

Program EVALPLOT
(Version 2018-1)

by

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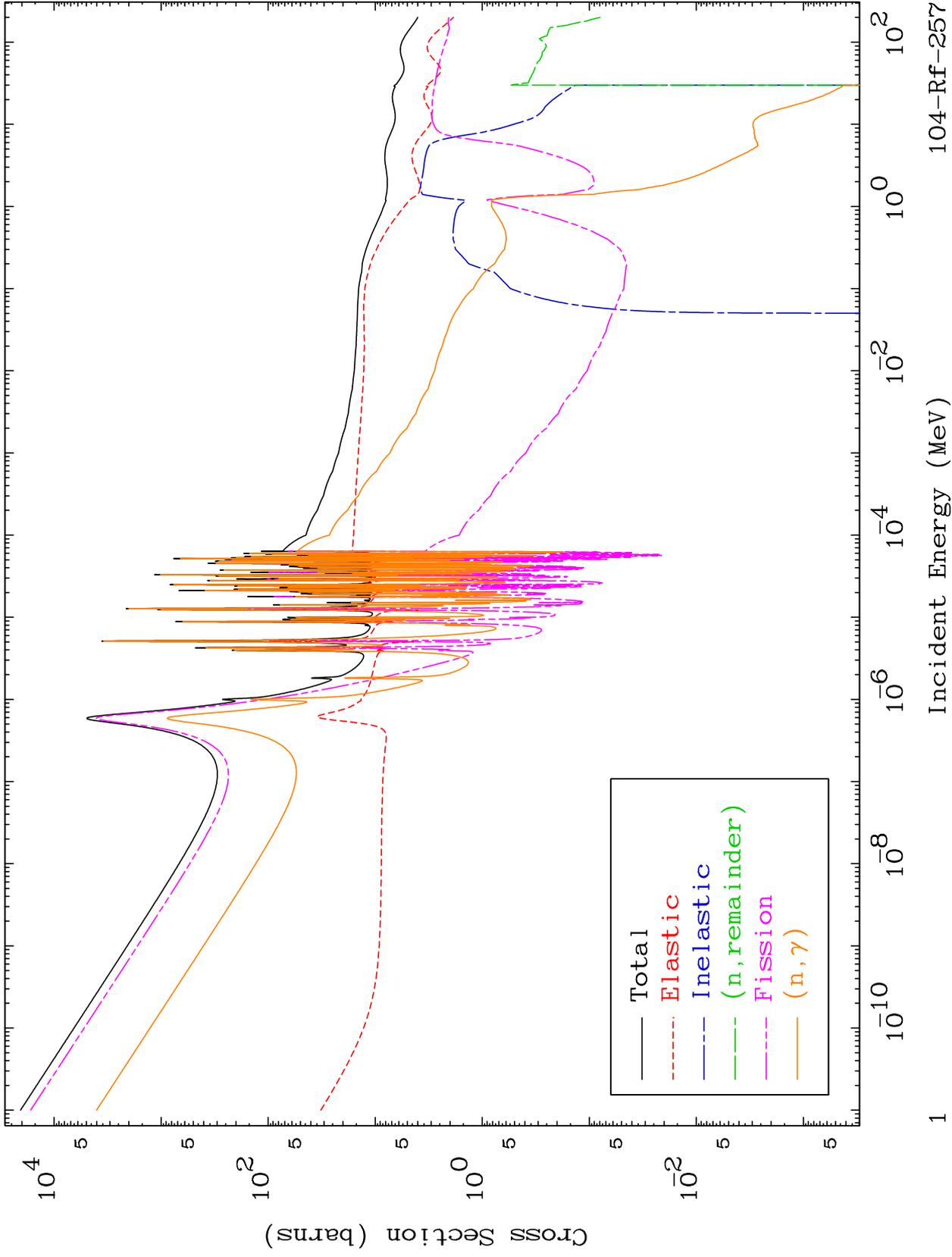
Press Mouse Button to Start

