

Program EVALPLOT
(Version 2018-1)

by

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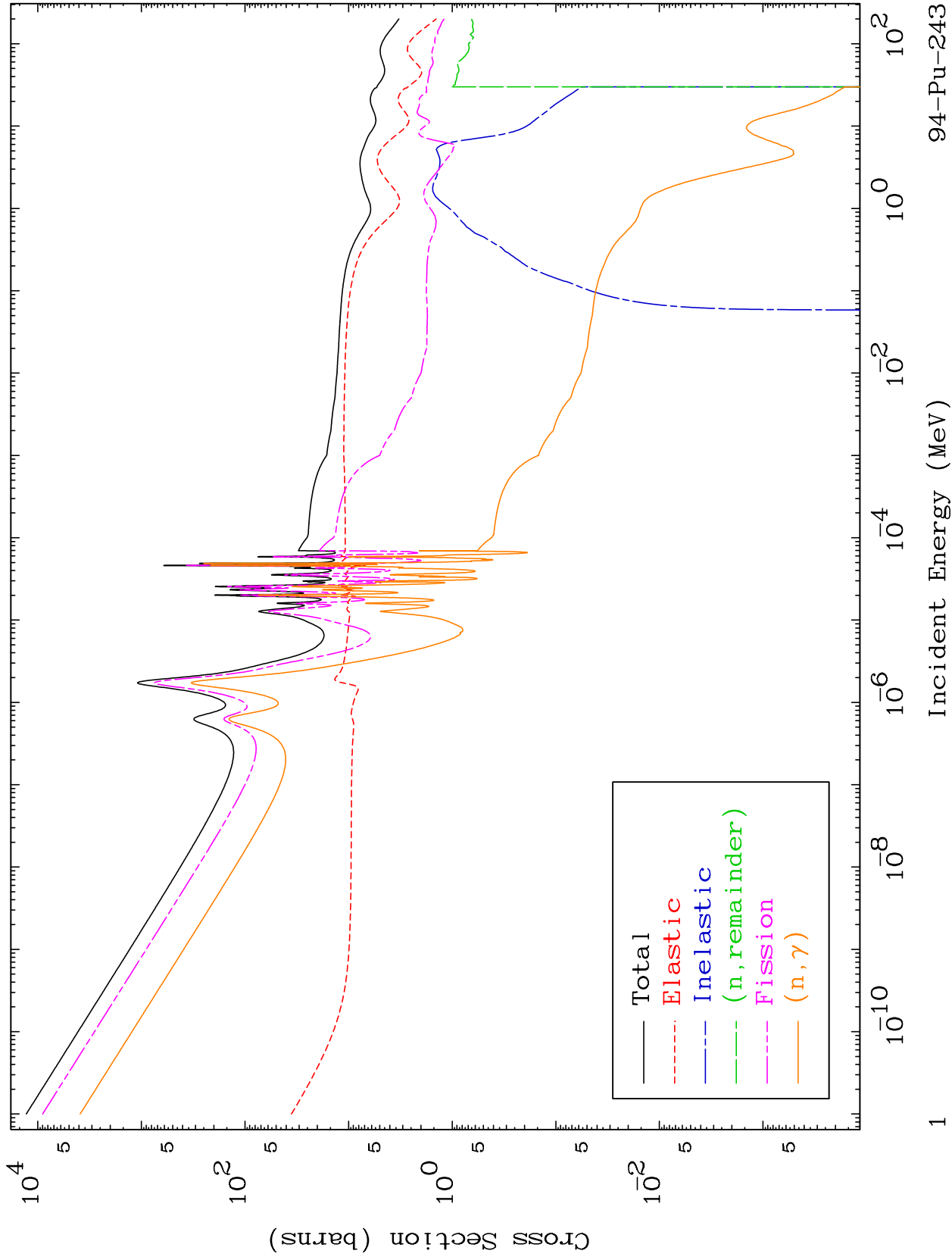
Press Mouse Button to Start

MAT 9449

Major

293 Kelvin Cross Sections

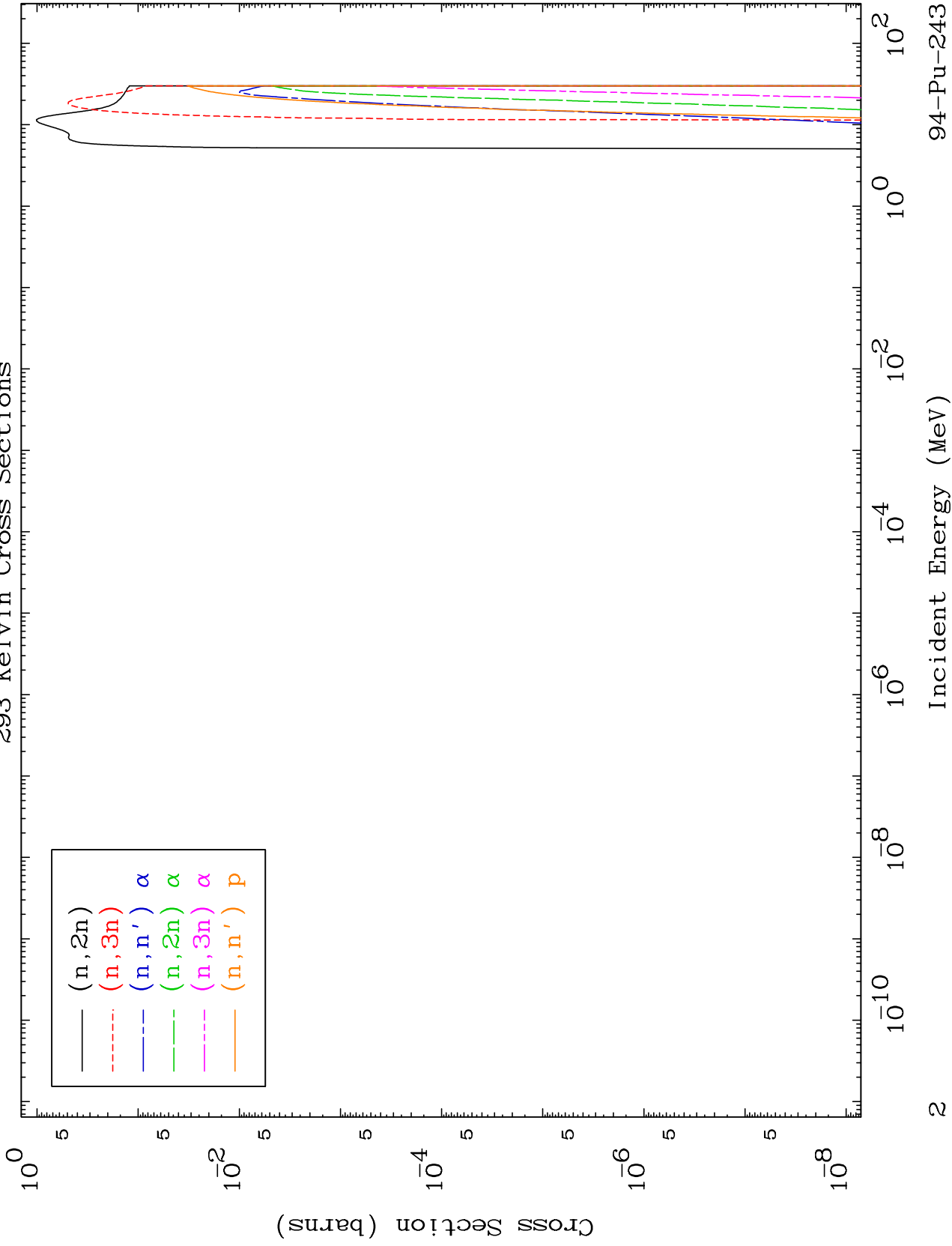
94-Pu-243



MAT 9449

Neutron Production
293 Kelvin Cross Sections

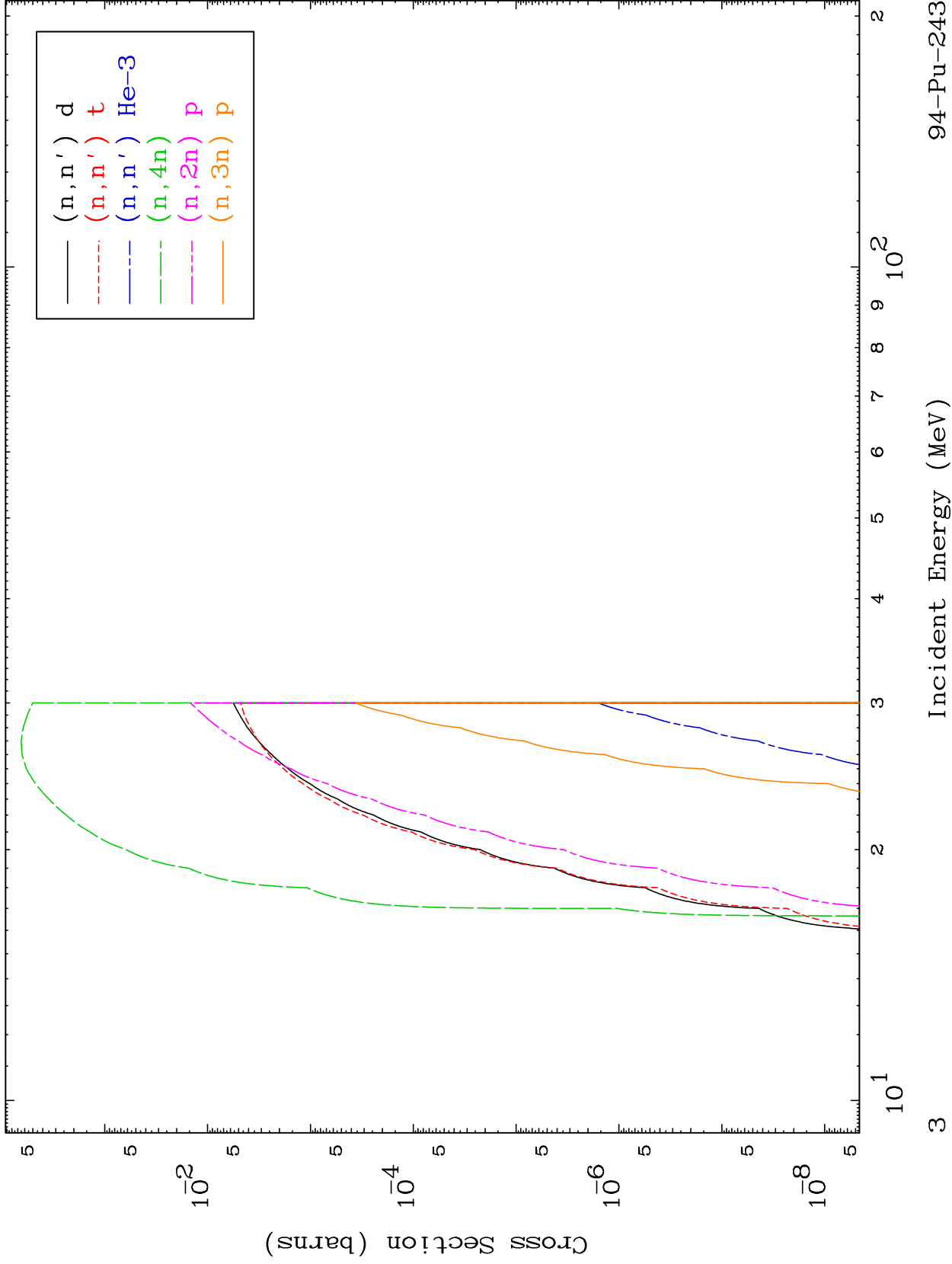
94-Pu-243



MAT 9449

Neutron Production
293 Kelvin Cross Sections

94-Pu-243



94-Pu-243

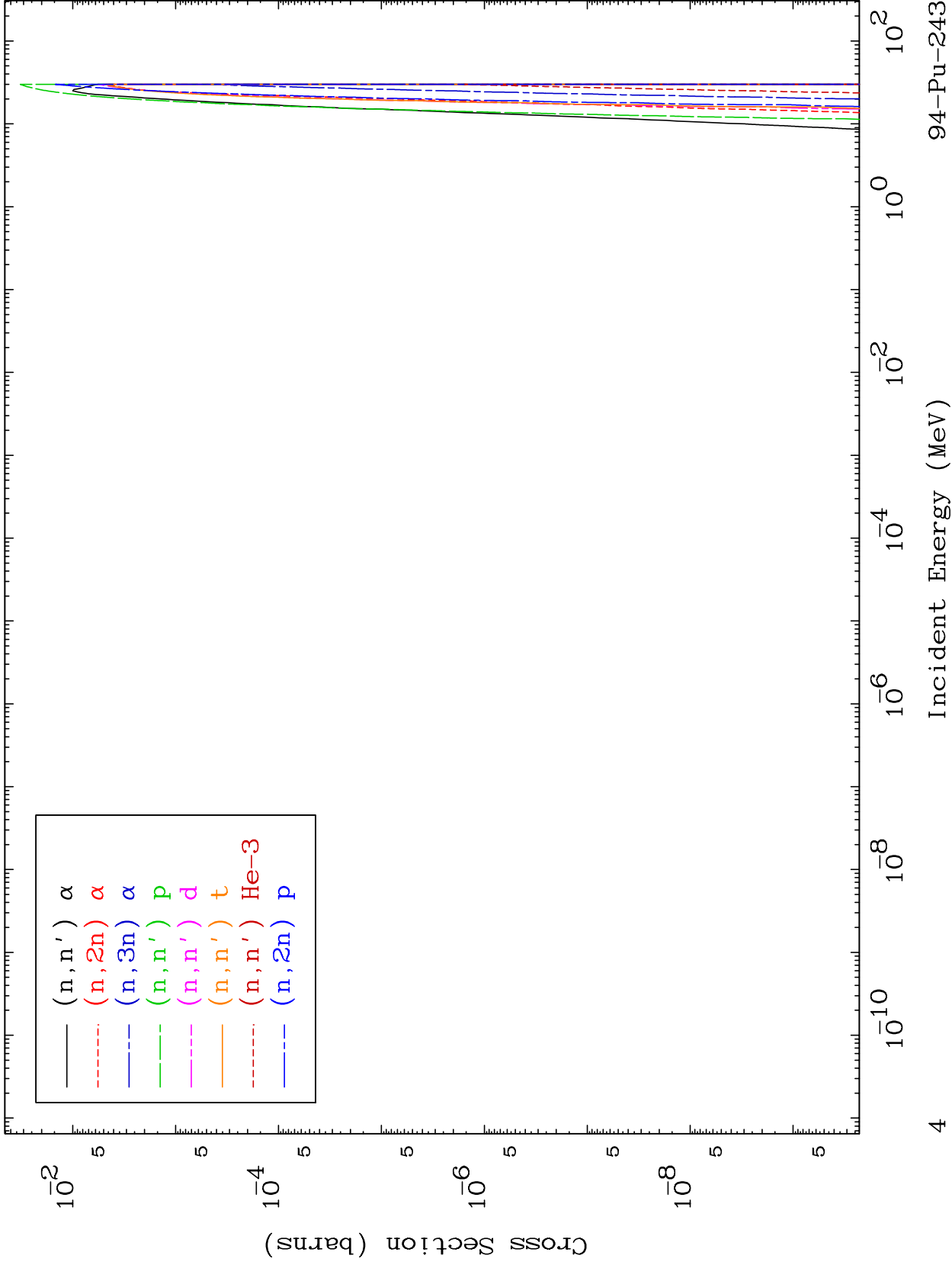
Incident Energy (MeV)

