

Program EVALPLOT  
(Version 2018-1)

by

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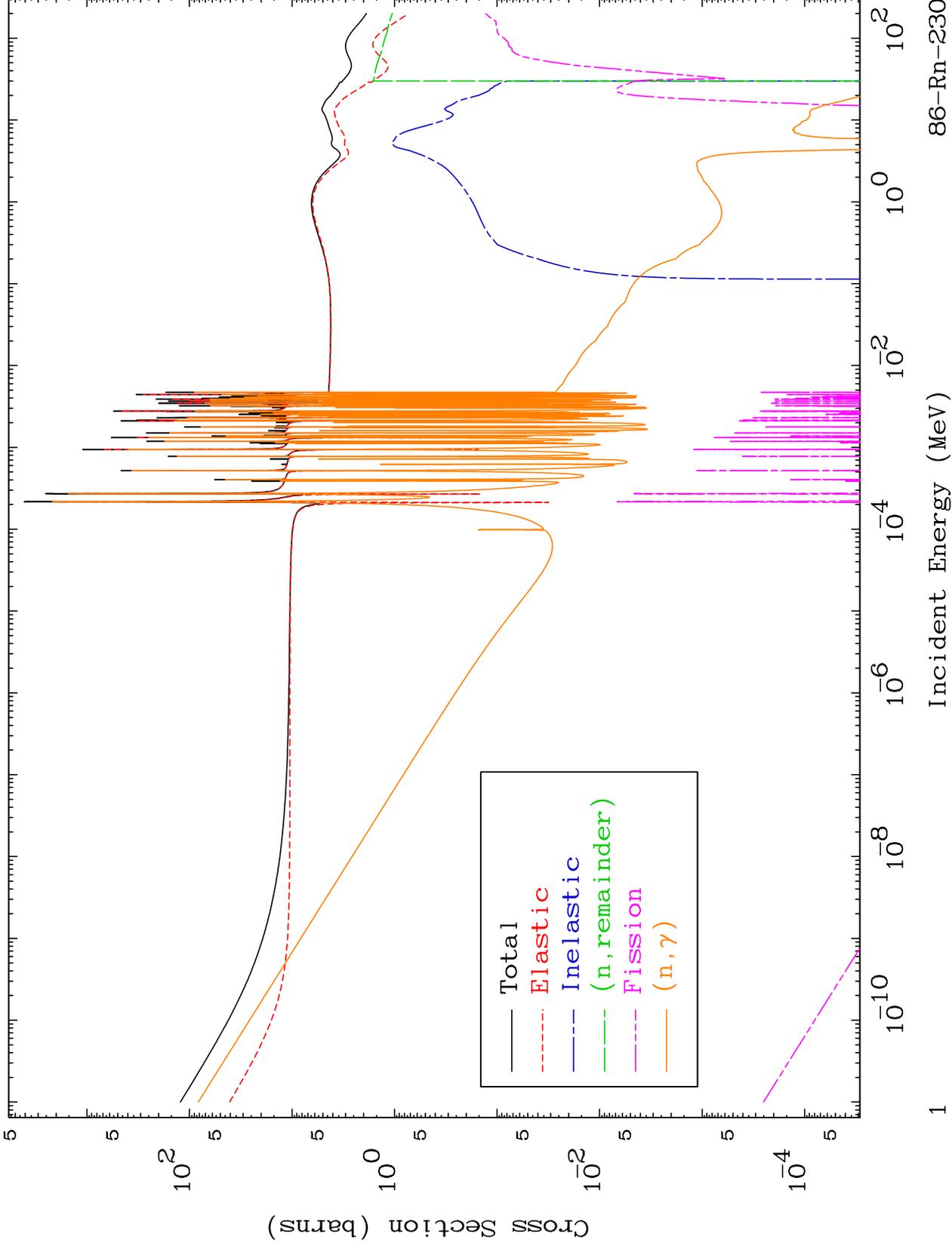
Press Mouse Button to Start