MAT 457

Major

293 Kelvin Cross Sections

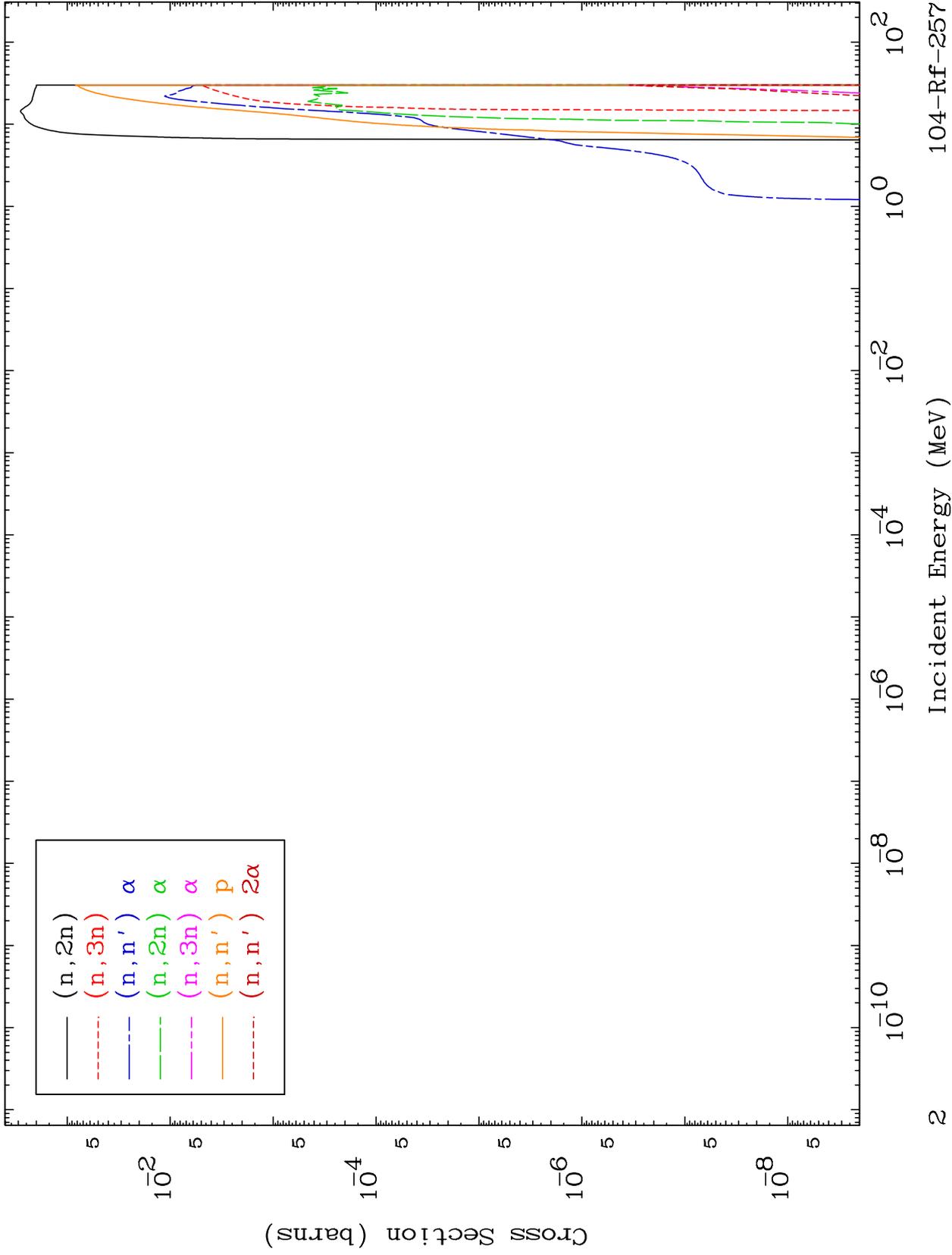
104-Rf-257



MAT 457

Neutron Production
293 Kelvin Cross Sections

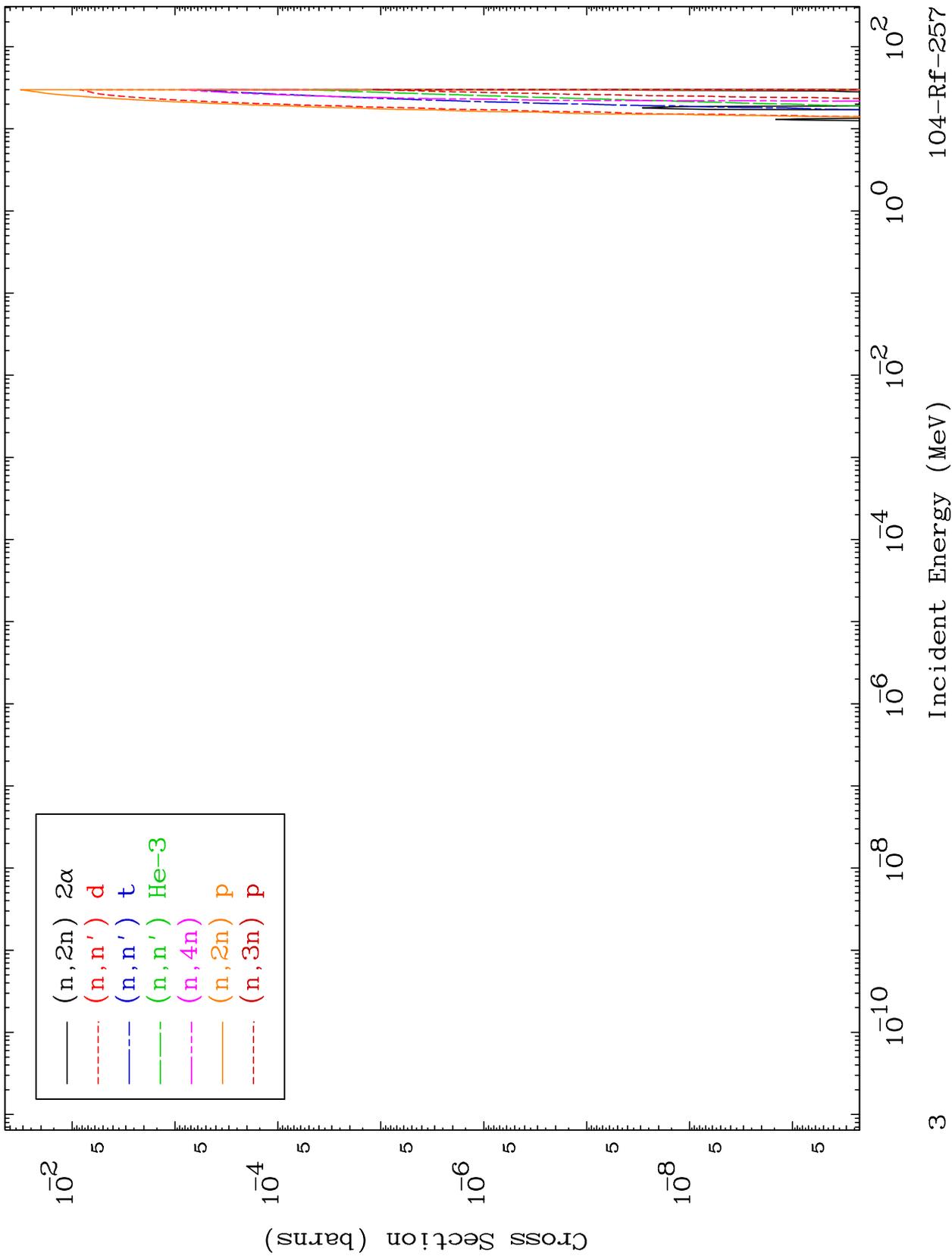
104-Rf-257



MAT 457

Neutron Production
293 Kelvin Cross Sections

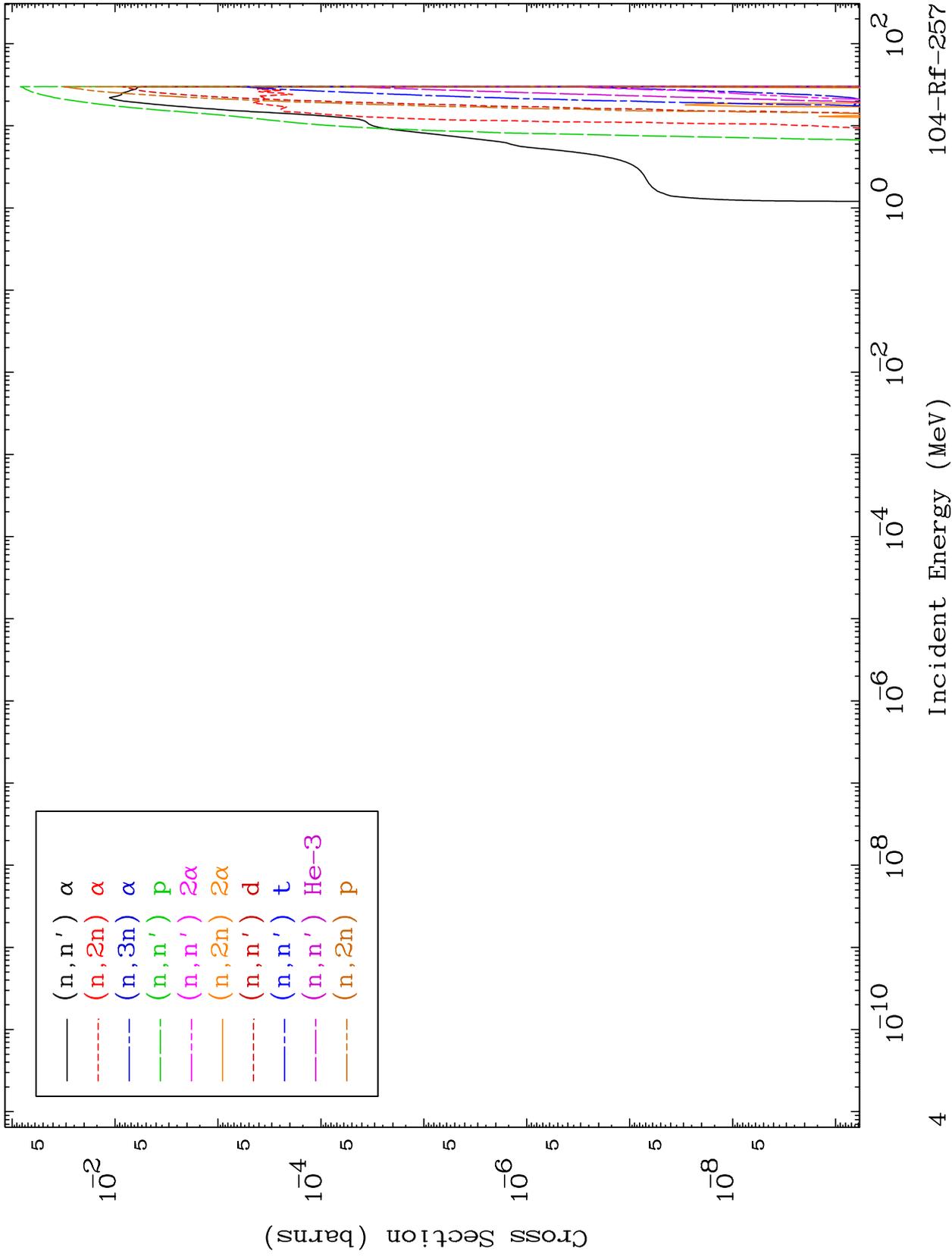
104-Rf-257



MAT 457

Charged Particle
293 Kelvin Cross Sections

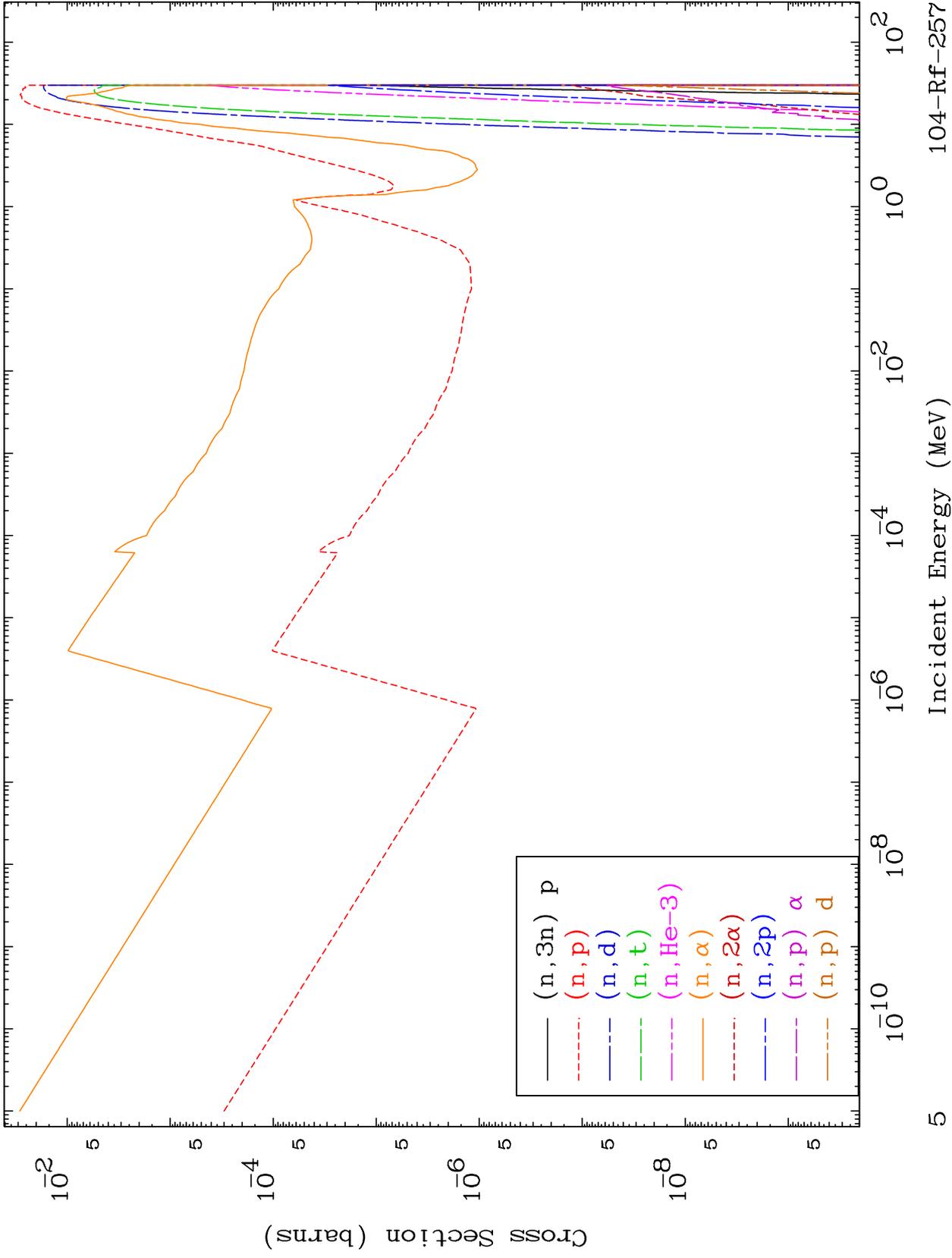
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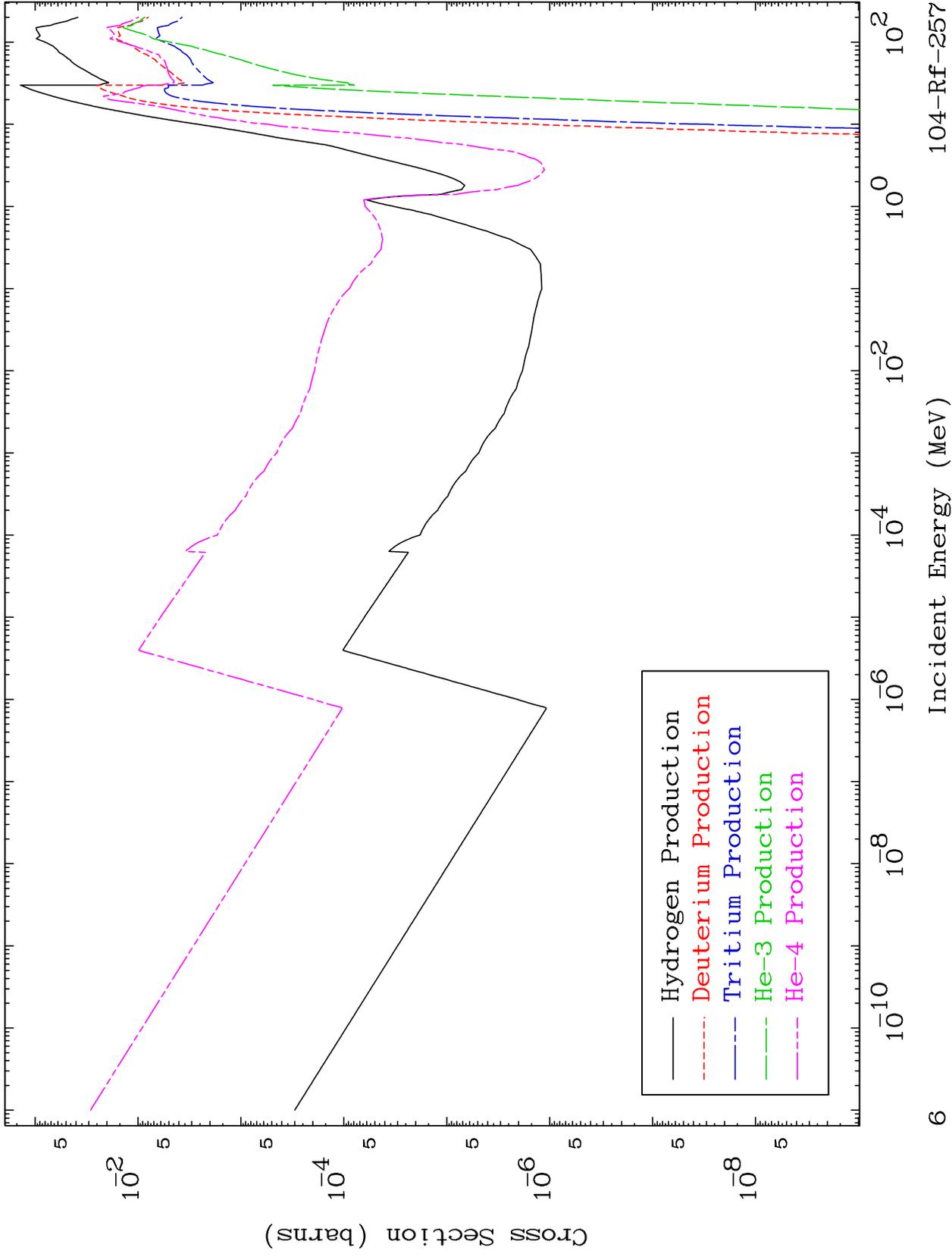