3

MAT 9449

Charged Particle
293 Kelvin Cross Sections

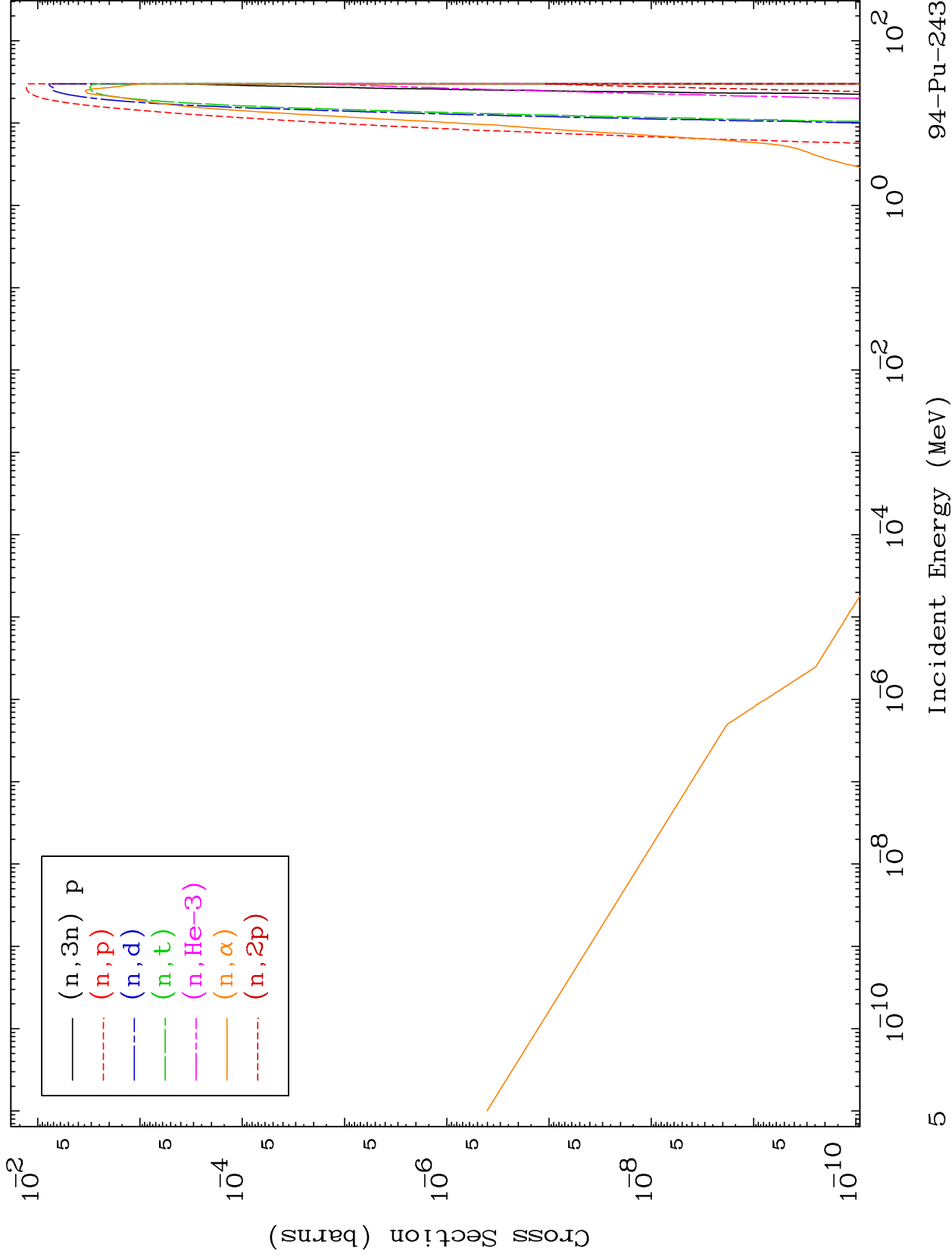
94-Pu-243



MAT 9449

Charged Particle
293 Kelvin Cross Sections

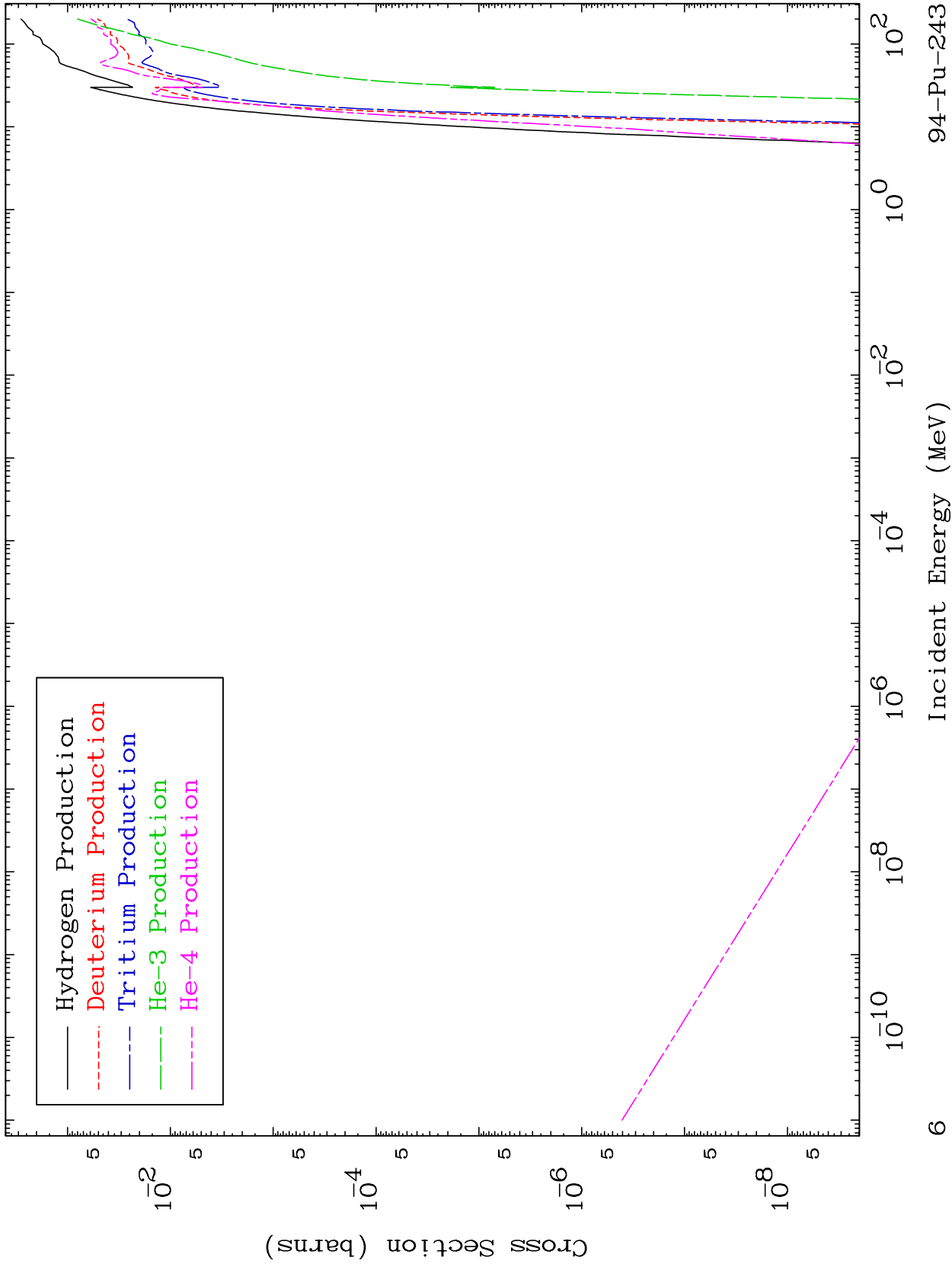
94-Pu-243



MAT 9449

Particle Production
293 Kelvin Cross Sections

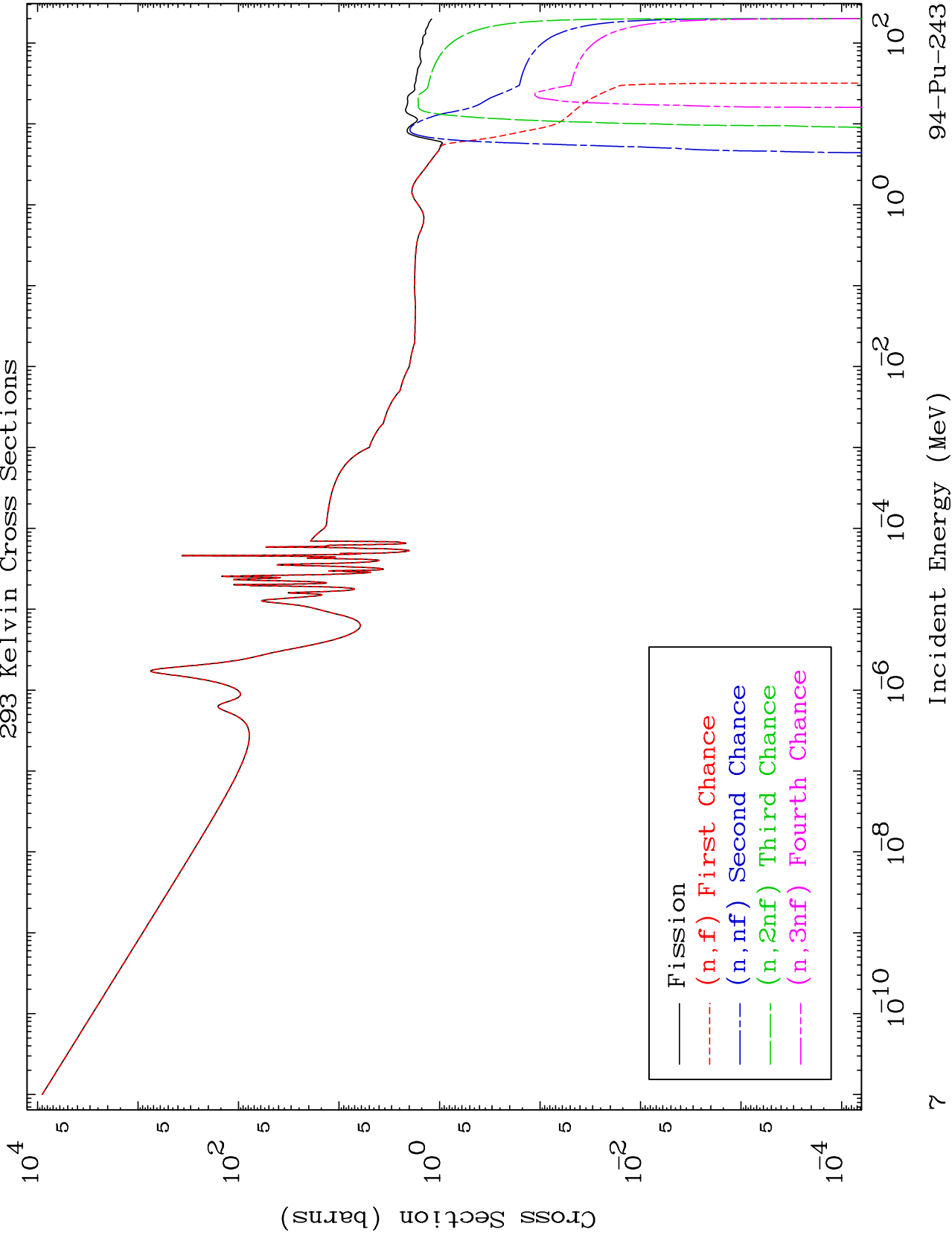
94-Pu-243



MAT 9449

Fission
293 Kelvin Cross Sections

