

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)

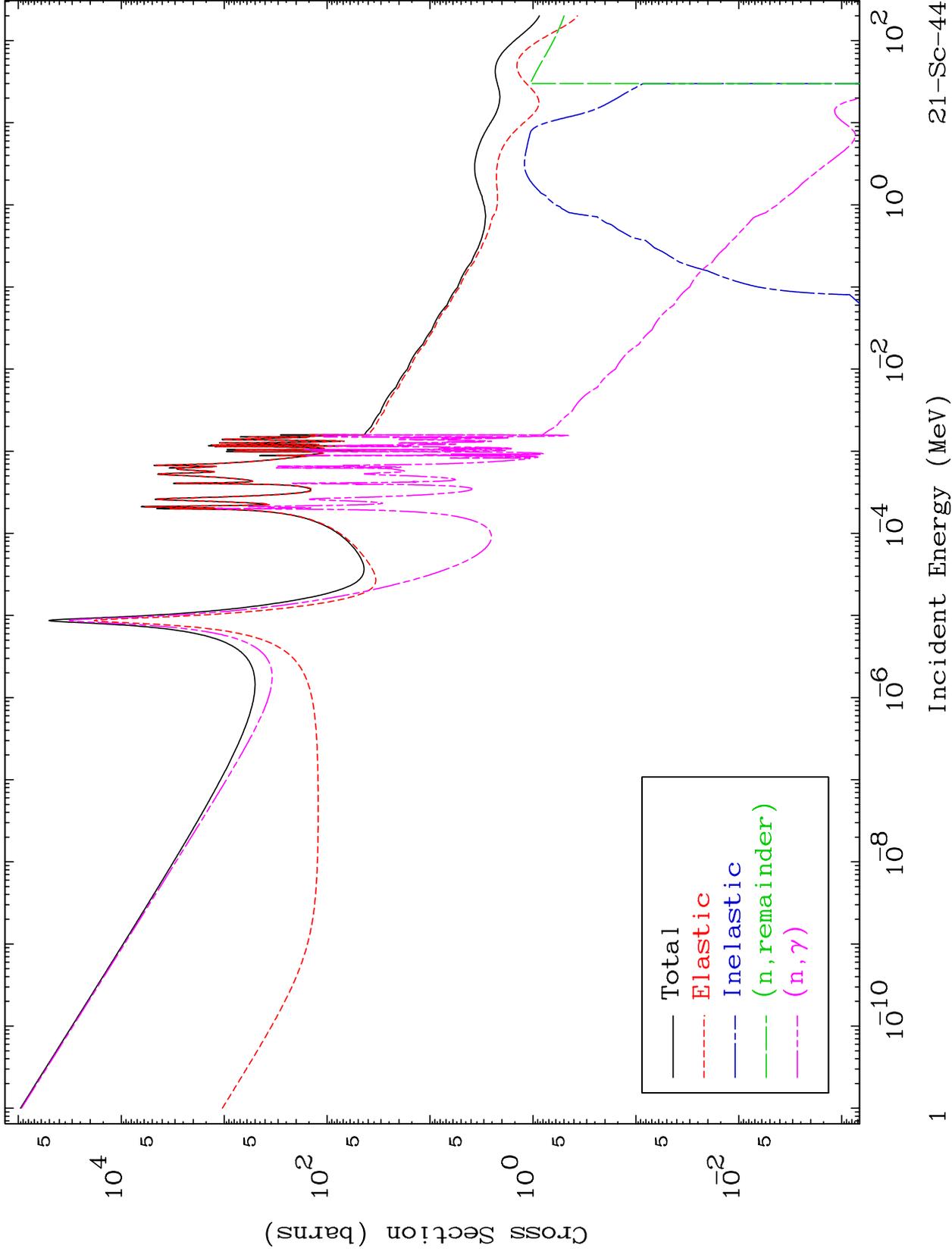
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 2123