MAT 457

Charged Particle
293 Kelvin Cross Sections

104-Rf-257

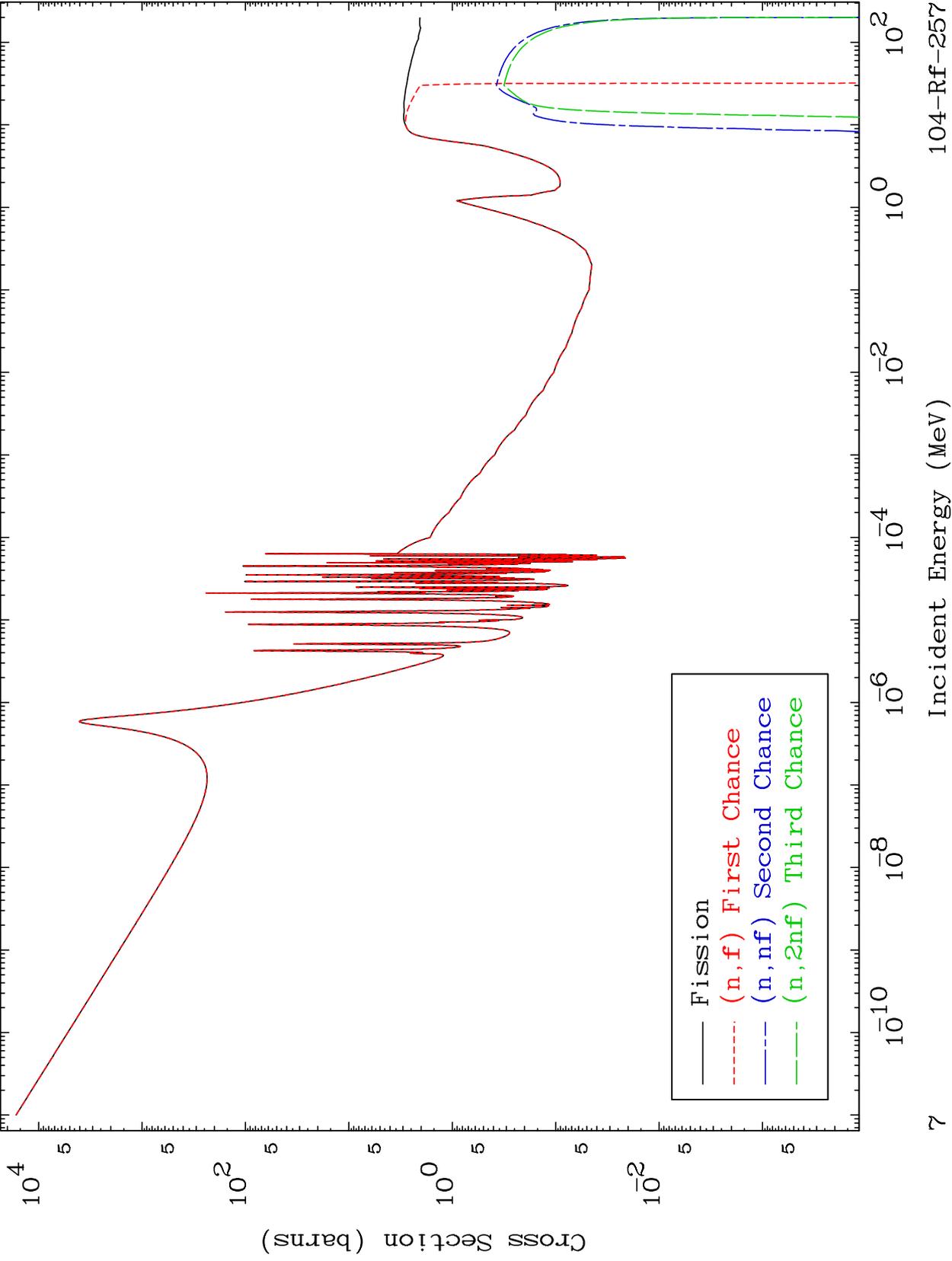




MAT 457

Fission
293 Kelvin Cross Sections

104-Rf-257

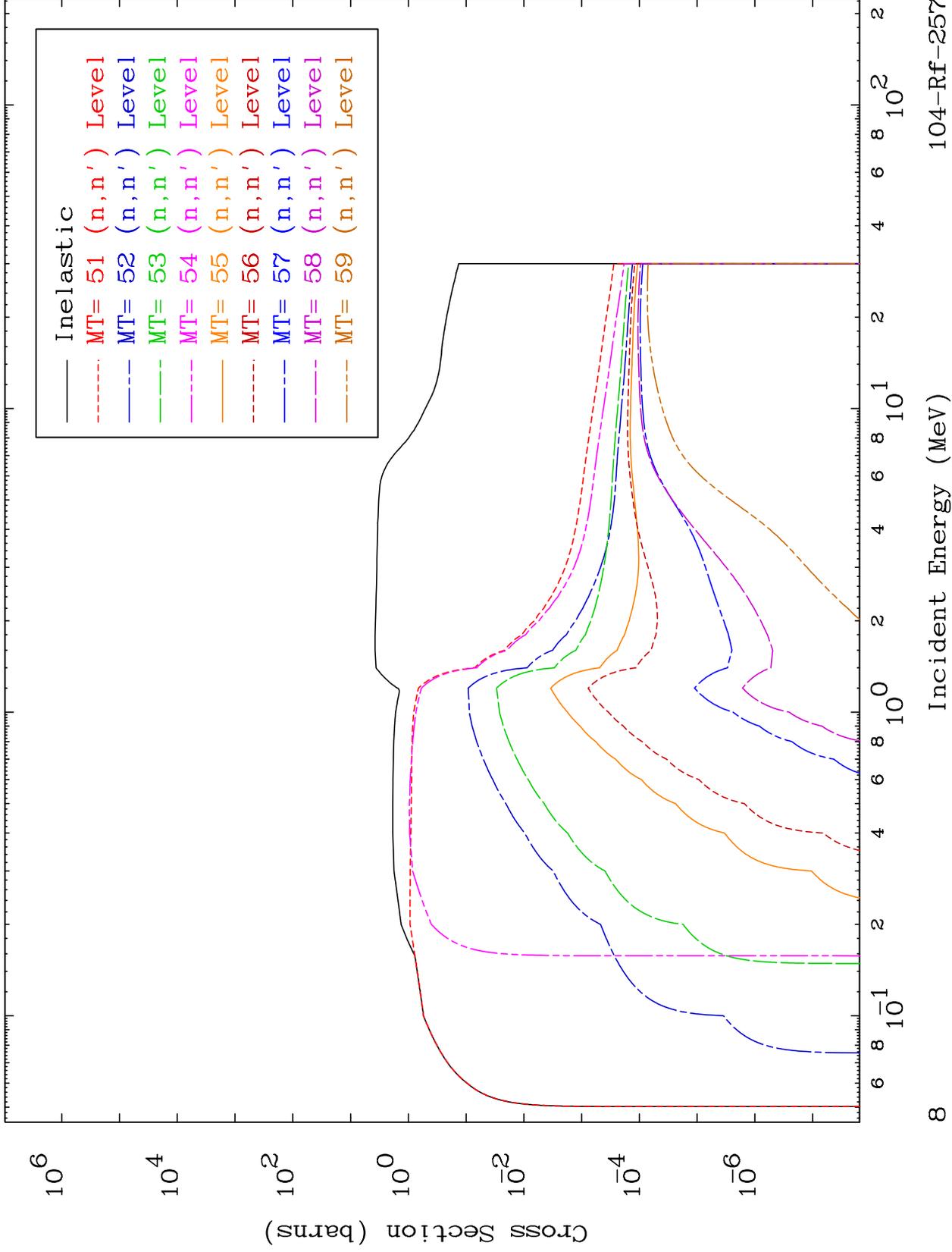


MAT 457

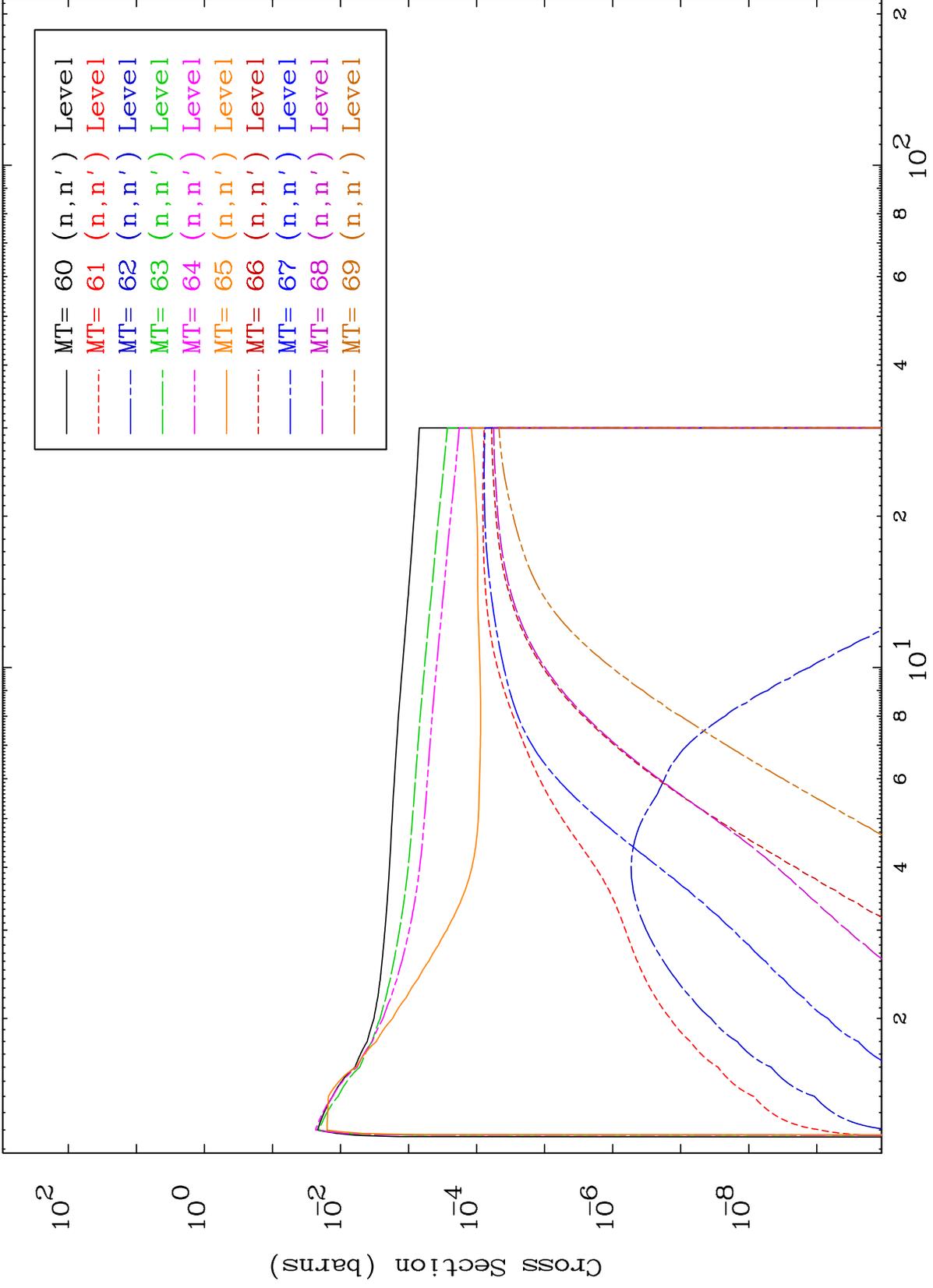
(n,n') Level

104-Rf-257

293 Kelvin Cross Sections



293 Kelvin Cross Sections

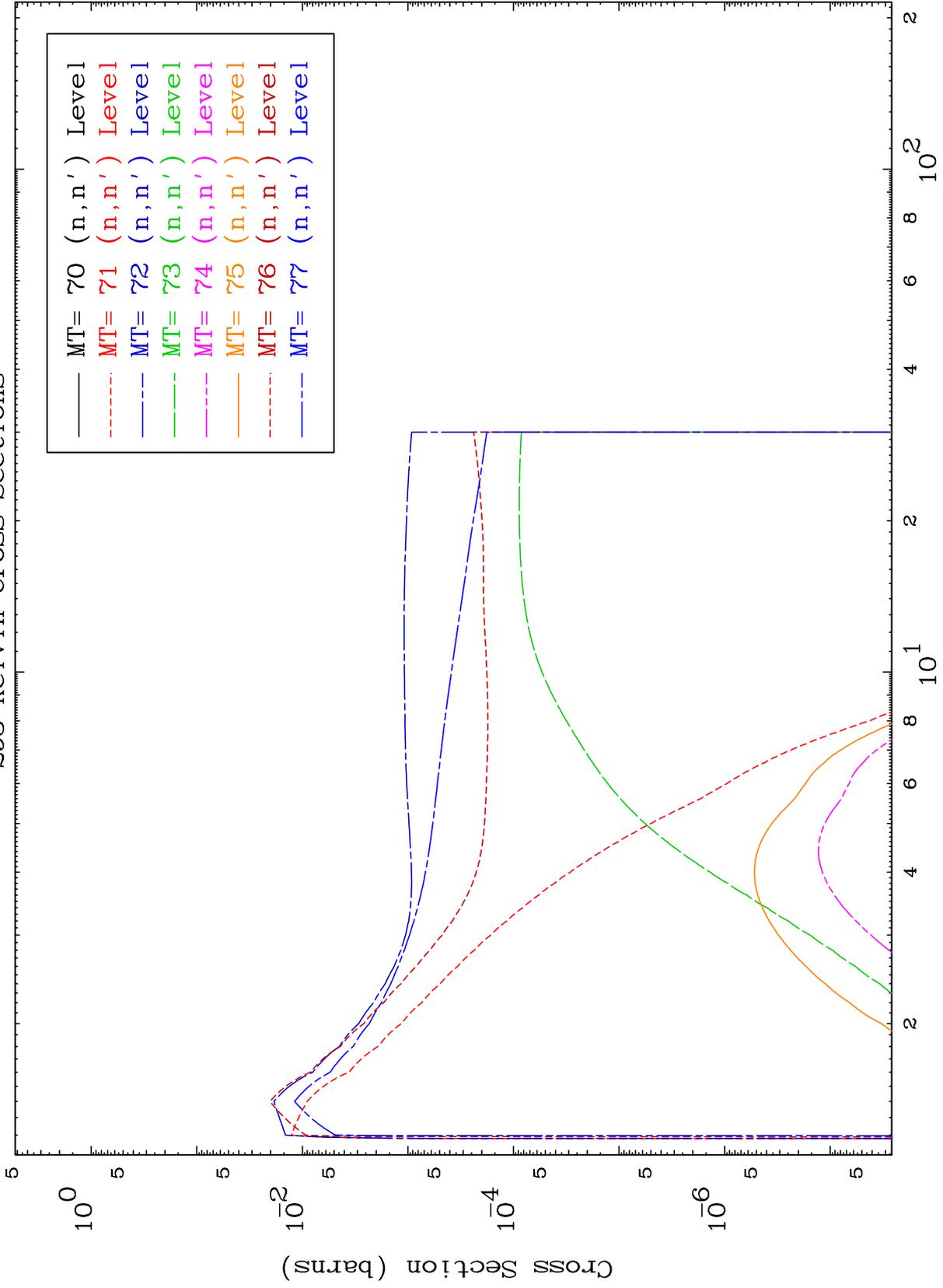


MAT 457

(n,n') Level

293 Kelvin Cross Sections

104-Rf-257



10

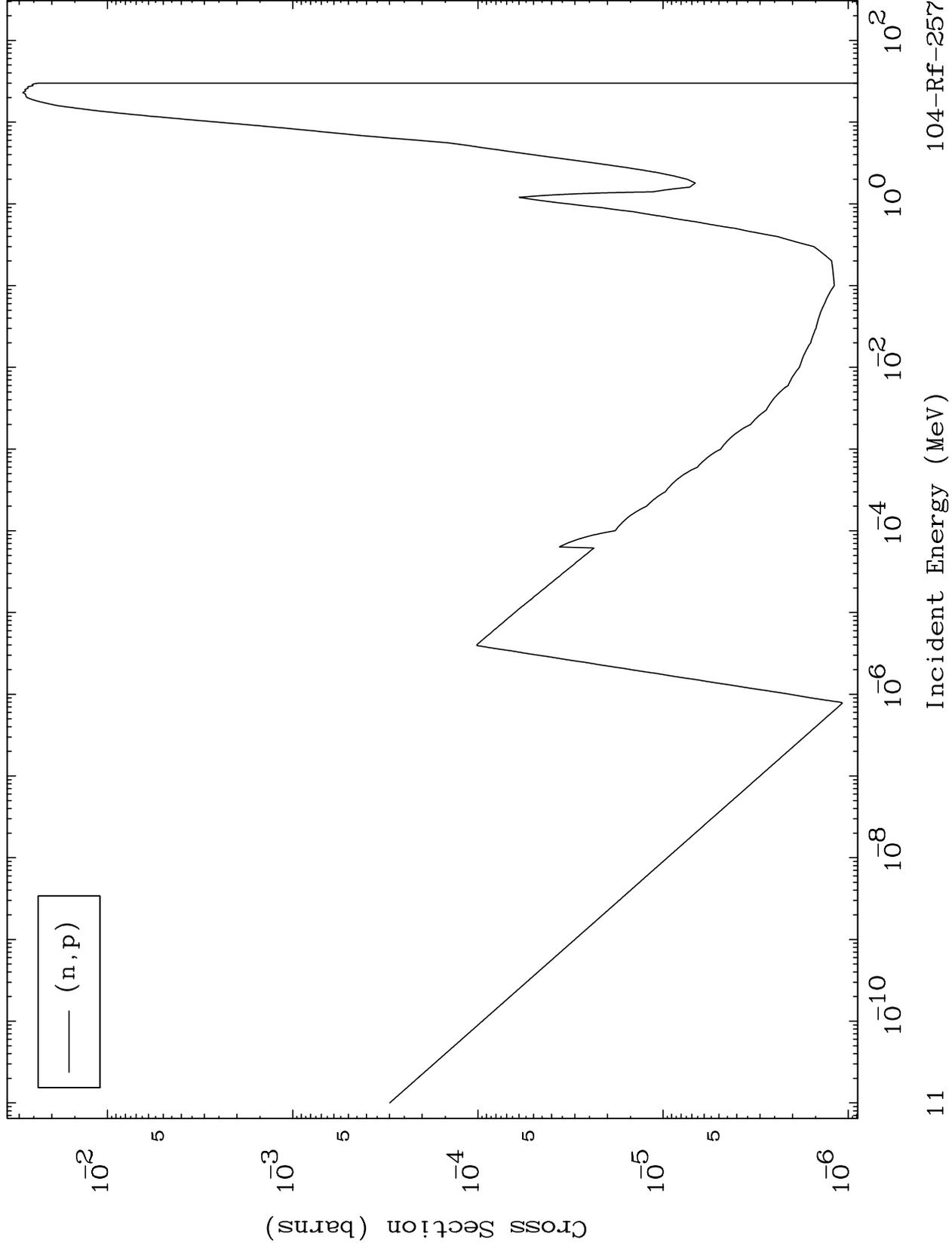
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,p) Levels
293 Kelvin Cross Sections

