

Program EVALPLOT  
(Version 2018-1)

by

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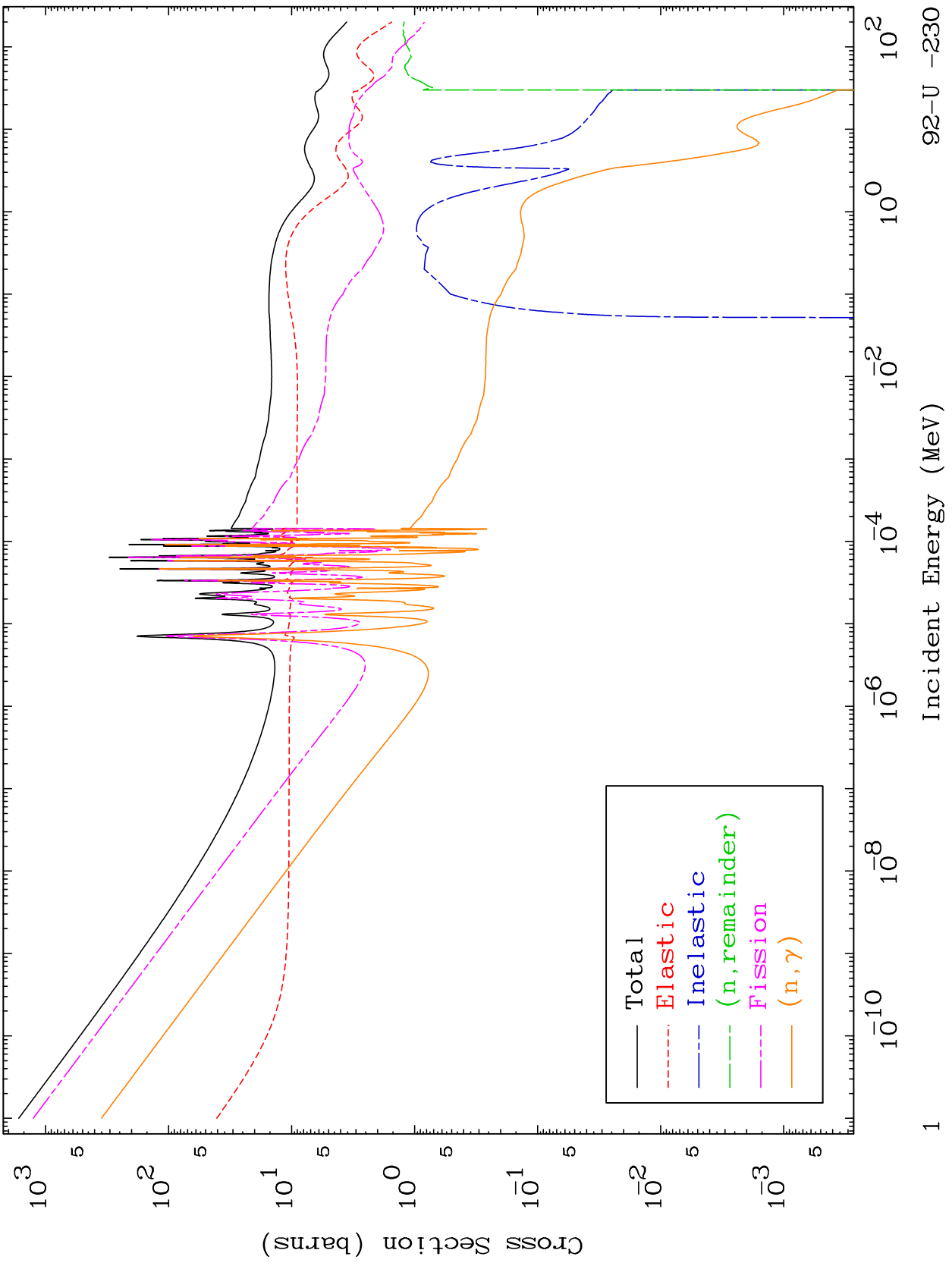
Press Mouse Button to Start