Major  
293 Kelvin Cross Sections

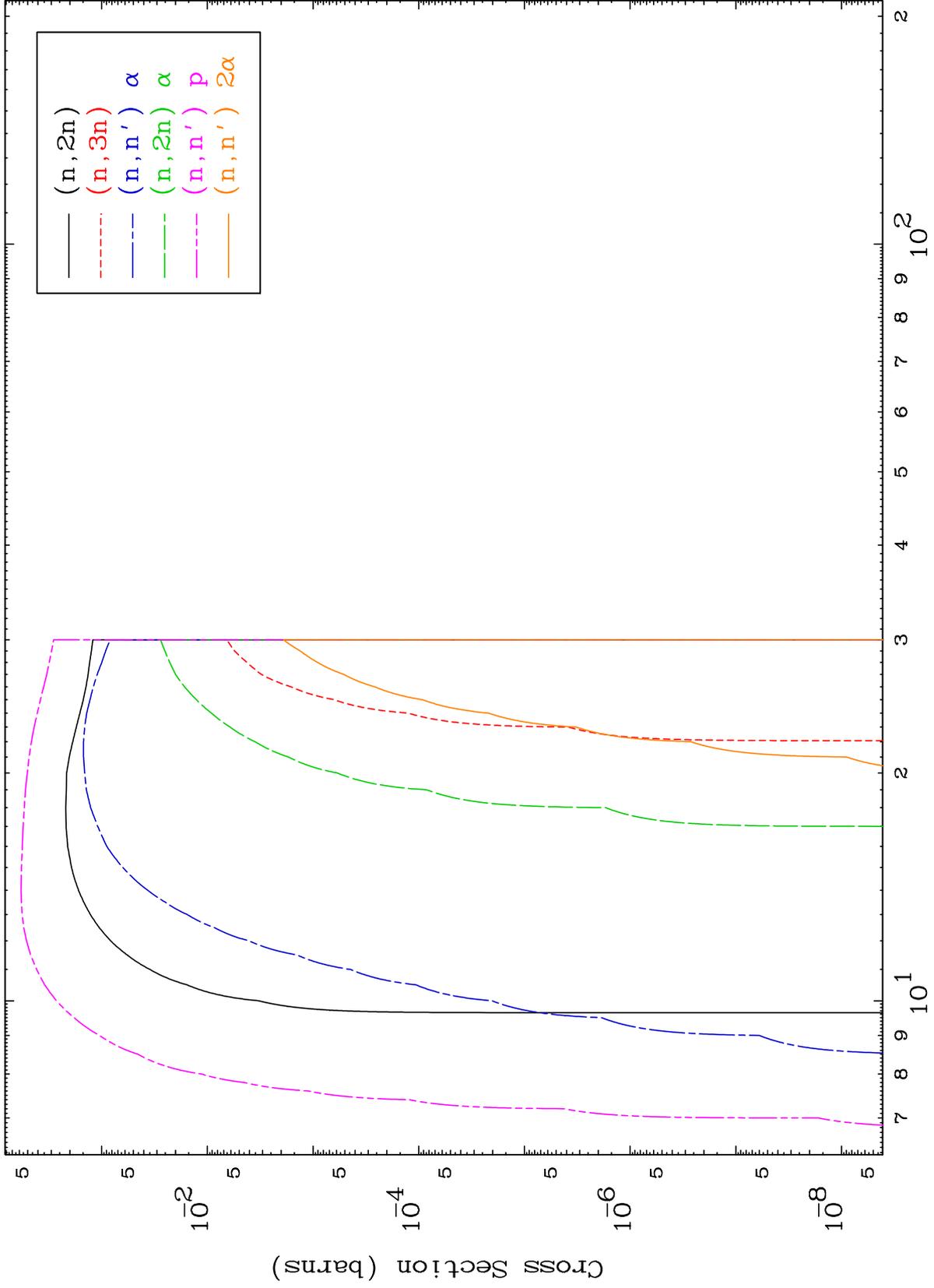
21-Sc-44



MAT 2123

Neutron Production  
293 Kelvin Cross Sections

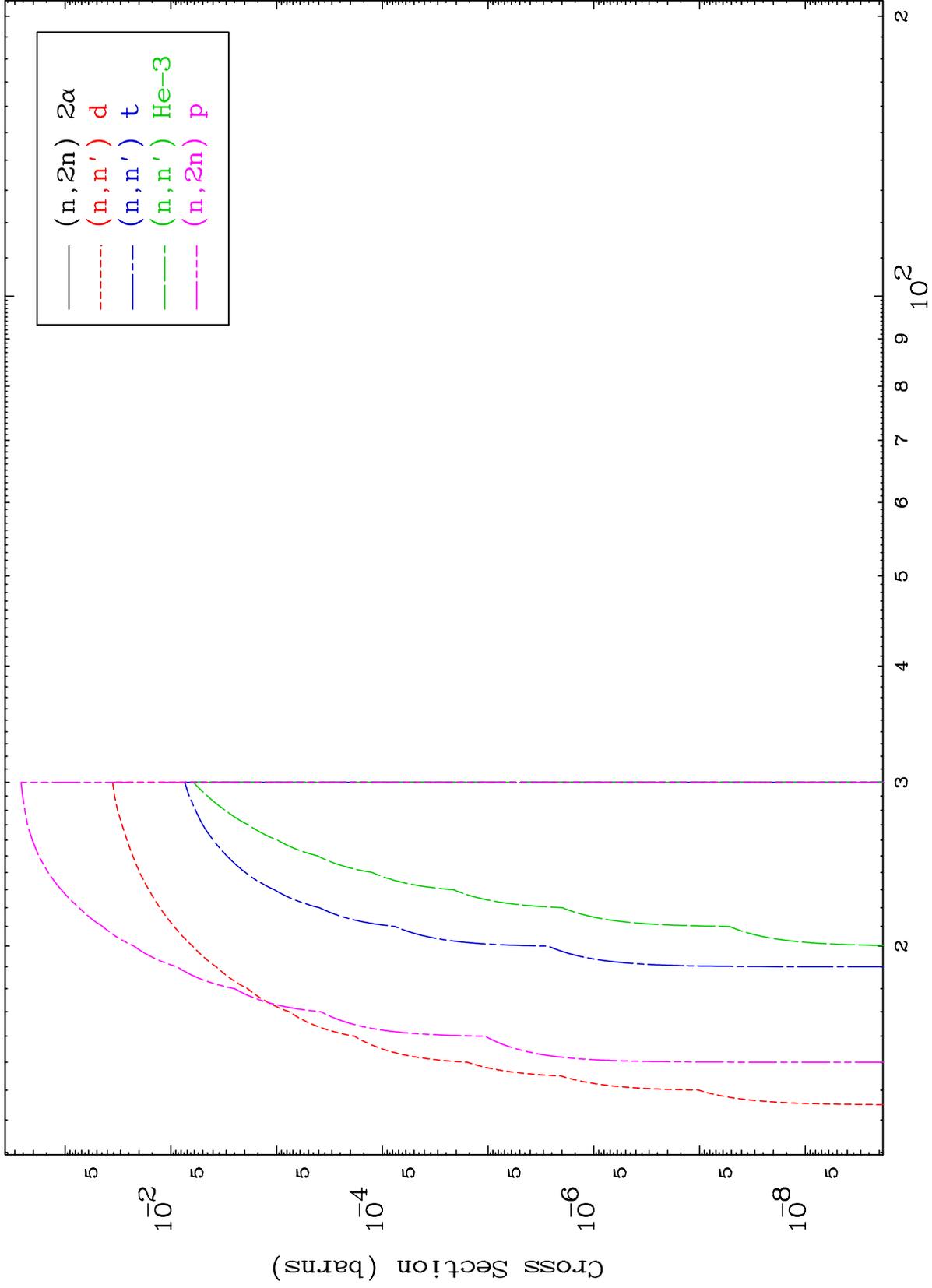
21-Sc-44



2

Incident Energy (MeV)