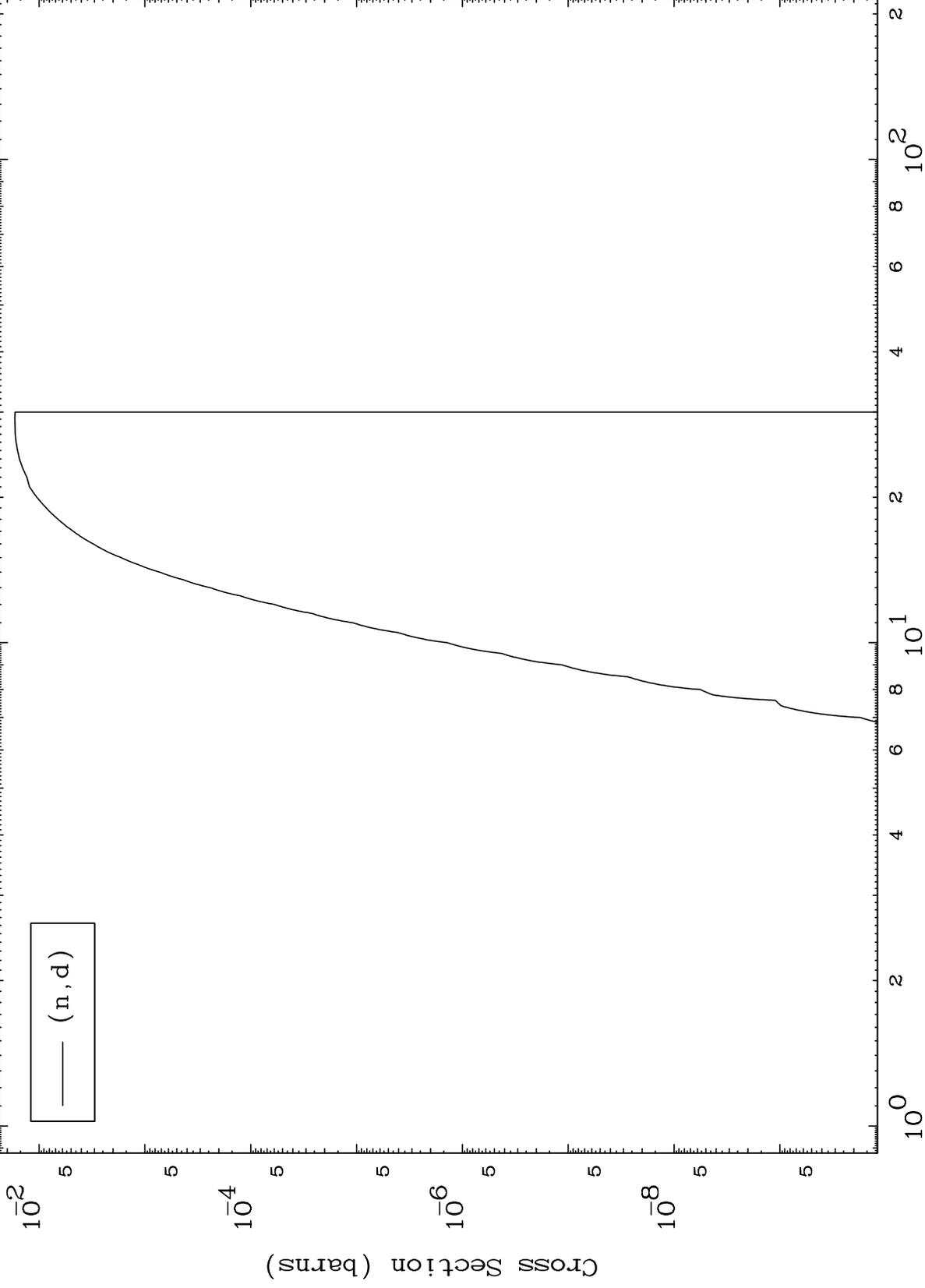
104-Rf-257



MAT 457

(n,d) Levels
293 Kelvin Cross Sections

104-Rf-257



12

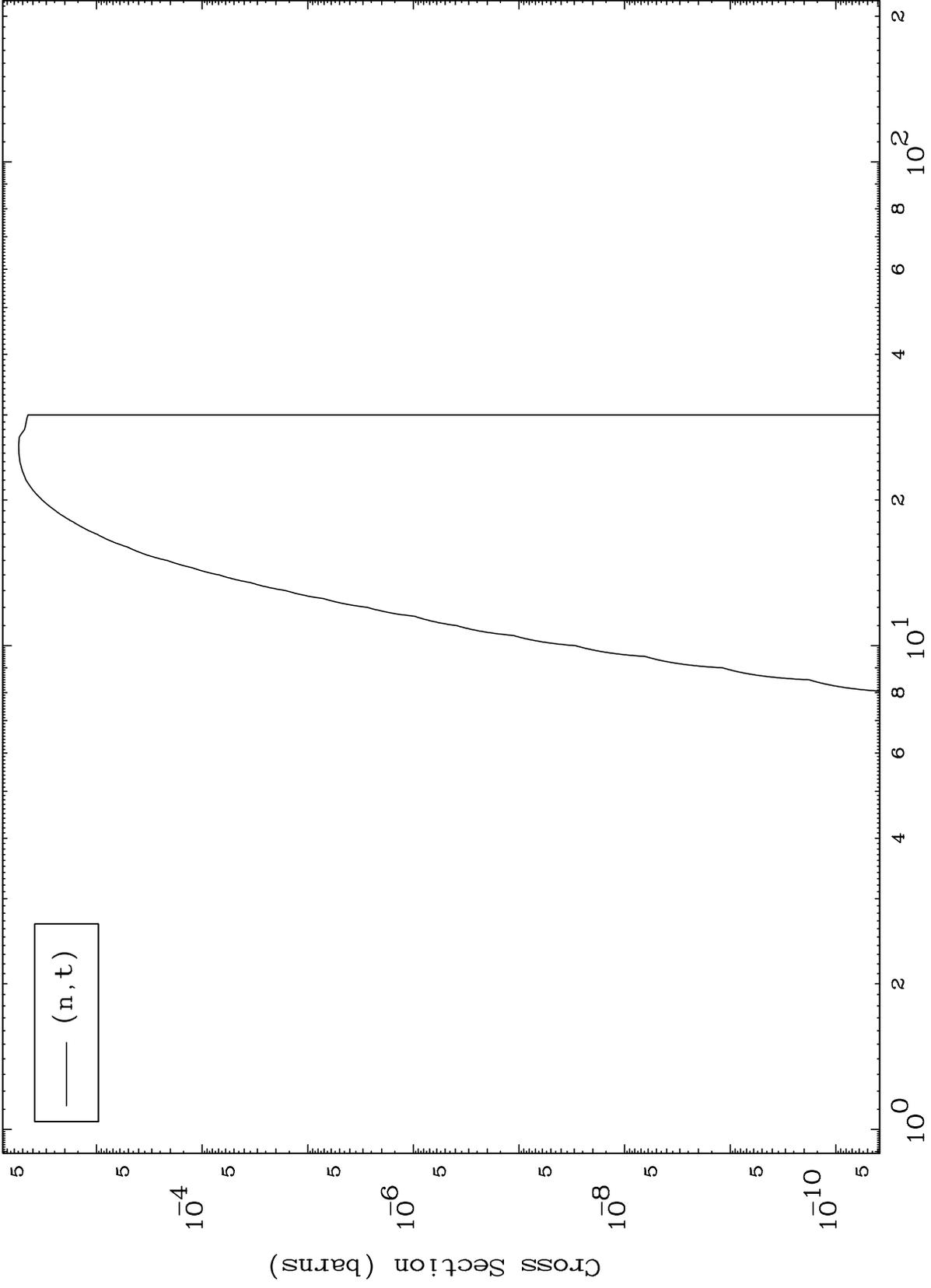
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,t) Levels
293 Kelvin Cross Sections

104-Rf-257



13

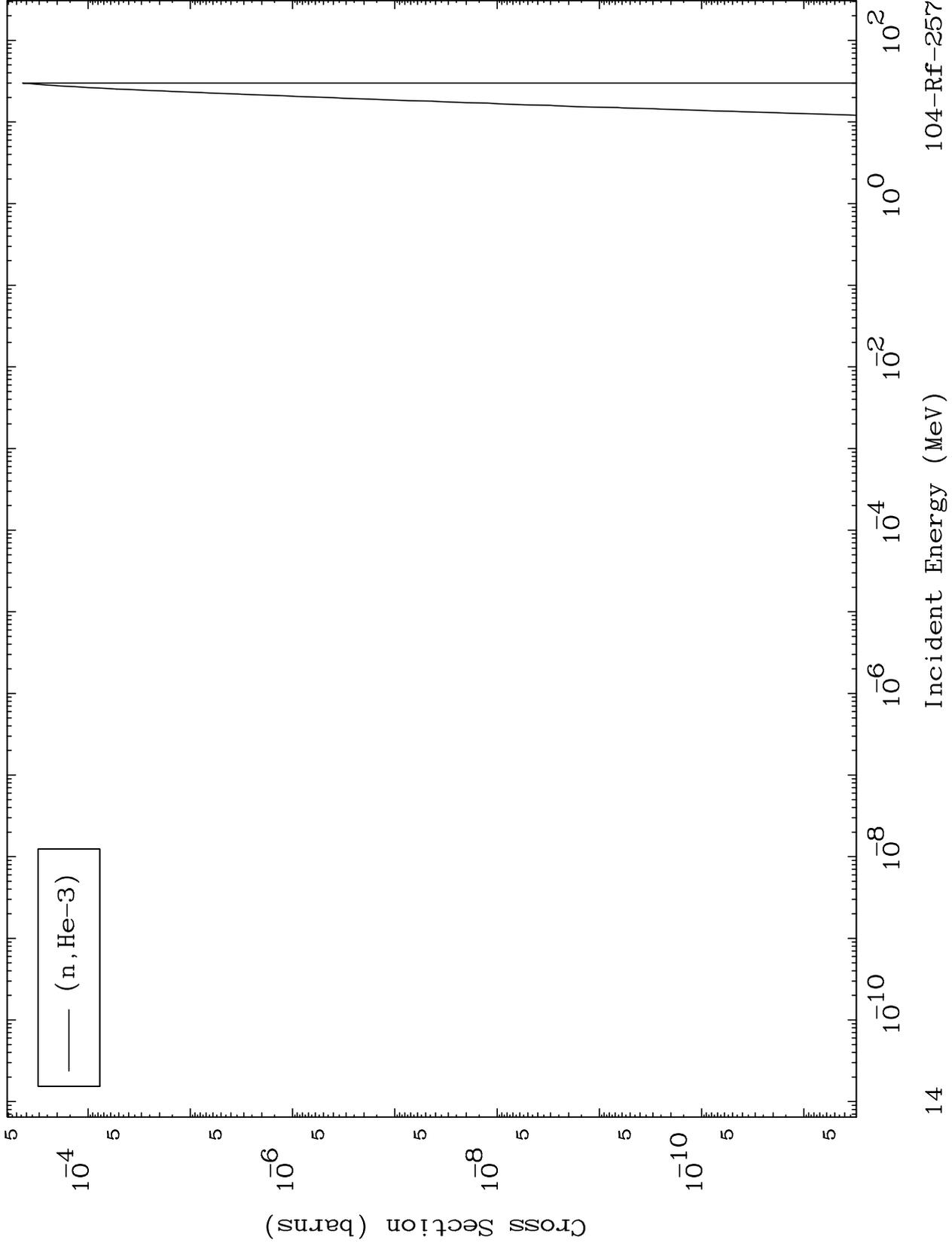
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,He3) Levels
293 Kelvin Cross Sections