MAT 9213

Major

293 Kelvin Cross Sections

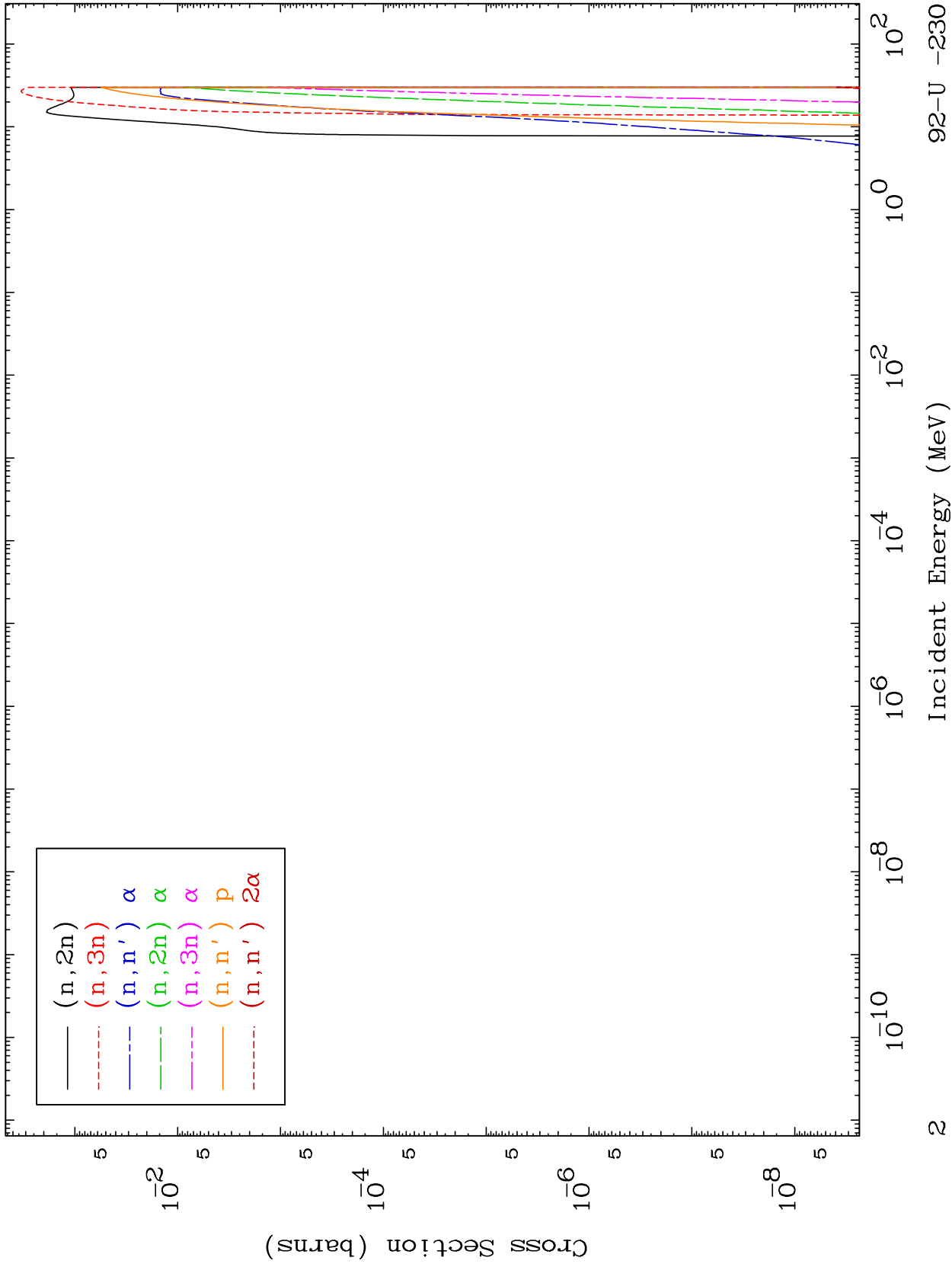
92-U -230



MAT 9213

Neutron Production  
