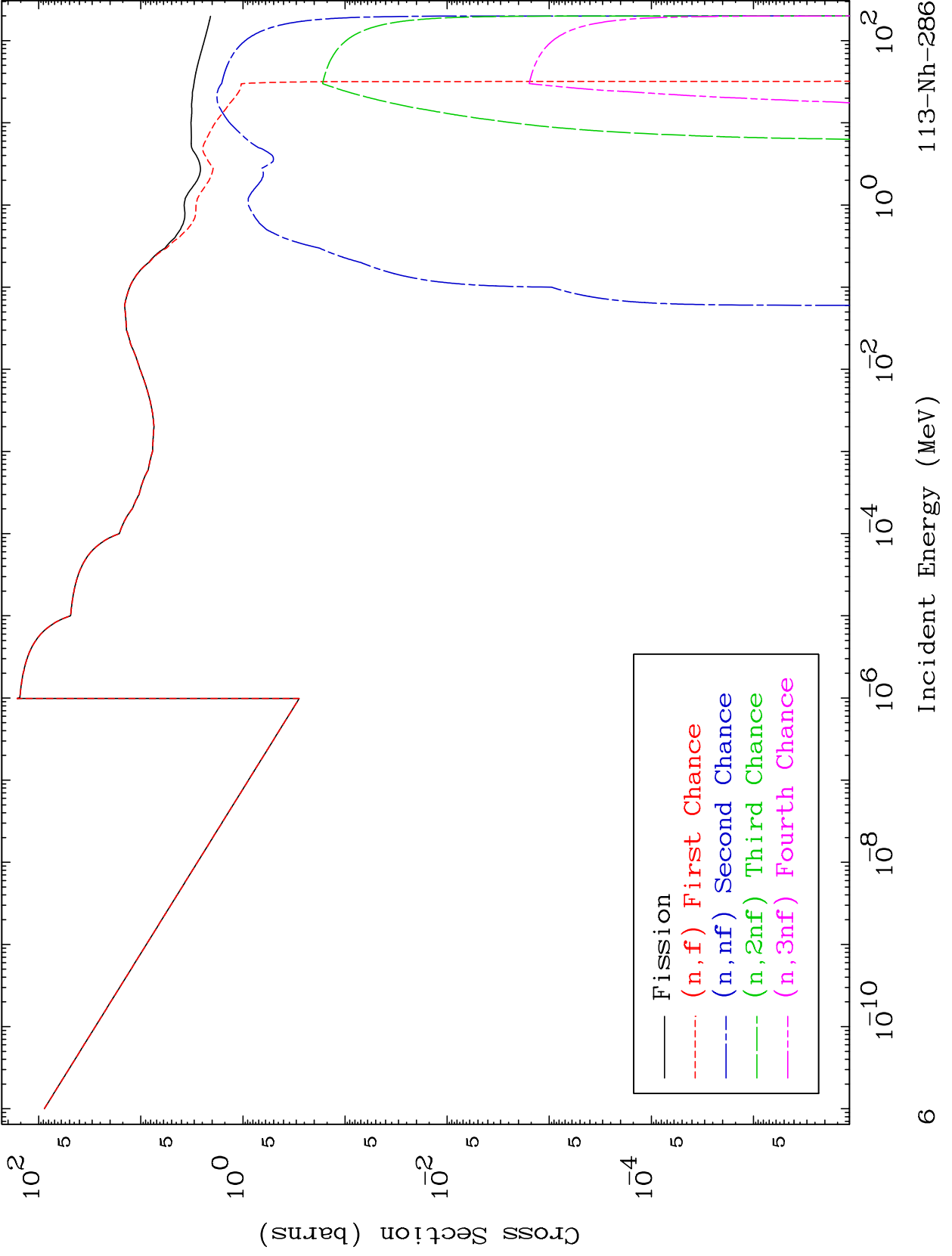
113-Nh-286



MAT 1386

Fission  
293 Kelvin Cross Sections

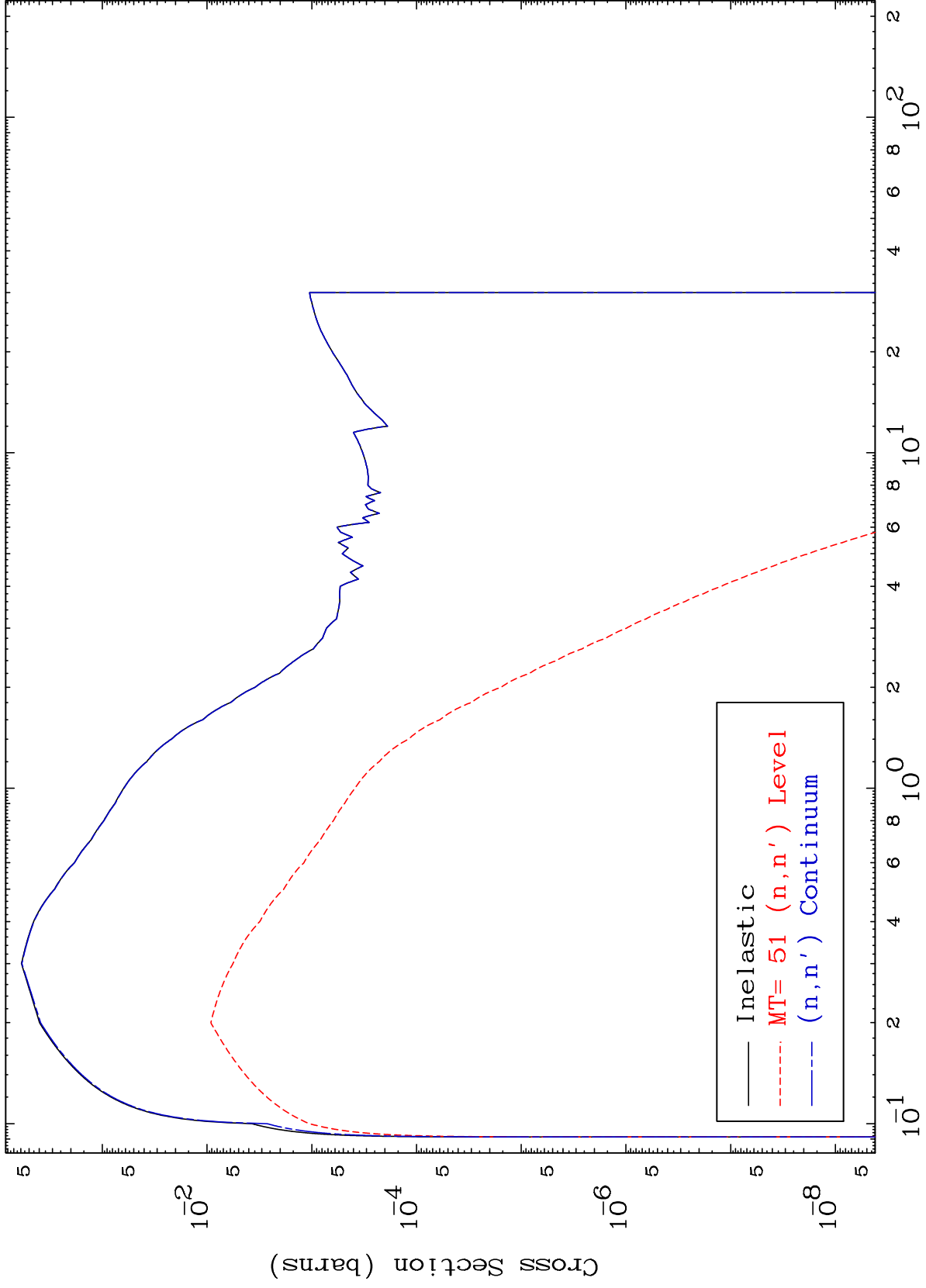
113-Nh-286



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(n,n') Level  
293 Kelvin Cross Sections

113-Nh-286



Incident Energy (MeV)