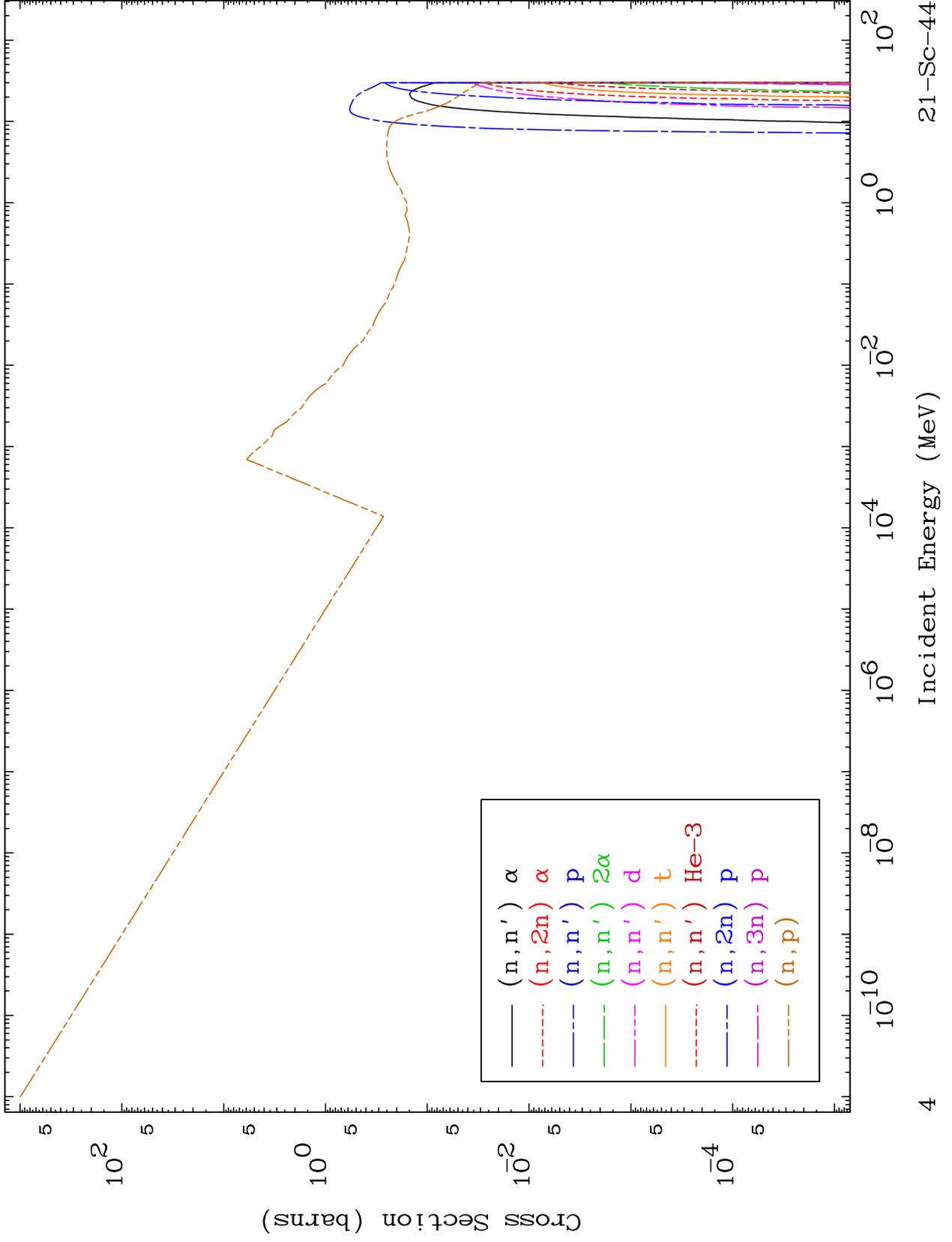
21-Sc-44



MAT 2123

Charged Particle  
293 Kelvin Cross Sections

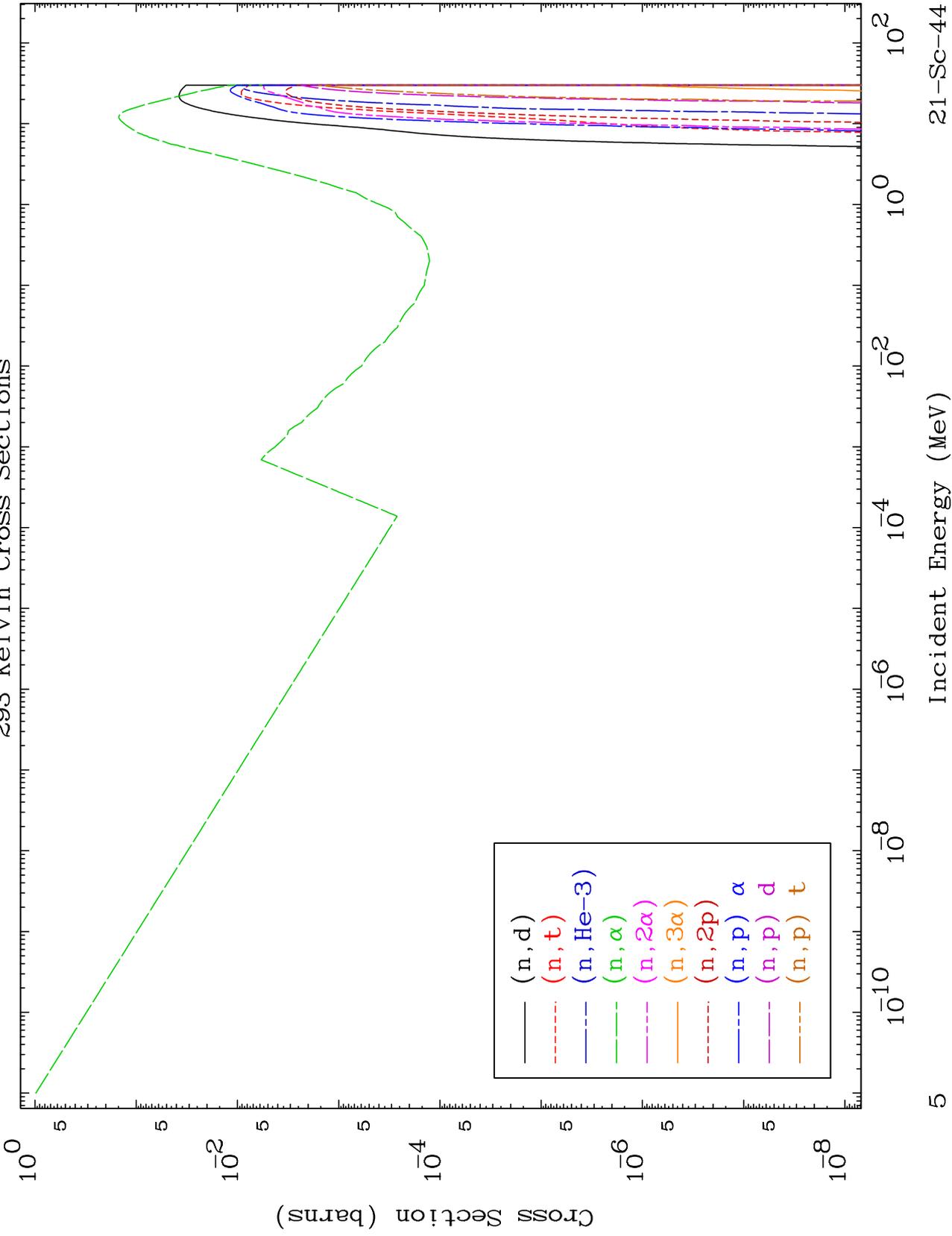
21-Sc-44



MAT 2123

Charged Particle  
293 Kelvin Cross Sections

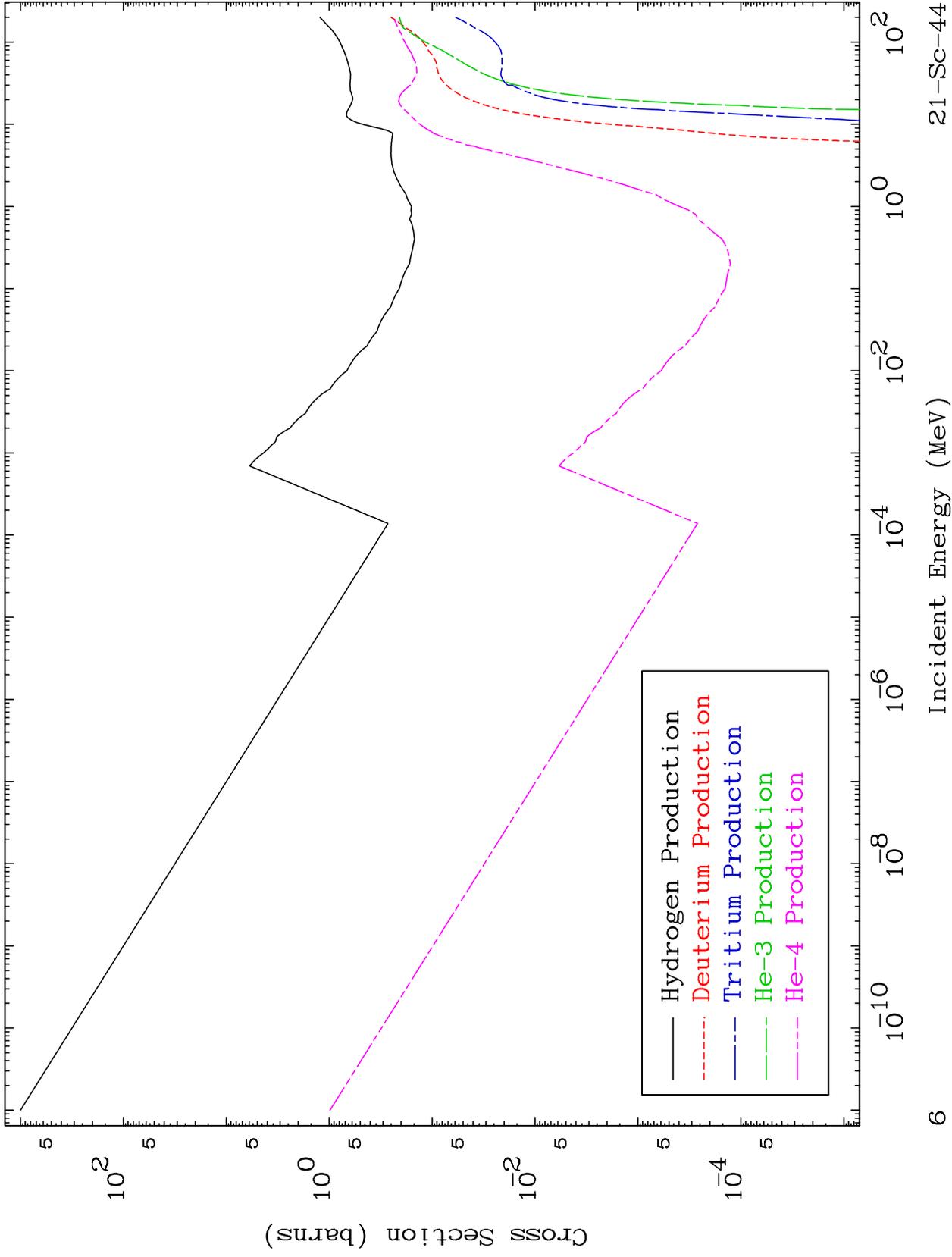
21-Sc-44



MAT 2123

Particle Production  
293 Kelvin Cross Sections

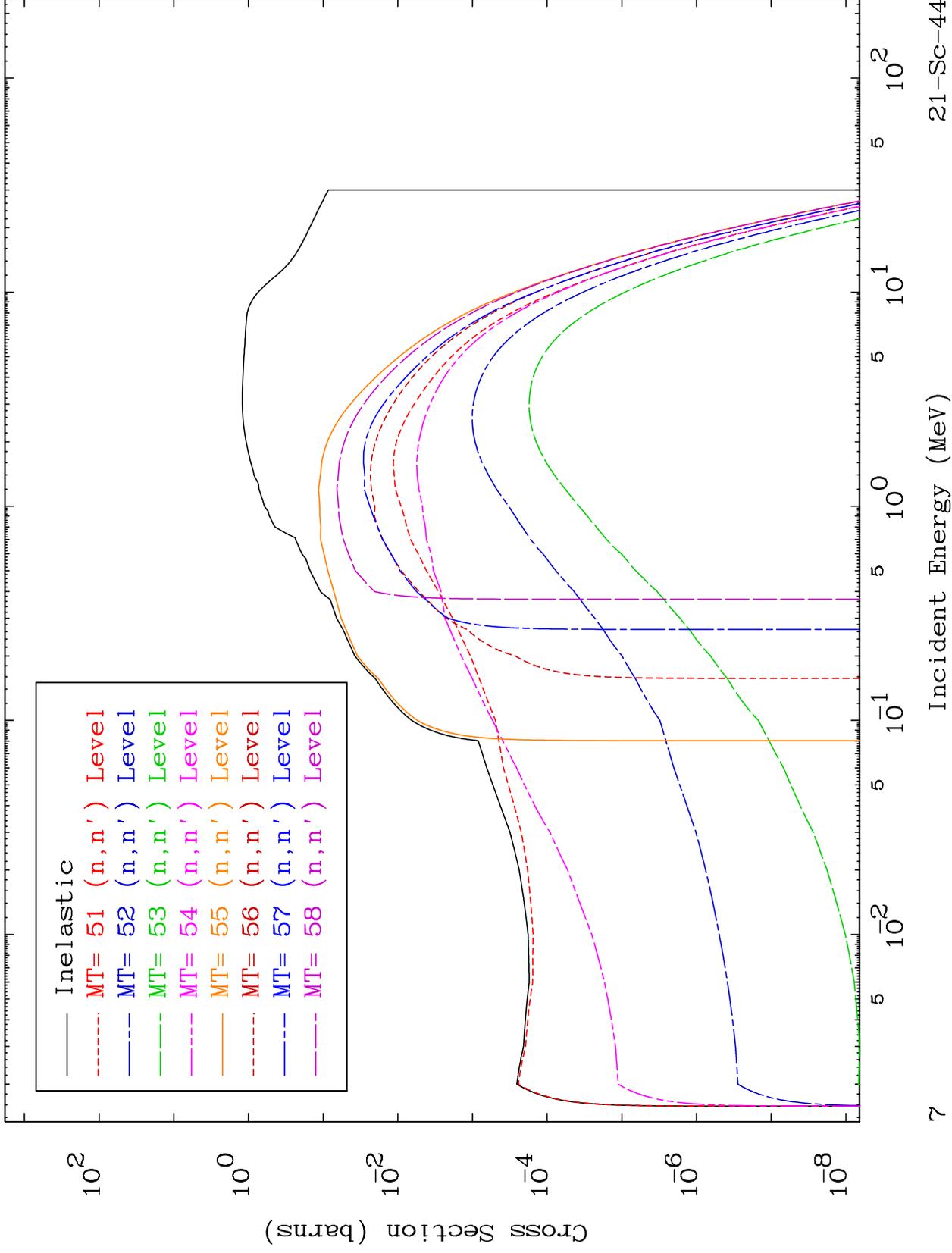
21-Sc-44



MAT 2123

(n,n') Level  