293 Kelvin Cross Sections

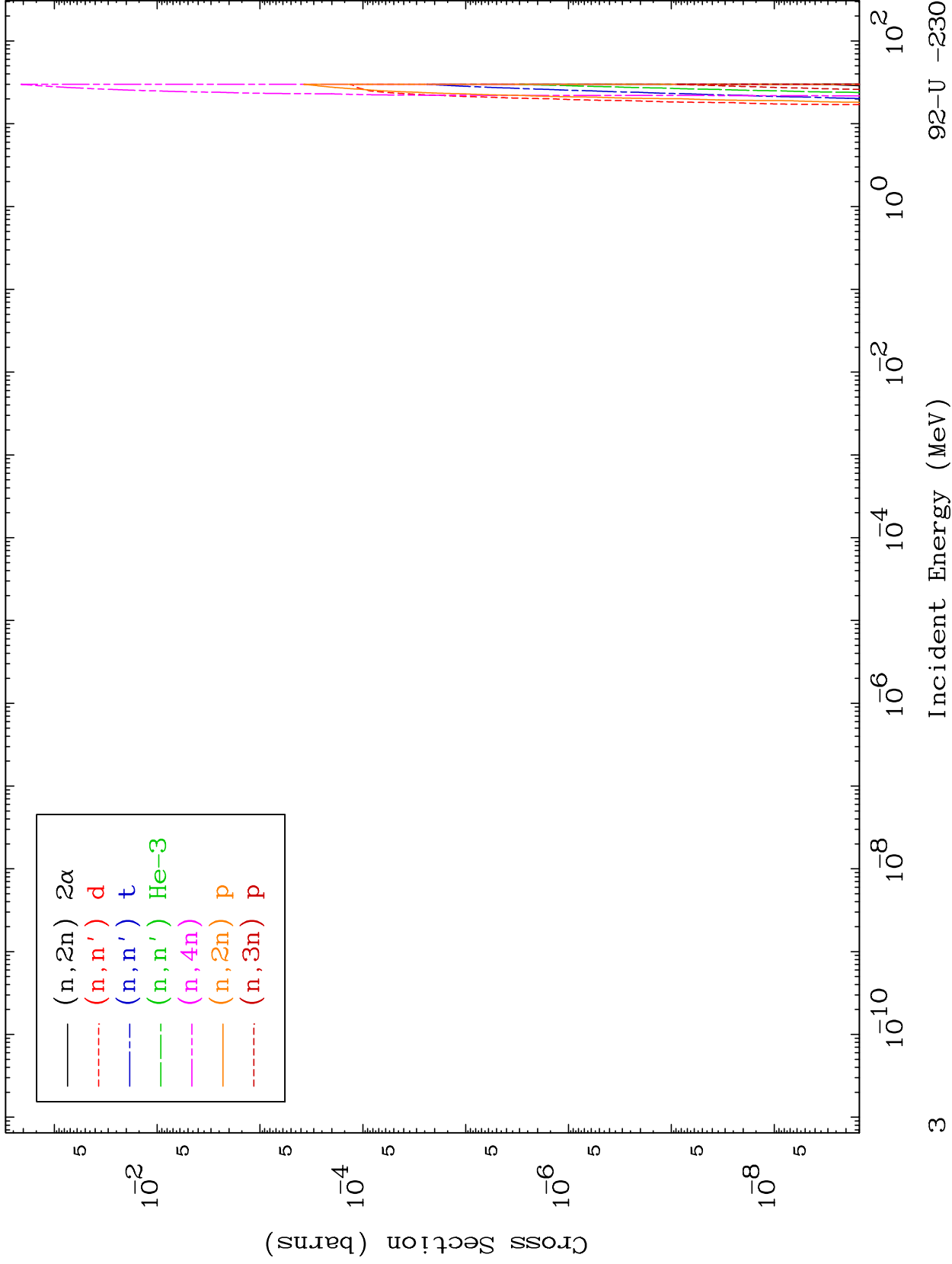
92-U -230



MAT 9213

Neutron Production  