MAT 8682

Major

293 Kelvin Cross Sections

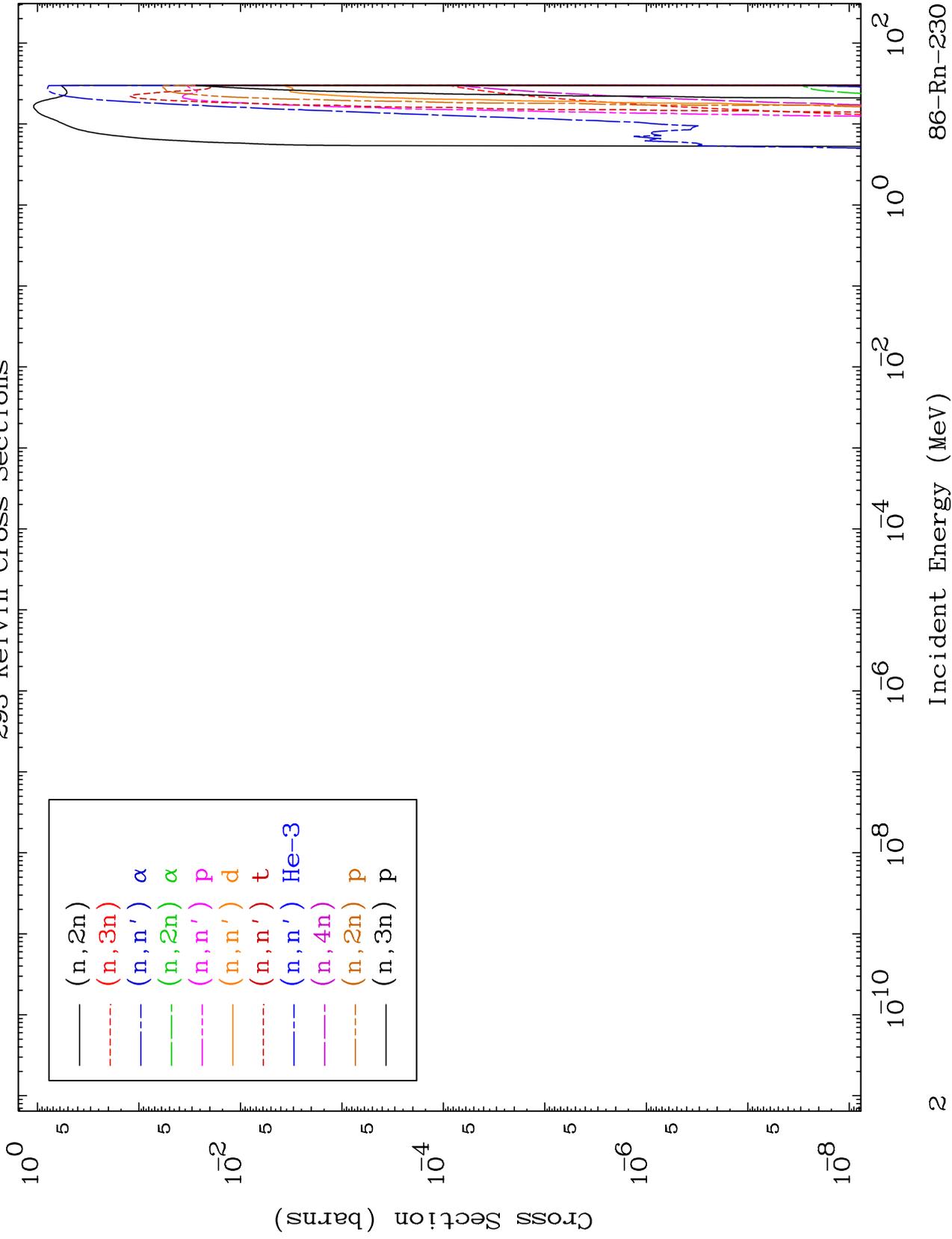
86-Rn-230



MAT 8682

Neutron Production  
293 Kelvin Cross Sections

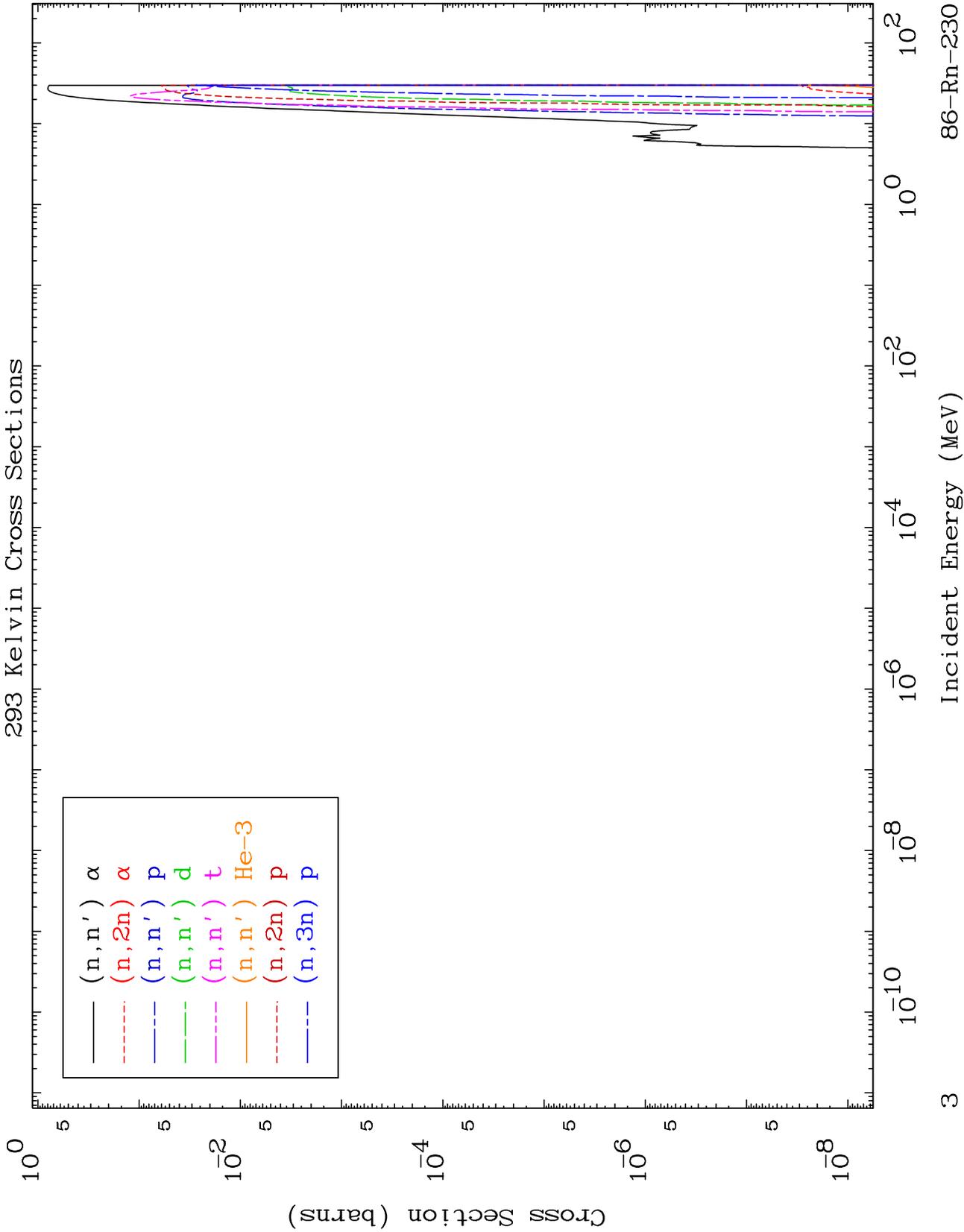
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Charged Particle  
293 Kelvin Cross Sections

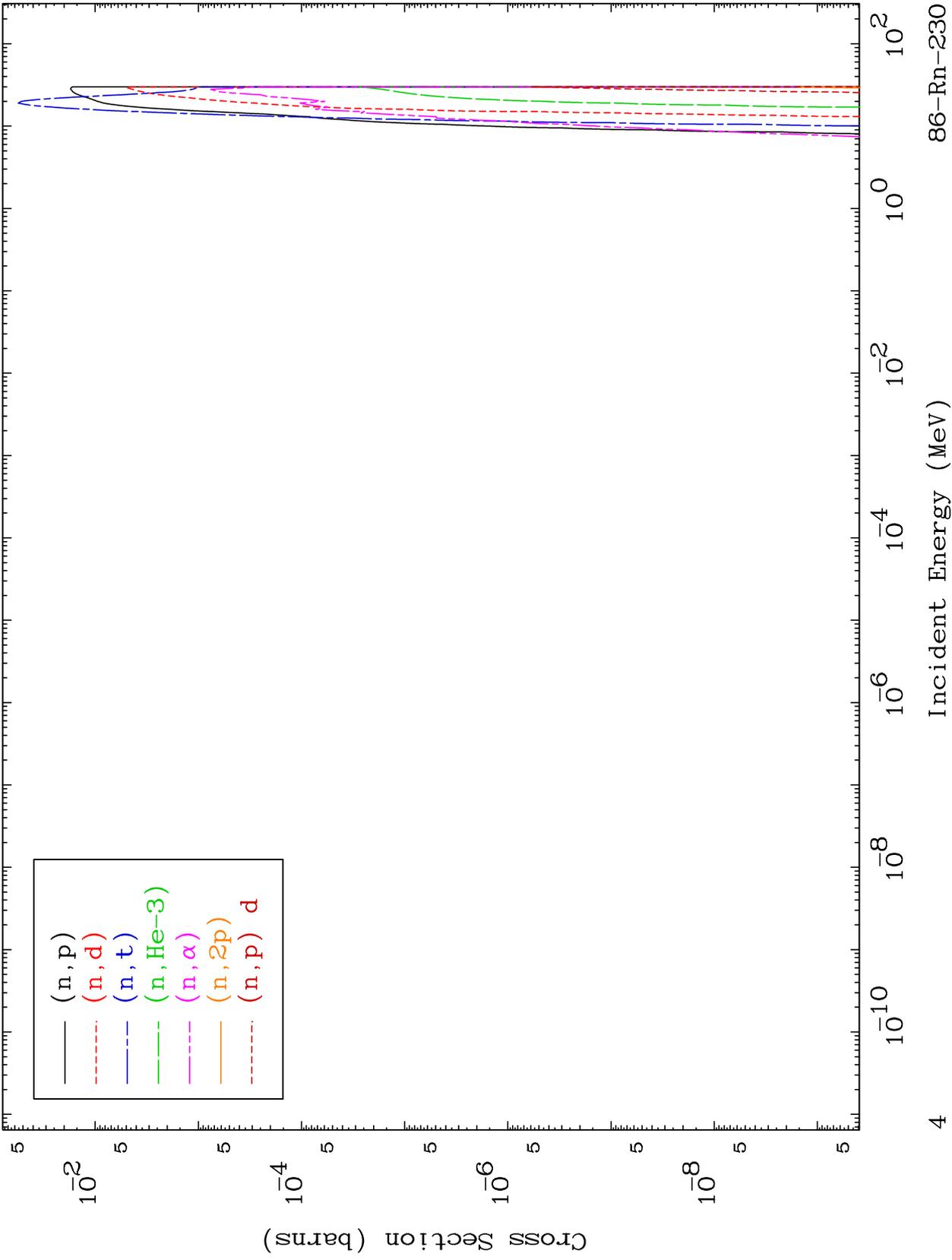
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Charged Particle  
293 Kelvin Cross Sections

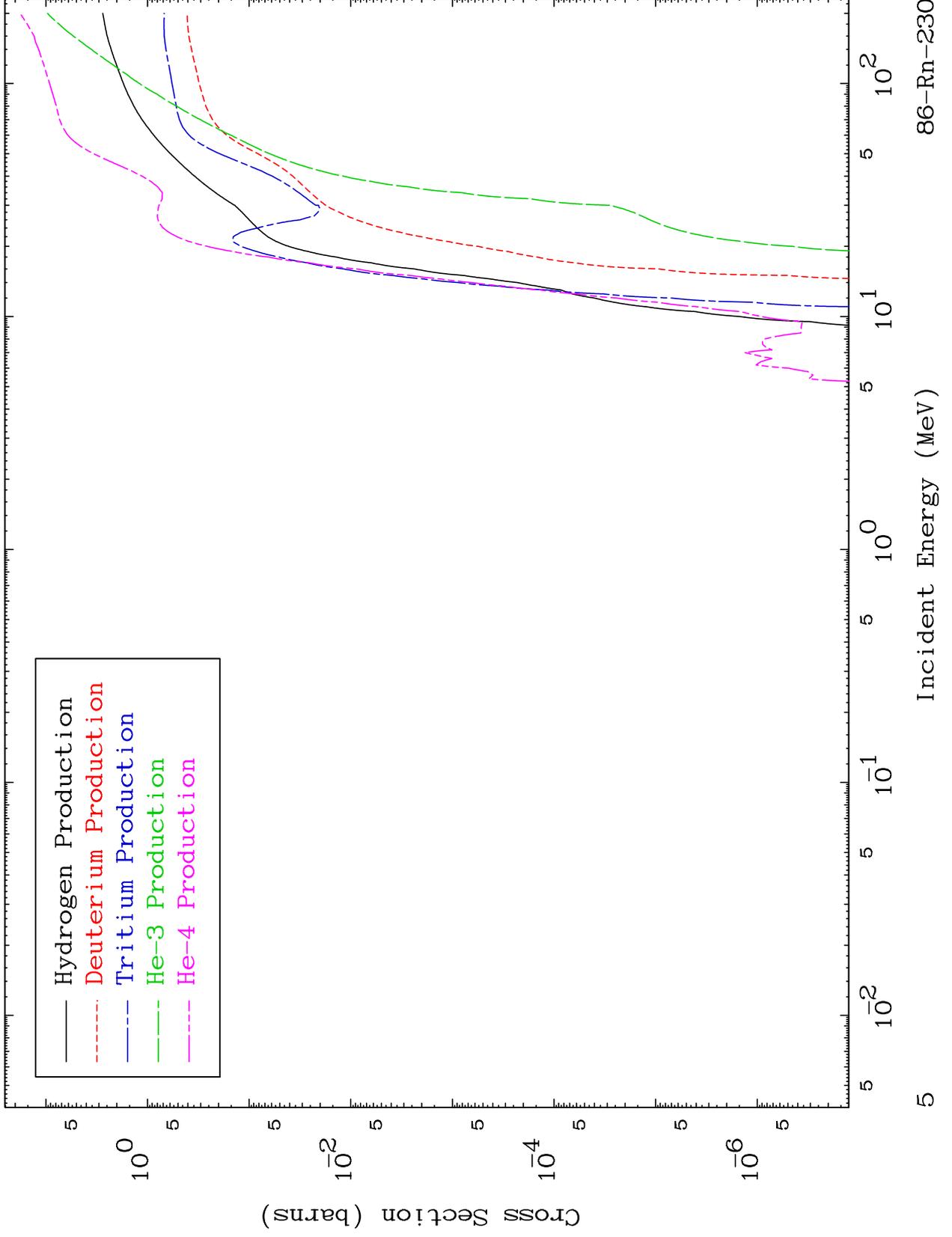
86-Rn-230



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Particle Production  
293 Kelvin Cross Sections