293 Kelvin Cross Sections

21-Sc-44

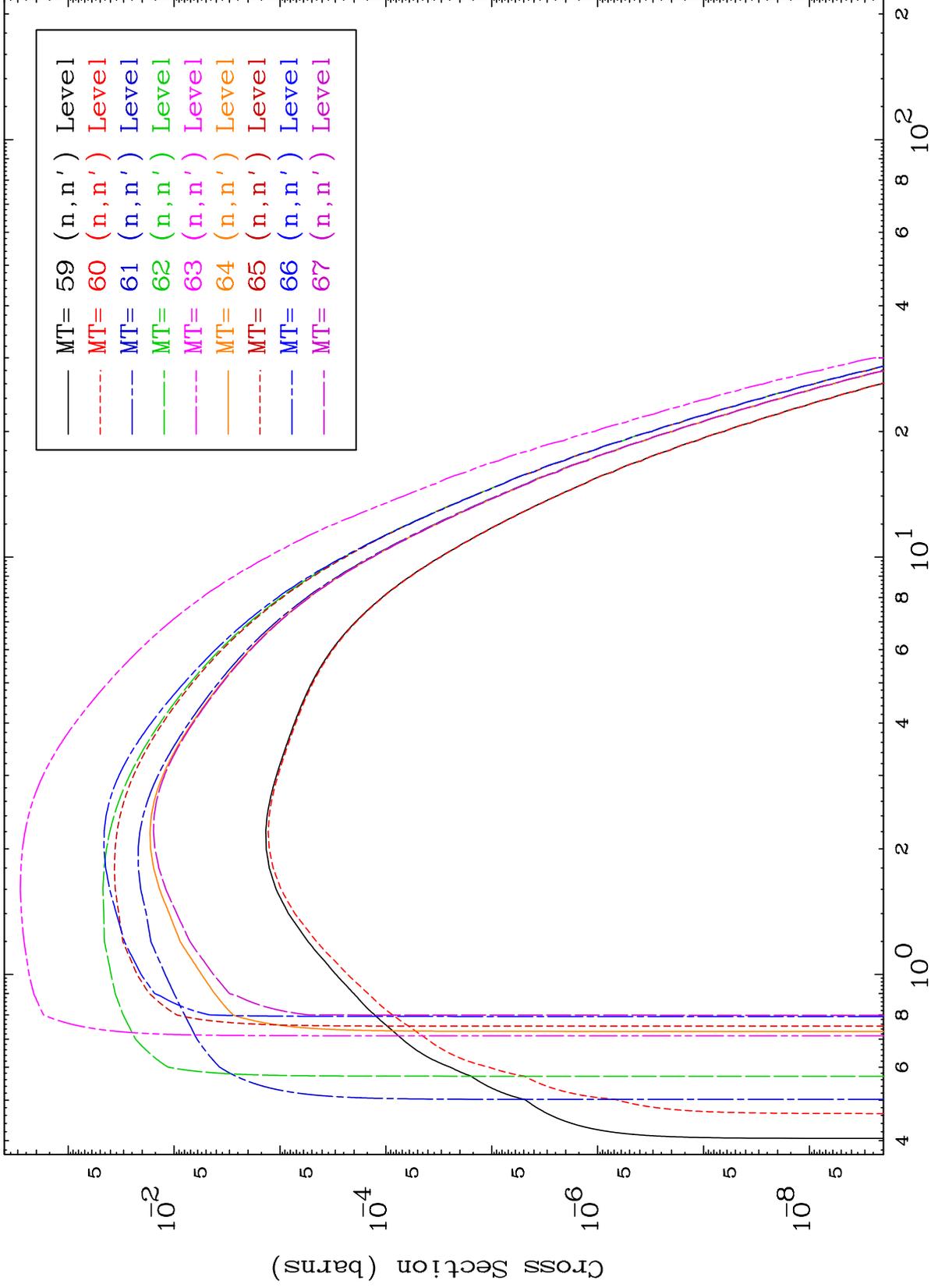


MAT 2123

(n,n') Level

21-Sc-44

293 Kelvin Cross Sections



Incident Energy (MeV)

21-Sc-44

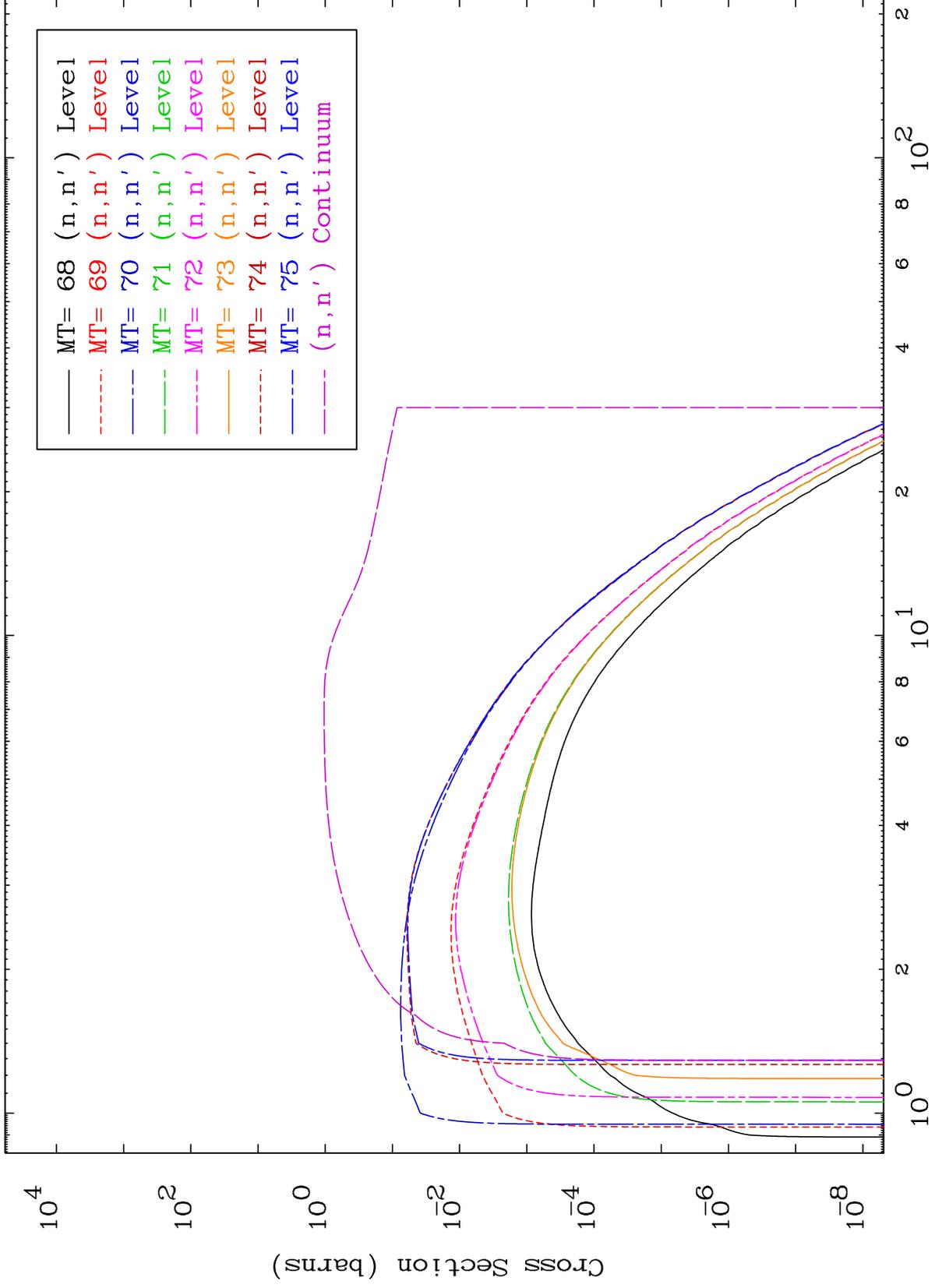
8

MAT 2123

(n,n') Level

21-Sc-44

293 Kelvin Cross Sections



9

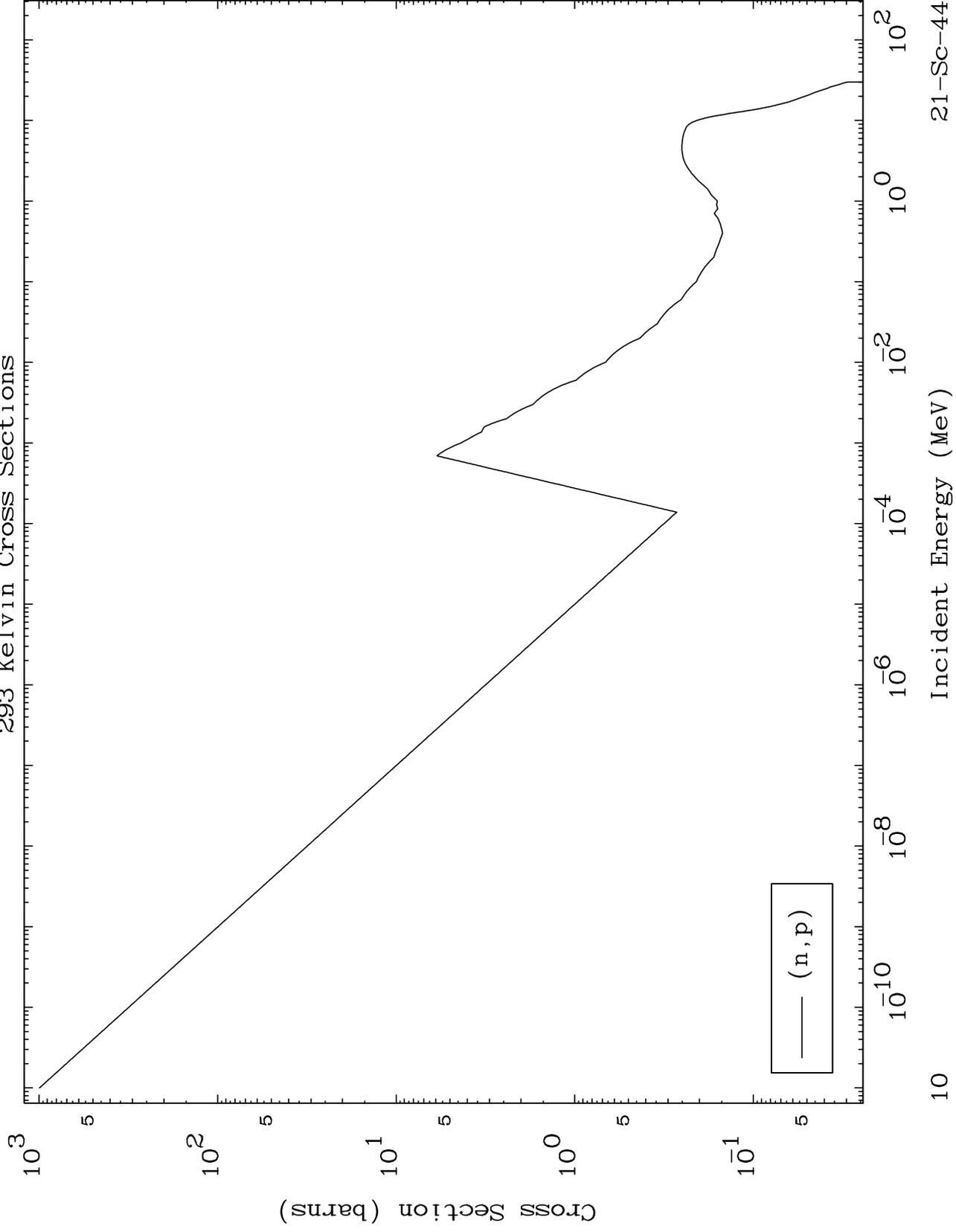
Incident Energy (MeV)

21-Sc-44

MAT 2123

(n,p) Levels  
293 Kelvin Cross Sections

21-Sc-44



(n,p)