293 Kelvin Cross Sections

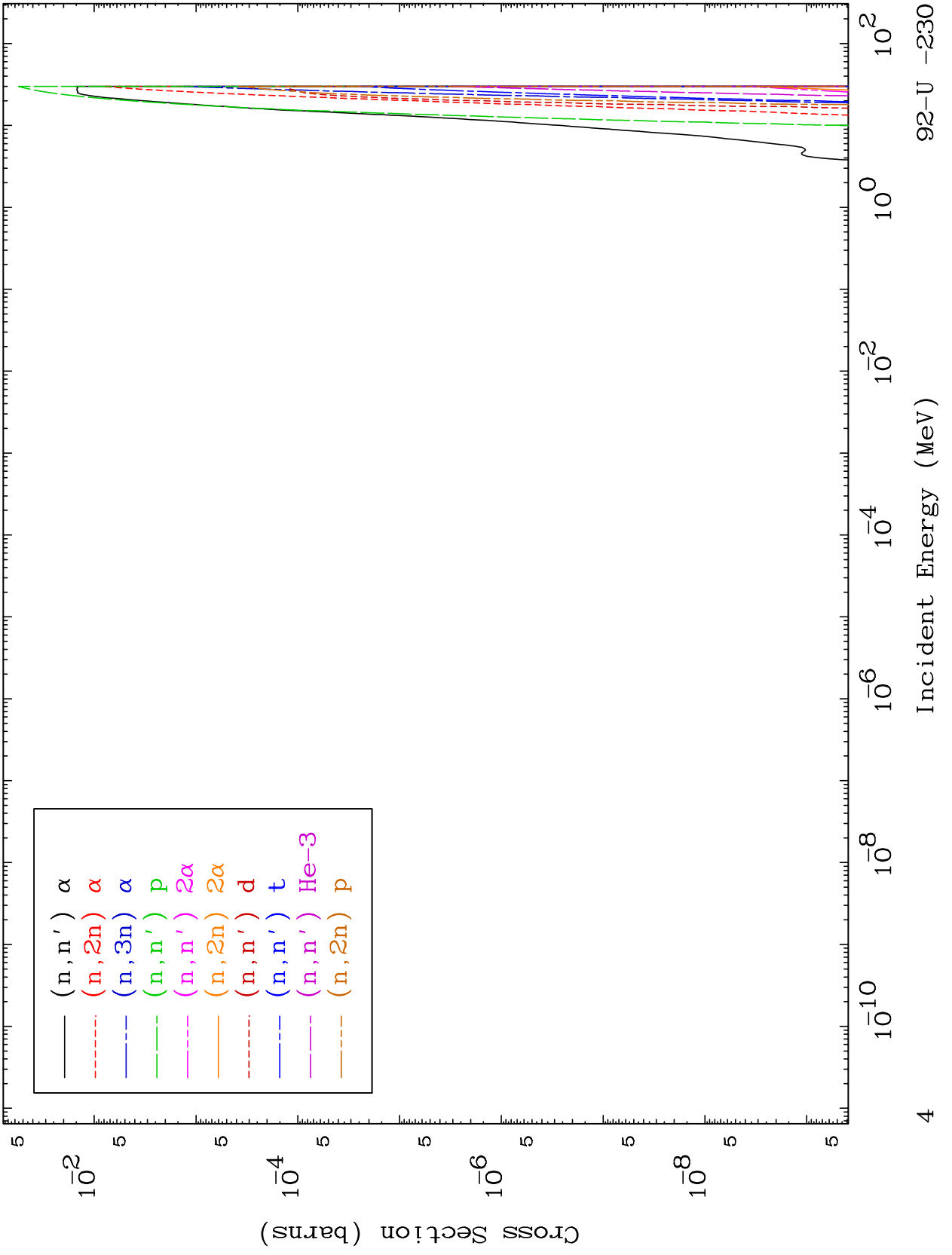
92-U -230



MAT 9213

Charged Particle  