86-Rn-230



86-Rn-230

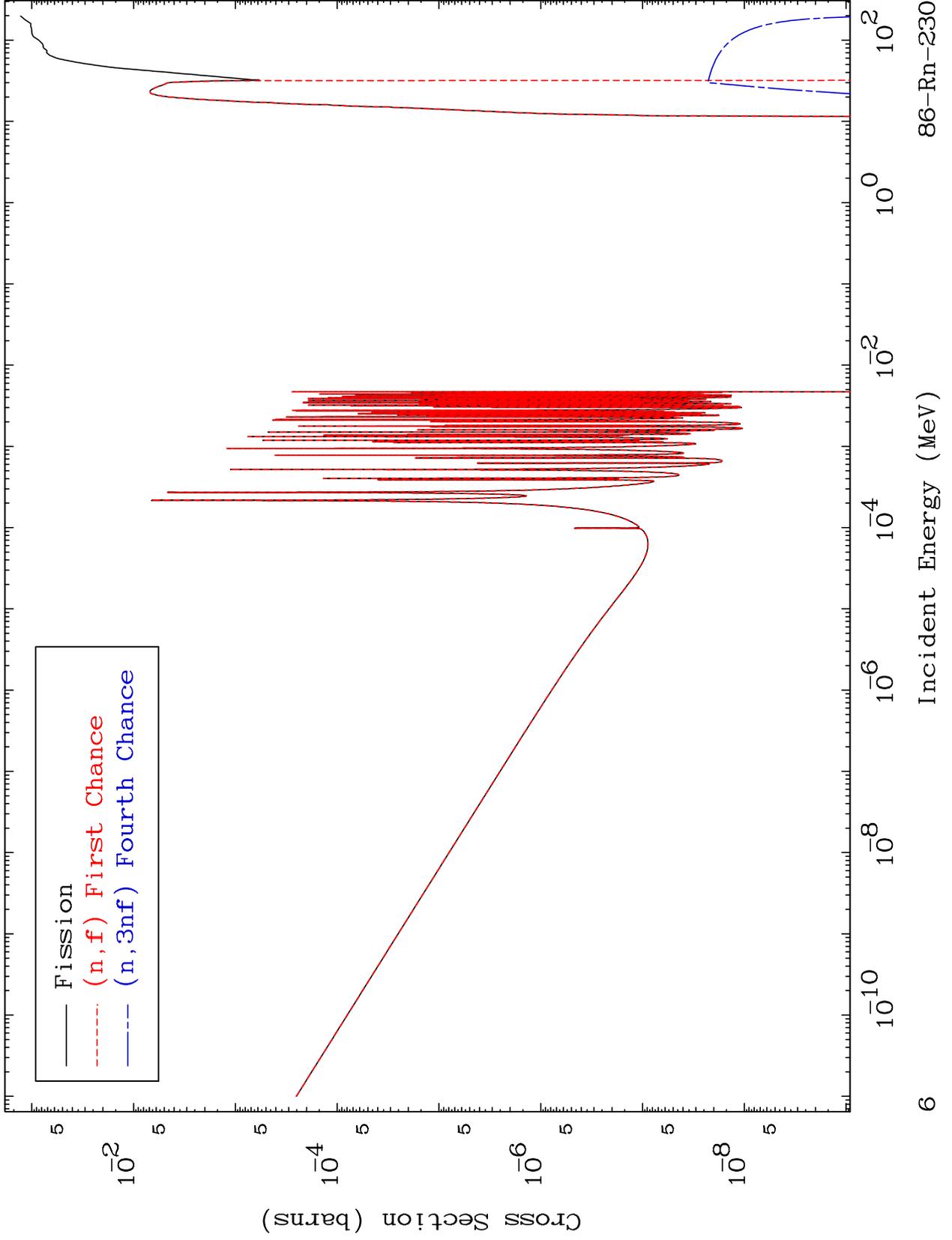
Incident Energy (MeV)

5

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Fission  
293 Kelvin Cross Sections