10

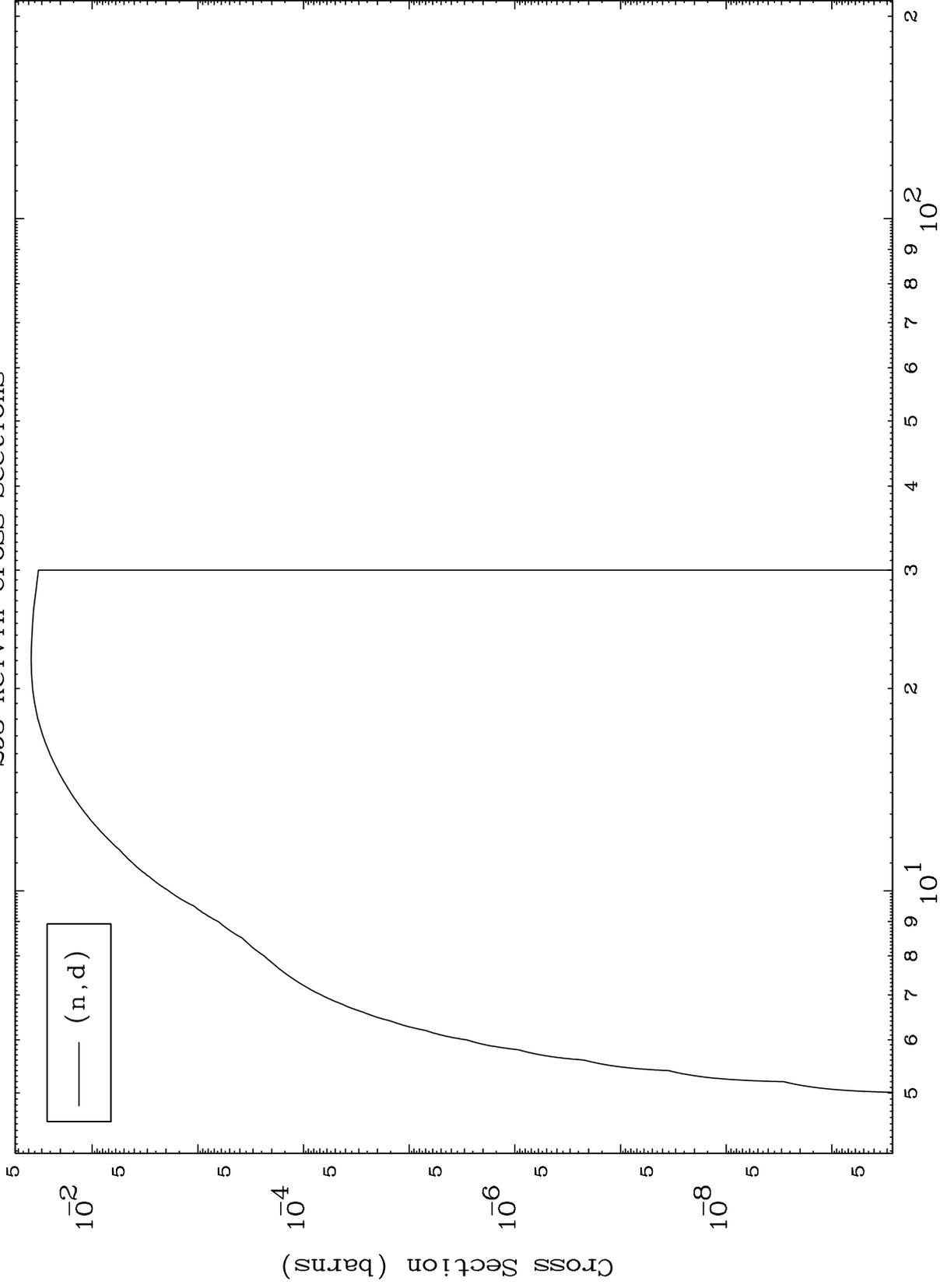
Incident Energy (MeV)

21-Sc-44

MAT 2123

(n,d) Levels  
293 Kelvin Cross Sections

21-Sc-44



11

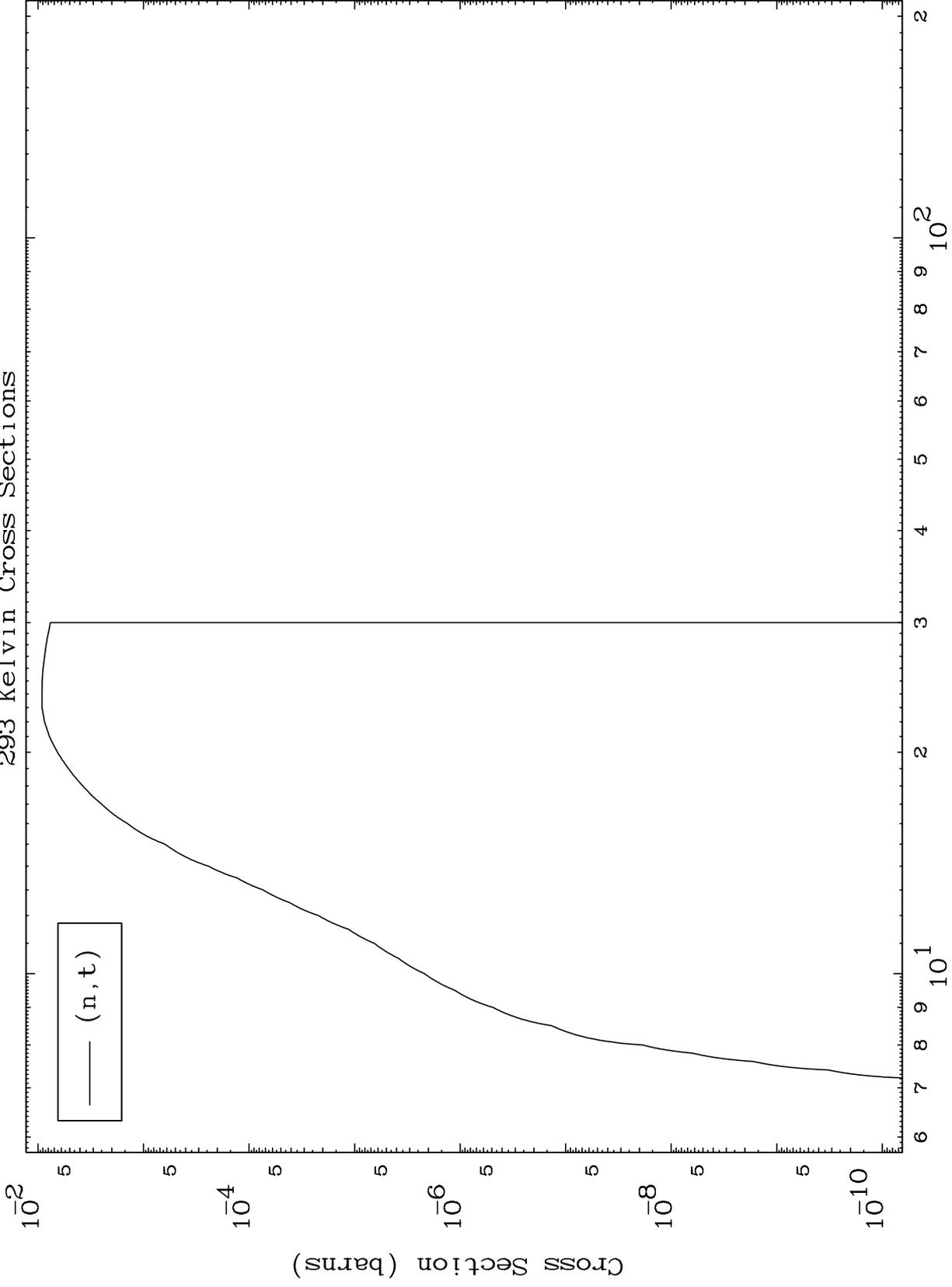
Incident Energy (MeV)