293 Kelvin Cross Sections

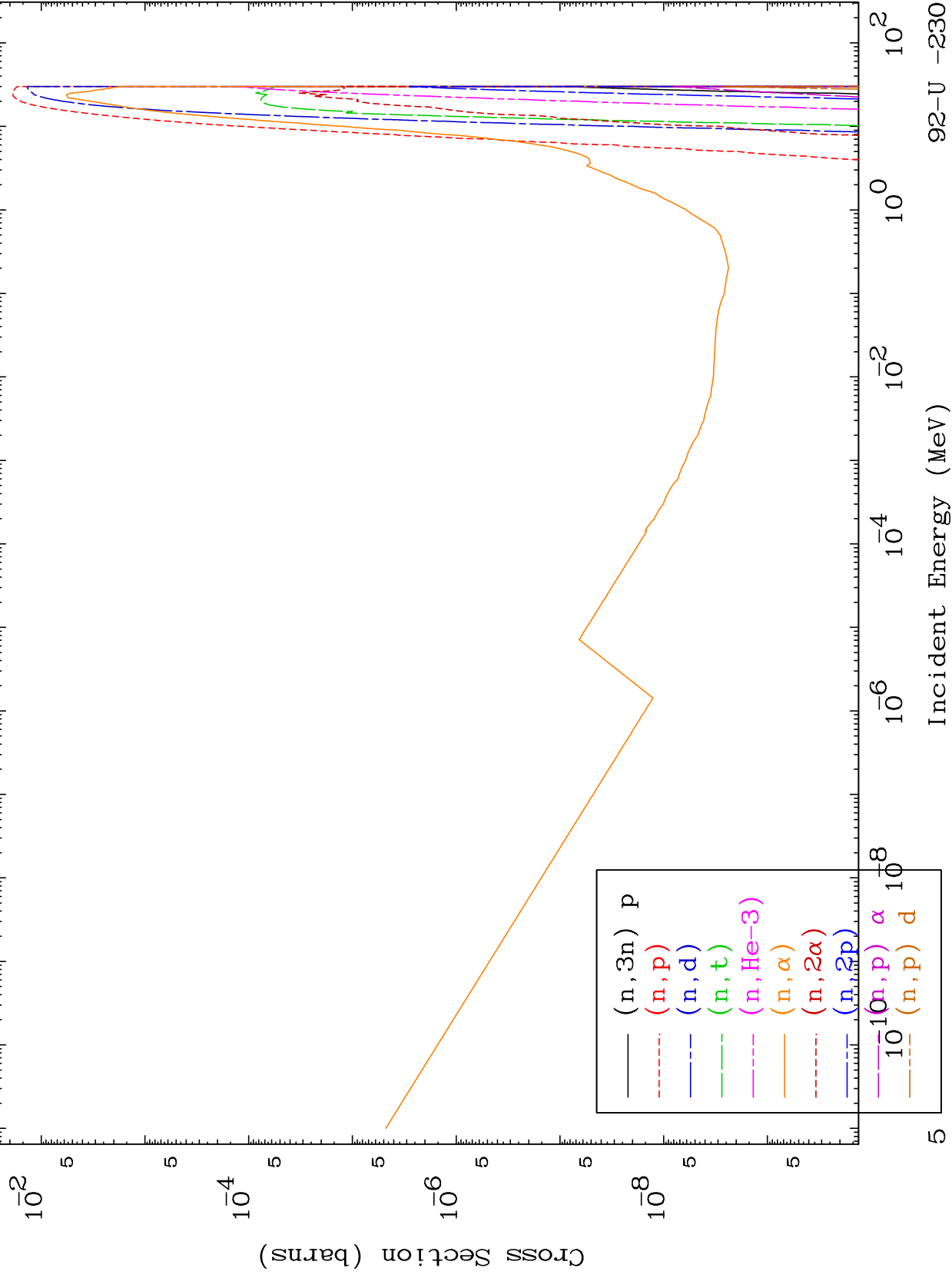
92-U -230



MAT 9213

Charged Particle  
293 Kelvin Cross Sections