86-Rn-230

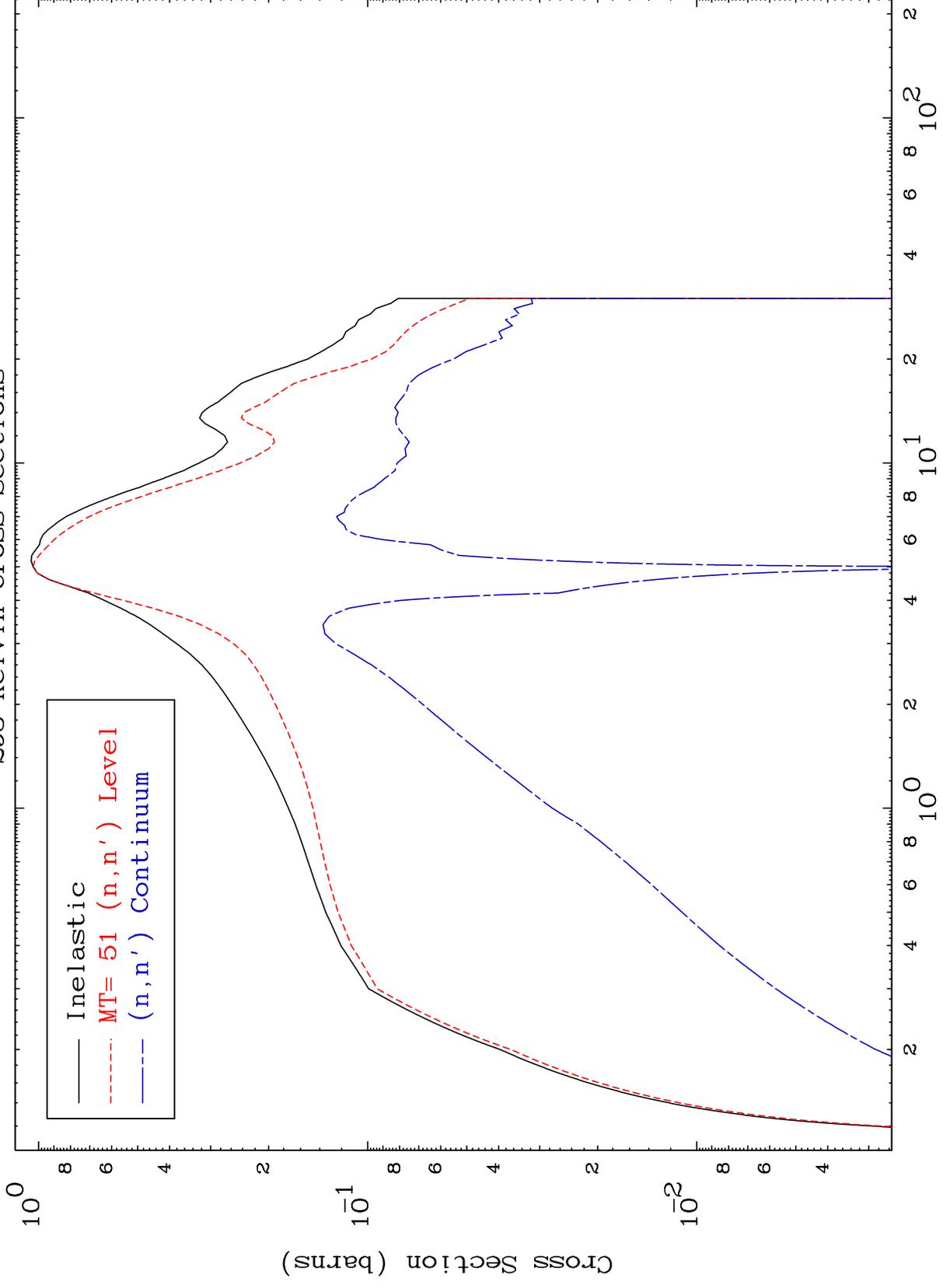


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293 Kelvin Cross Sections

(n,n') Level

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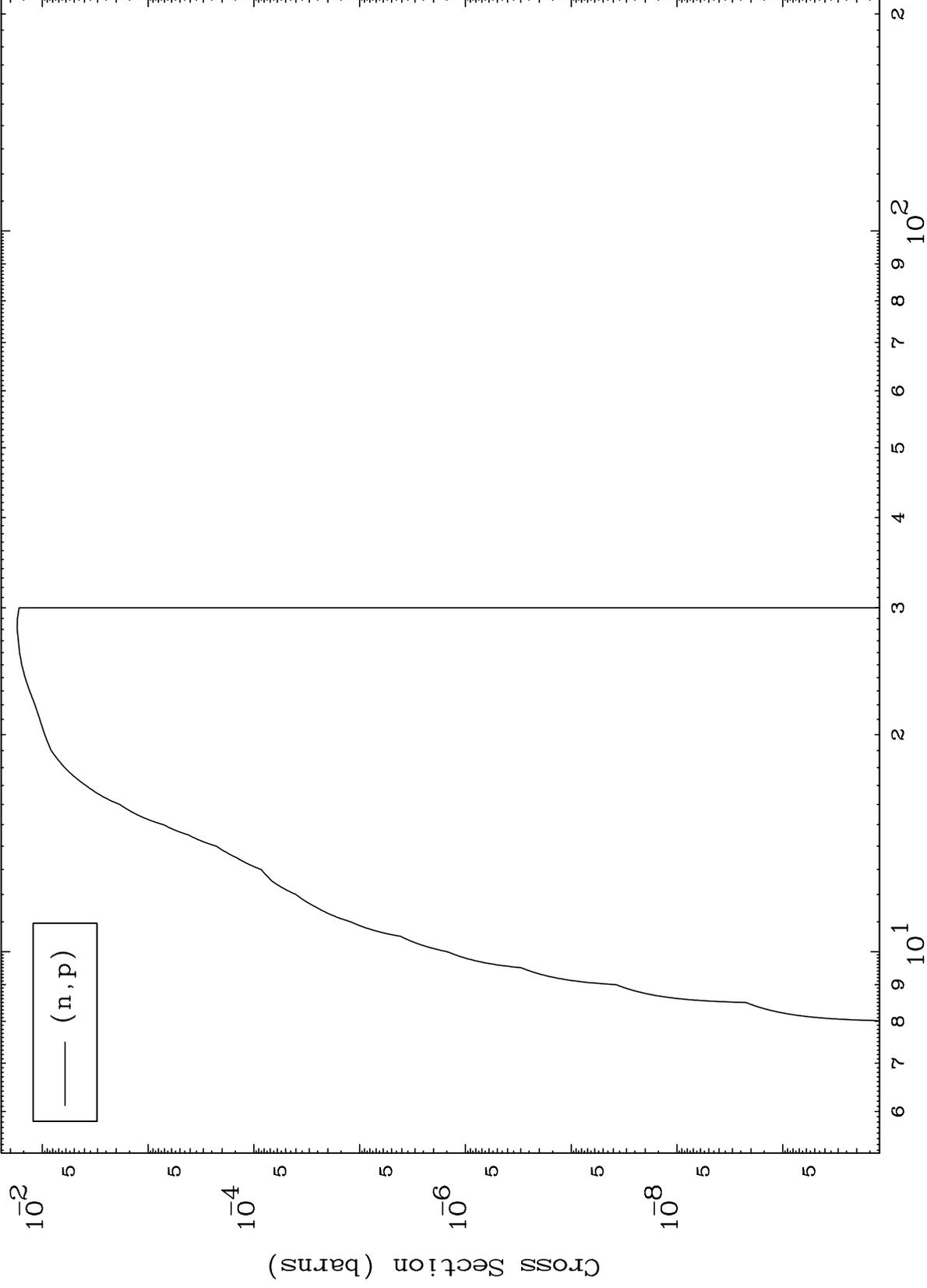


Legend:  
— Inelastic  
- - - MT= 51 (n,n') Level  
- . - (n,n') Continuum

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(n,p) Levels  
293 Kelvin Cross Sections

86-Rn-230



8

Incident Energy (MeV)

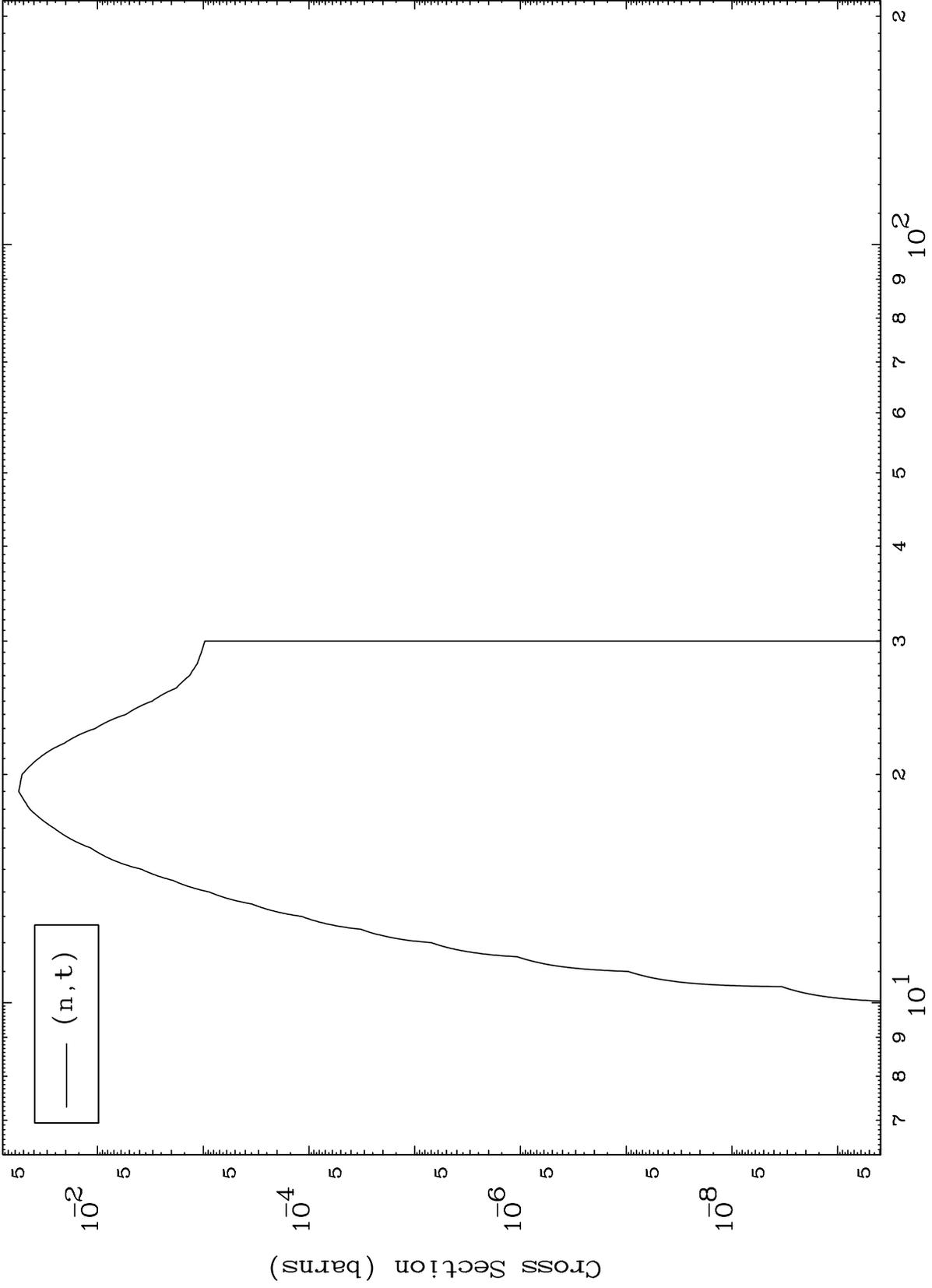
86-Rn-230



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(n,t) Levels  
293 Kelvin Cross Sections

86-Rn-230



10

Incident Energy (MeV)