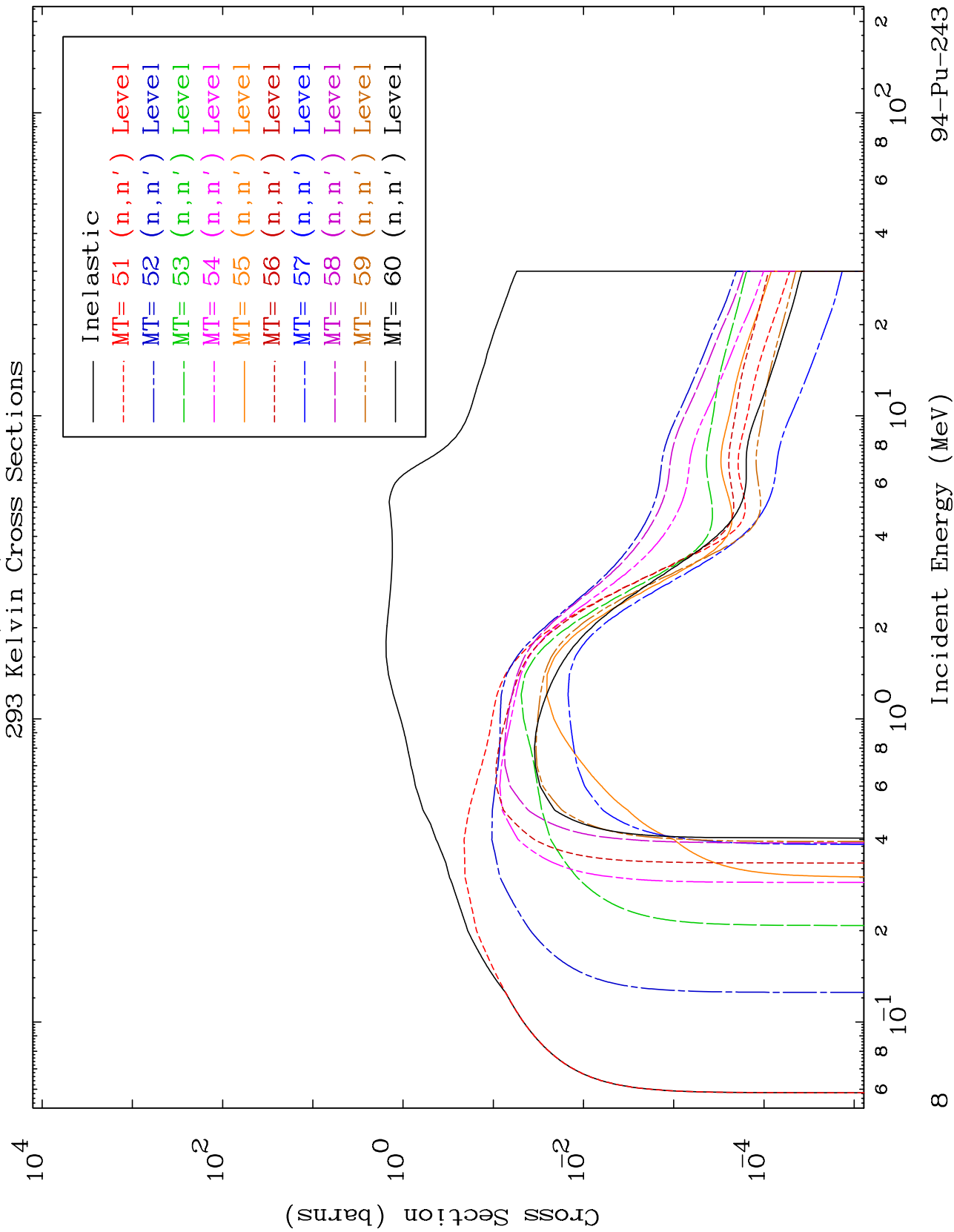
94-Pu-243



MAT 9449

(n,n') Level

94-Pu-243

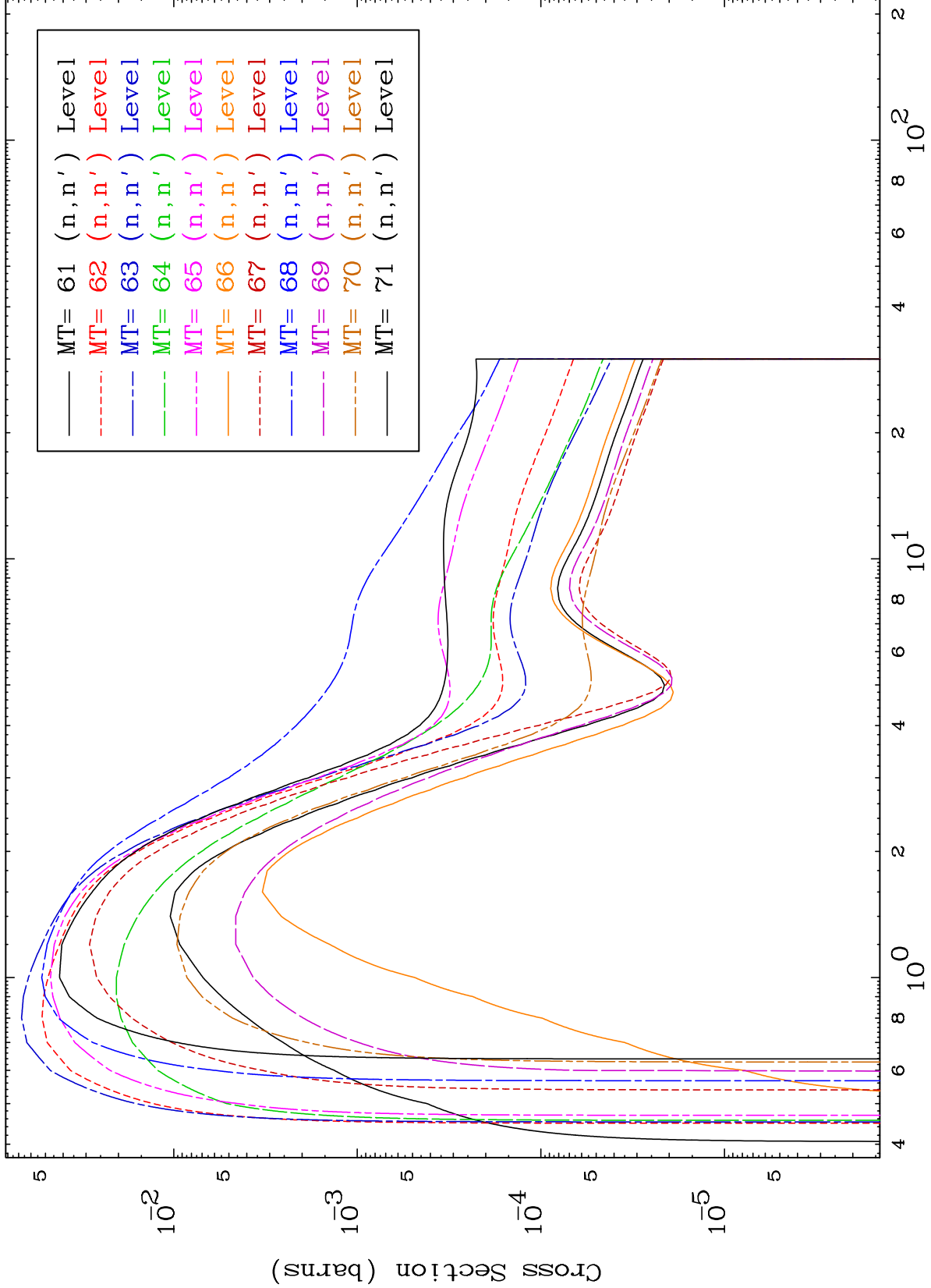


MAT 9449

(n,n') Level

94-Pu-243

293 Kelvin Cross Sections



9

Incident Energy (MeV)

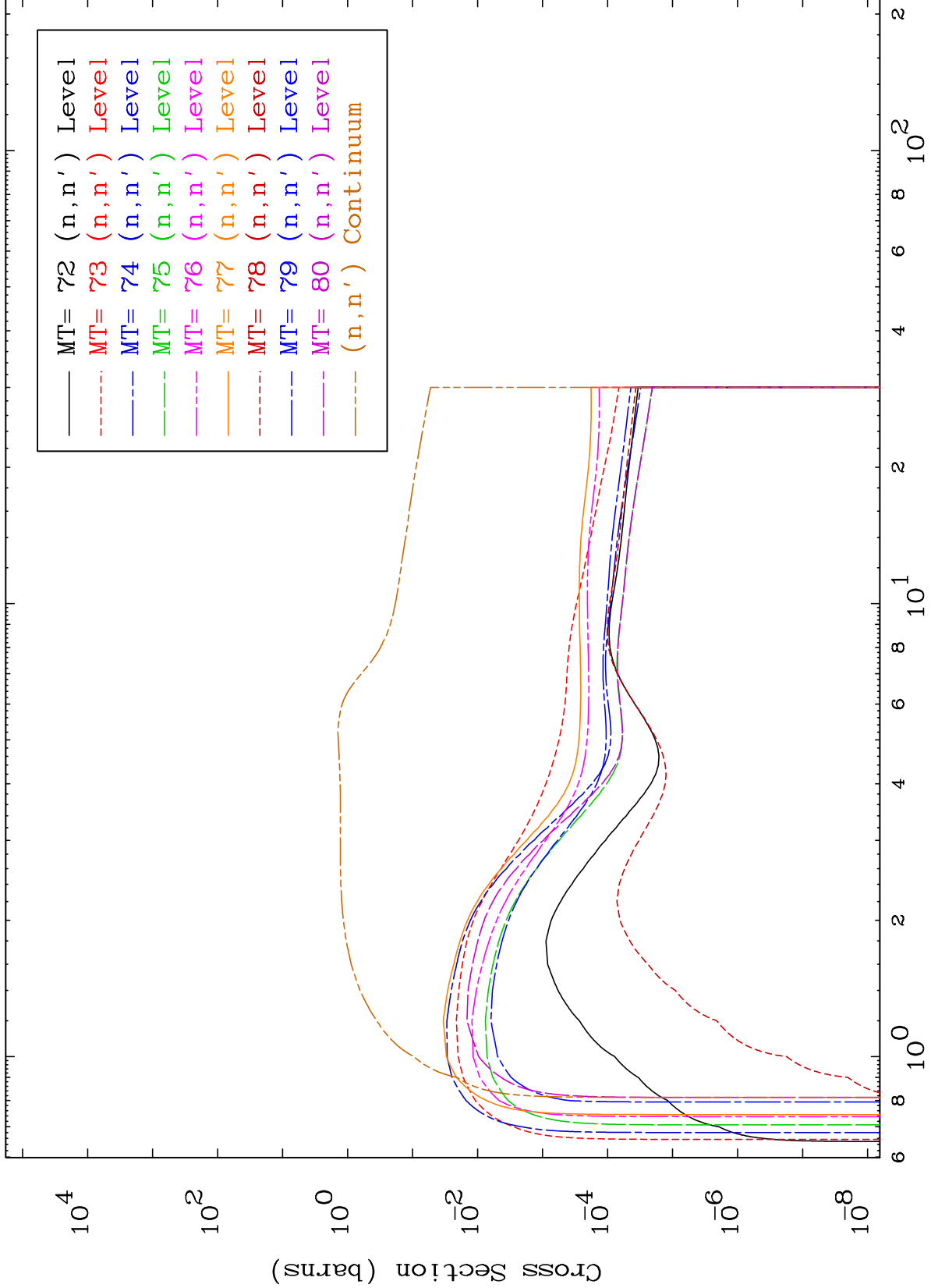
94-Pu-243

MAT 9449

(n,n') Level

94-Pu-243

293 Kelvin Cross Sections



10

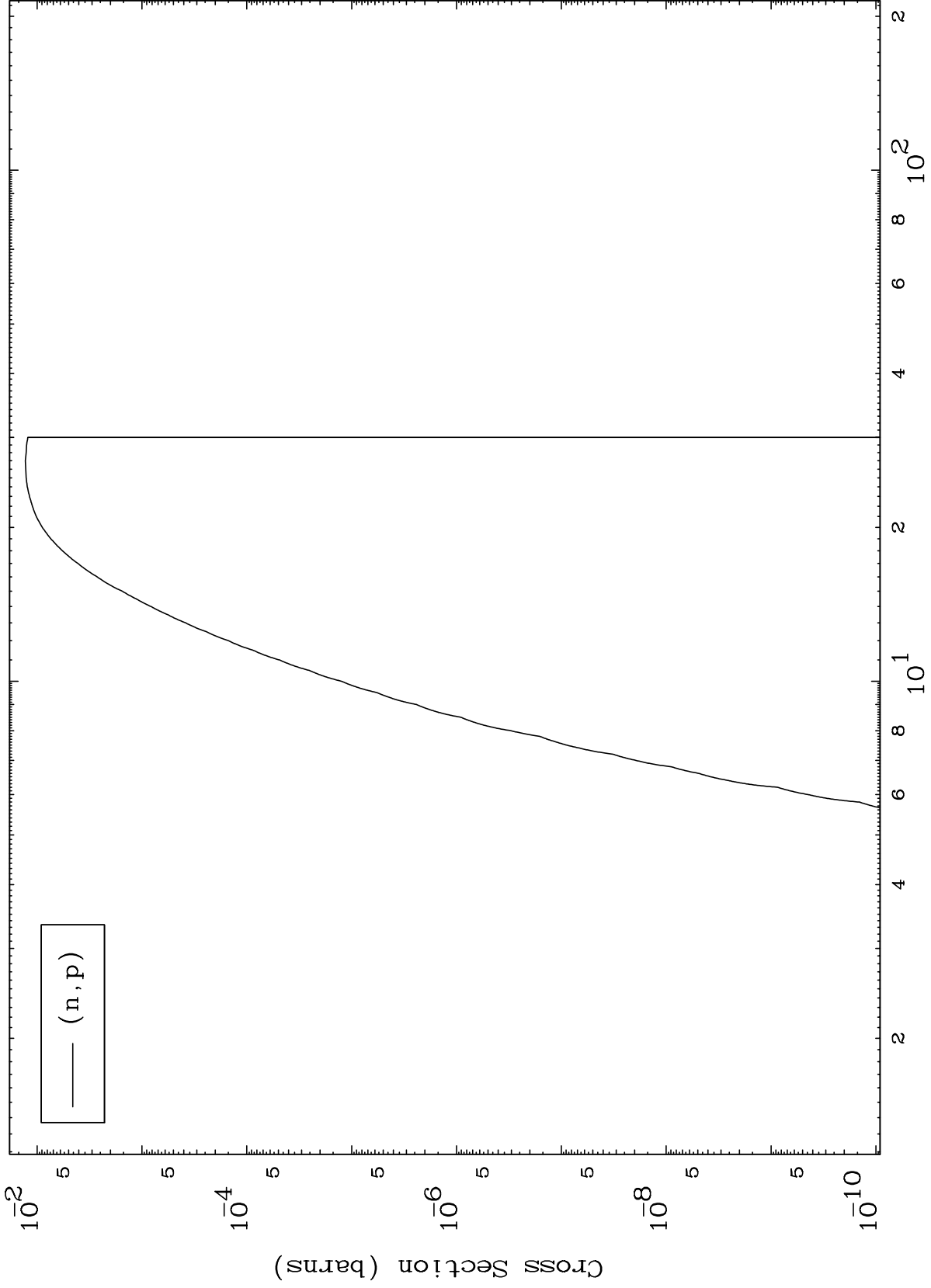
Incident Energy (MeV)