21-Sc-44

MAT 2123

(n,t) Levels  
293 Kelvin Cross Sections

21-Sc-44



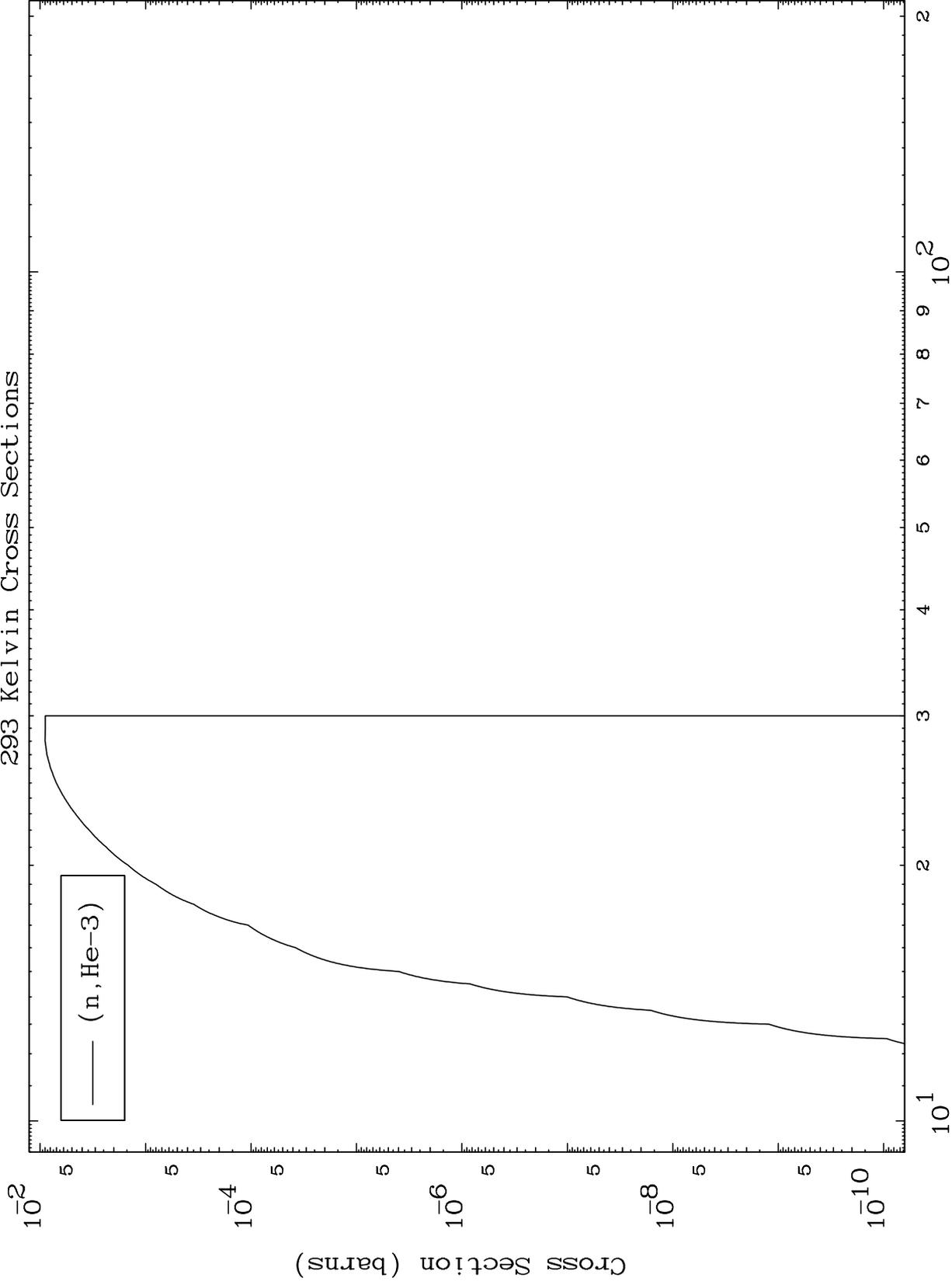
12

21-Sc-44

MAT 2123

(n,He3) Levels  
293 Kelvin Cross Sections

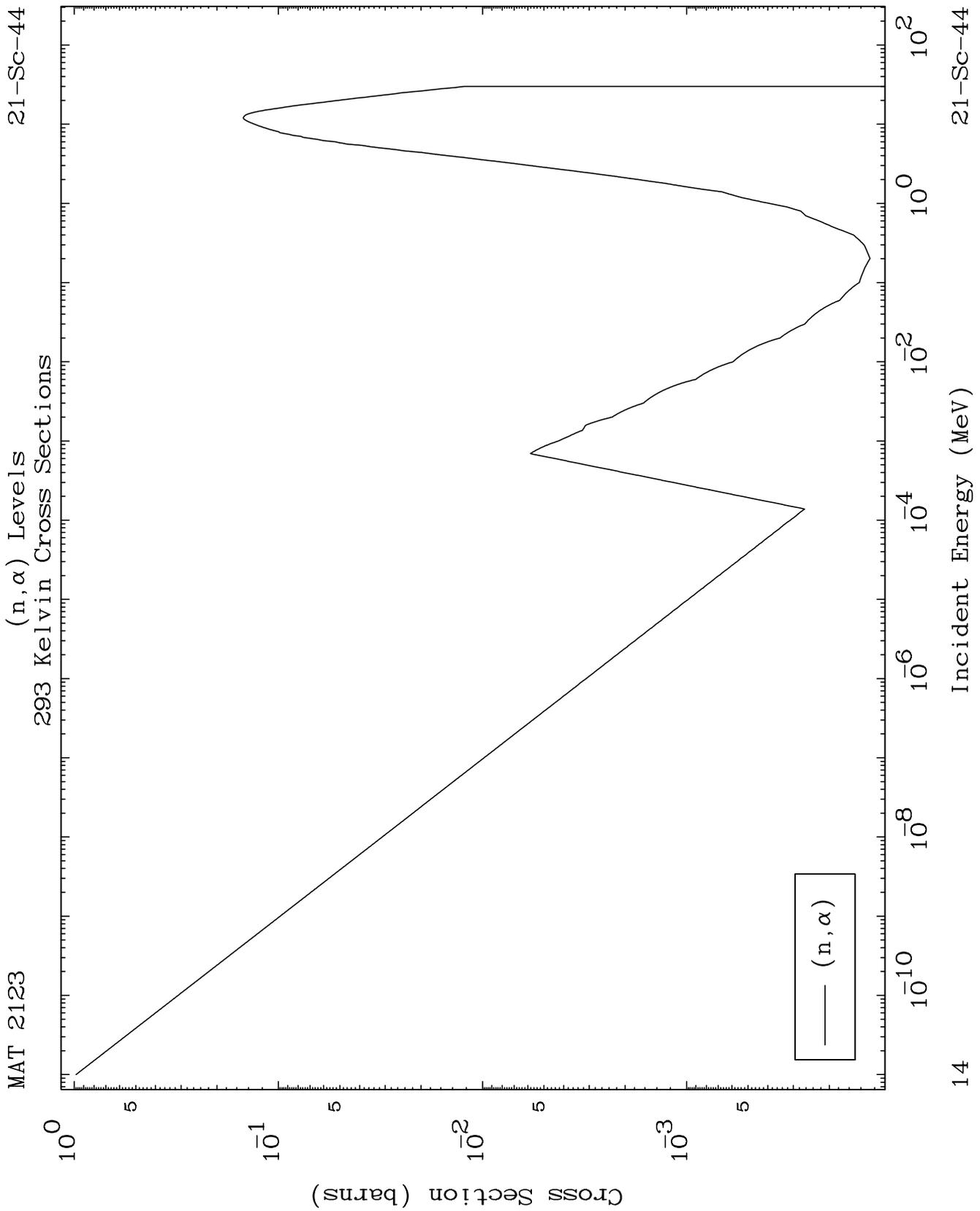
21-Sc-44

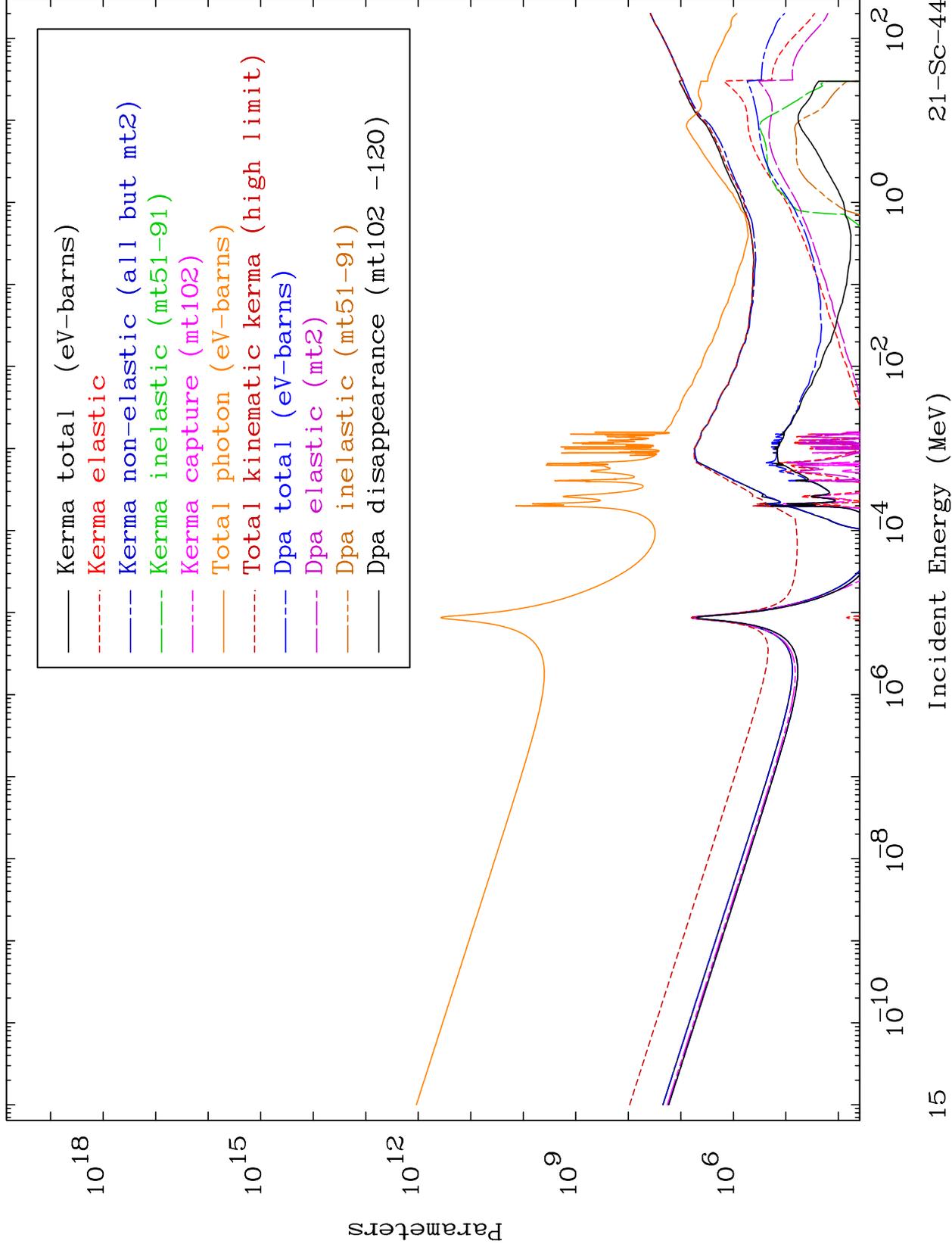


13

Incident Energy (MeV)