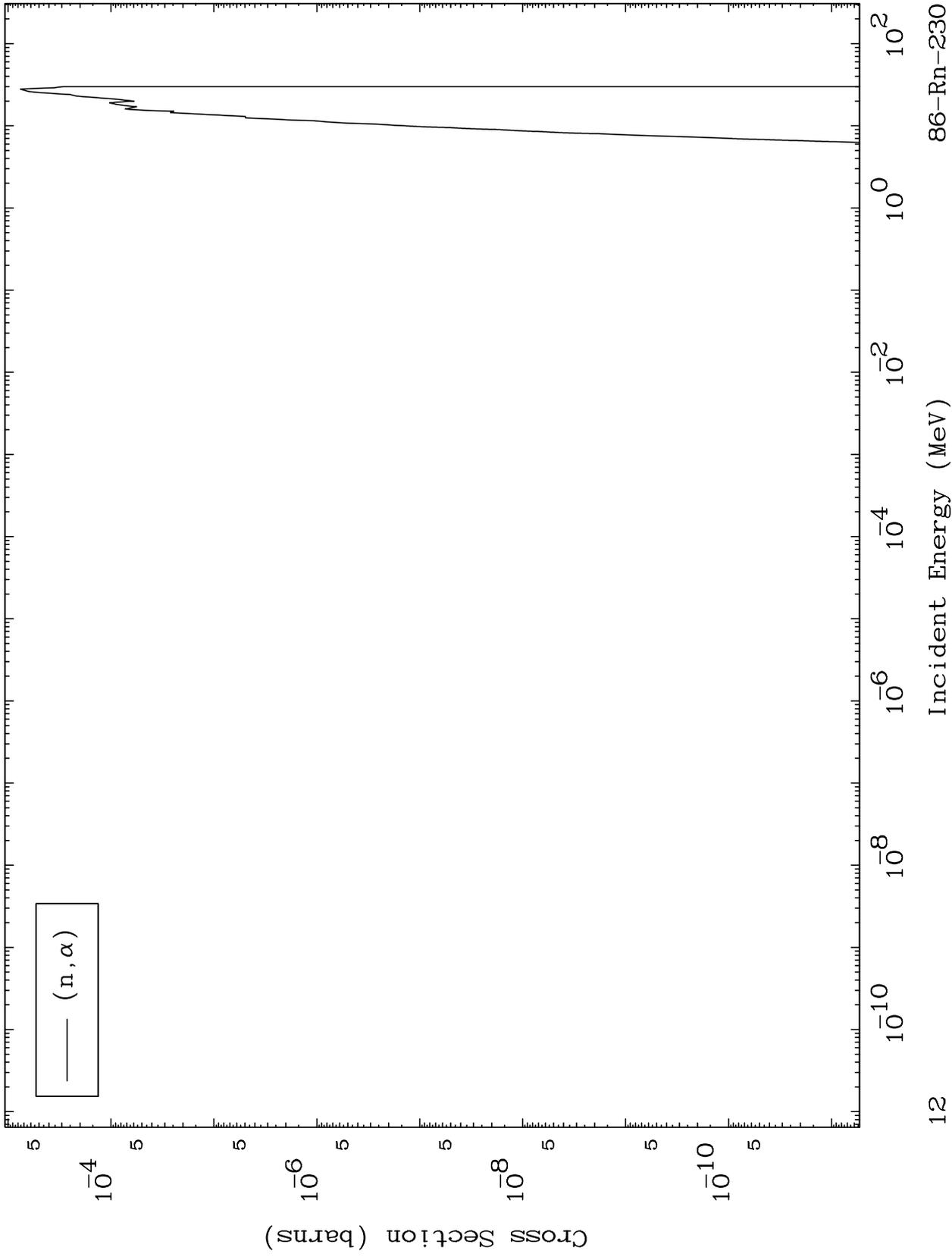
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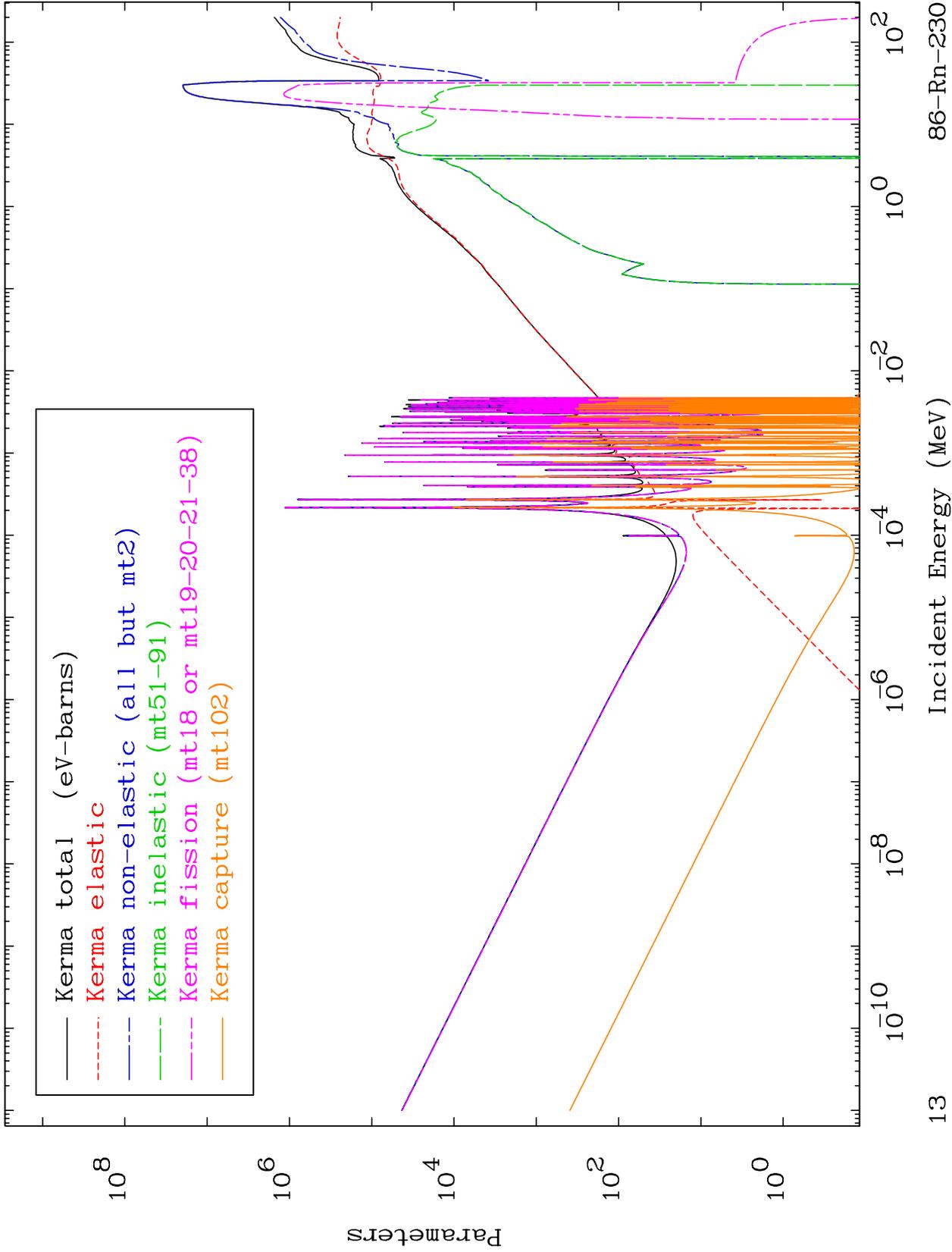


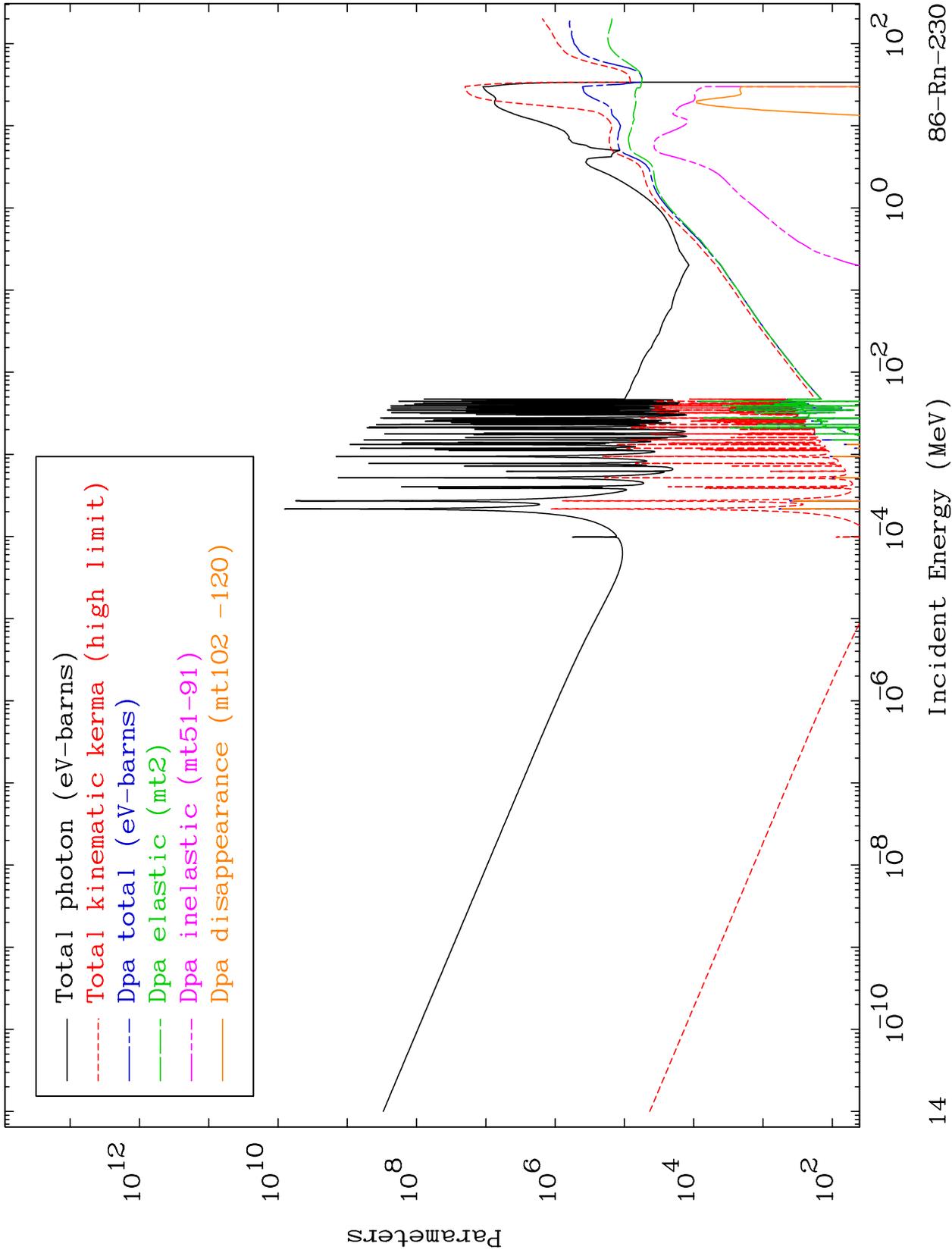
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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