94-Pu-243

MAT 9449

(n,p) Levels
293 Kelvin Cross Sections

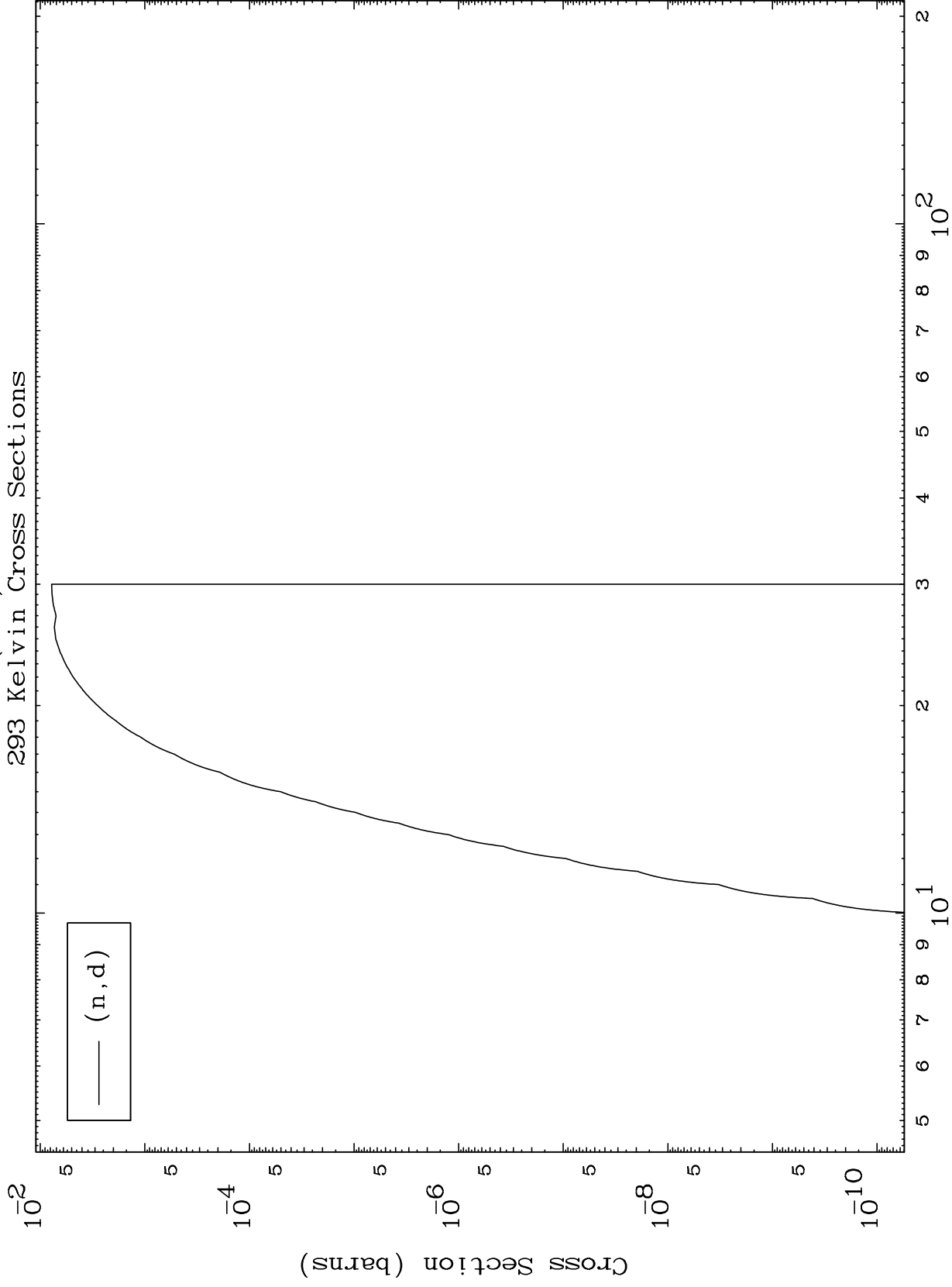
94-Pu-243



MAT 9449

(n,d) Levels
293 Kelvin Cross Sections

94-Pu-243



12

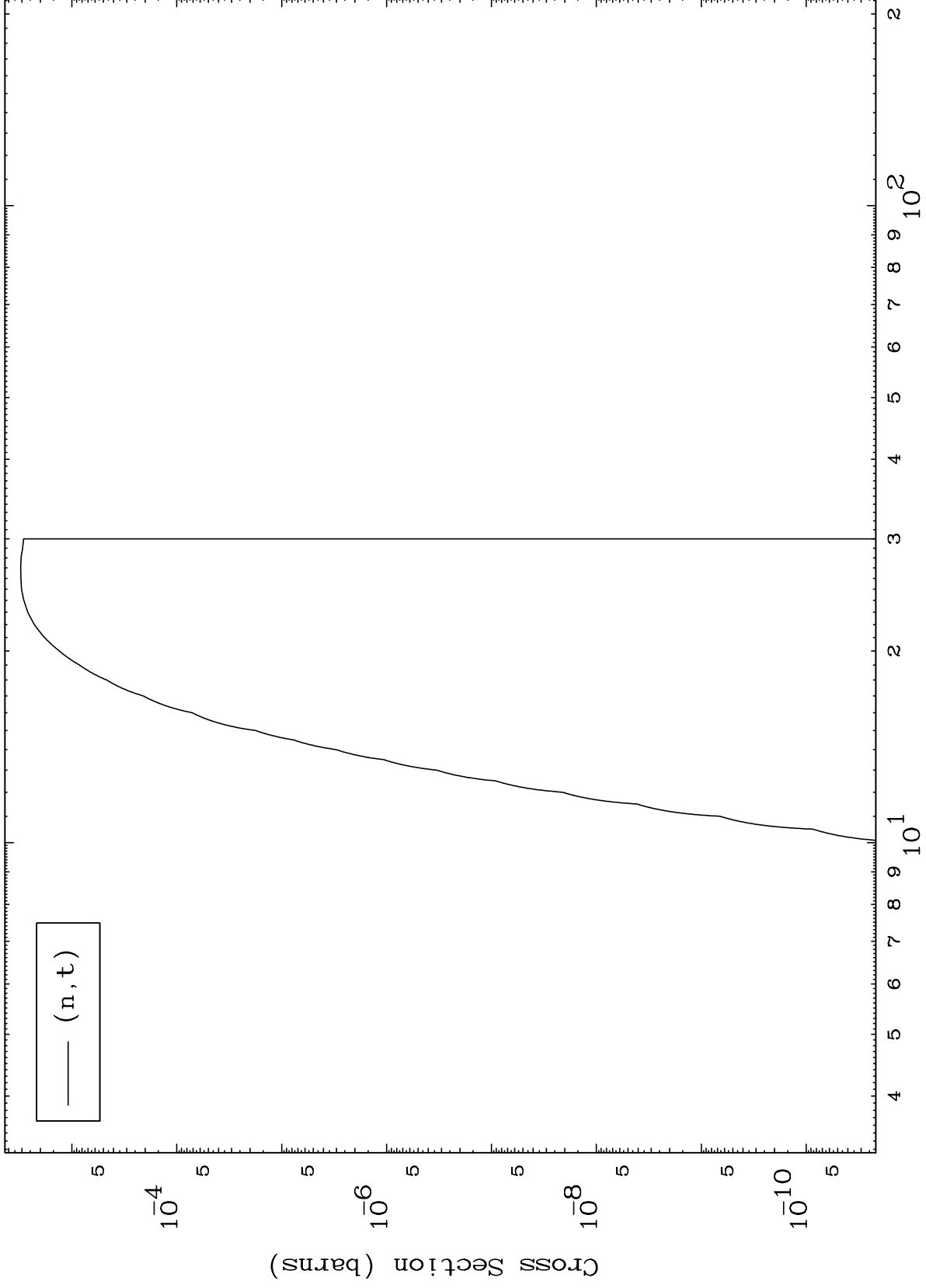
Incident Energy (MeV)

94-Pu-243

MAT 9449

(n,t) Levels
293 Kelvin Cross Sections

94-Pu-243



13

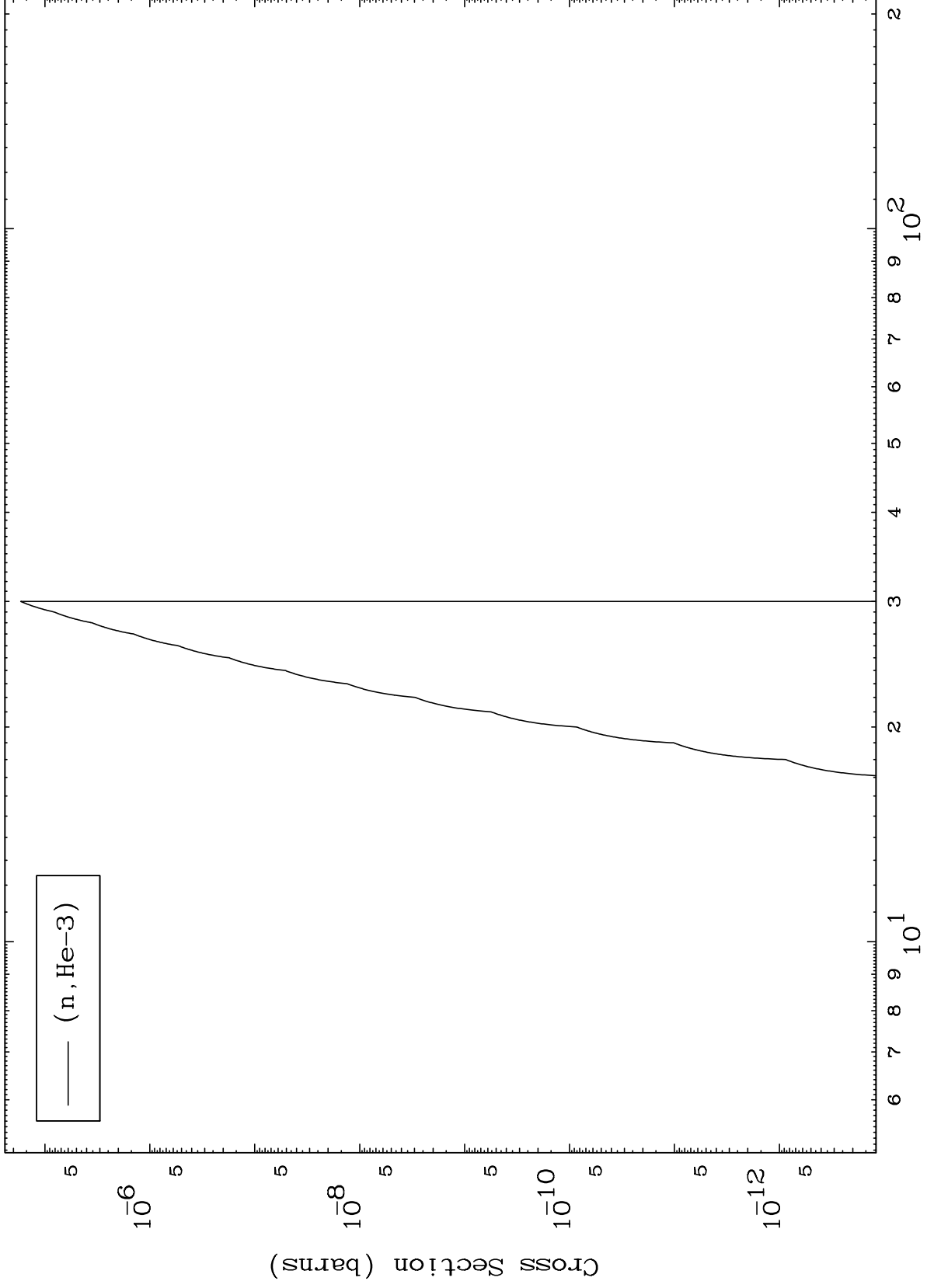
Incident Energy (MeV)

94-Pu-243

MAT 9449

(n,He3) Levels
293 Kelvin Cross Sections

94-Pu-243



14

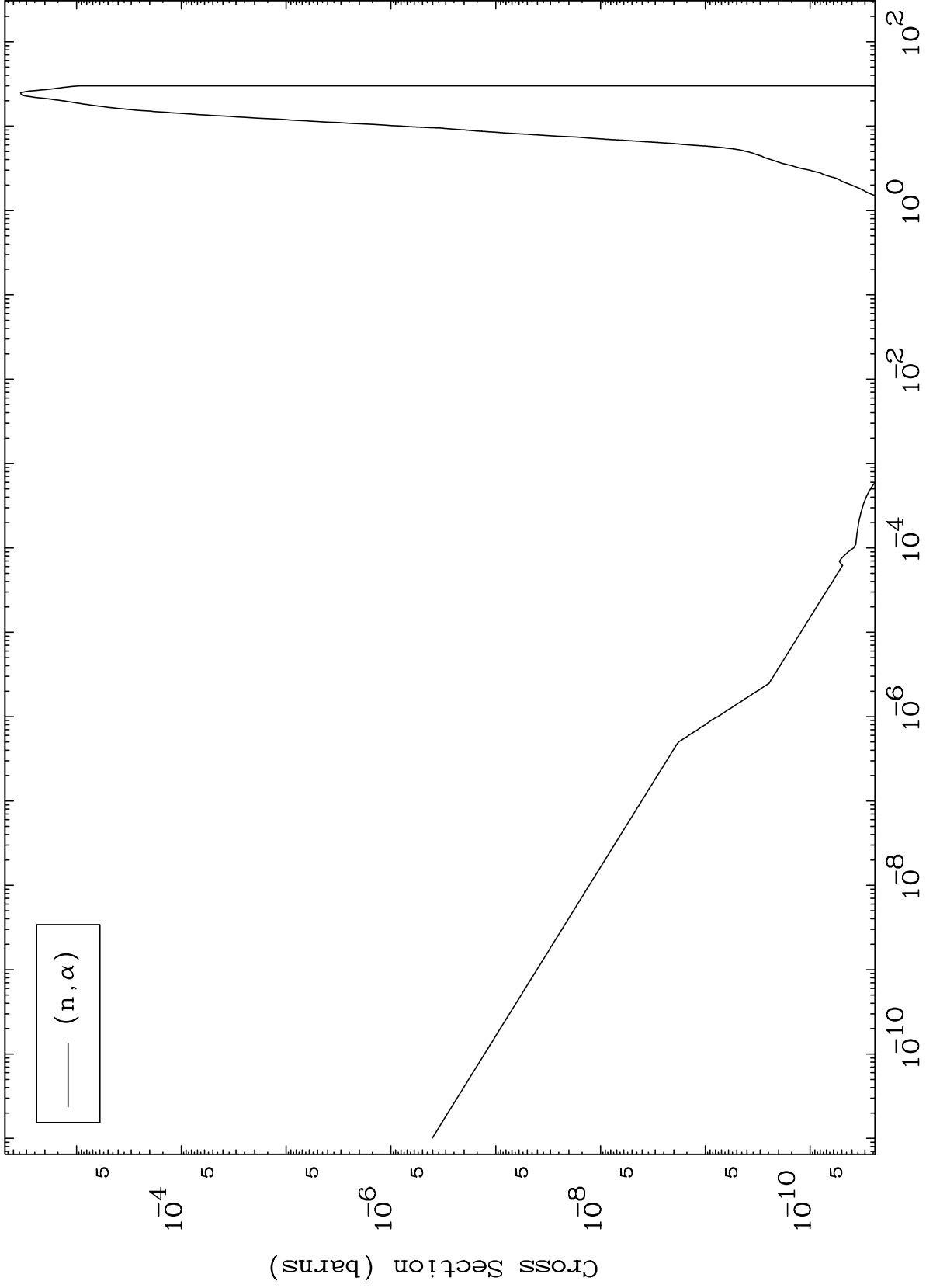
Incident Energy (MeV)