113-Nh-286

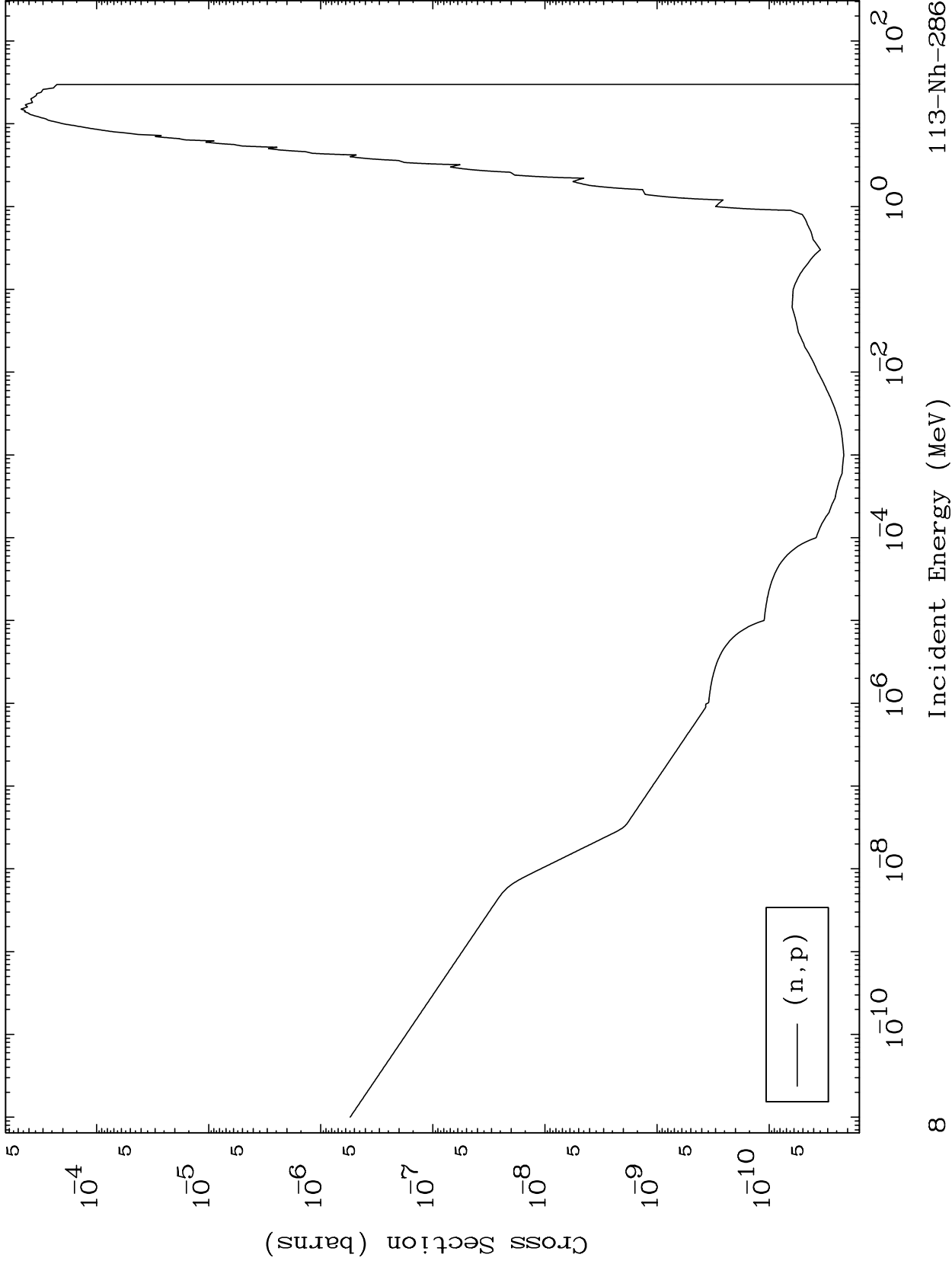
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(n,p) Levels  
293 Kelvin Cross Sections