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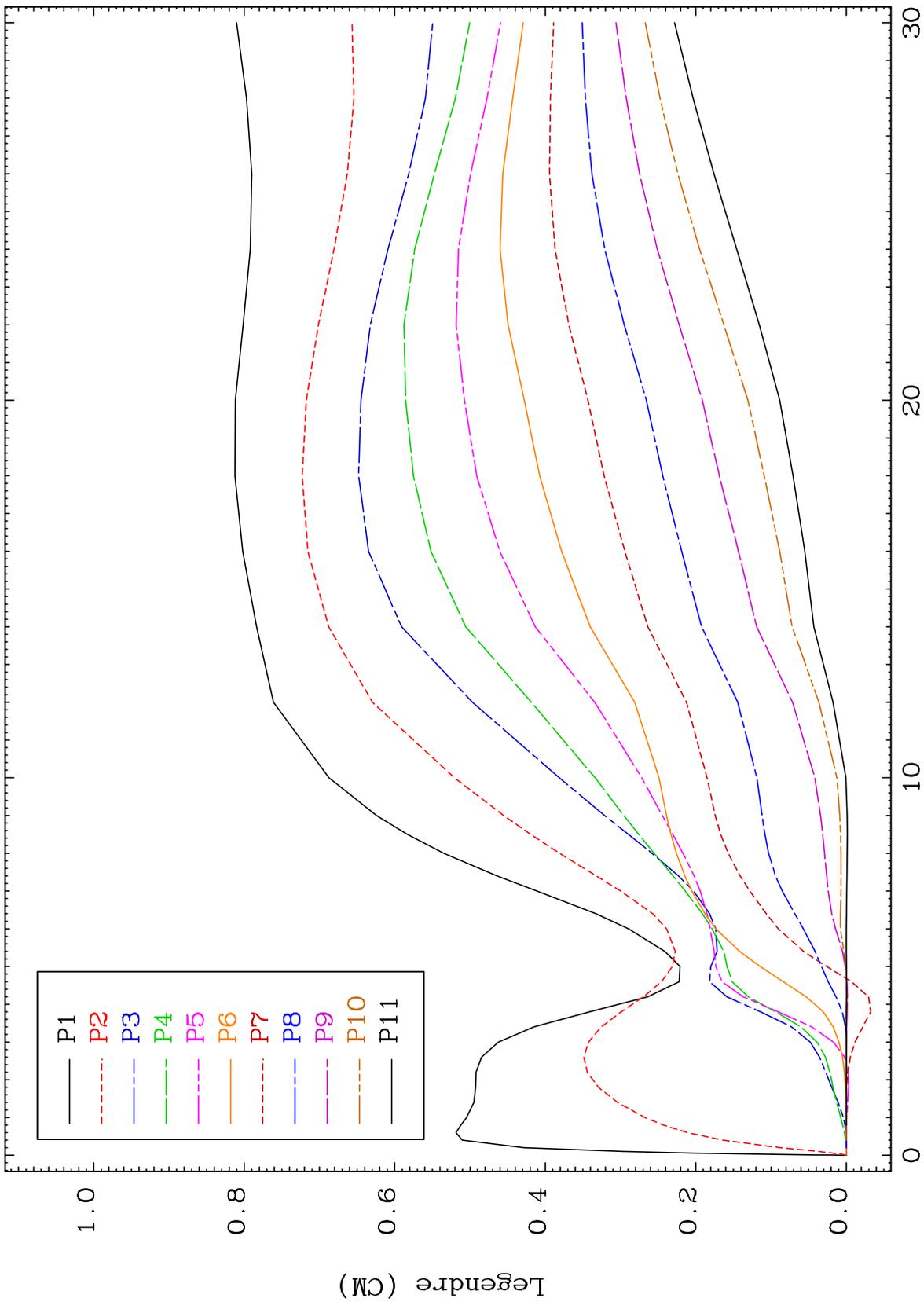




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Elastic Legendre Coefficients

86-Rn-230

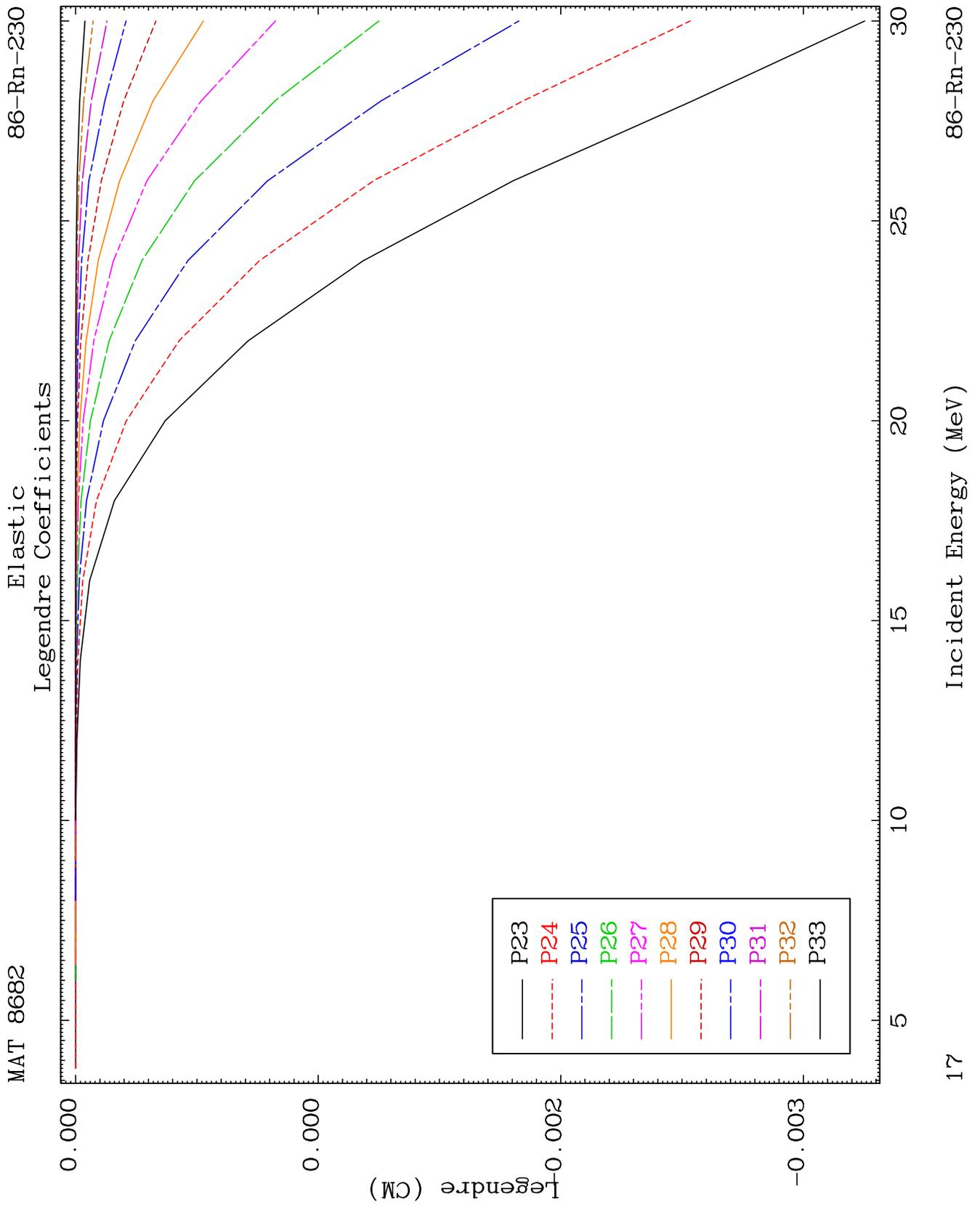


15

Incident Energy (MeV)