21-Sc-44



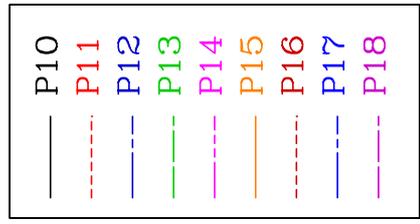




MAT 2123

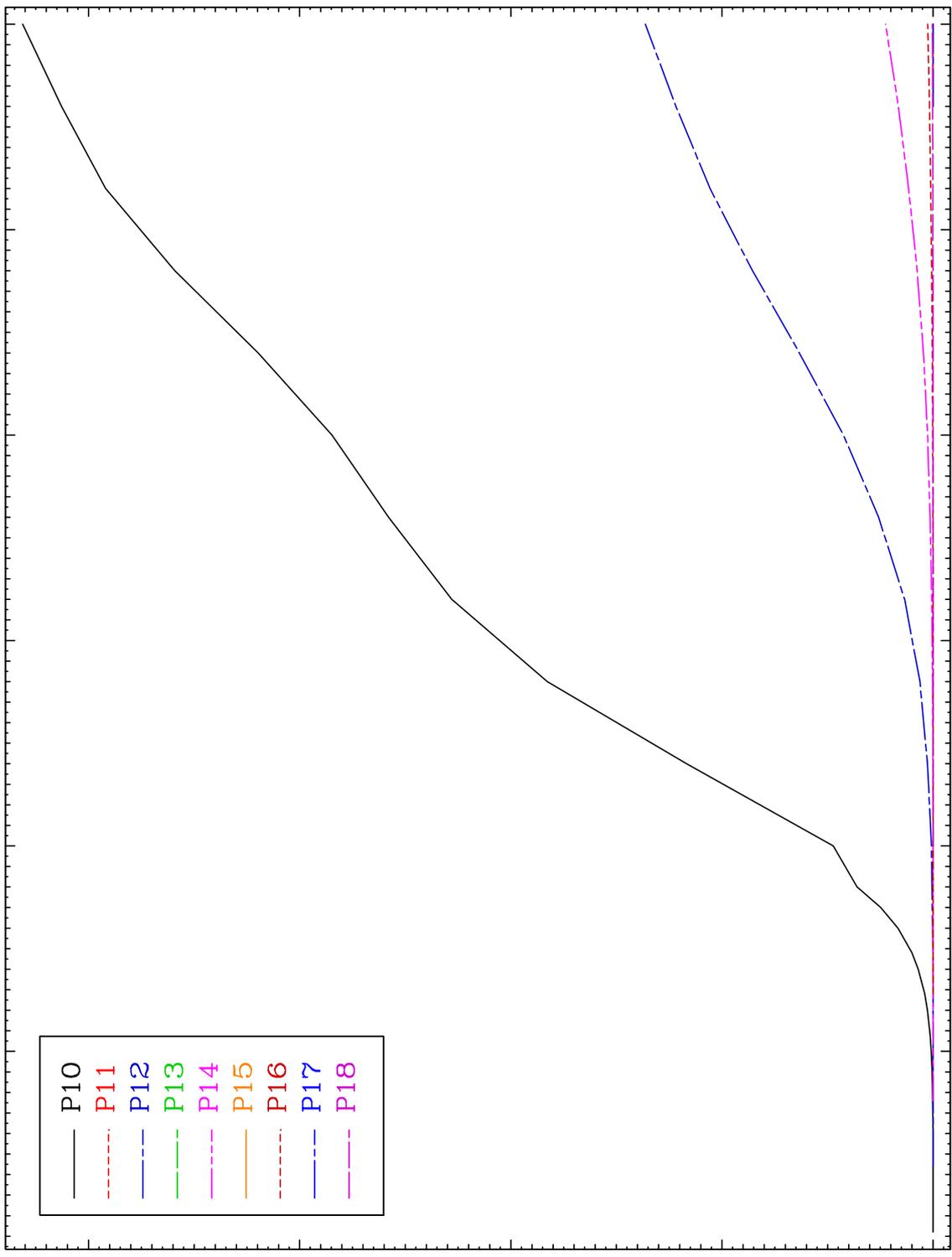
Elastic Legendre Coefficients

21-Sc-44



$\times 10^{-4}$

Legendre (CM)



17

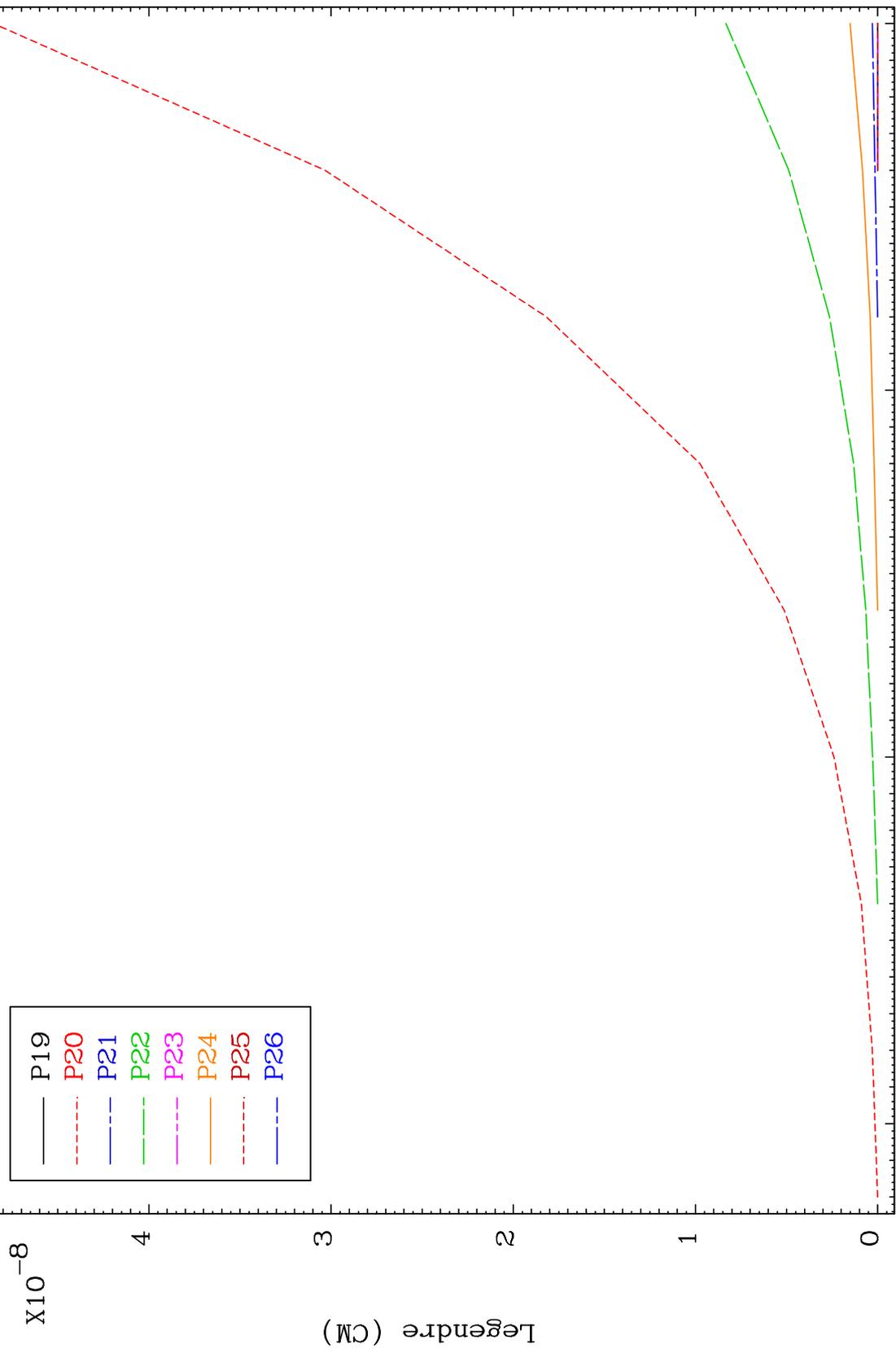
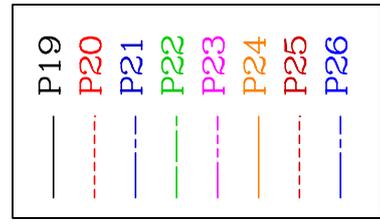
21-Sc-44

Incident Energy (MeV)

MAT 2123

Elastic Legendre Coefficients

21-Sc-44



18

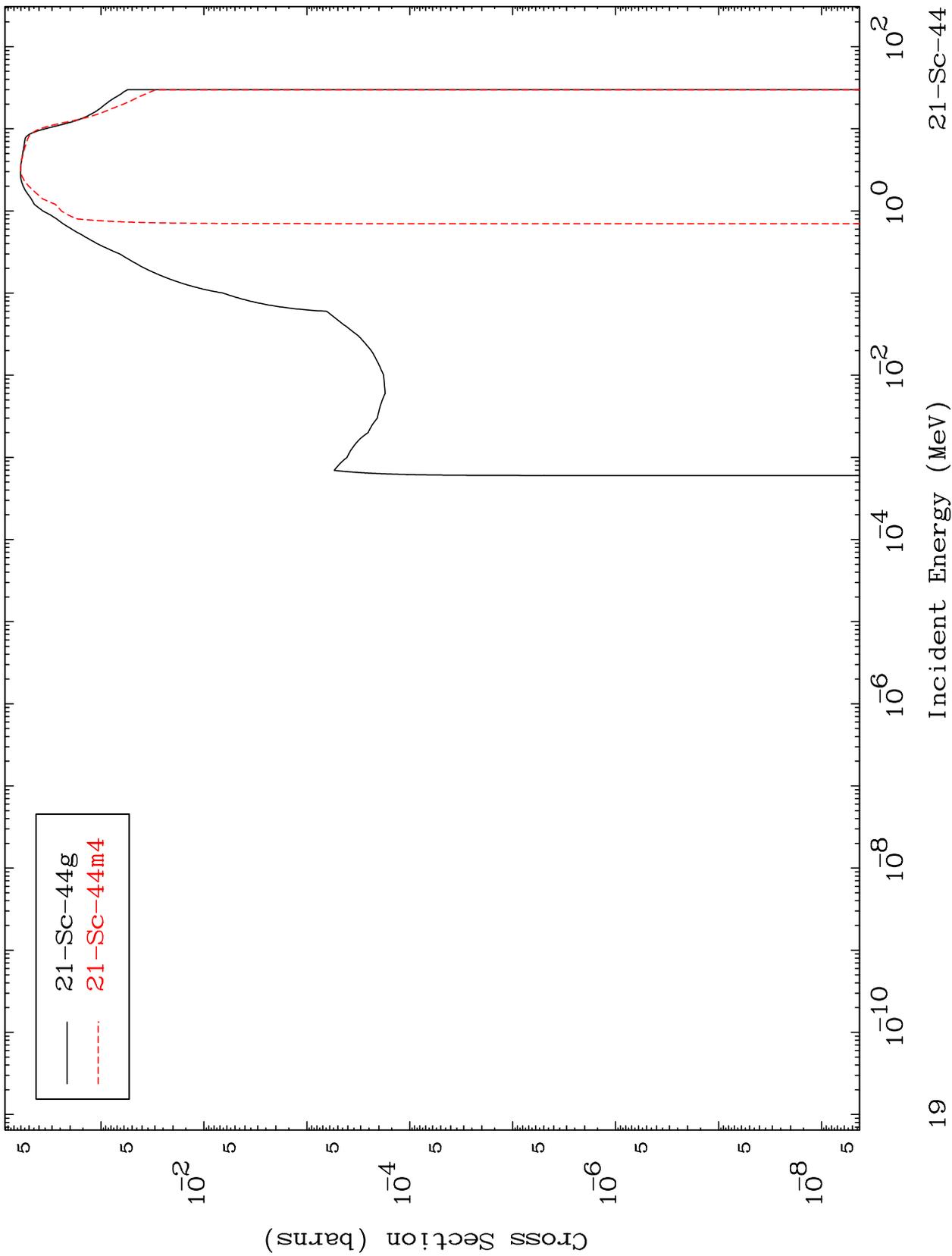
Incident Energy (MeV)