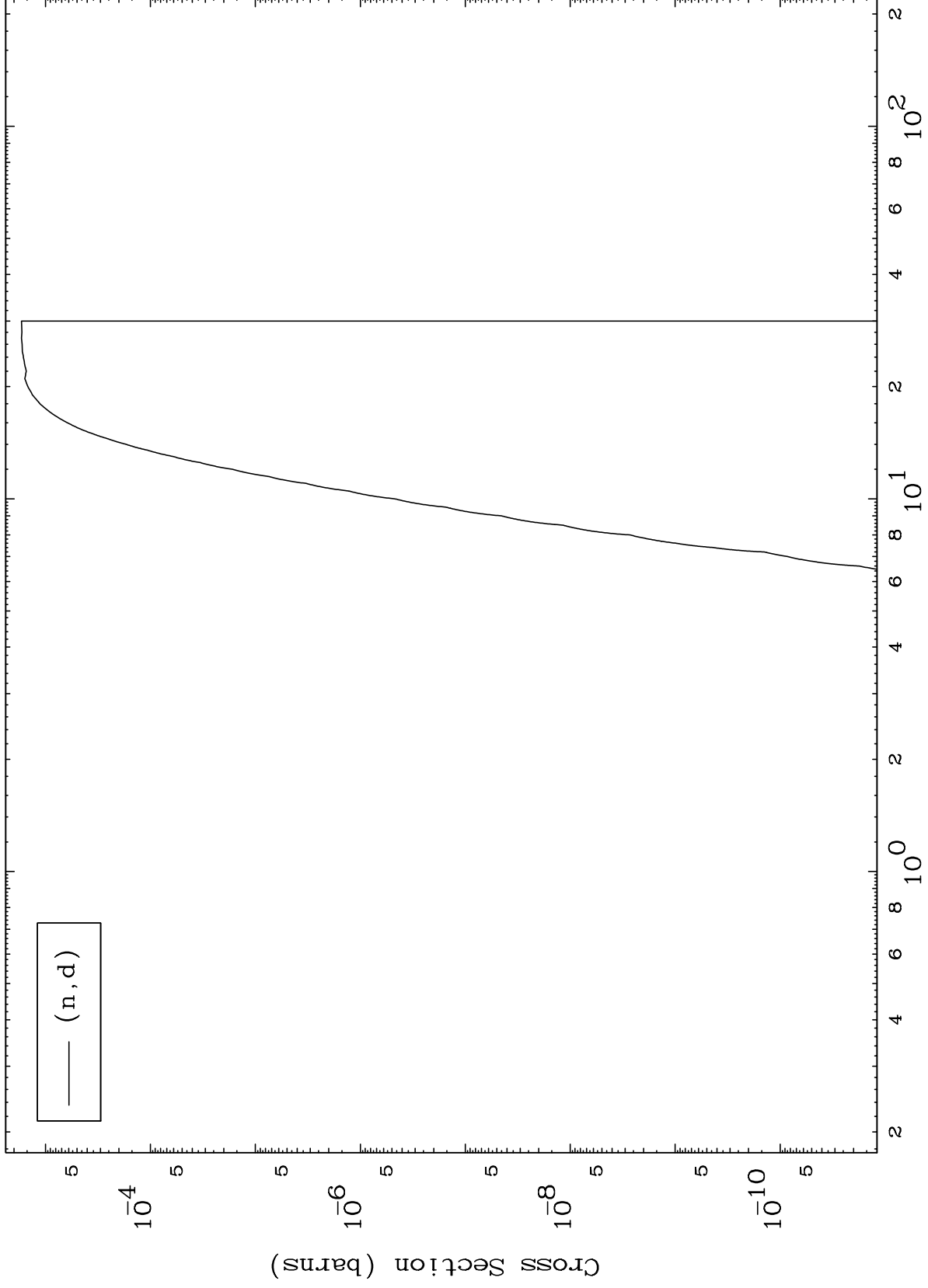
113-Nh-286



MAT 1386

(n,d) Levels  
293 Kelvin Cross Sections

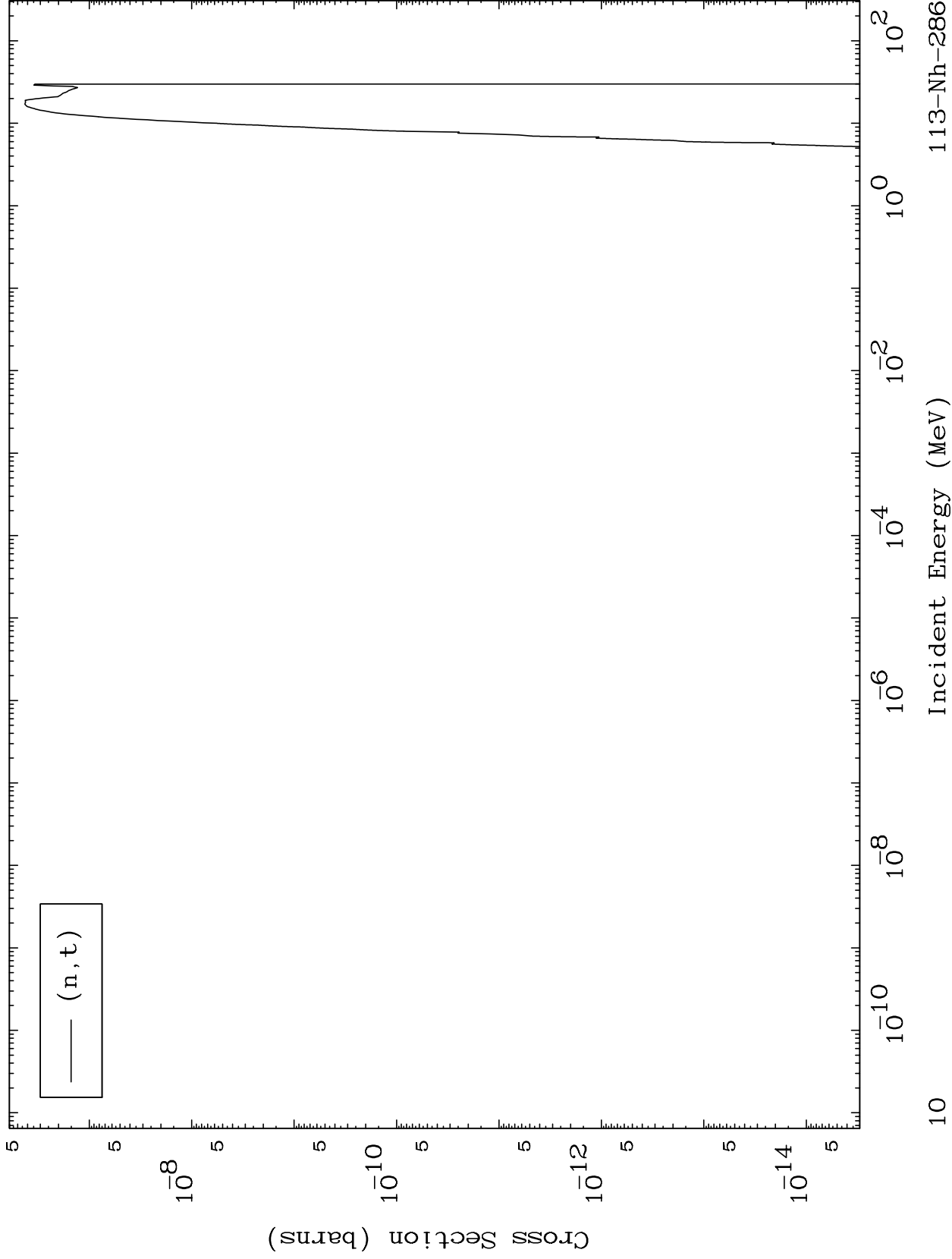
113-Nh-286



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(n,t) Levels  
293 Kelvin Cross Sections

113-Nh-286



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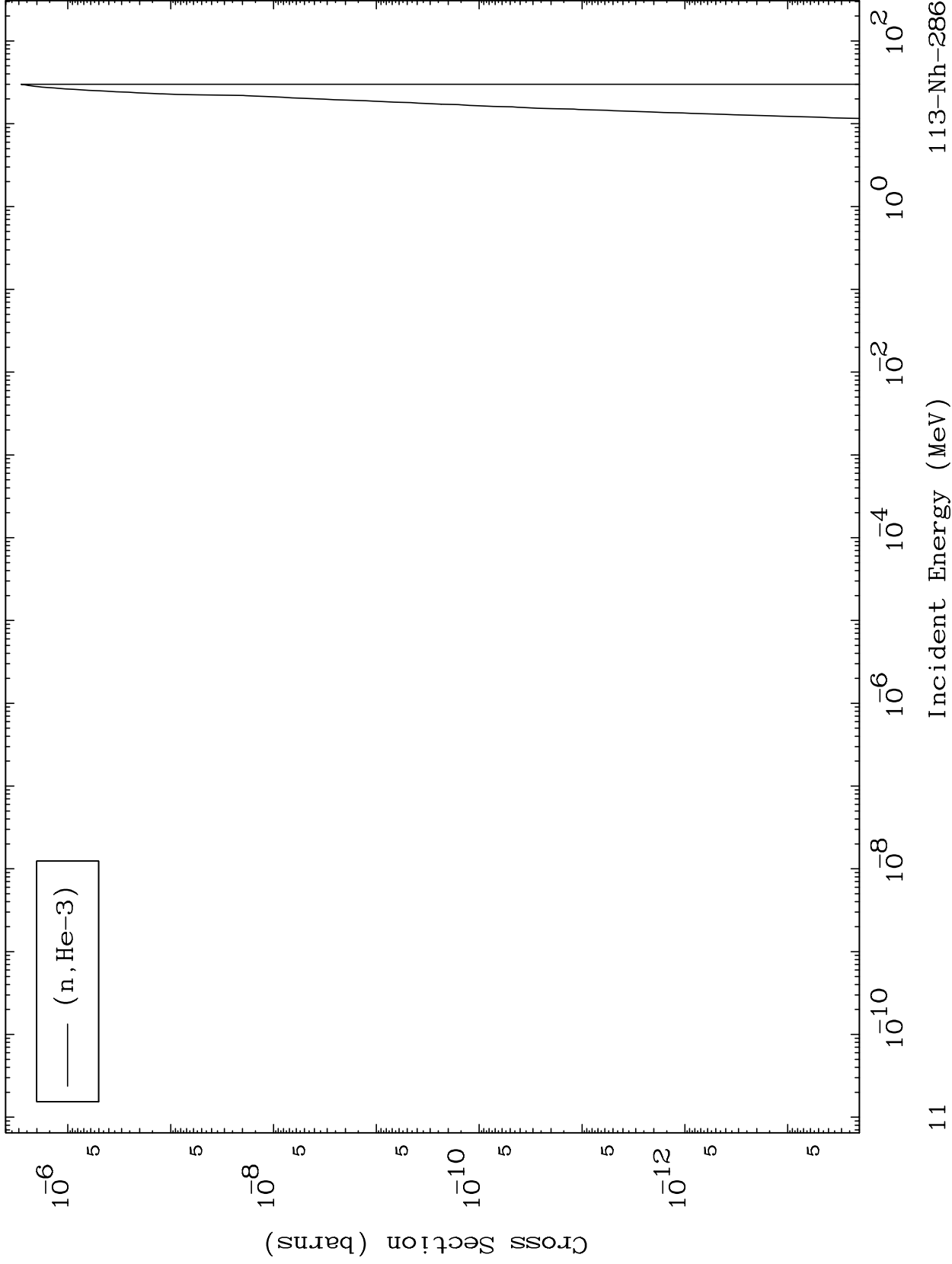
Incident Energy (MeV)