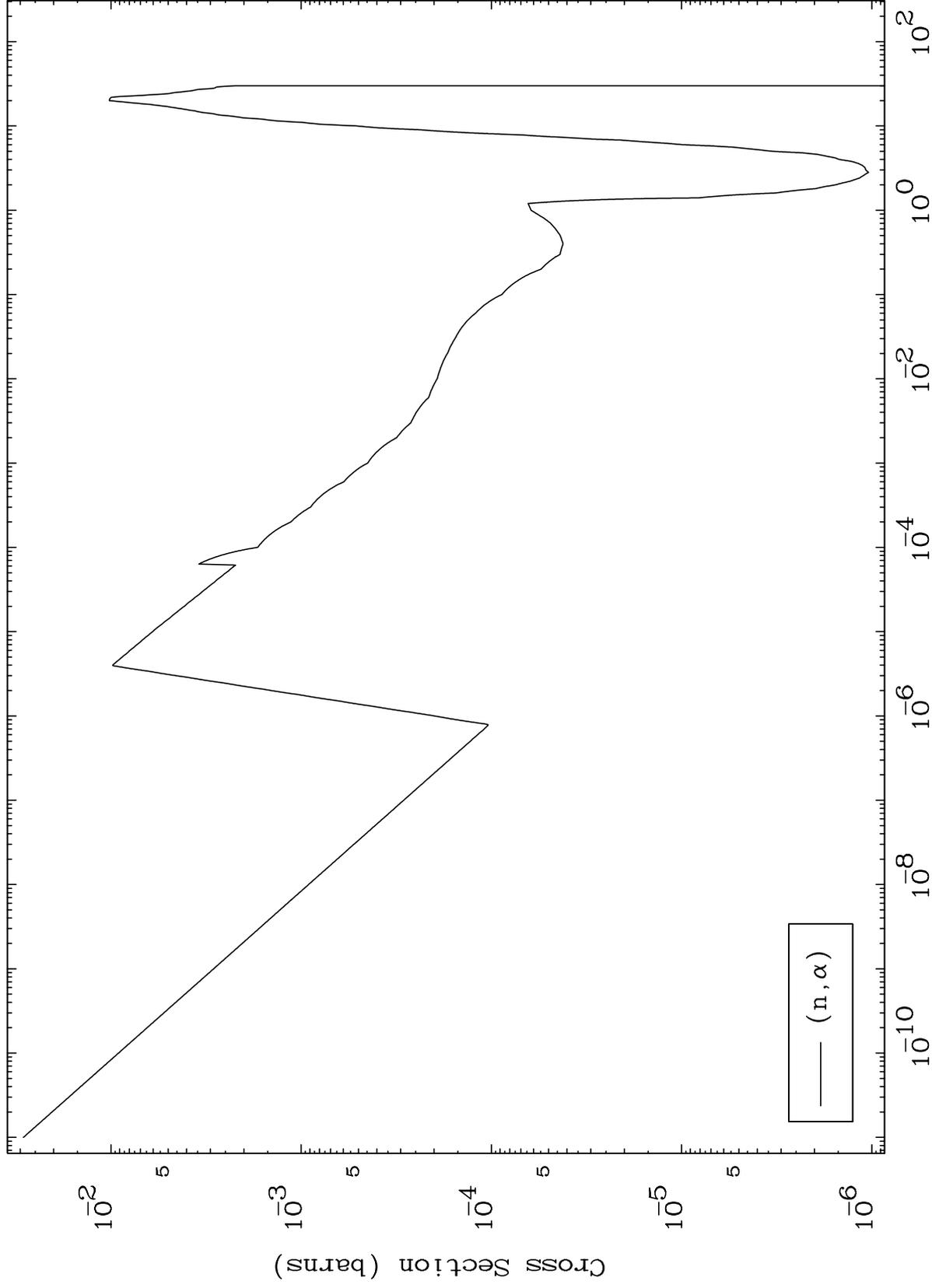
104-Rf-257



MAT 457

(n, α) Levels
293 Kelvin Cross Sections

104-Rf-257

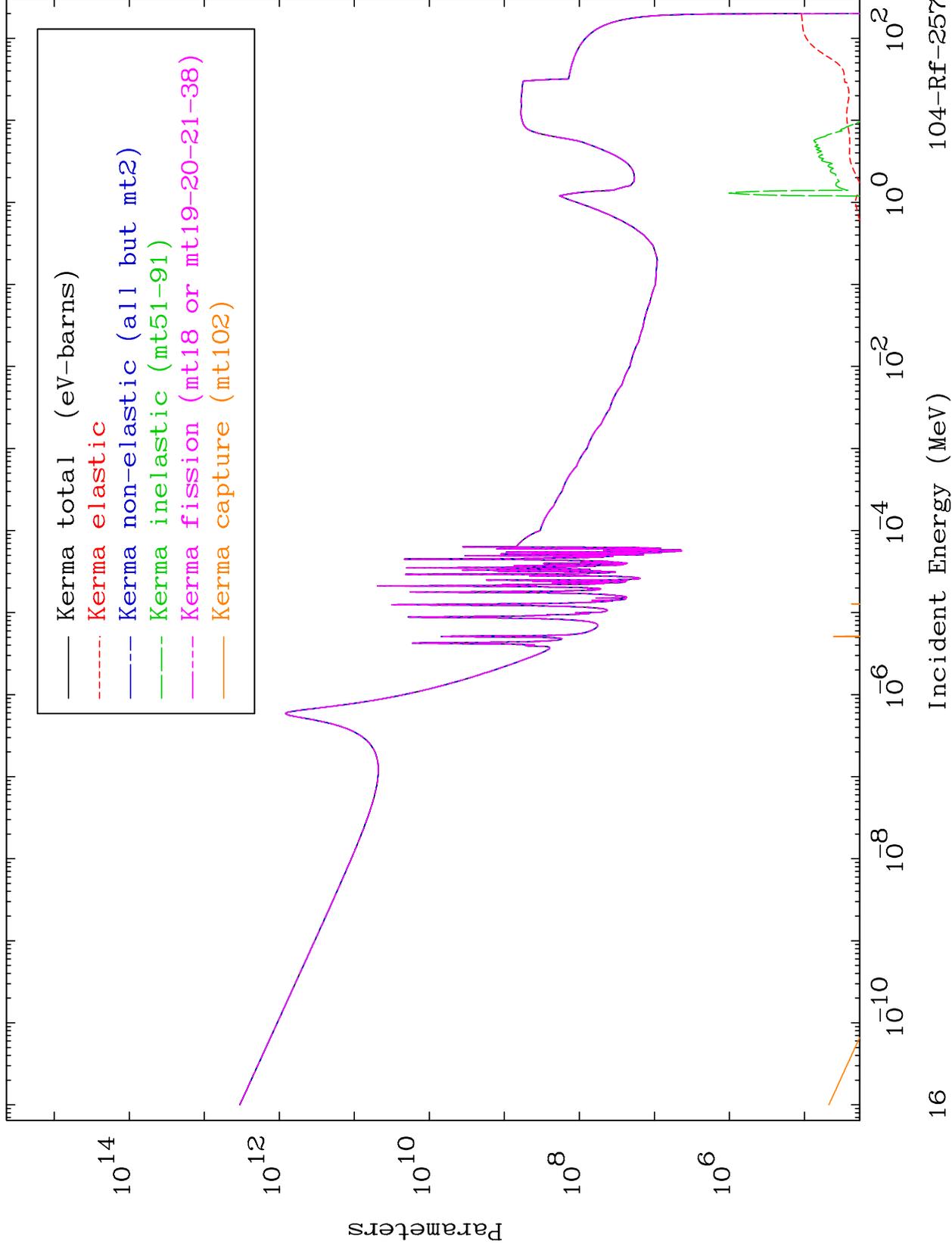


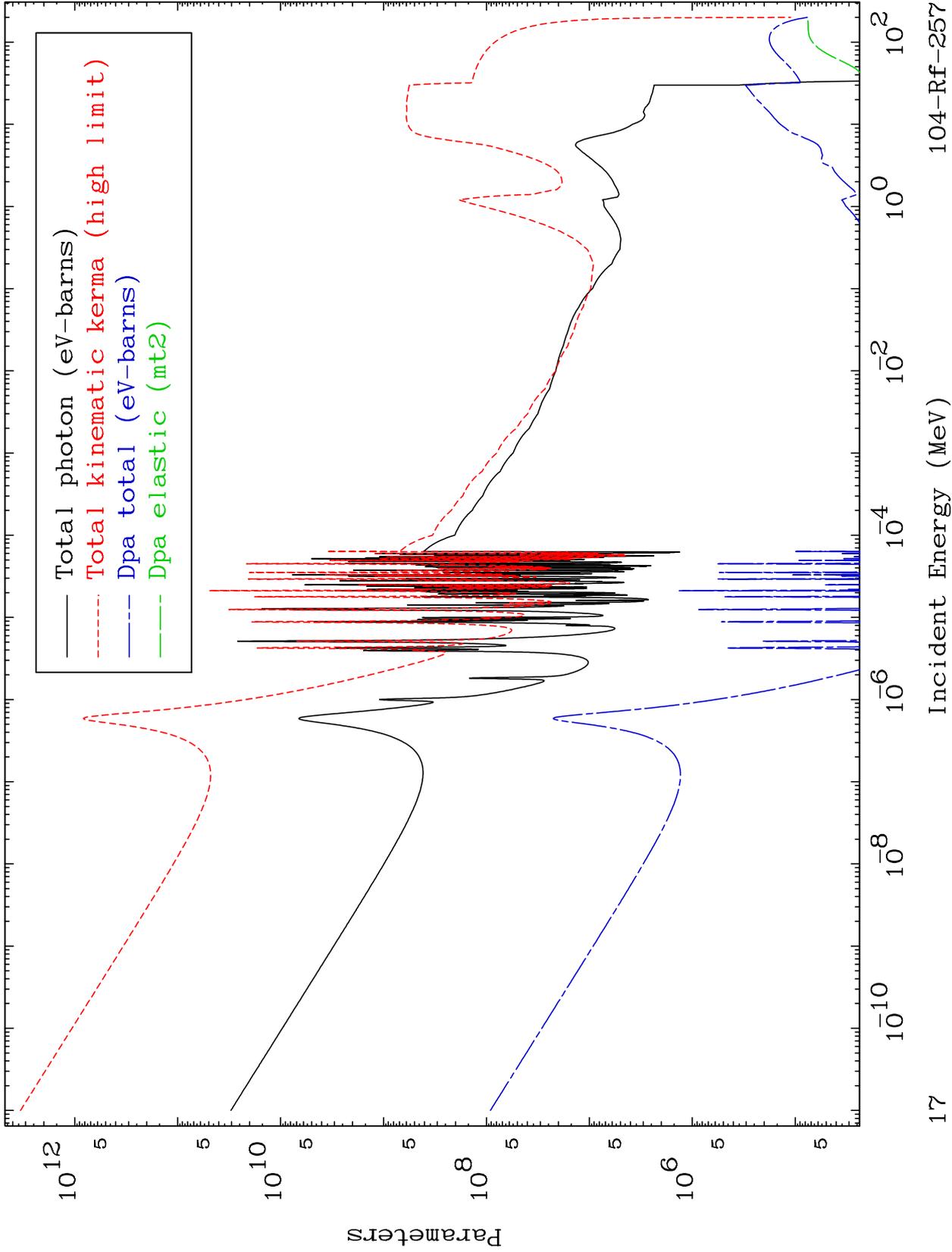
(n, α)

Incident Energy (MeV)

104-Rf-257

15

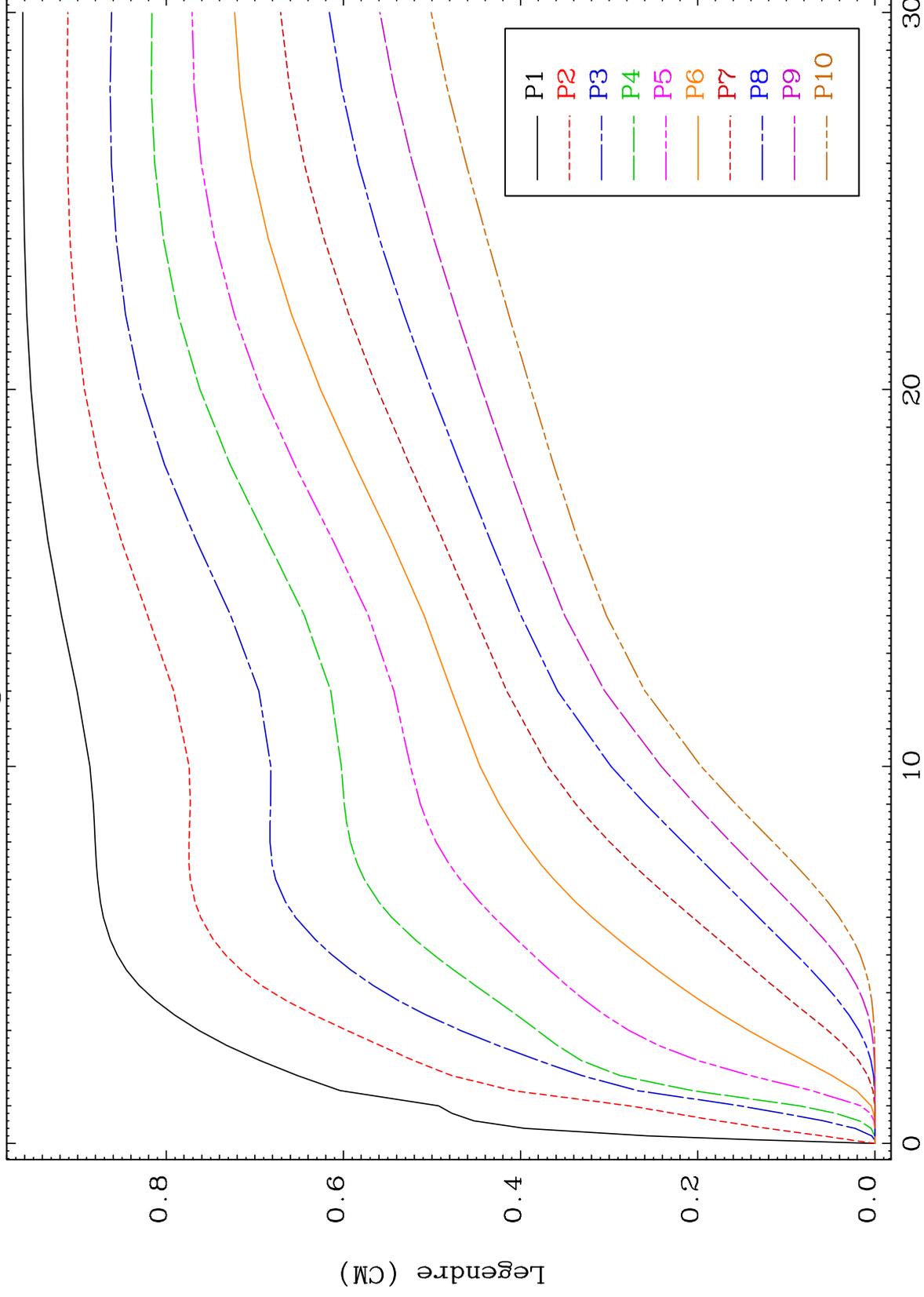




MAT 457

Elastic Legendre Coefficients

104-Rf-257

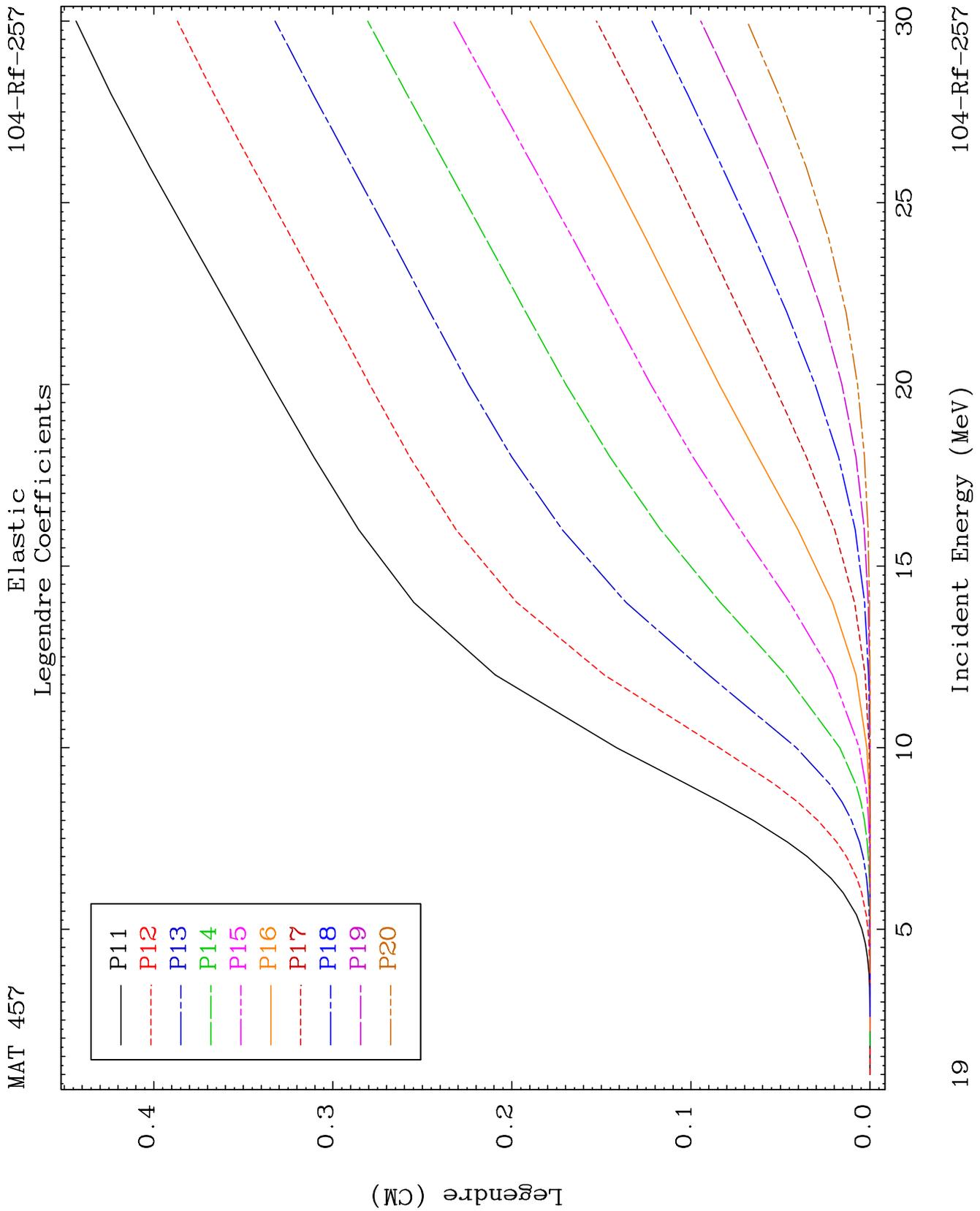


18

Incident Energy (MeV)