21-Sc-44

MAT 2123

21-Sc-44

Inelastic  
Radionuclide Production Cross Section

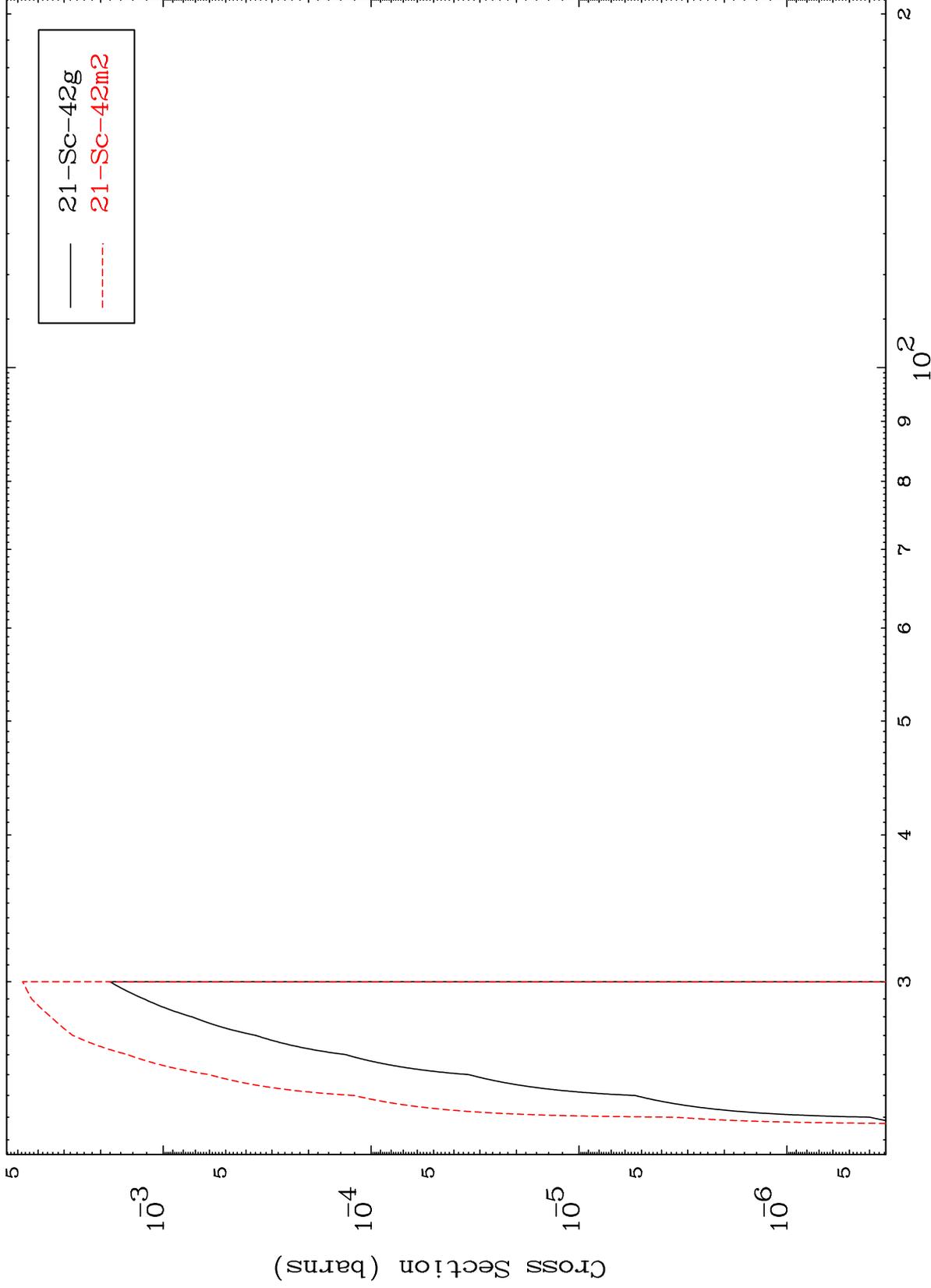


MAT 2123

(n,3n)

21-Sc-44

Radionuclide Production Cross Section



20

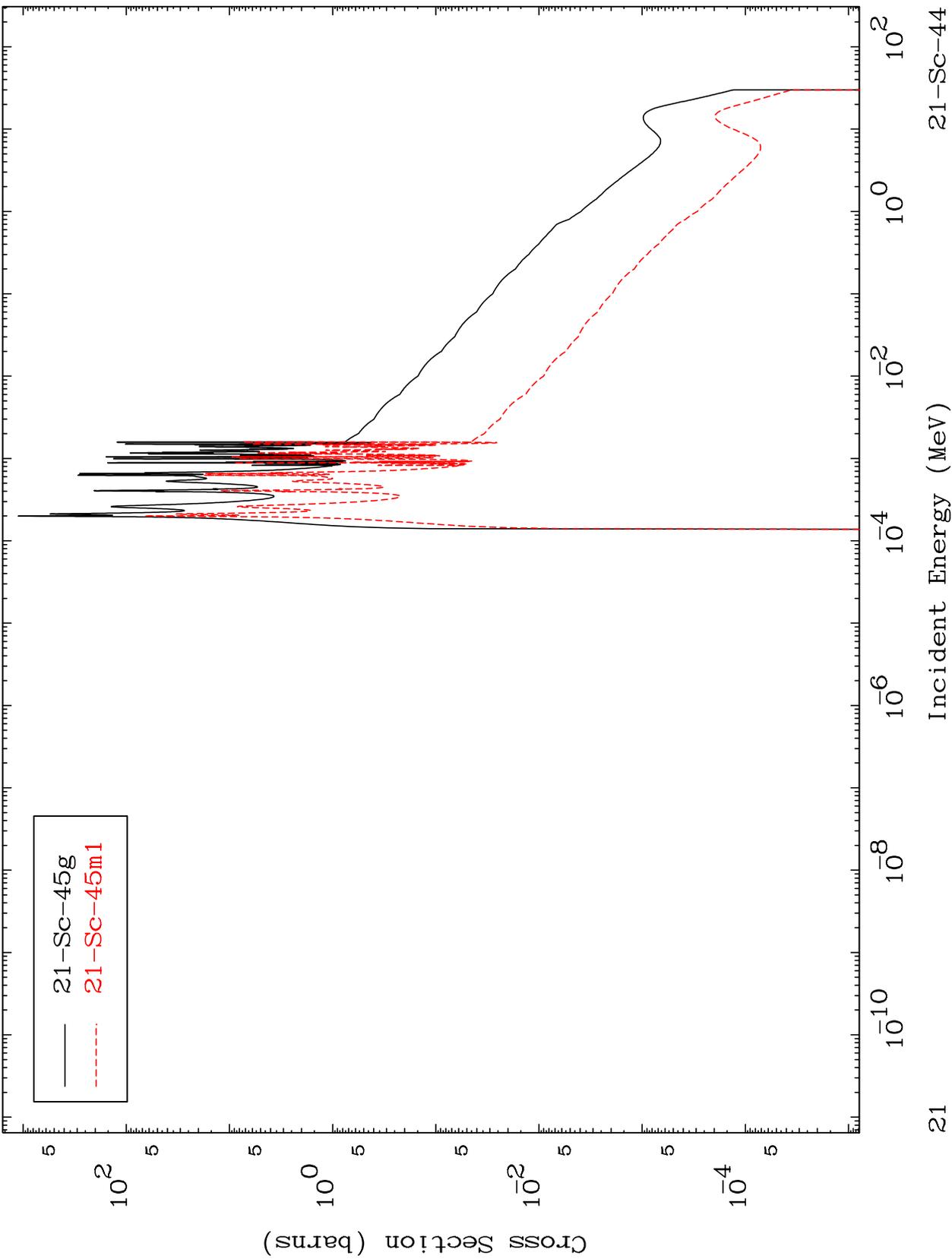
Incident Energy (MeV)

21-Sc-44

MAT 2123

21-Sc-44

Radionuclide Production Cross Section



— 21-Sc-45g  
- - - 21-Sc-45m1

21

21-Sc-44