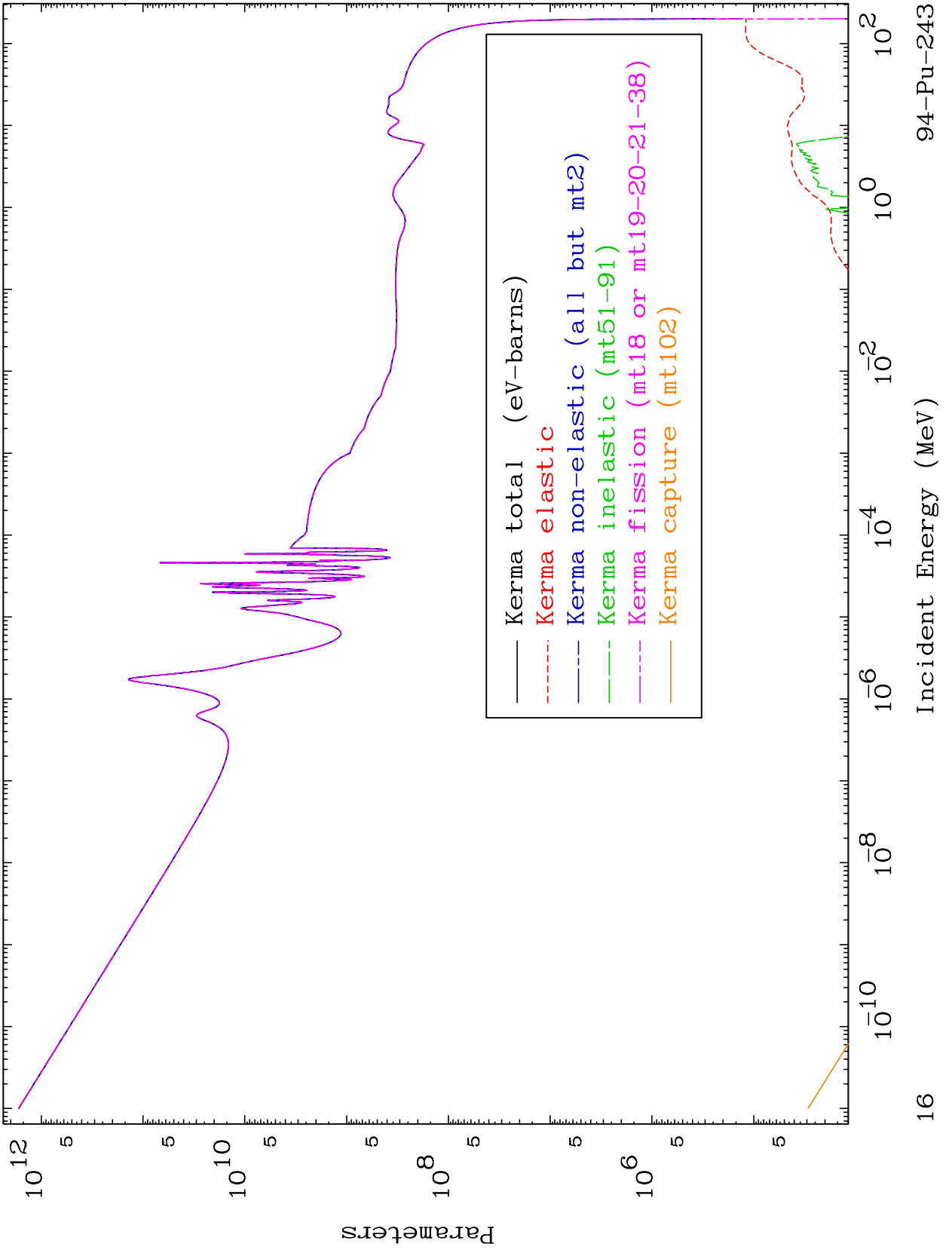
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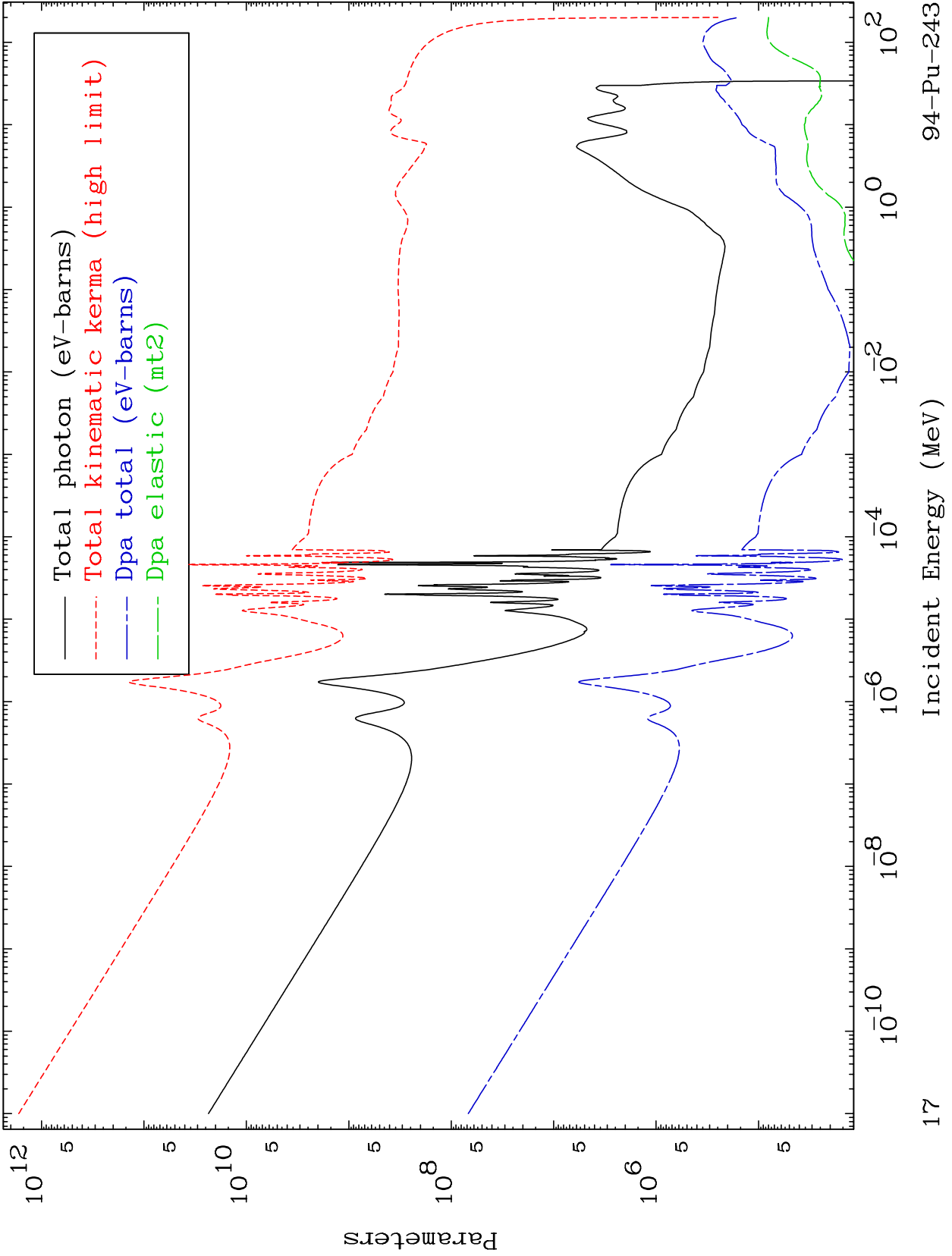
MAT 9449

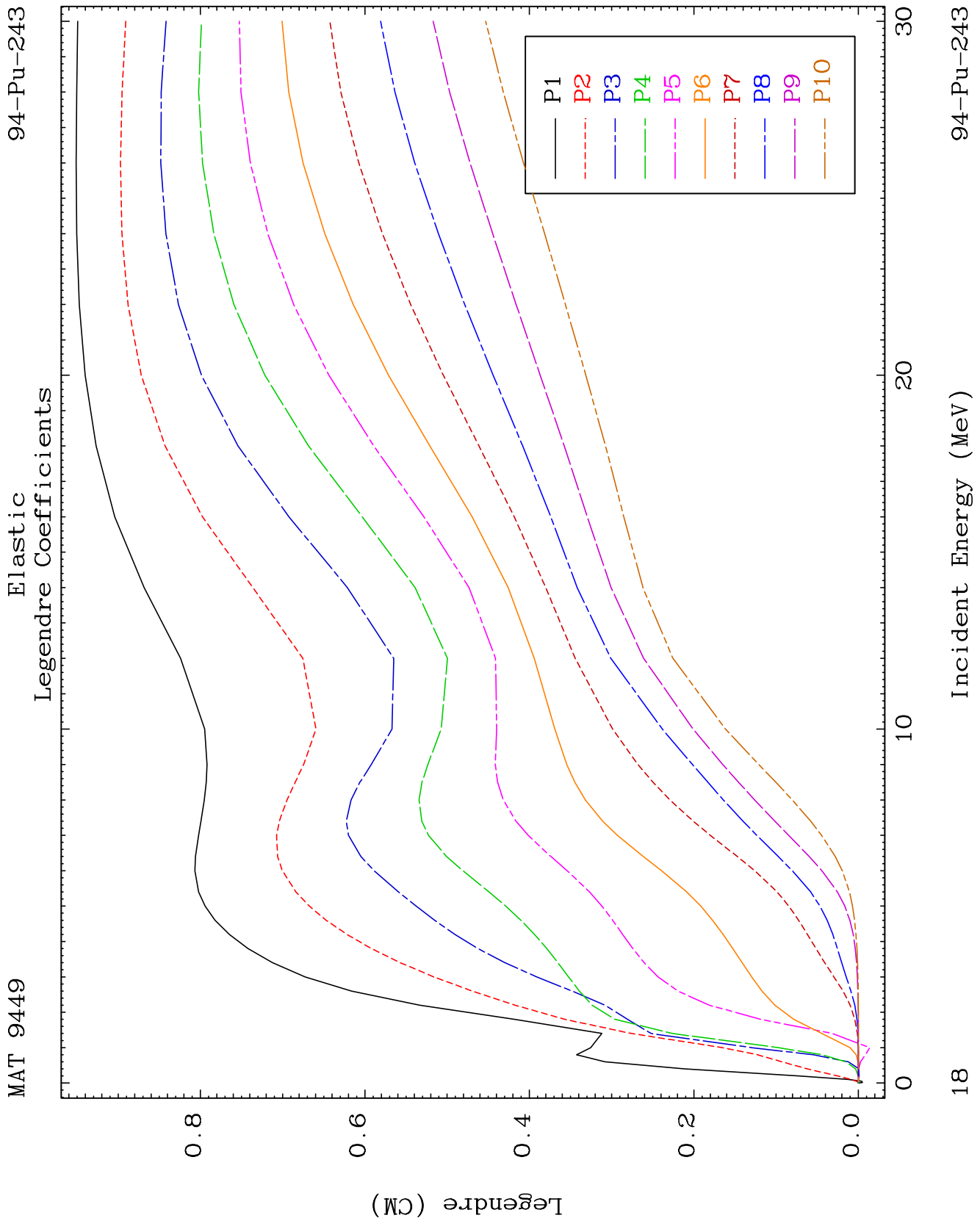
(n, α) Levels
293 Kelvin Cross Sections

94-Pu-243





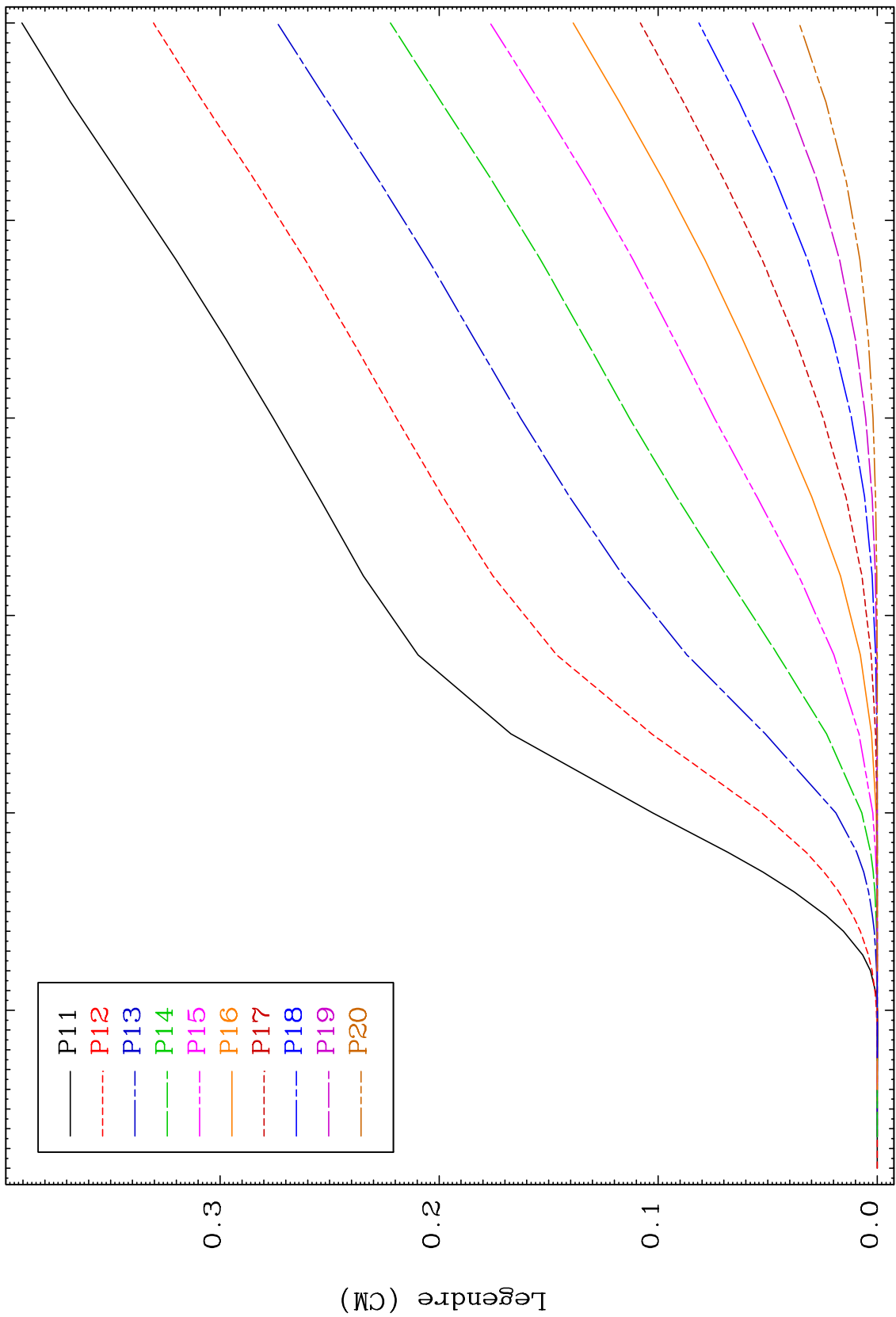




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Elastic Legendre Coefficients

94-Pu-243



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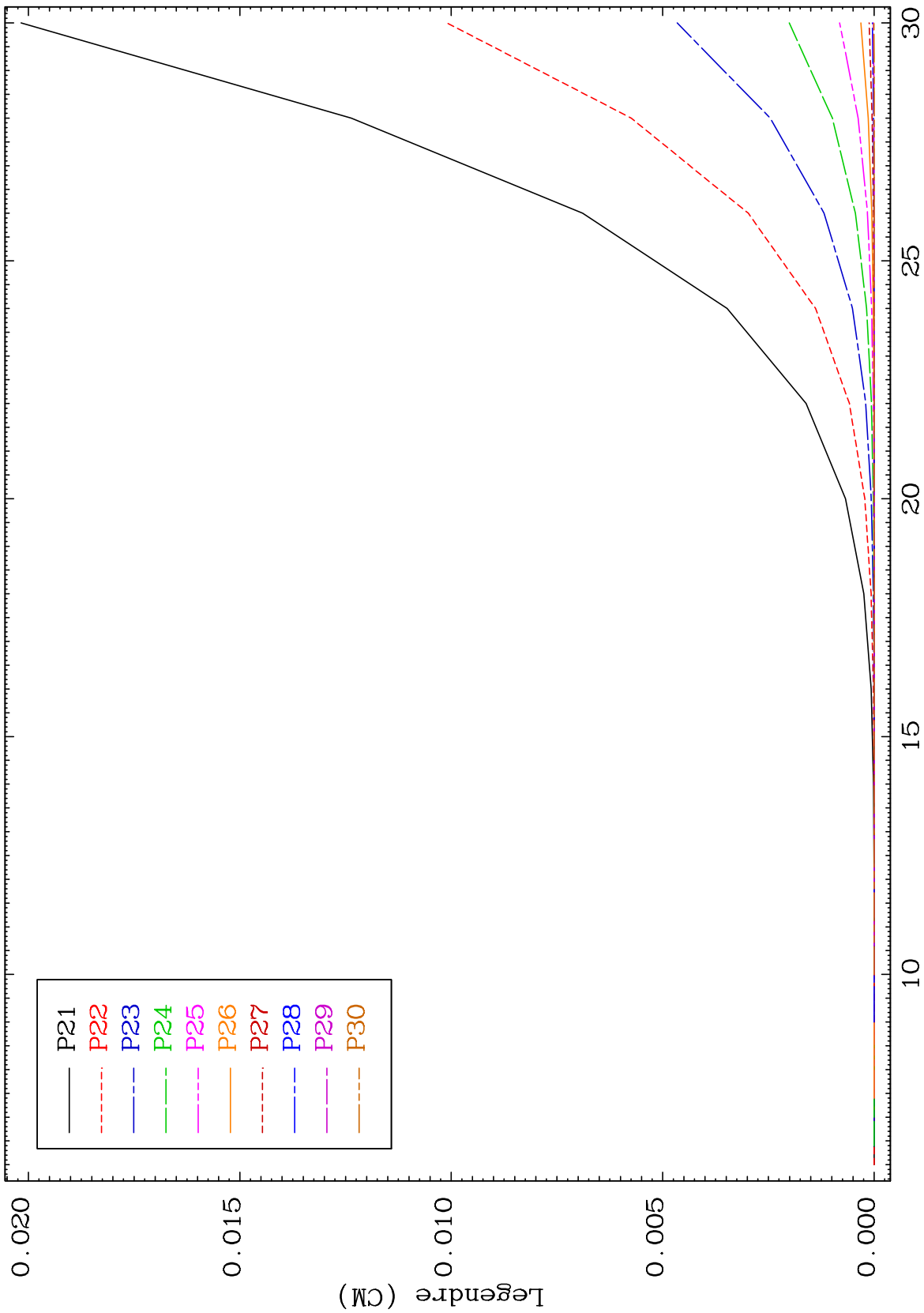
Incident Energy (MeV)