104-Rf-257

30



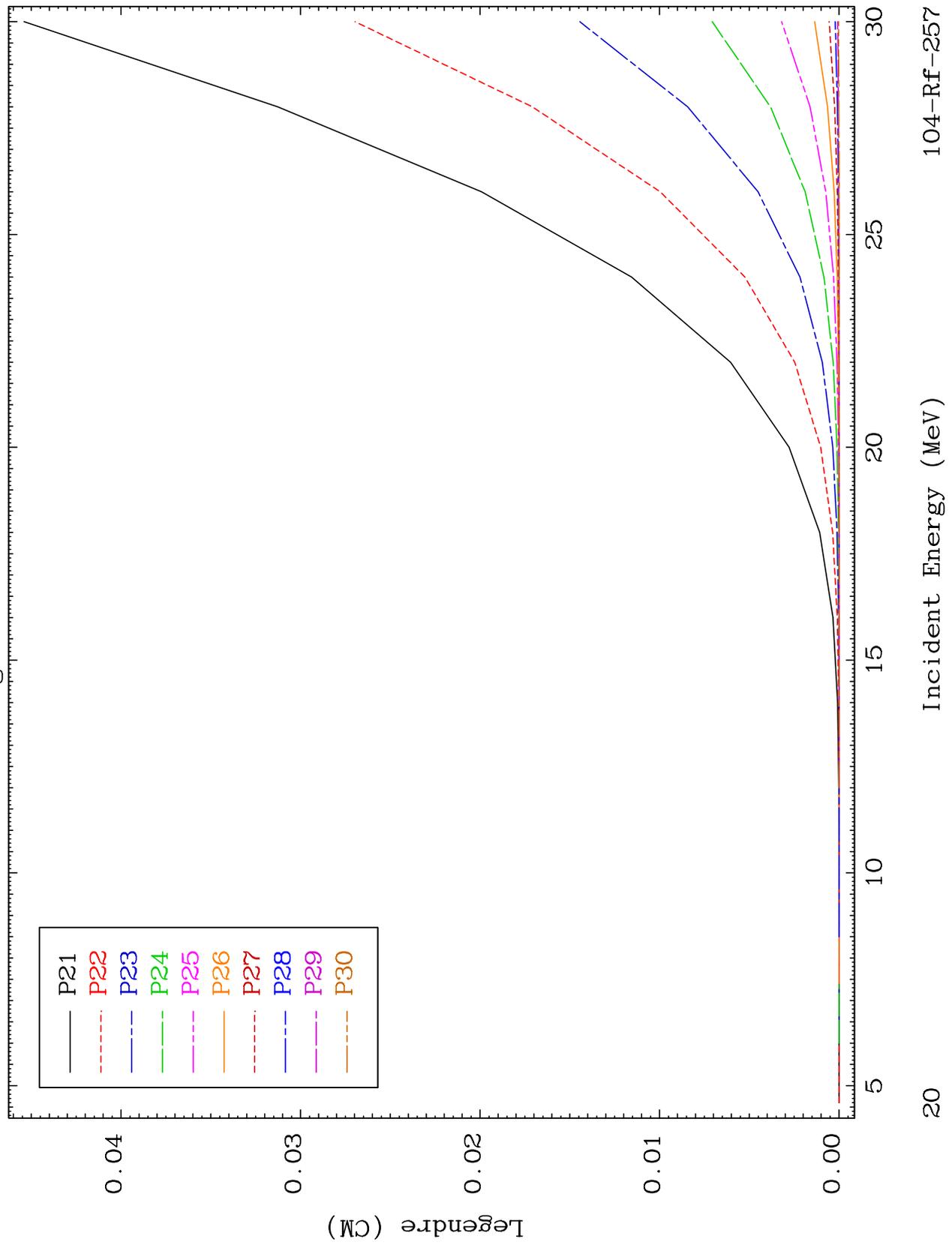
104-Rf-257

104-Rf-257

MAT 457

Elastic Legendre Coefficients

104-Rf-257

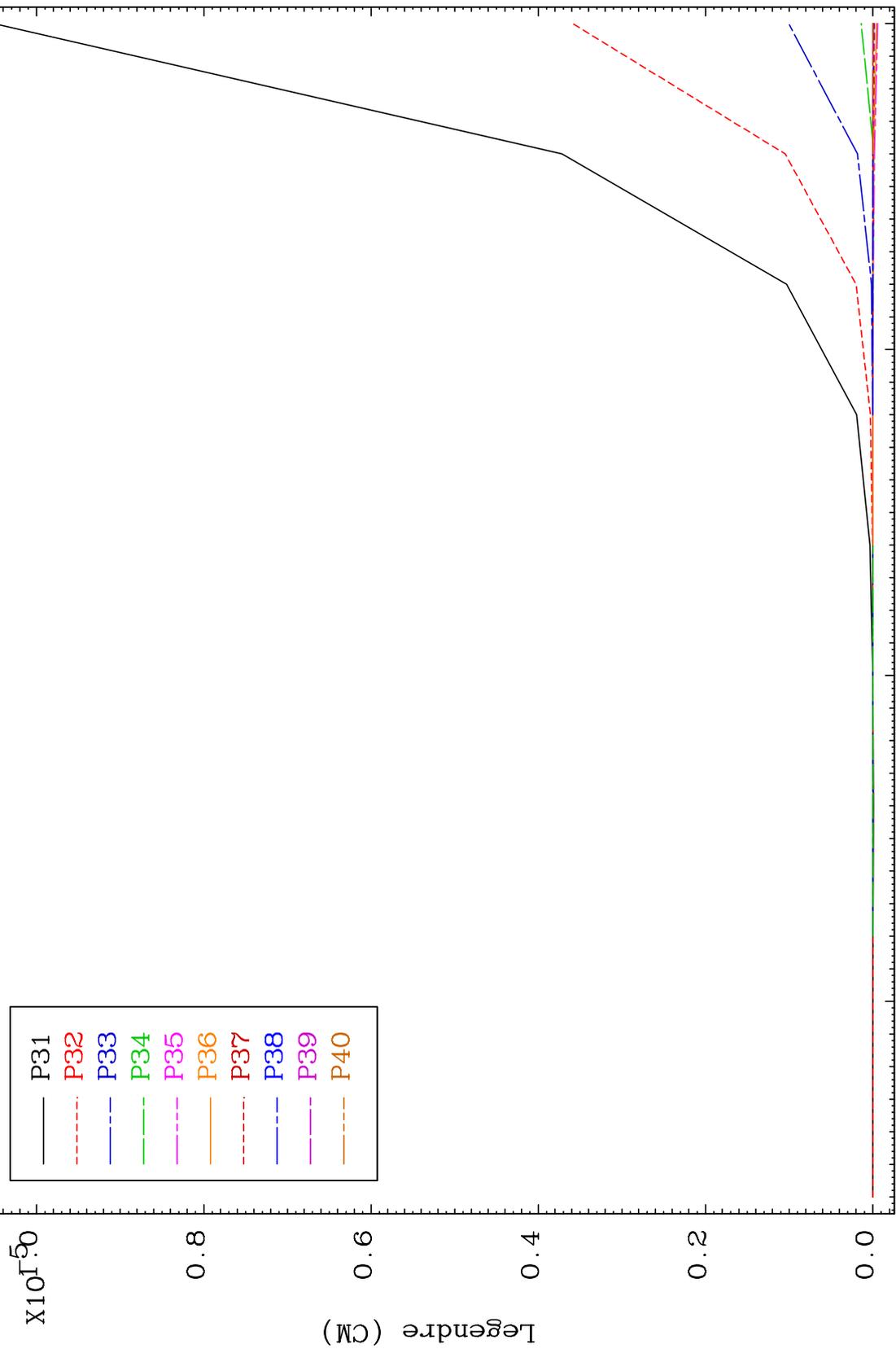
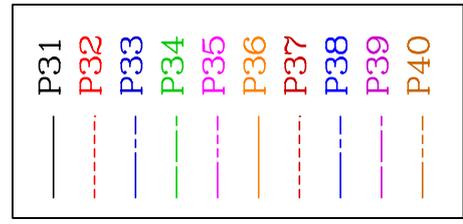


104-Rf-257

MAT 457

Elastic Legendre Coefficients

104-Rf-257



21

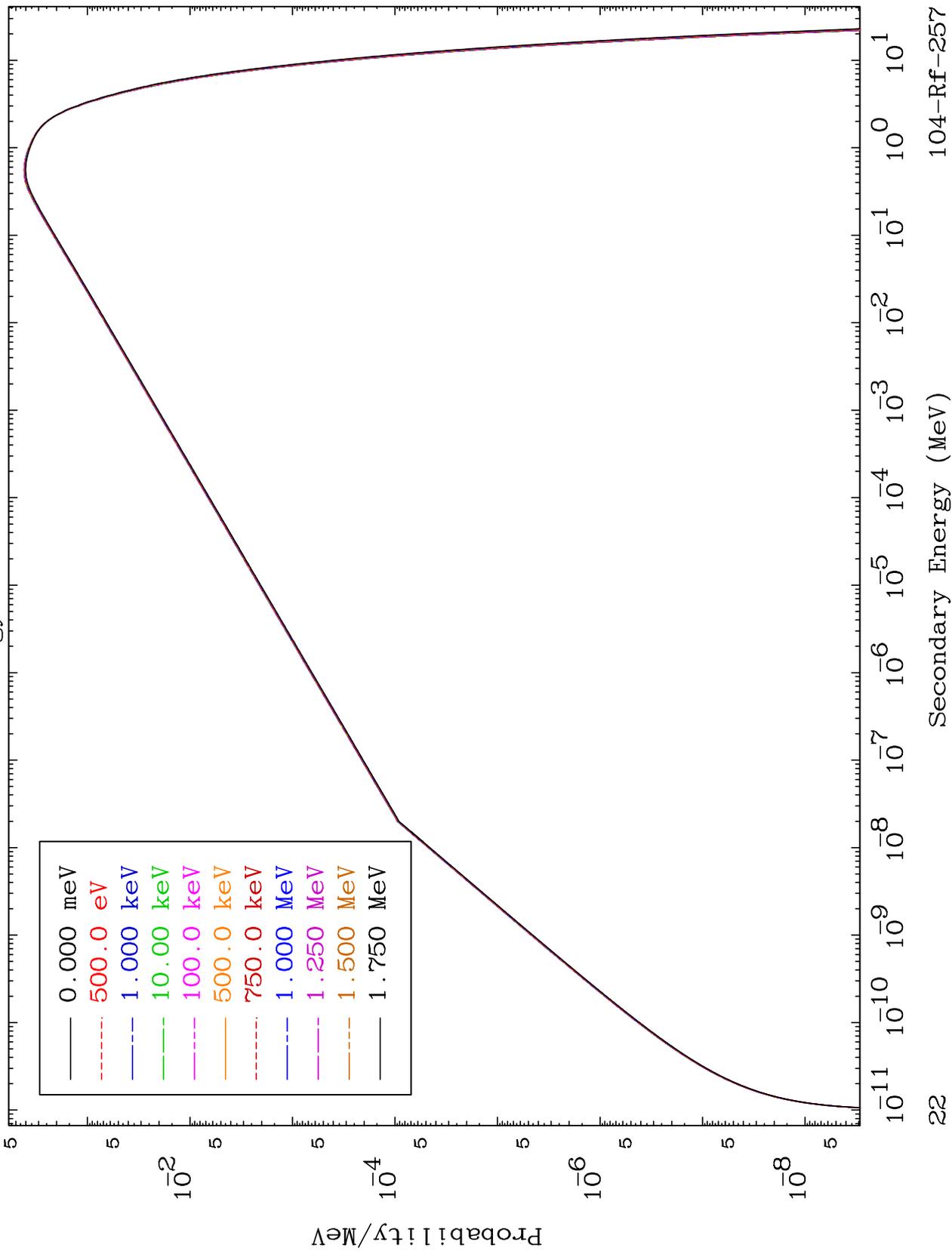
104-Rf-257

Incident Energy (MeV)