113-Nh-286

MAT 1386

(n,He3) Levels  
293 Kelvin Cross Sections

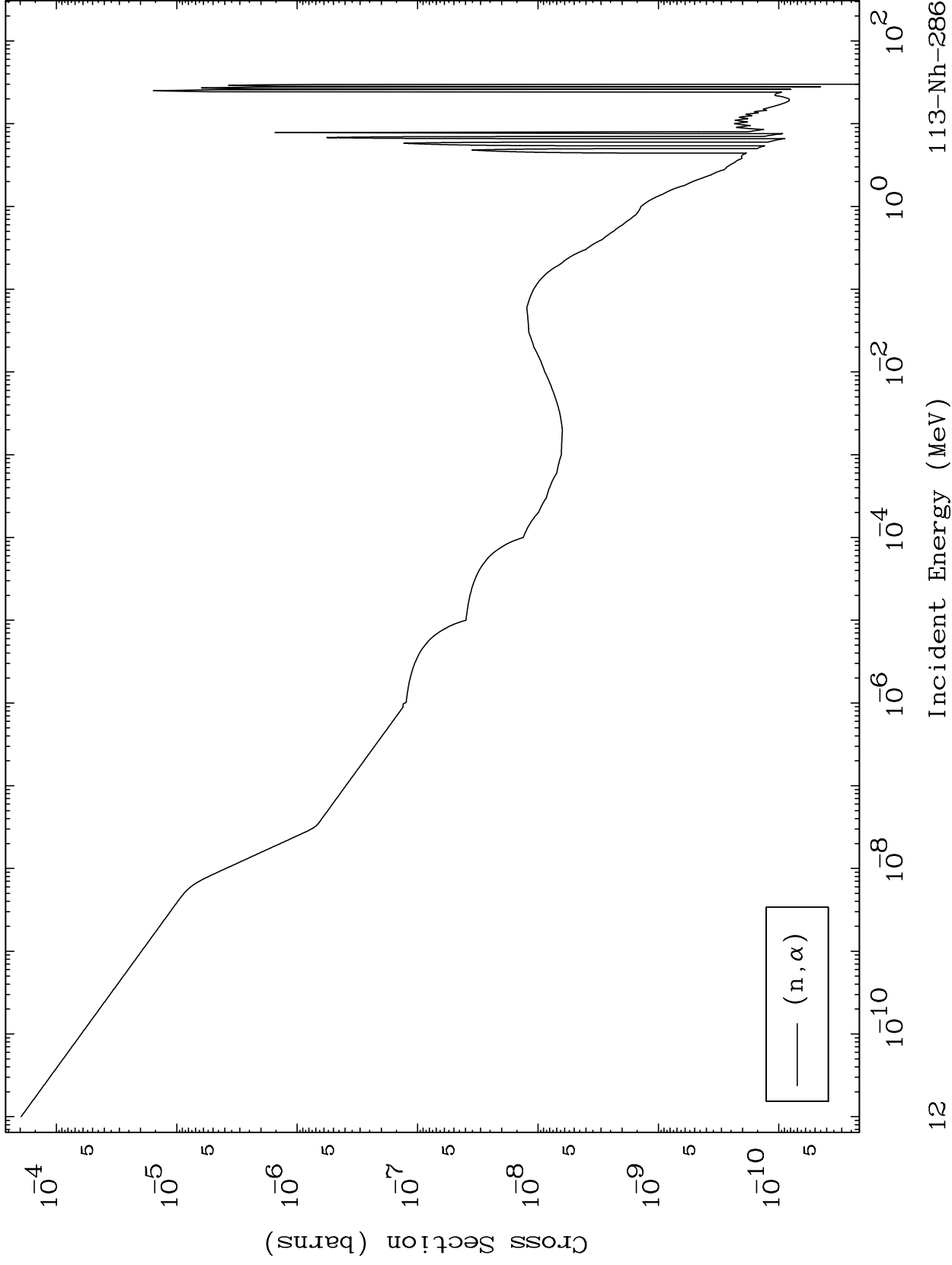
113-Nh-286

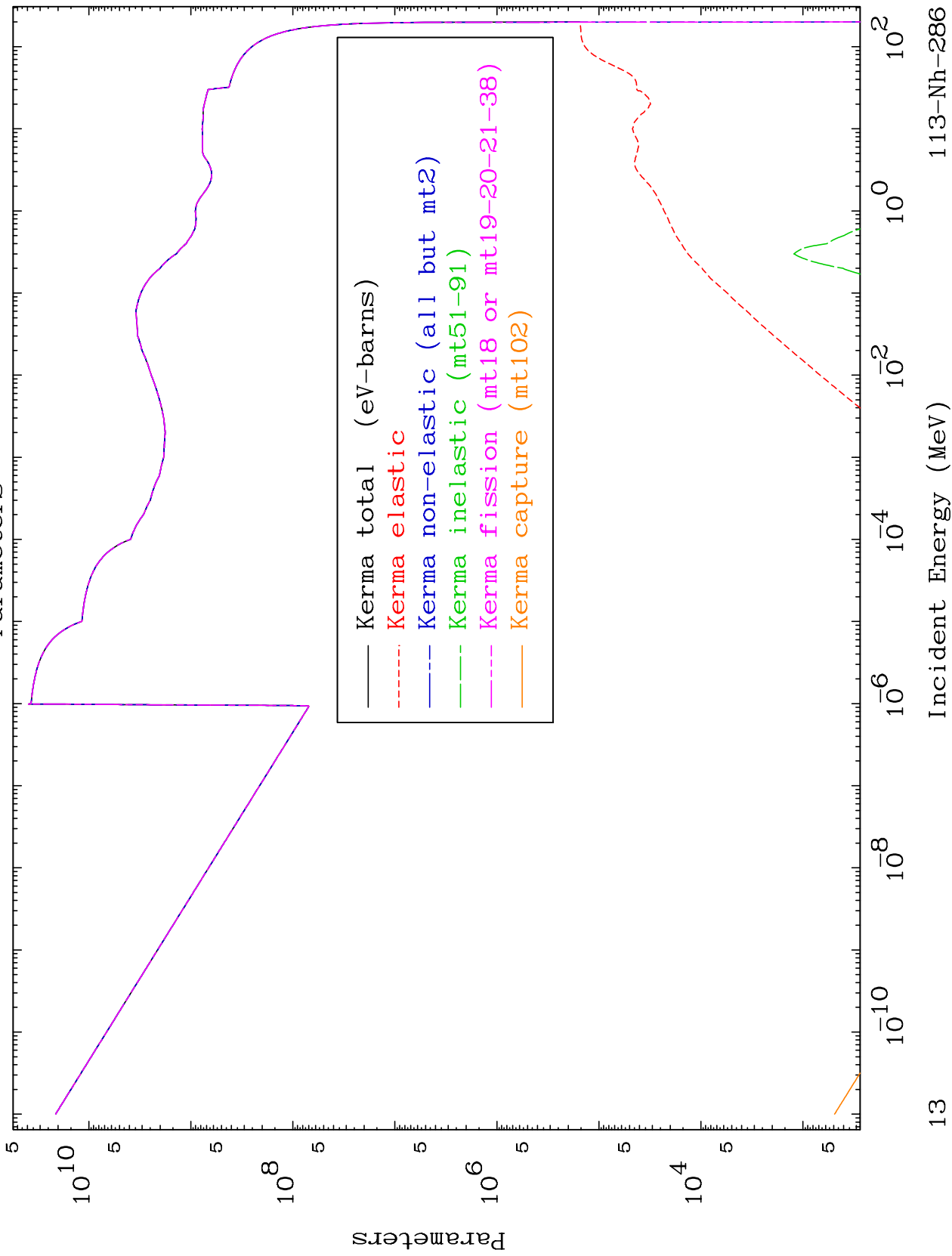


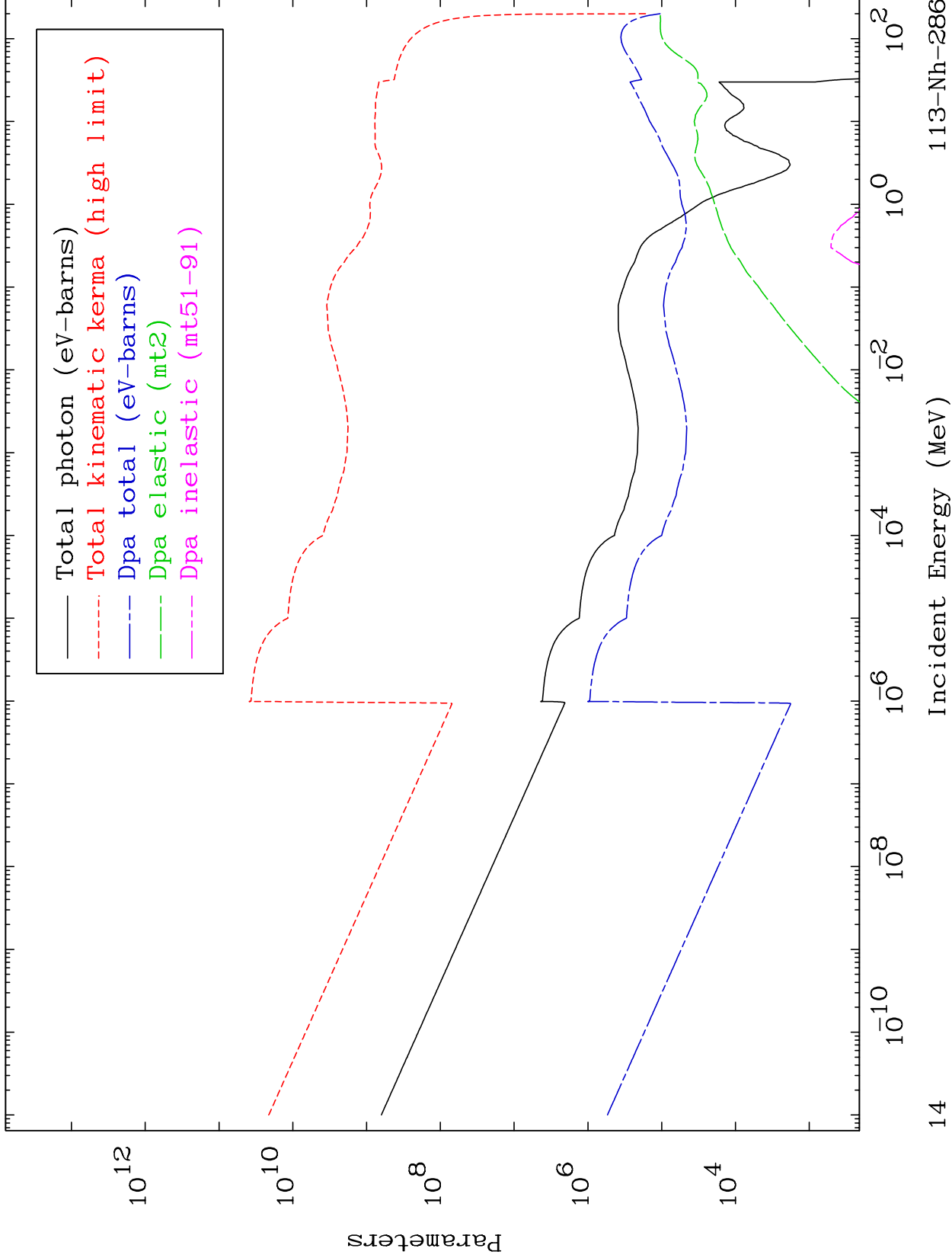
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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