94-Pu-243

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Elastic
Legendre Coefficients

94-Pu-243



20

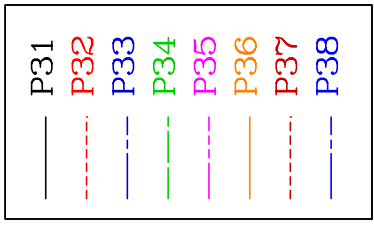
Incident Energy (MeV)

94-Pu-243

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Elastic Legendre Coefficients

94-Pu-243



$\times 10^{-6}$

1.2

0.8

0.4

0.0

Legendre (CM)

15

20

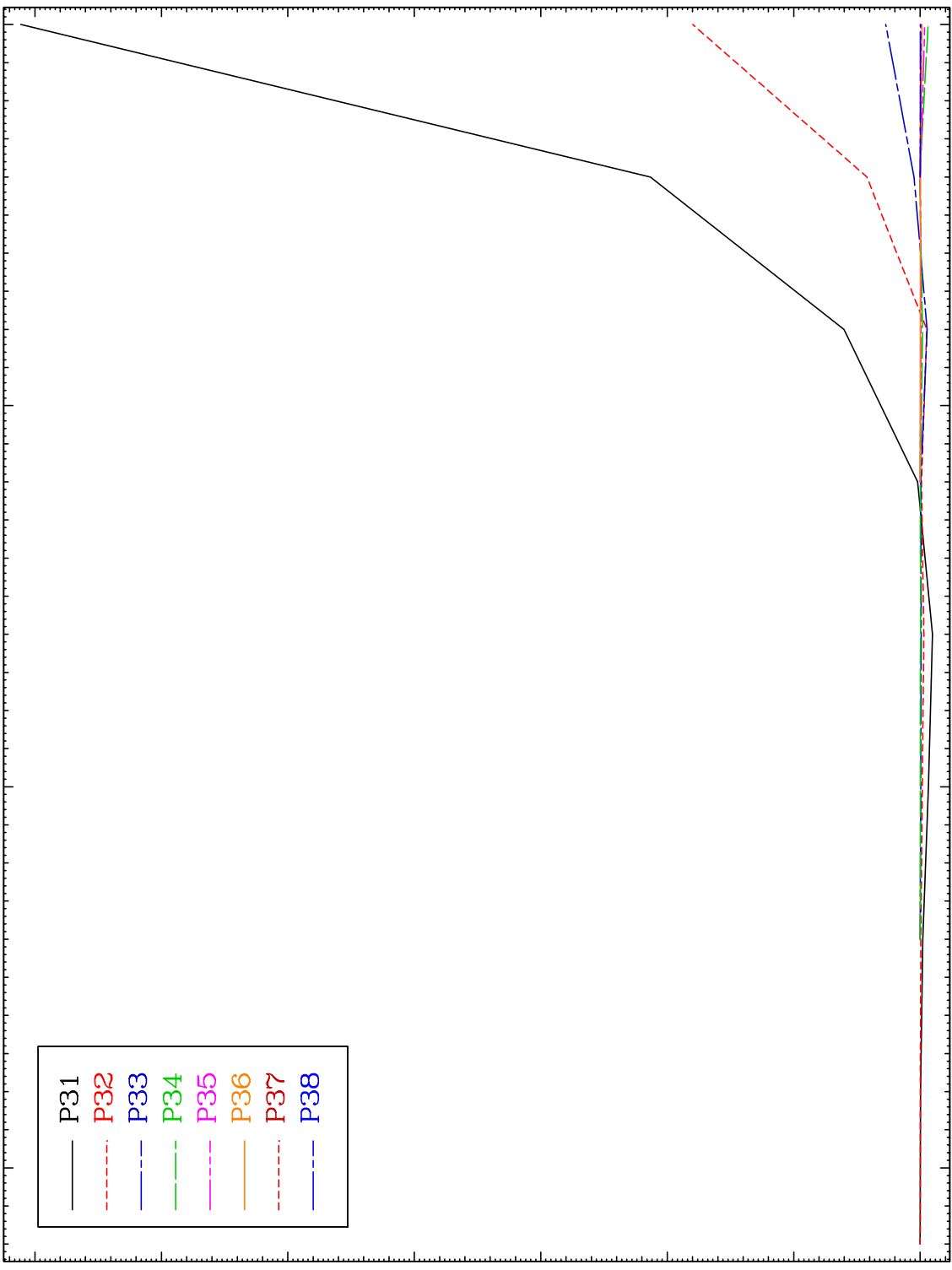
25

30

21

Incident Energy (MeV)