MAT 457

Fission Energy Distribution

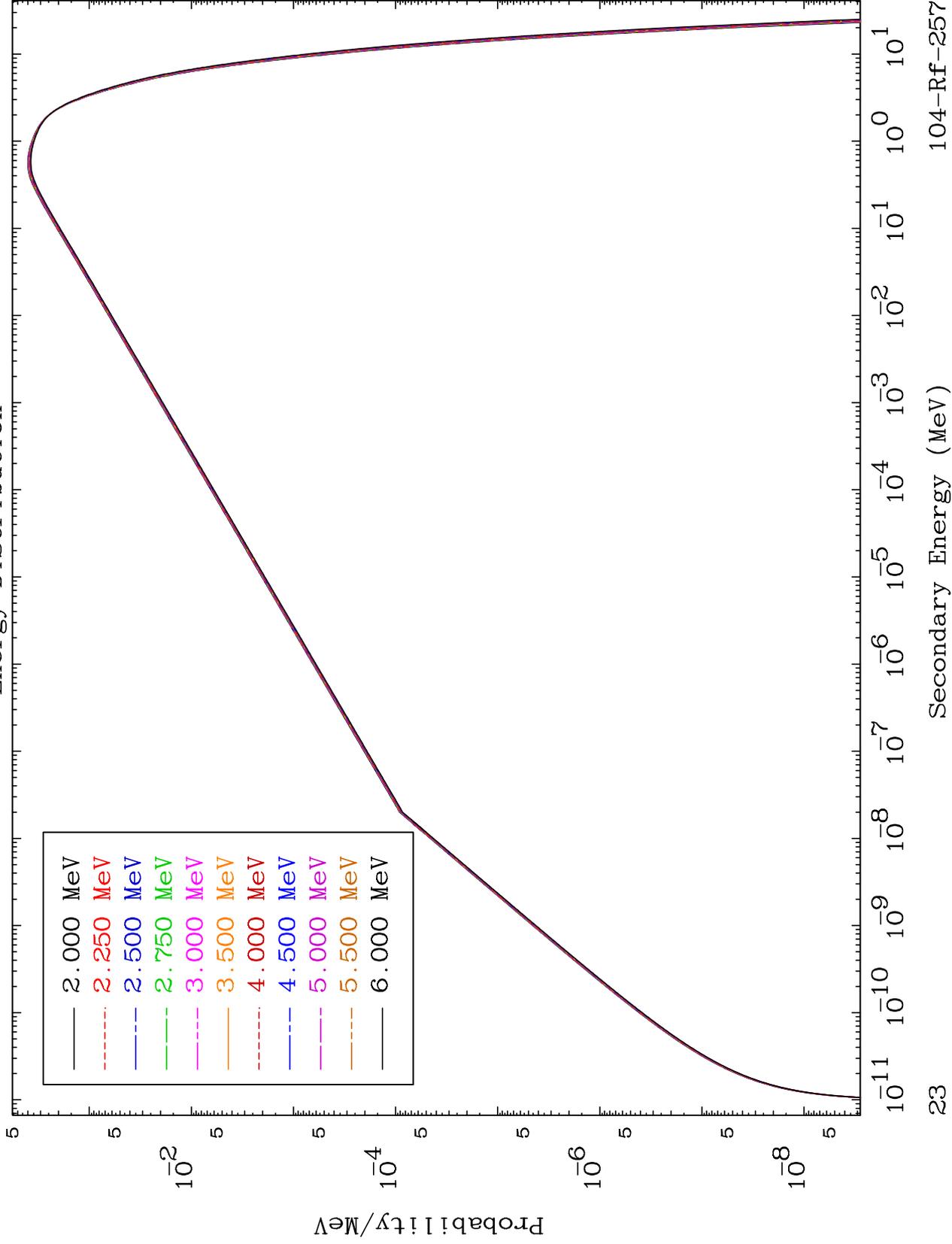
104-Rf-257



MAT 457

Fission Energy Distribution

104-Rf-257



104-Rf-257

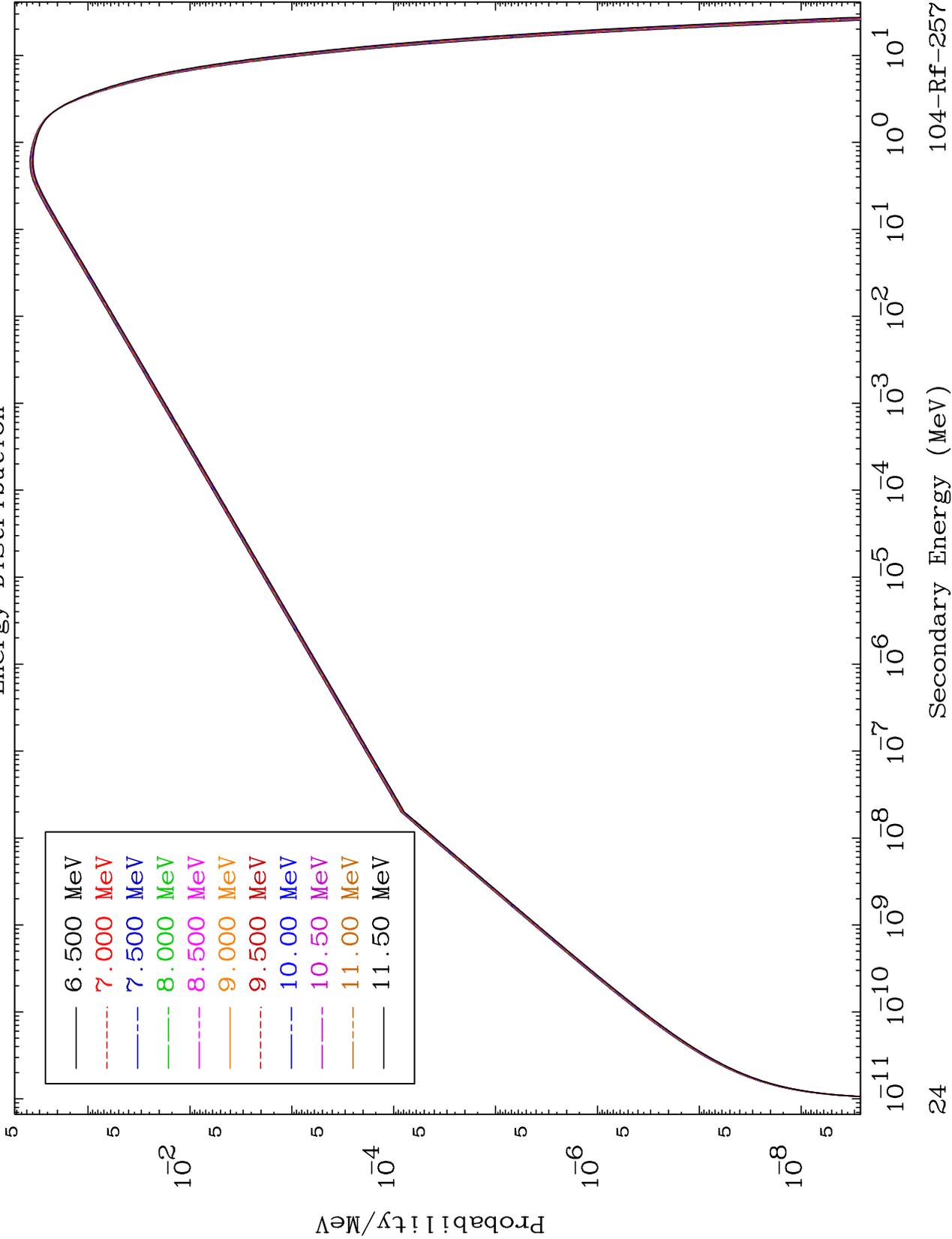
Secondary Energy (MeV)

23

MAT 457

Fission Energy Distribution

104-Rf-257



104-Rf-257

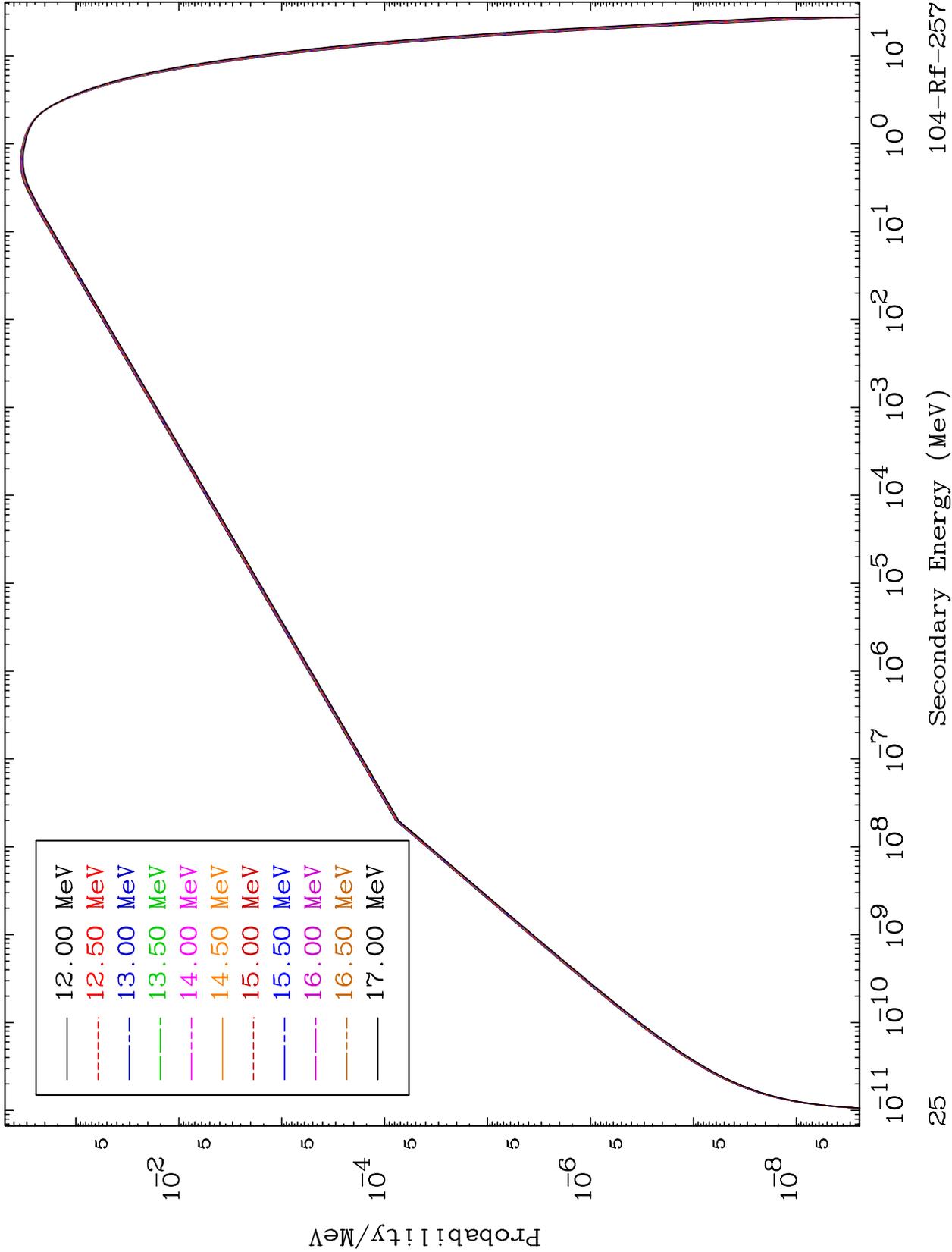
Secondary Energy (MeV)