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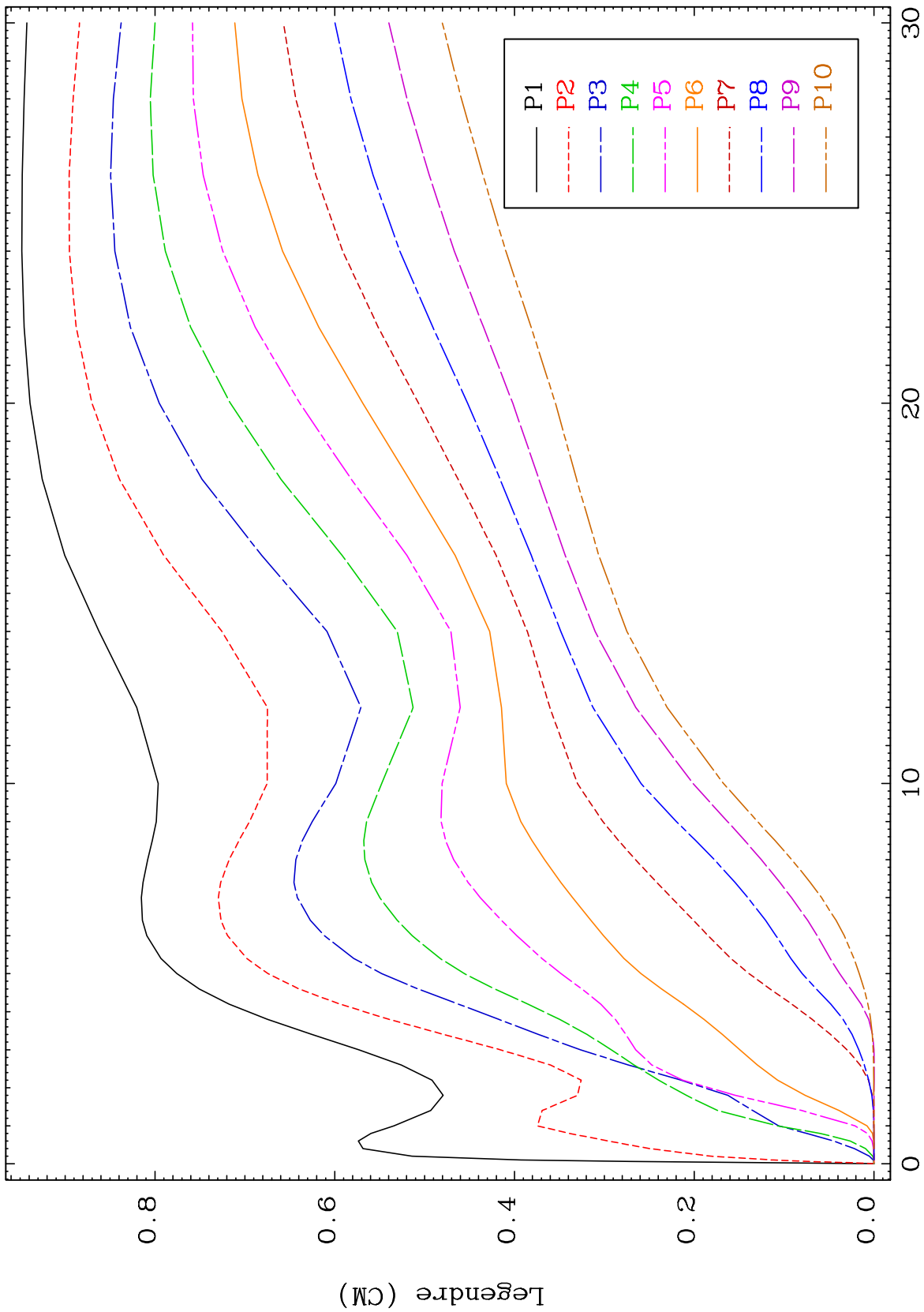




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Elastic Legendre Coefficients



113-Nh-286

Incident Energy (MeV)

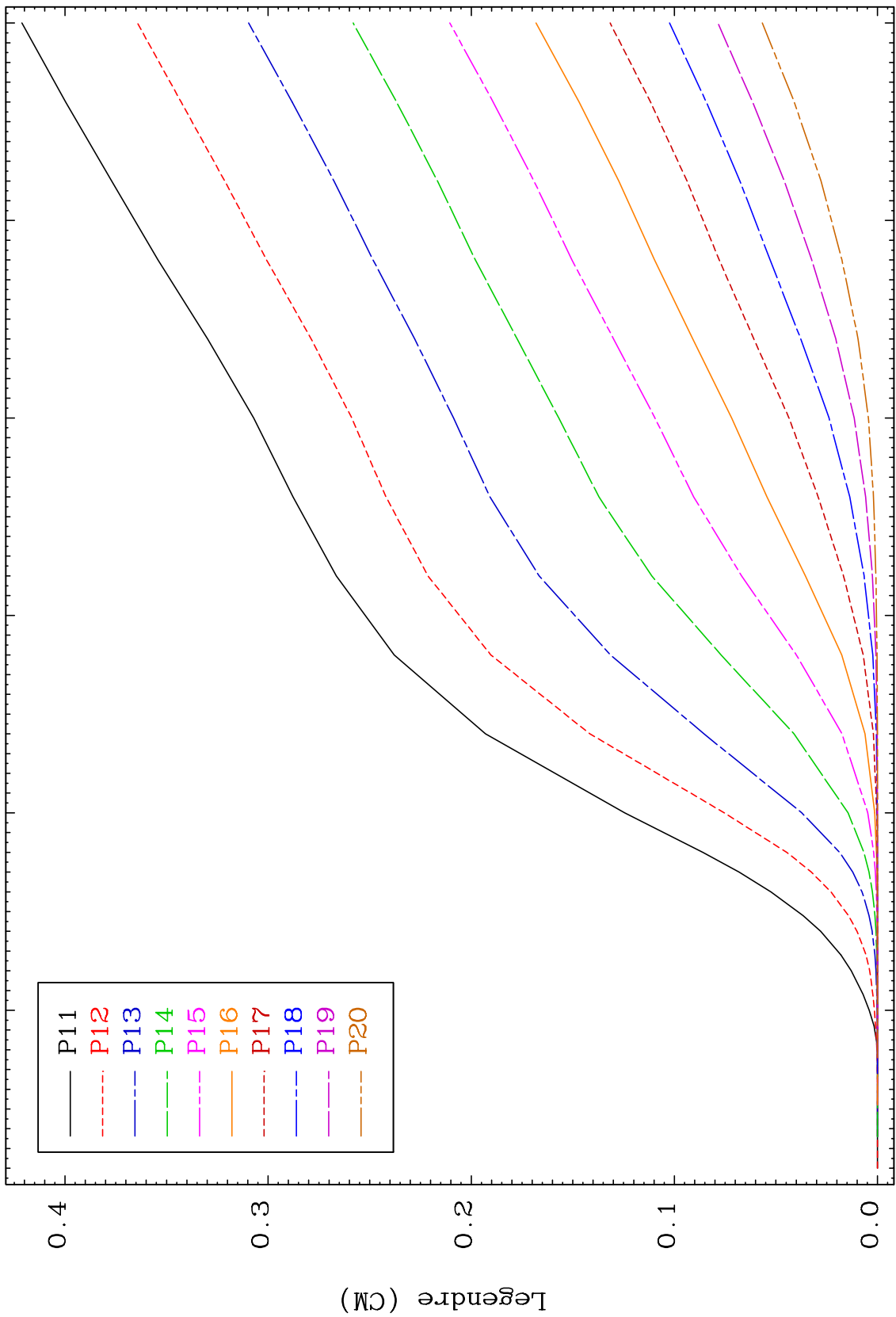
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Elastic Legendre Coefficients