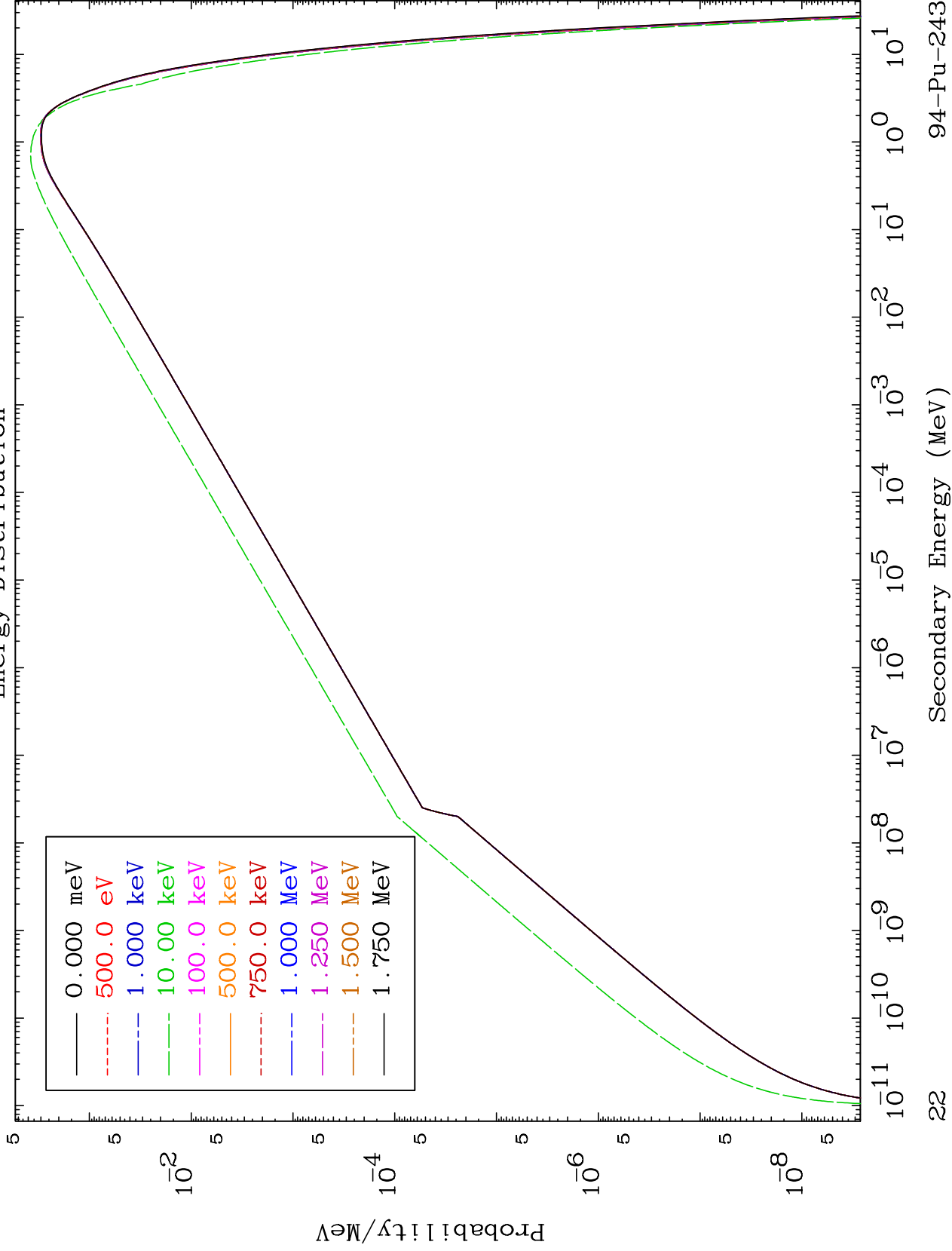
94-Pu-243



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Fission Energy Distribution

94-Pu-243

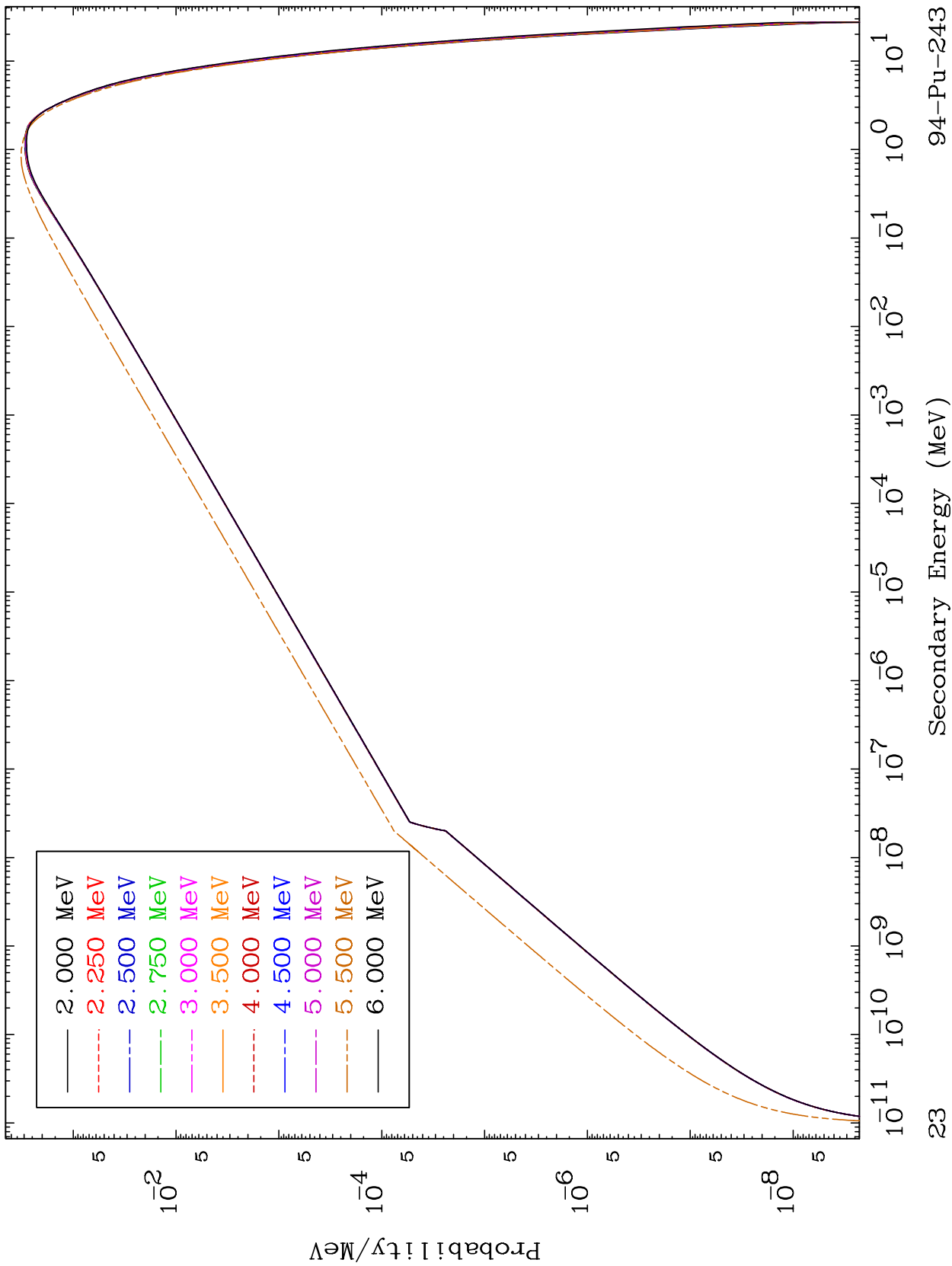


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Fission Energy Distribution

94-Pu-243



94-Pu-243

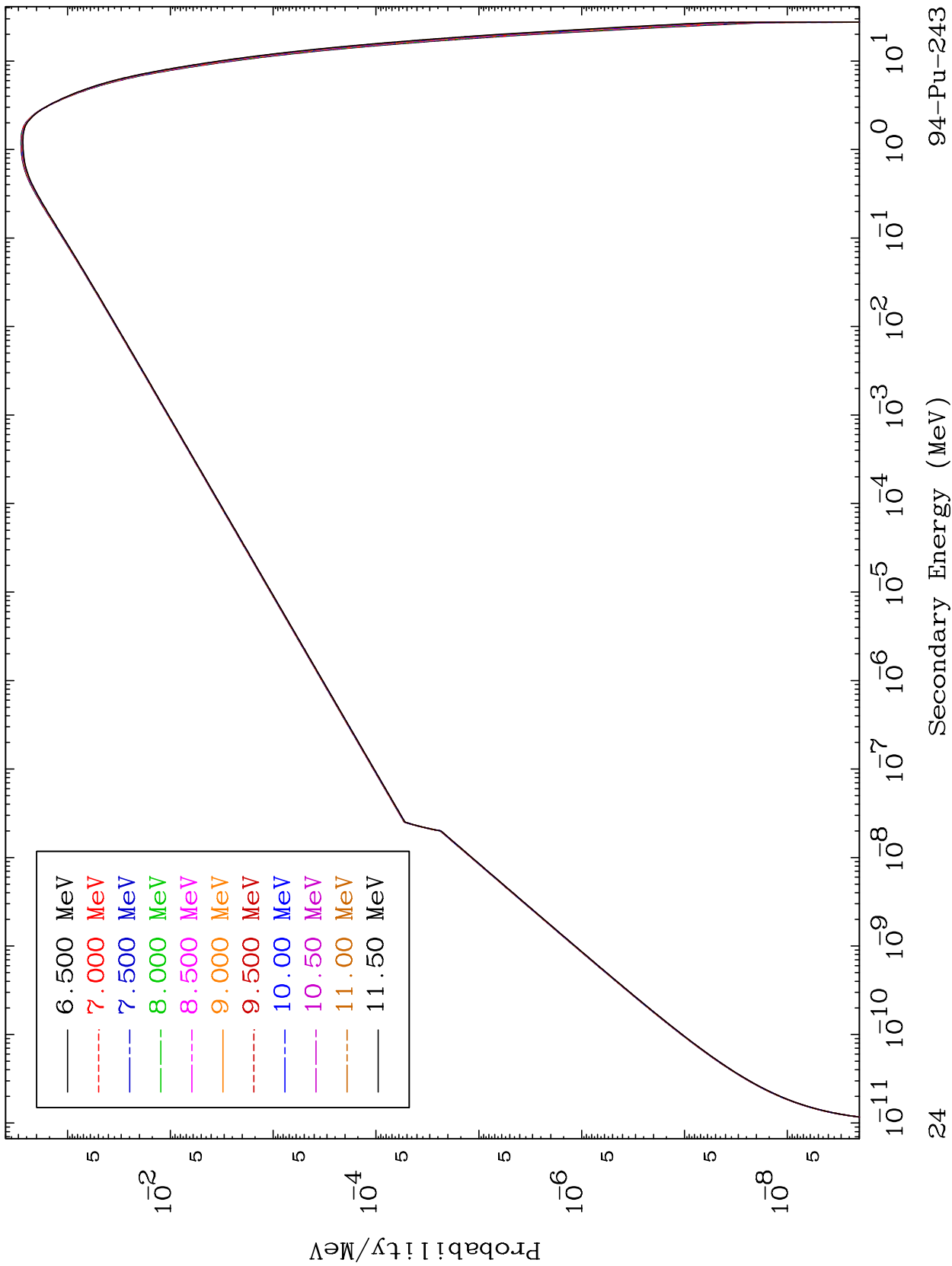
Secondary Energy (MeV)

23

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Fission Energy Distribution

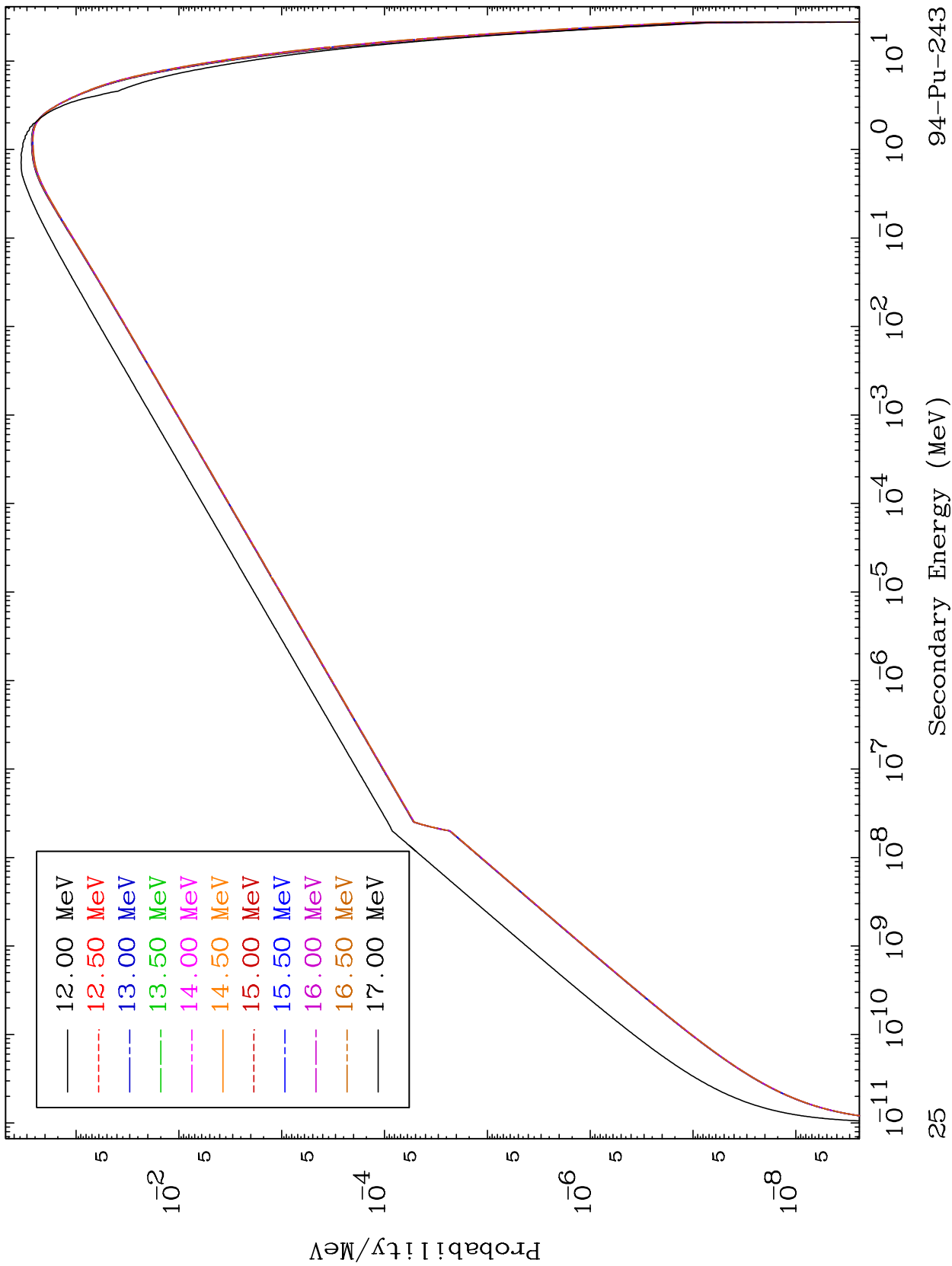
94-Pu-243



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Fission Energy Distribution

94-Pu-243



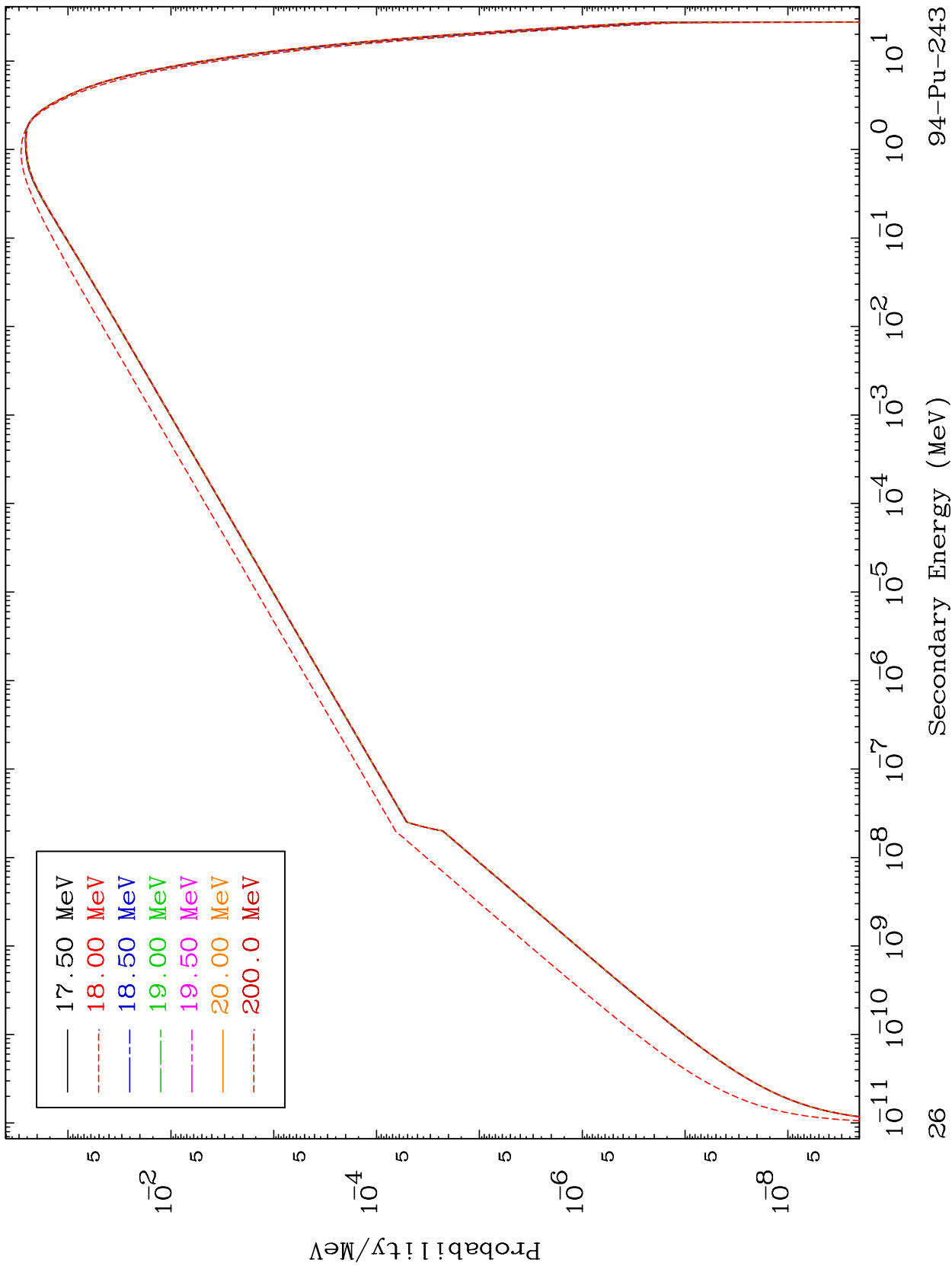
94-Pu-243

Secondary Energy (MeV)

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Fission Energy Distribution

94-Pu-243

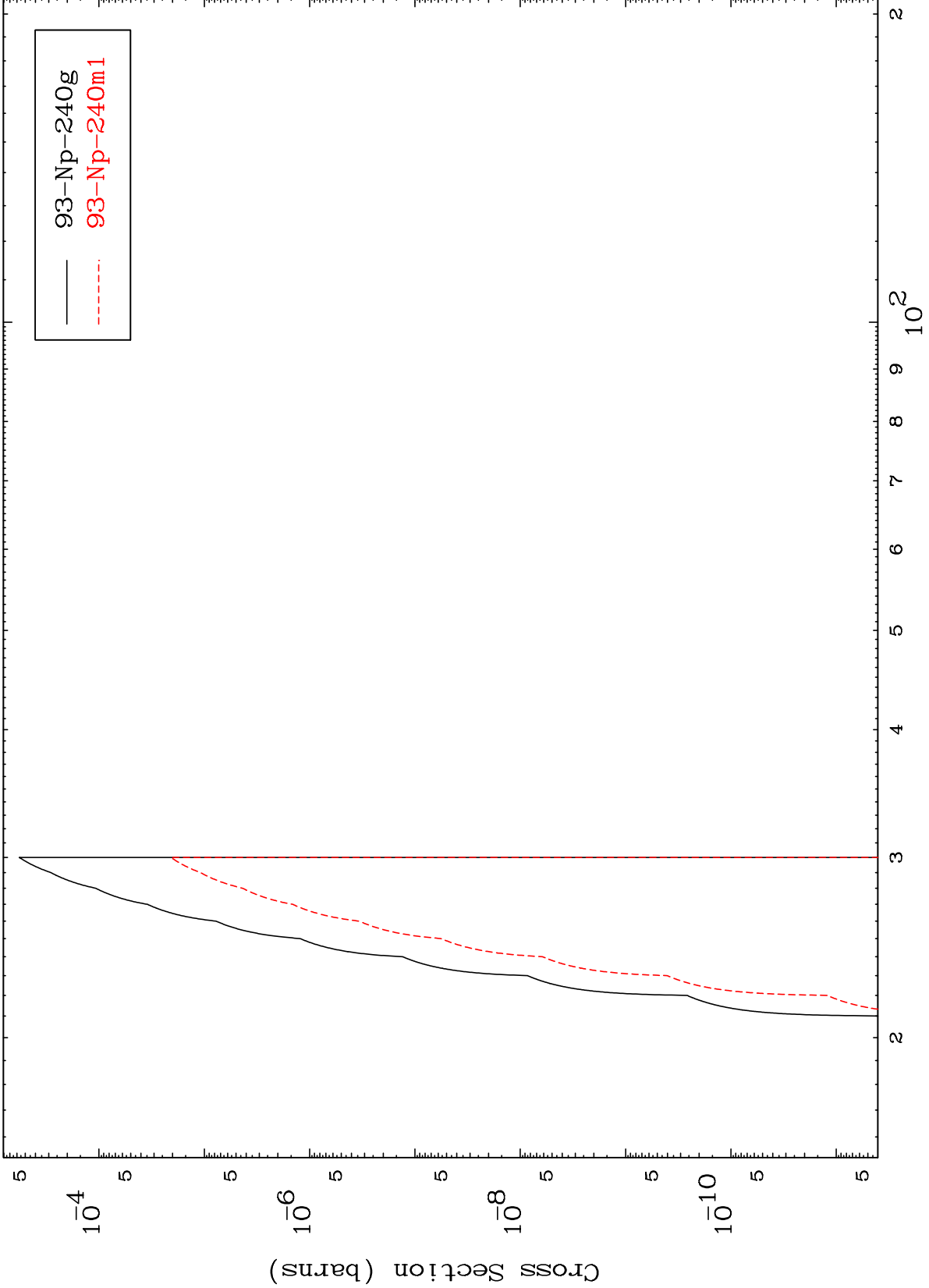


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(n,2n) d

94-Pu-243

Radionuclide Production Cross Section



27

Incident Energy (MeV)