86-Rn-230

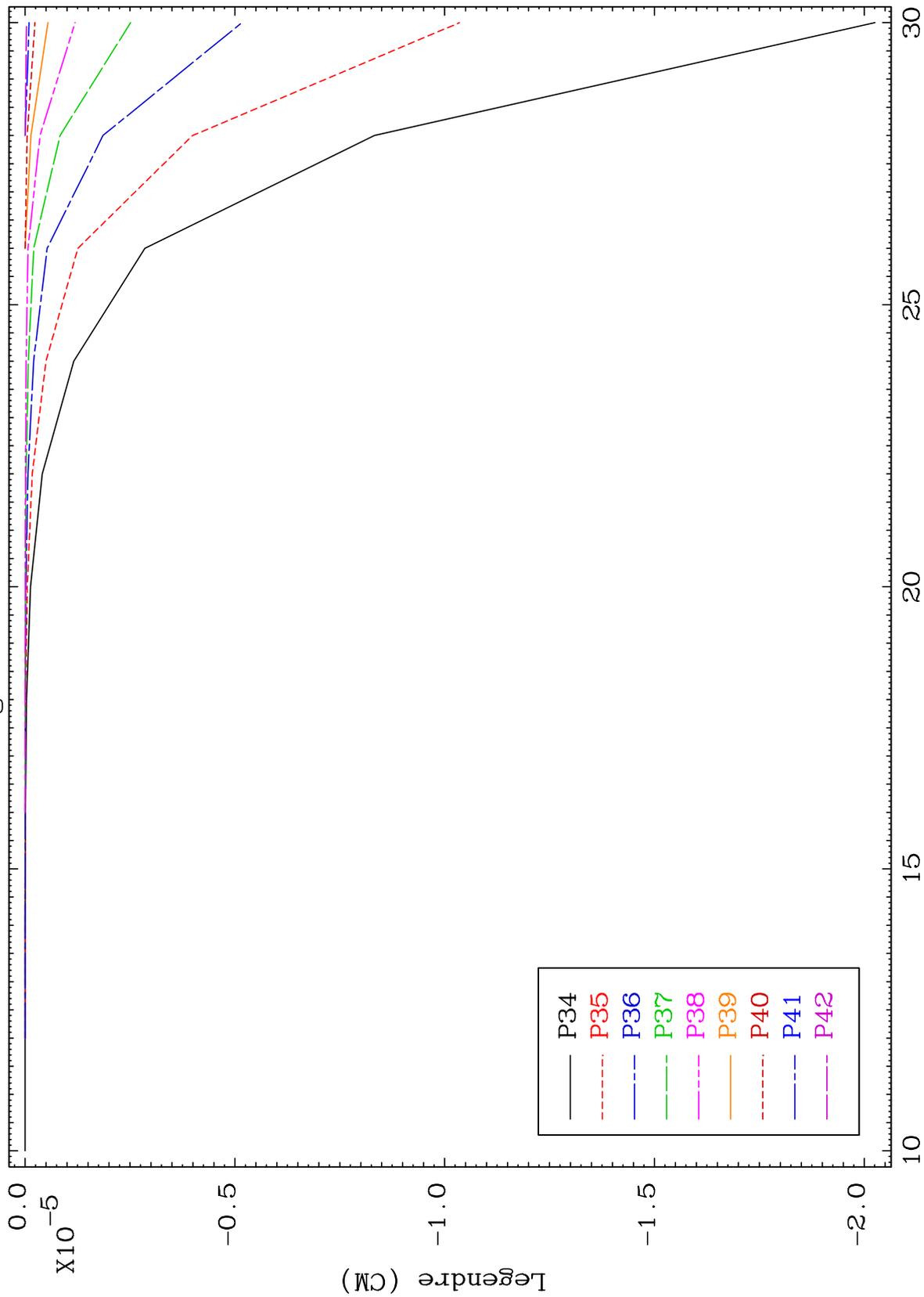




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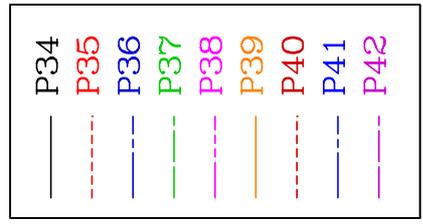
Elastic Legendre Coefficients

86-Rn-230



Incident Energy (MeV)

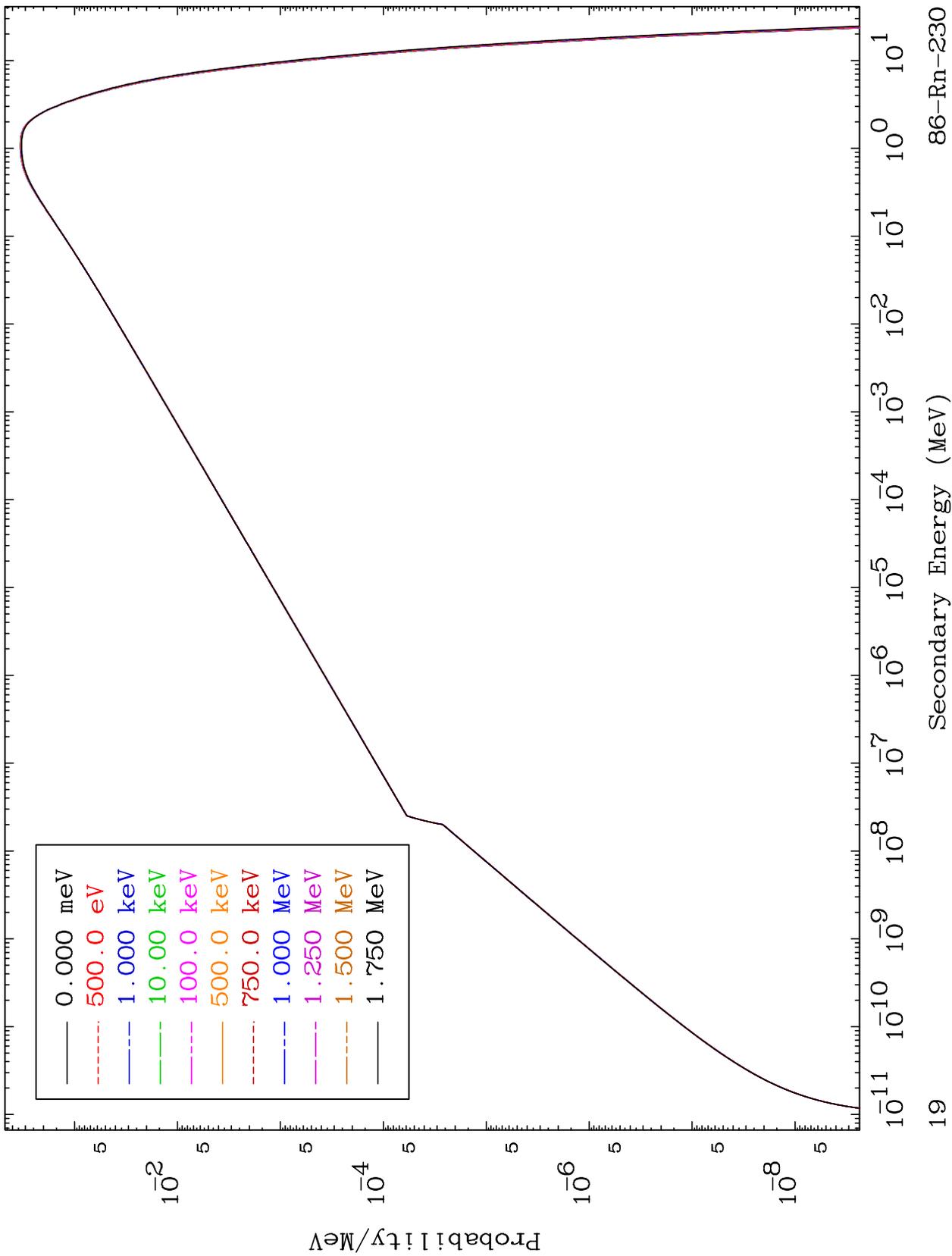
86-Rn-230



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### Fission Energy Distribution