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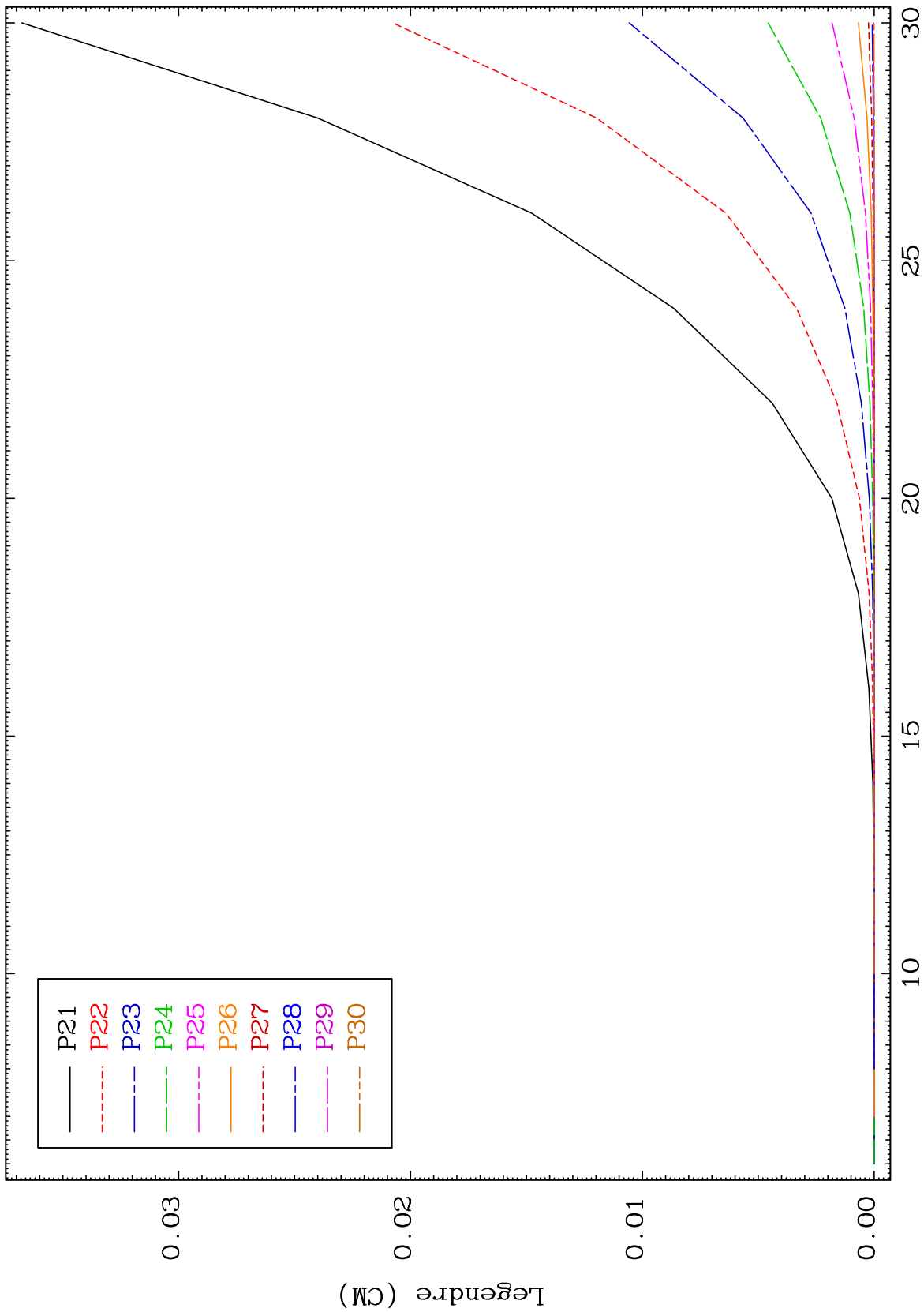
Incident Energy (MeV)

113-Nh-286

MAT 1386

Elastic Legendre Coefficients

113-Nh-286



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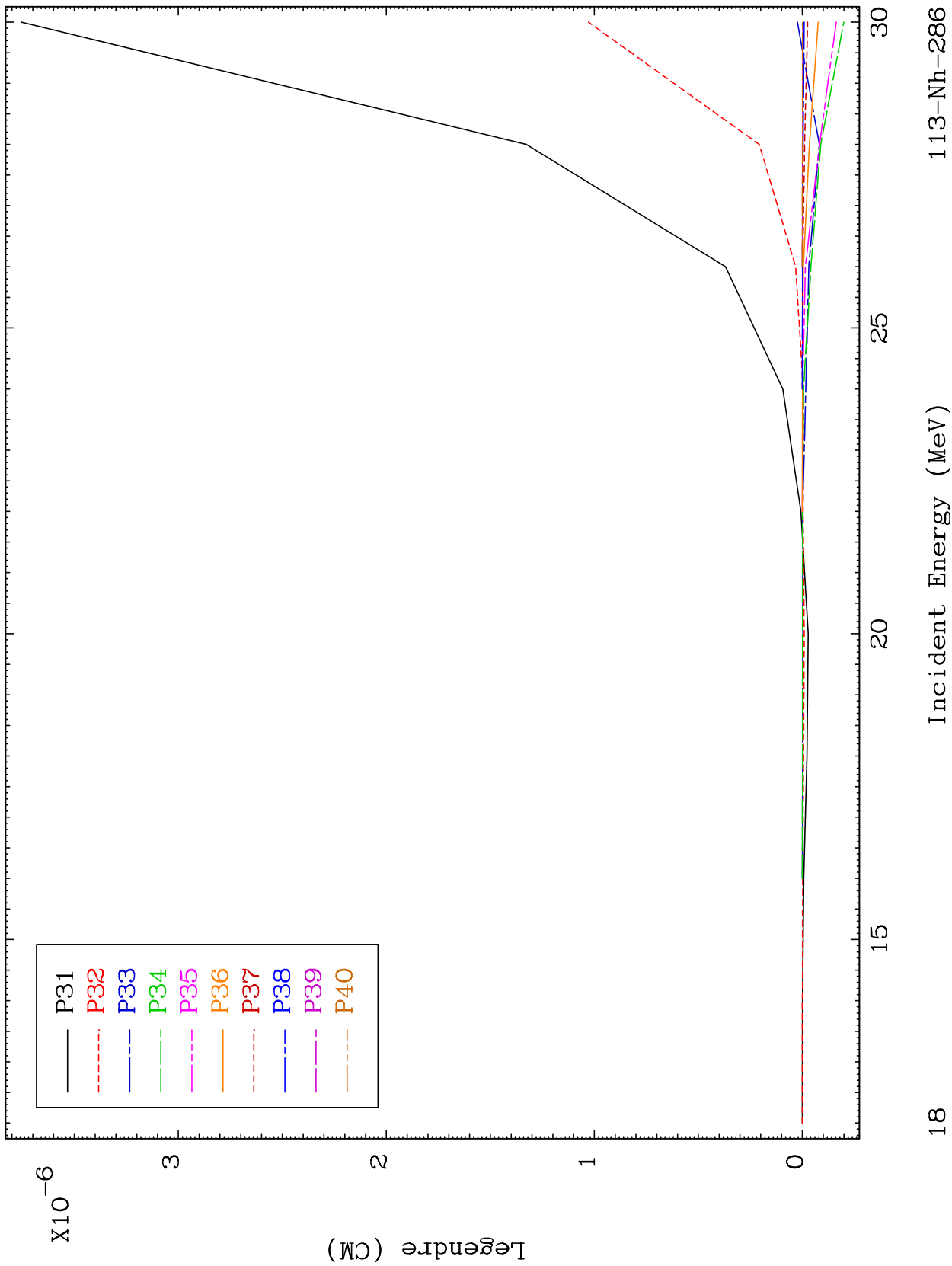
Incident Energy (MeV)