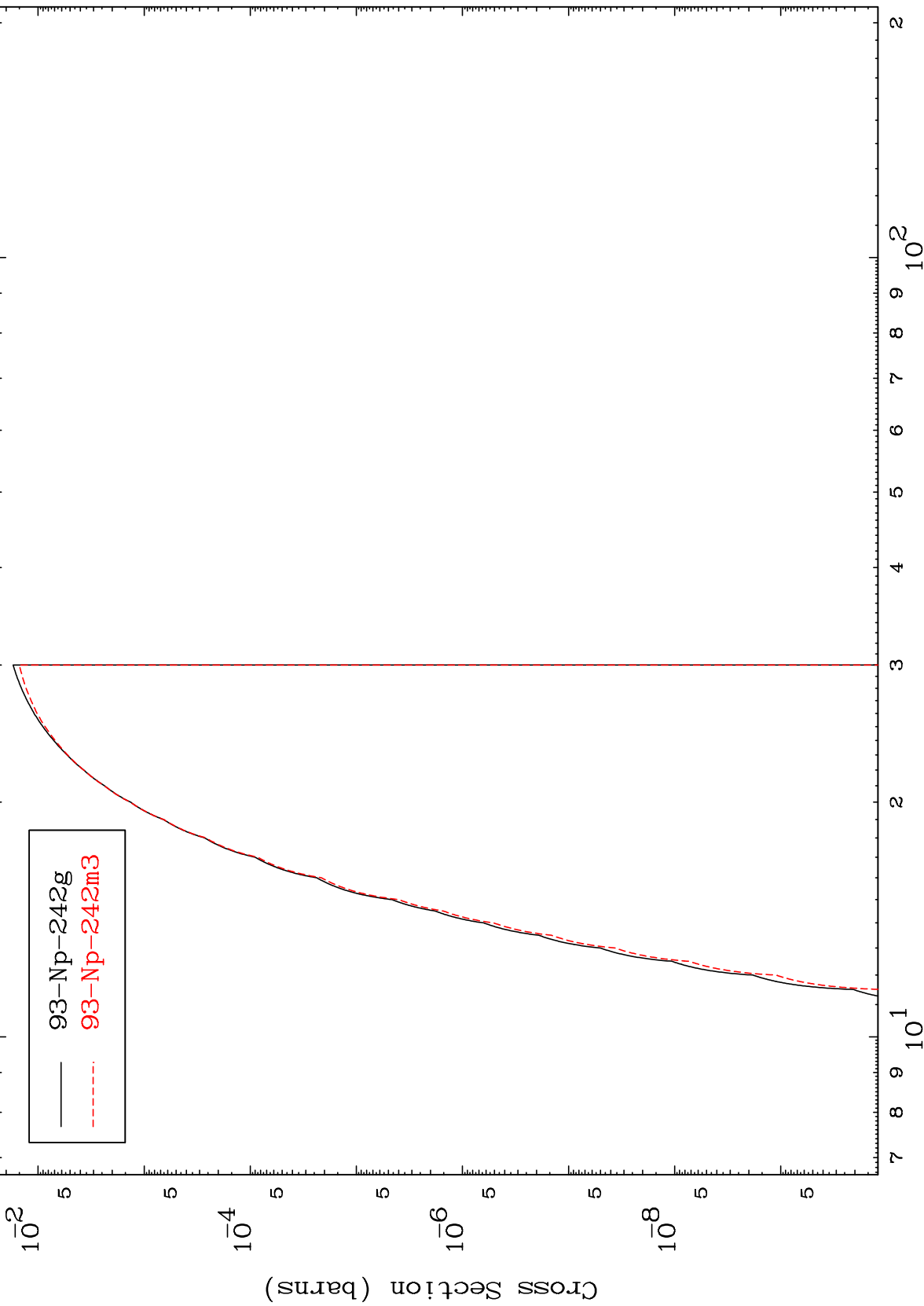
94-Pu-243

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(n,n') p

94-Pu-243

Radionuclide Production Cross Section



28

Incident Energy (MeV)

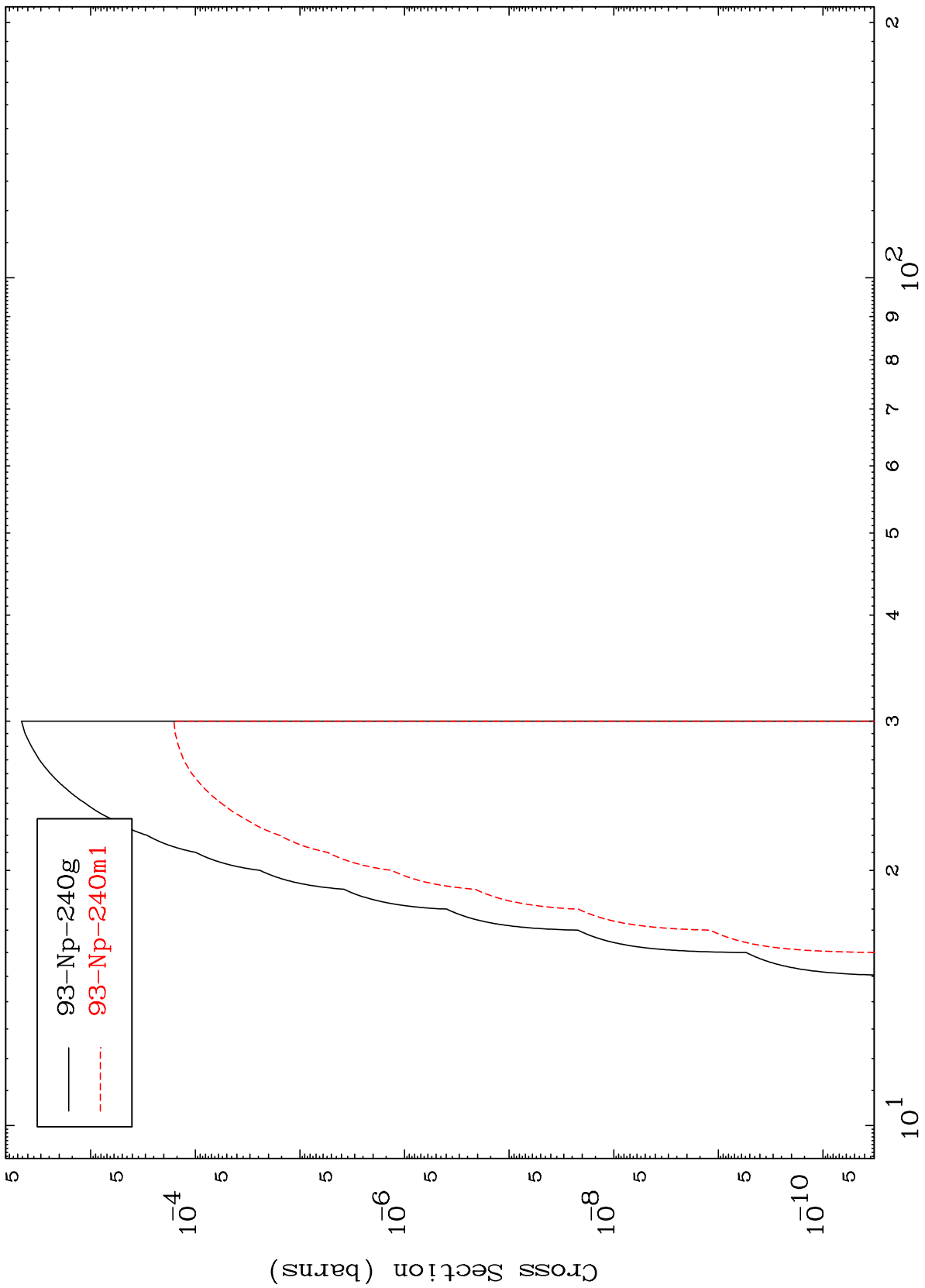
94-Pu-243

MAT 9449

(n,n') t

94-Pu-243

Radionuclide Production Cross Section



29

Incident Energy (MeV)

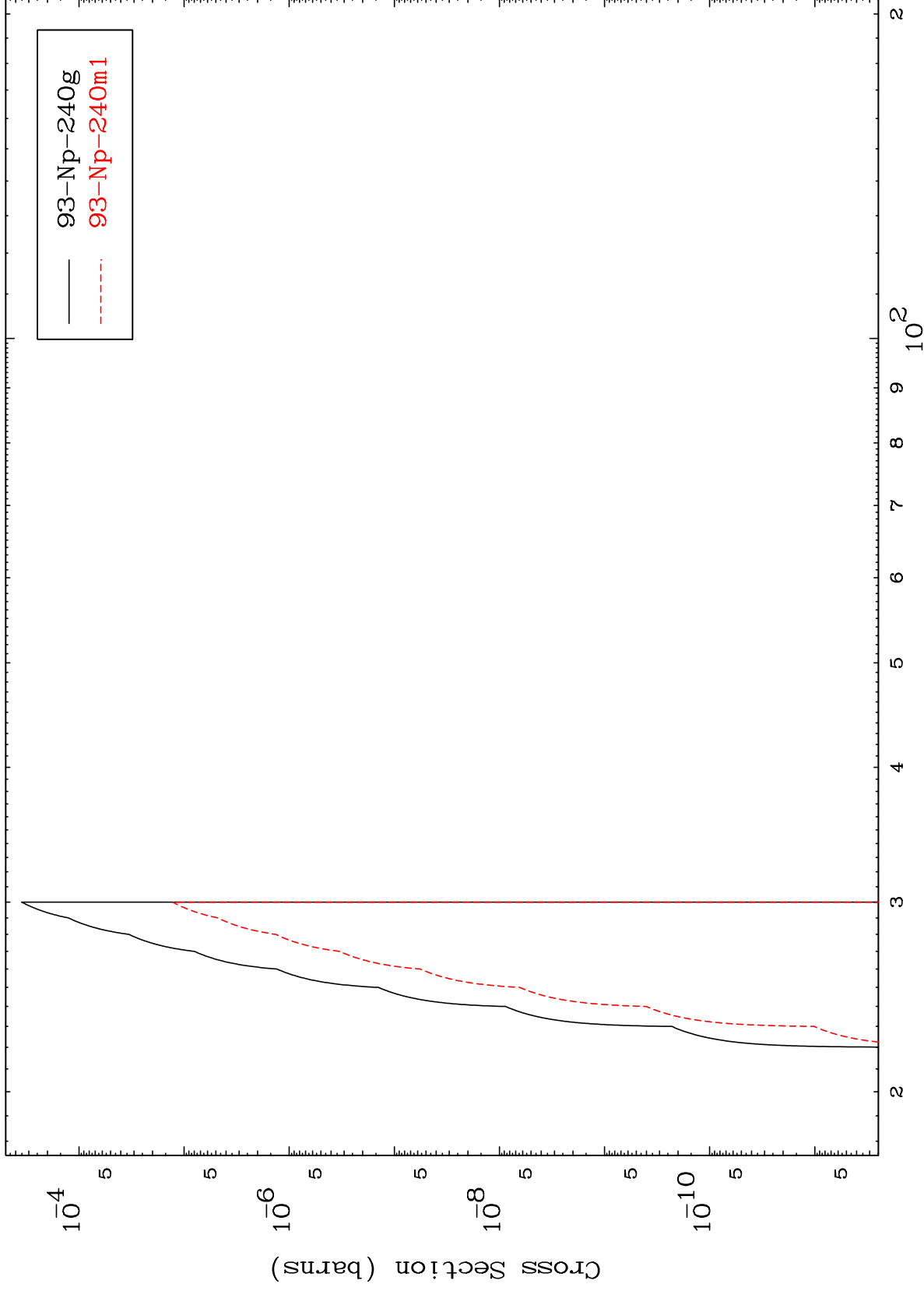
94-Pu-243

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(n,3n) p

94-Pu-243

Radionuclide Production Cross Section



30

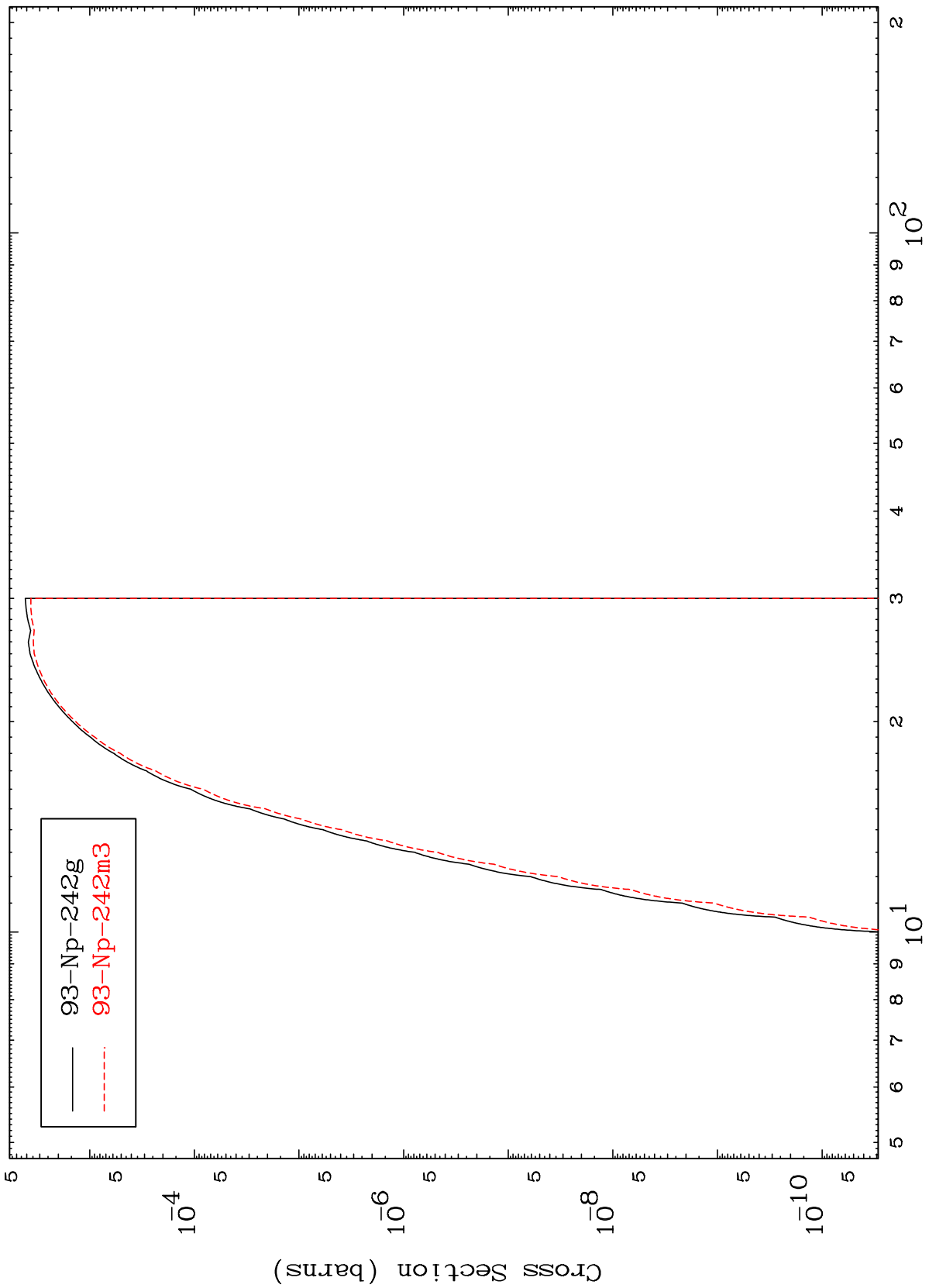
Incident Energy (MeV)

94-Pu-243

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94-Pu-243

(n,d)
Radionuclide Production Cross Section



31

Incident Energy (MeV)

94-Pu-243