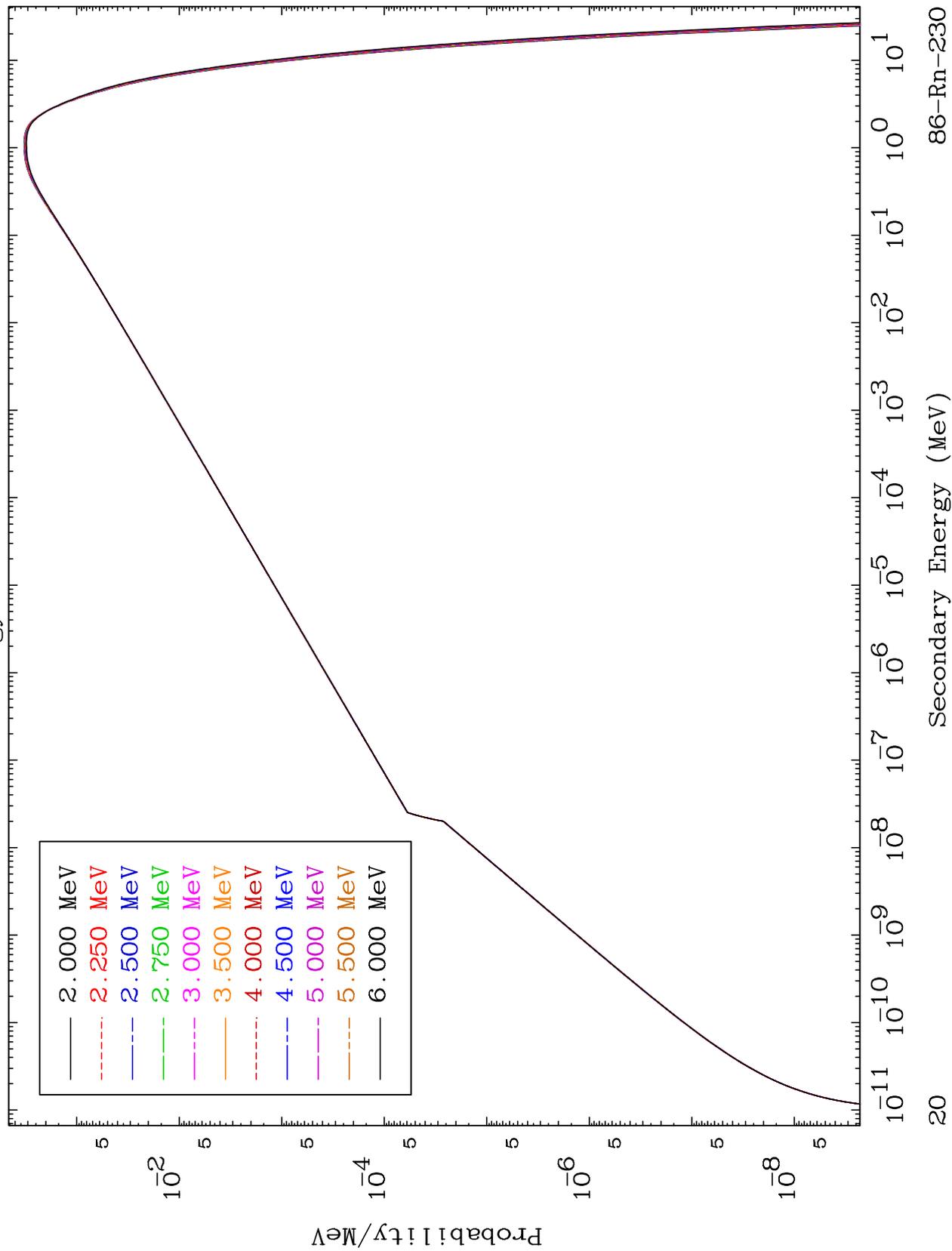
86-Rn-230



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### Fission Energy Distribution

86-Rn-230



86-Rn-230

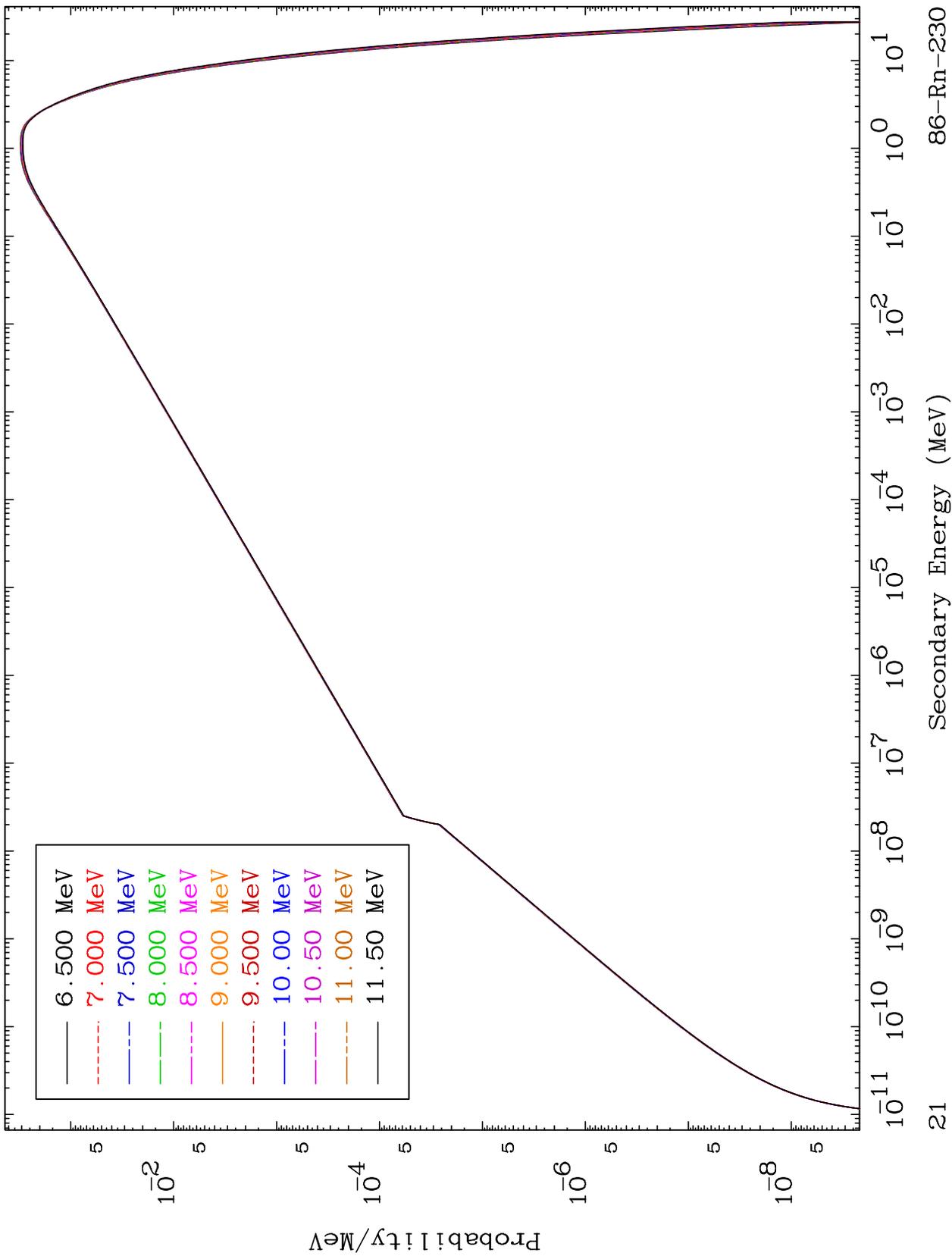
Secondary Energy (MeV)

20

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### Fission Energy Distribution

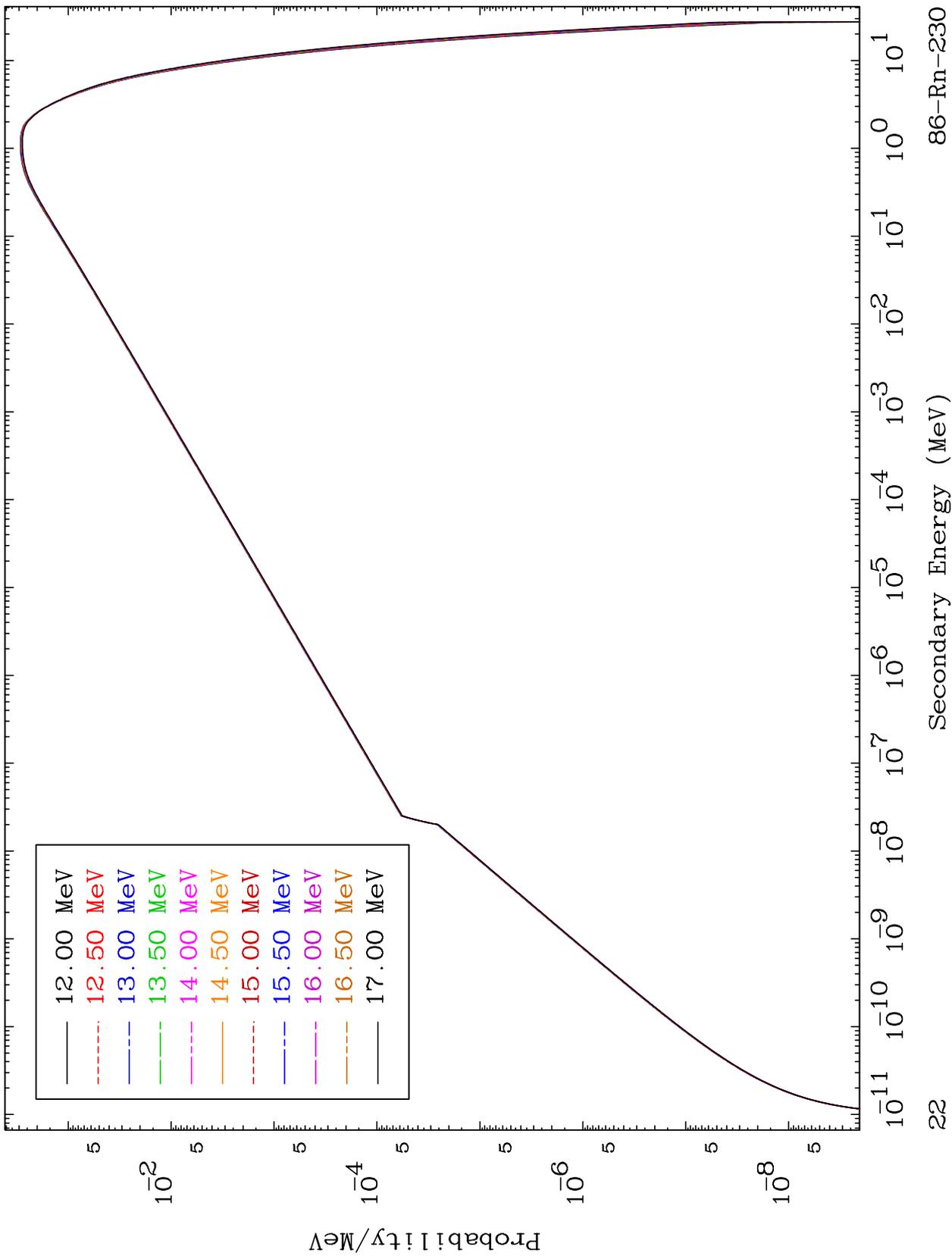
86-Rn-230



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### Fission Energy Distribution

86-Rn-230



MAT 8682

### Fission Energy Distribution

86-Rn-230