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Elastic  
Legendre Coefficients

113-Nh-286

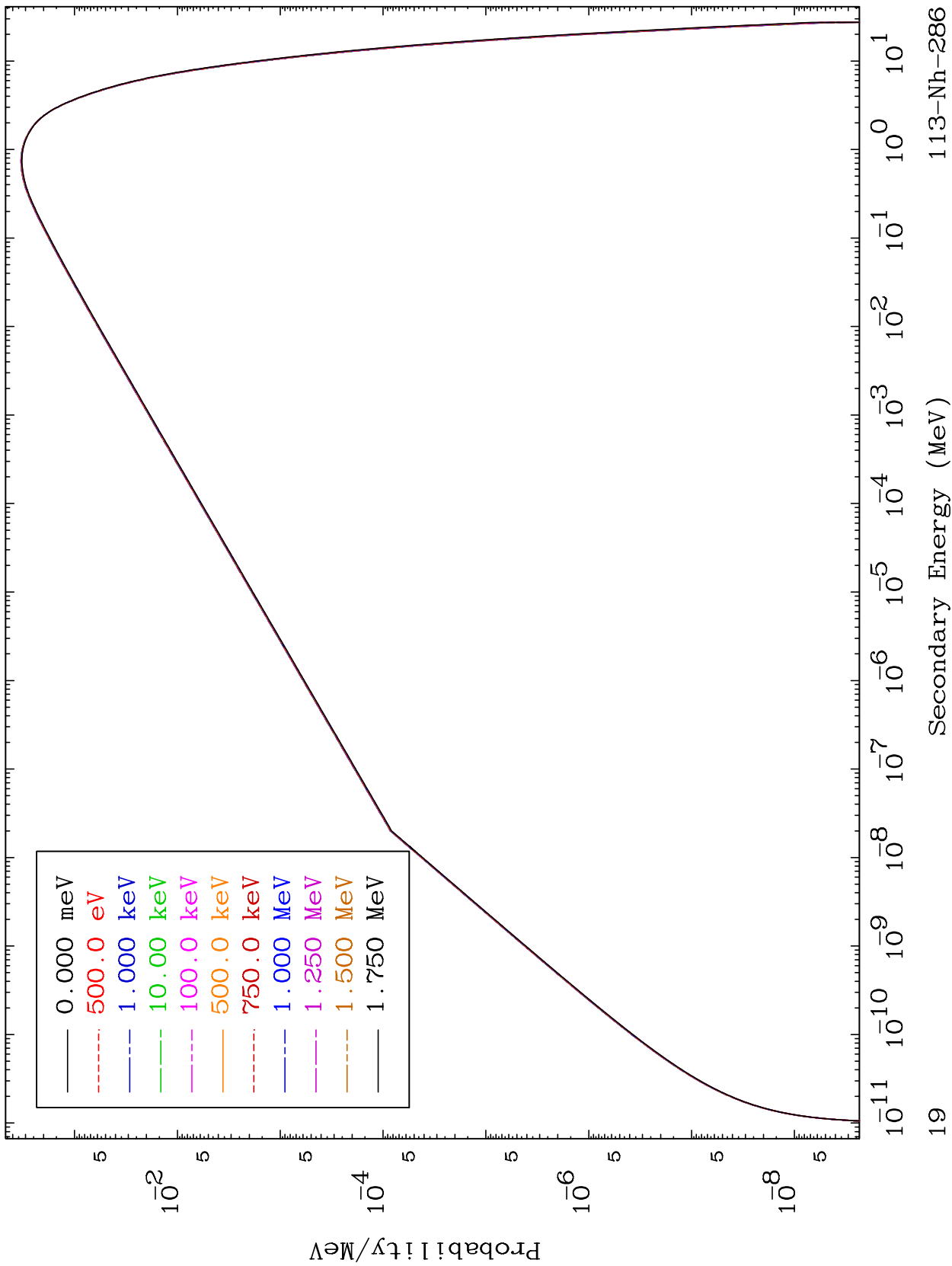


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Incident Energy (MeV)

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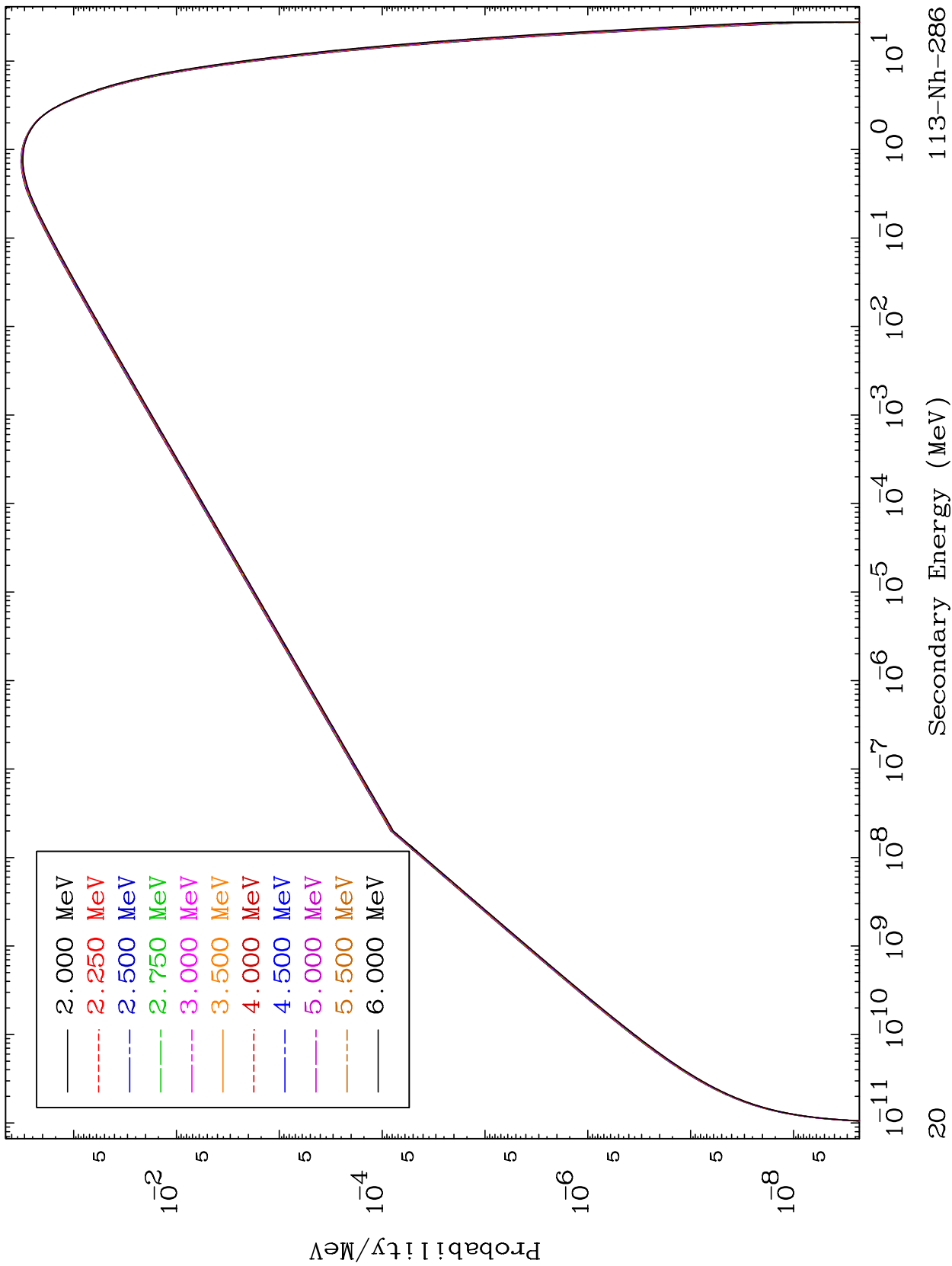
Fission Energy Distribution