24

MAT 457

Fission Energy Distribution

104-Rf-257



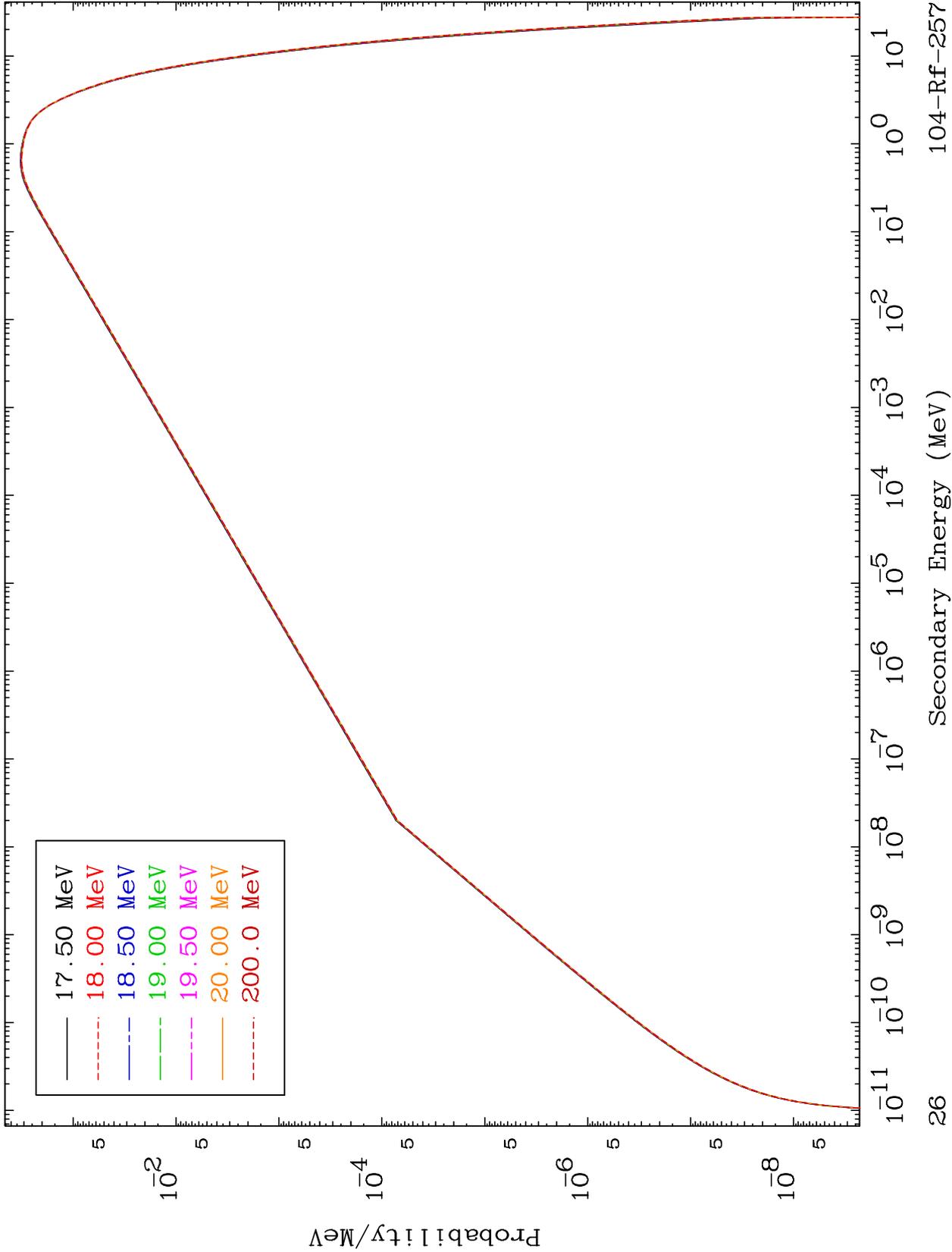
104-Rf-257

25

MAT 457

Fission Energy Distribution

104-Rf-257



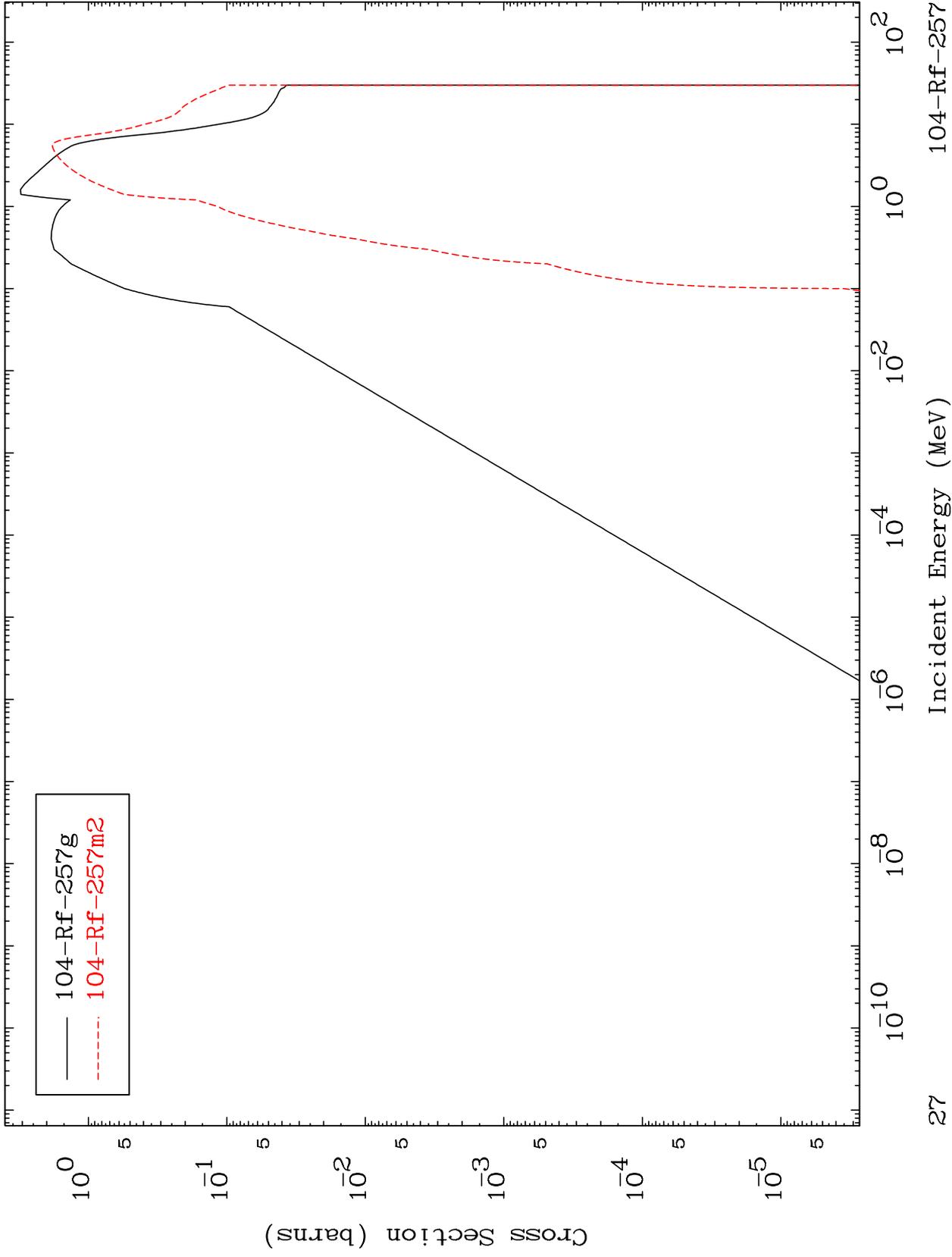
26

104-Rf-257

MAT 457

Inelastic
Radionuclide Production Cross Section

104-Rf-257



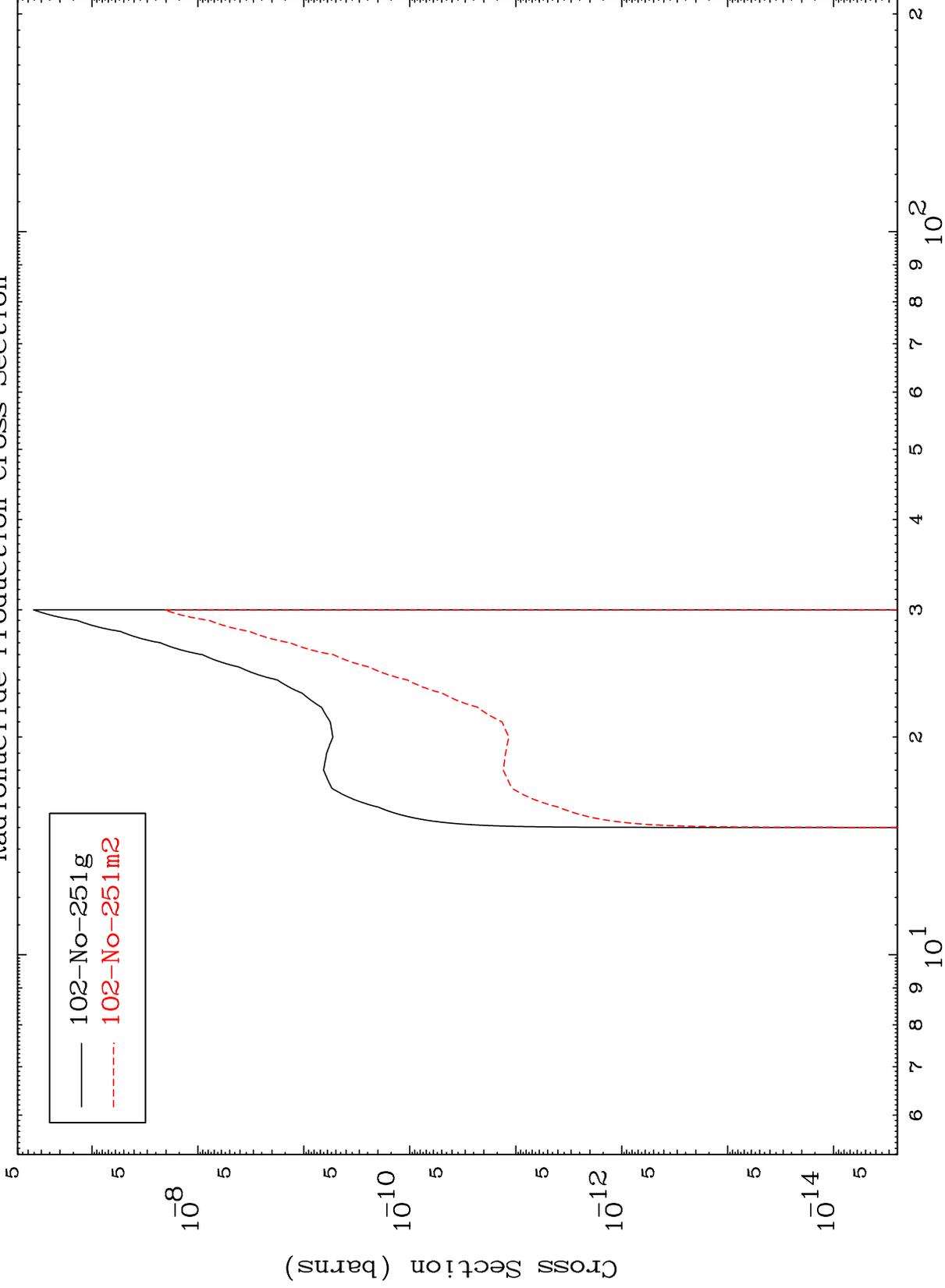
27

MAT 457

(n,3n) α

104-Rf-257

Radionuclide Production Cross Section



28

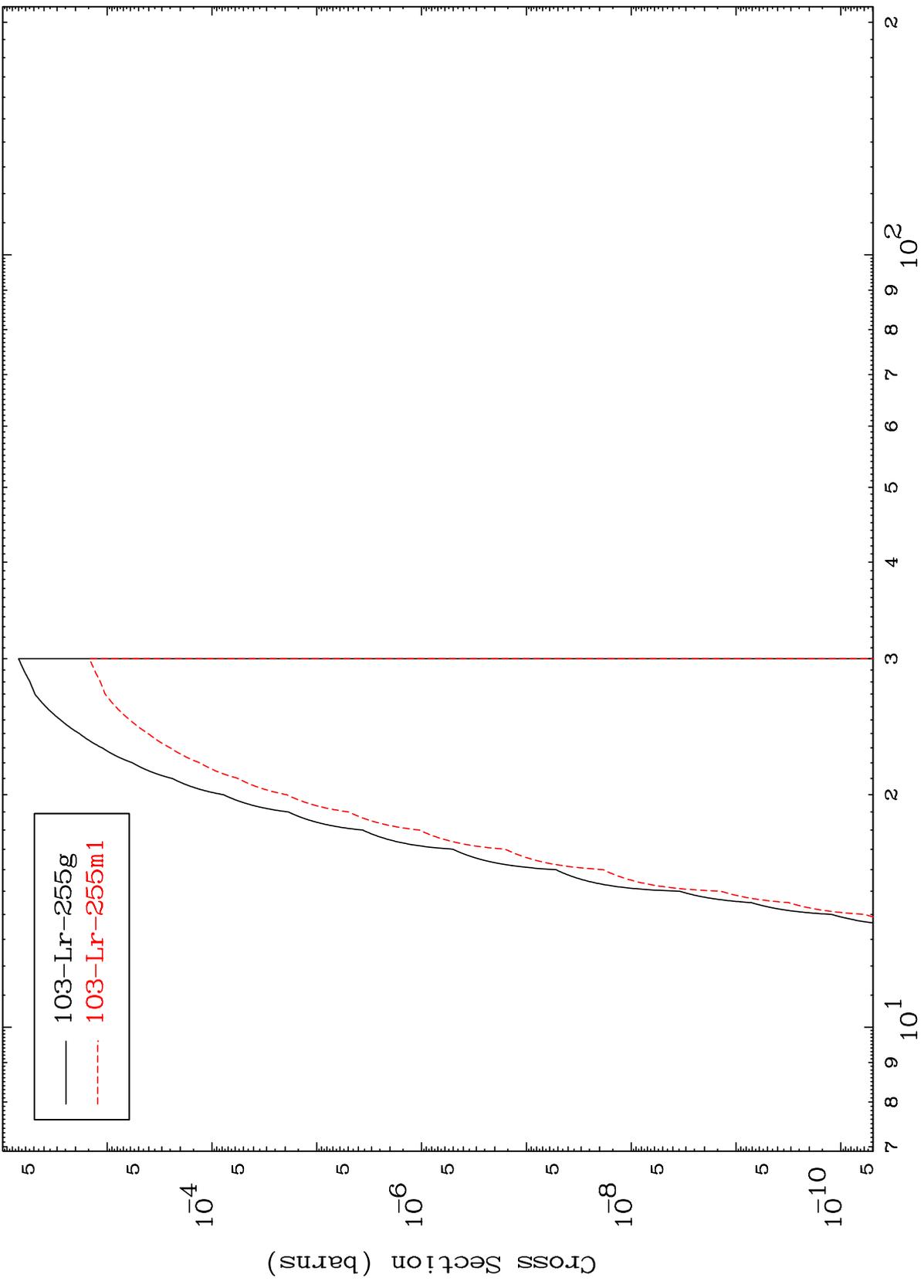
Incident Energy (MeV)

104-Rf-257

MAT 457

104-Rf-257

(n,n') d
Radionuclide Production Cross Section



29

Incident Energy (MeV)

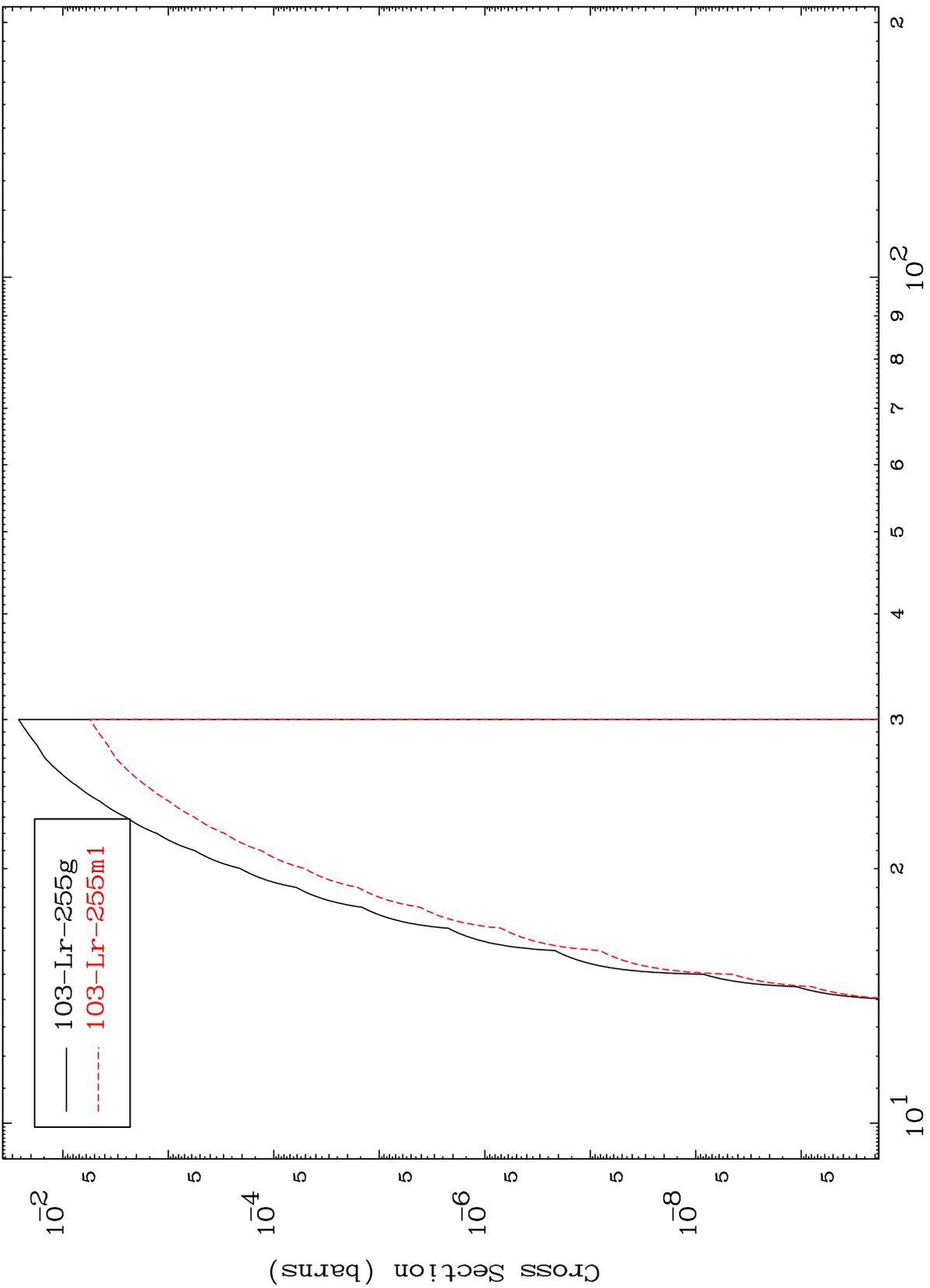
104-Rf-257

MAT 457

(n,2n) p

104-Rf-257

Radionuclide Production Cross Section



Incident Energy (MeV)

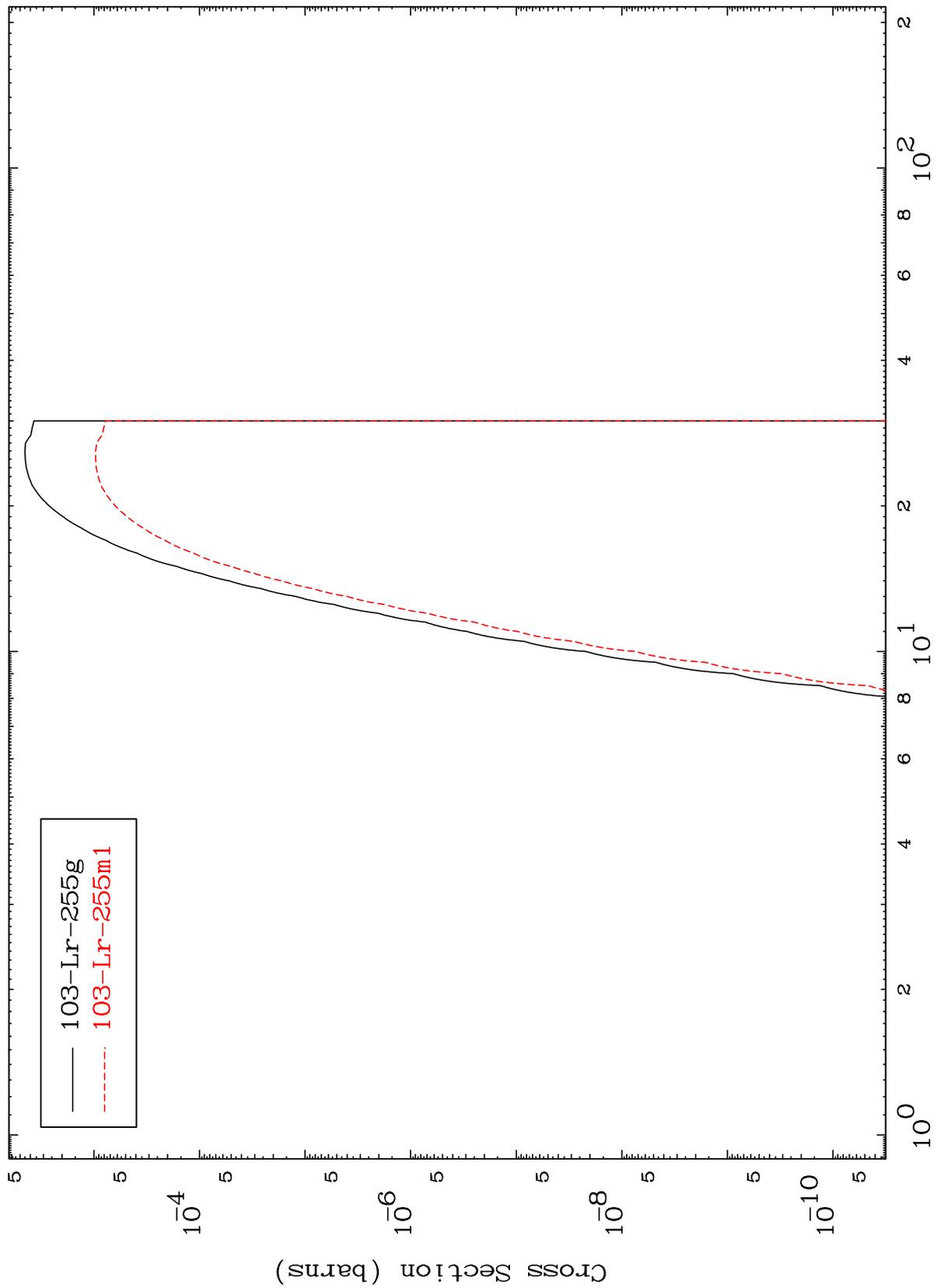
104-Rf-257

30

MAT 457

104-Rf-257

Radionuclide Production Cross Section



Incident Energy (MeV)

104-Rf-257

31

MAT 457

(n,2α)

104-Rf-257

Radionuclide Production Cross Section