MAT 1386

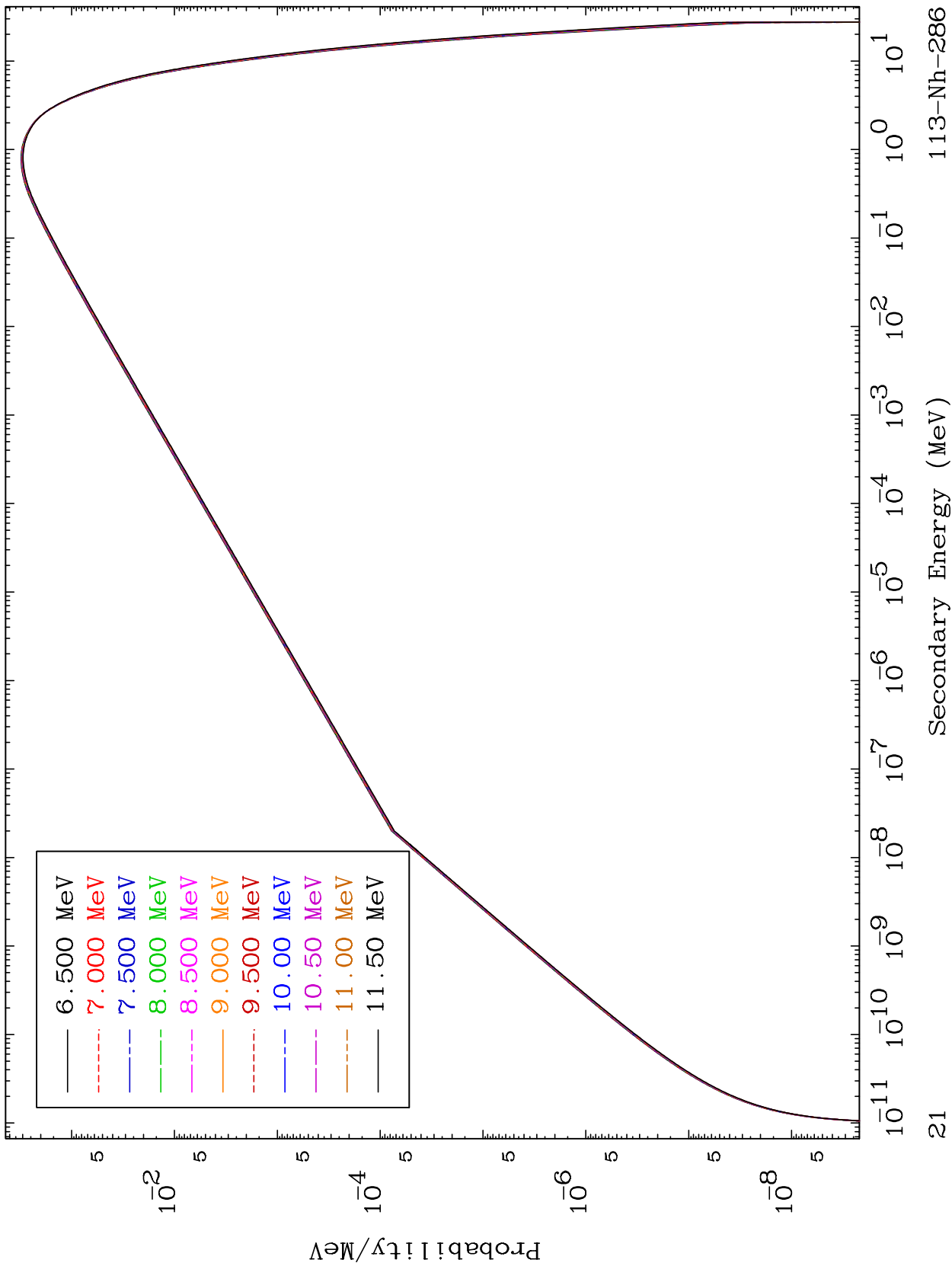
### Fission Energy Distribution

113-Nh-286



113-Nh-286

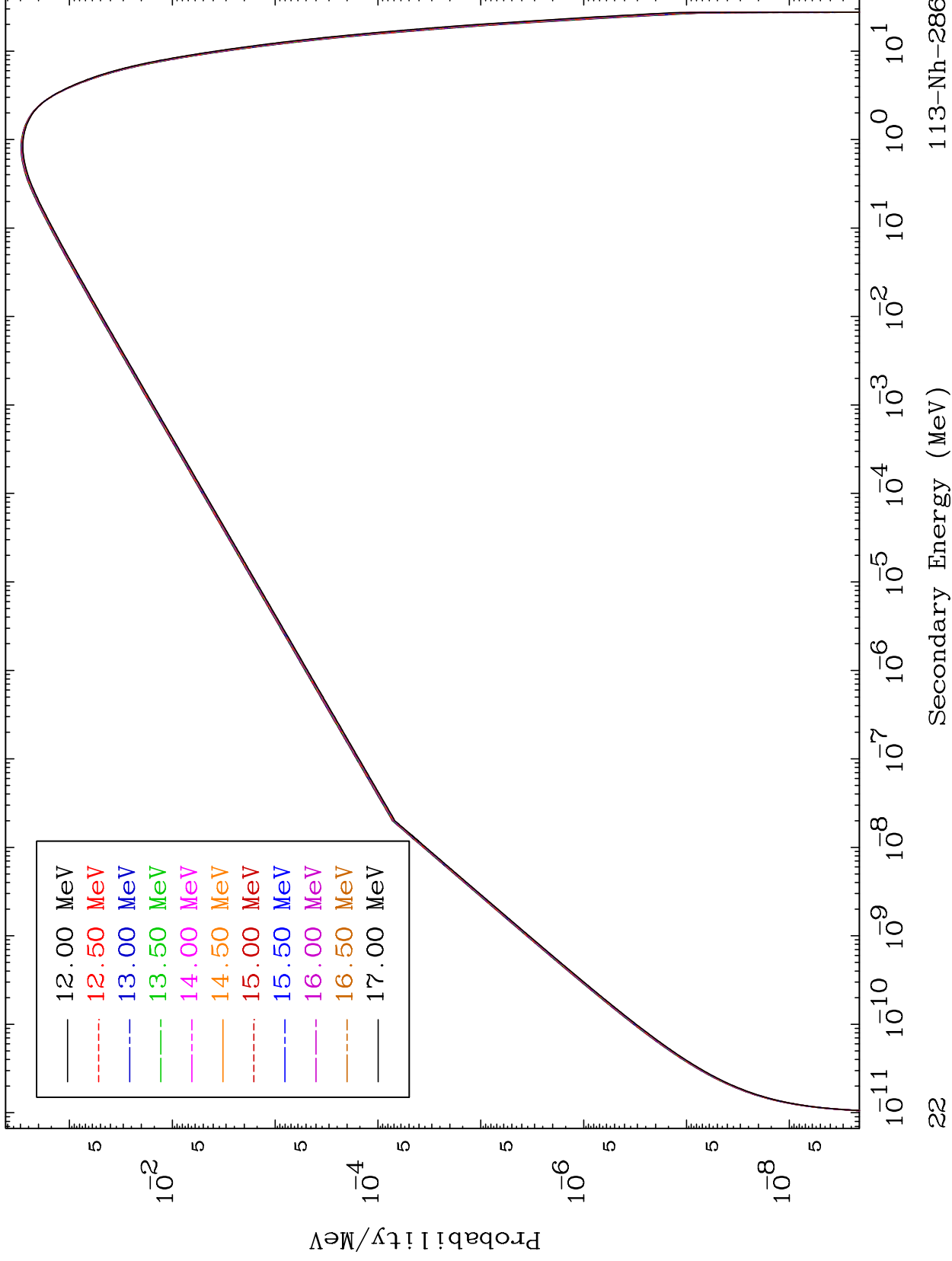
Secondary Energy (MeV)



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### Fission Energy Distribution

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<sup>113</sup>Nh-286

Fission Energy Distribution