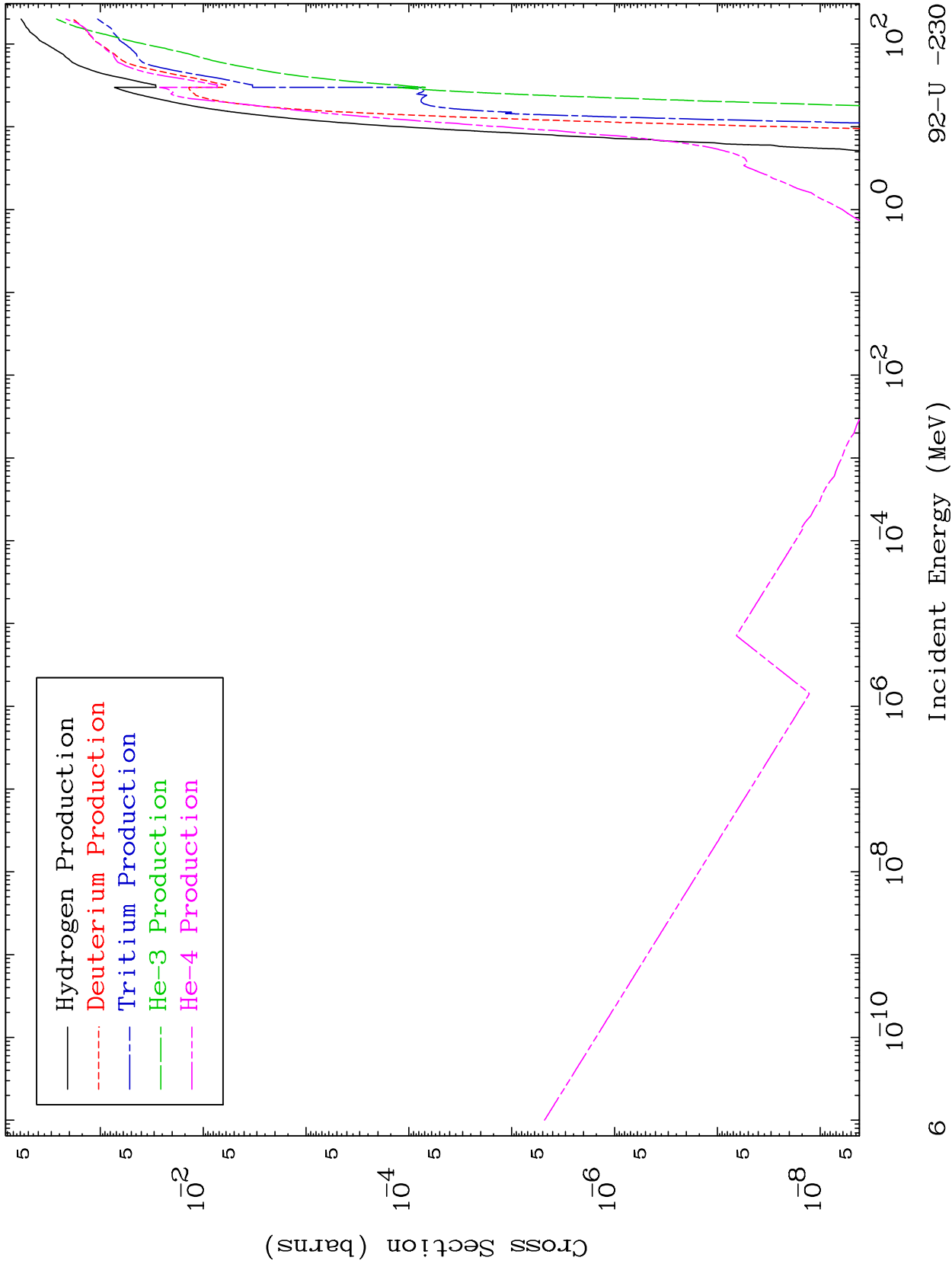
92-U -230



MAT 9213

Particle Production  
293 Kelvin Cross Sections

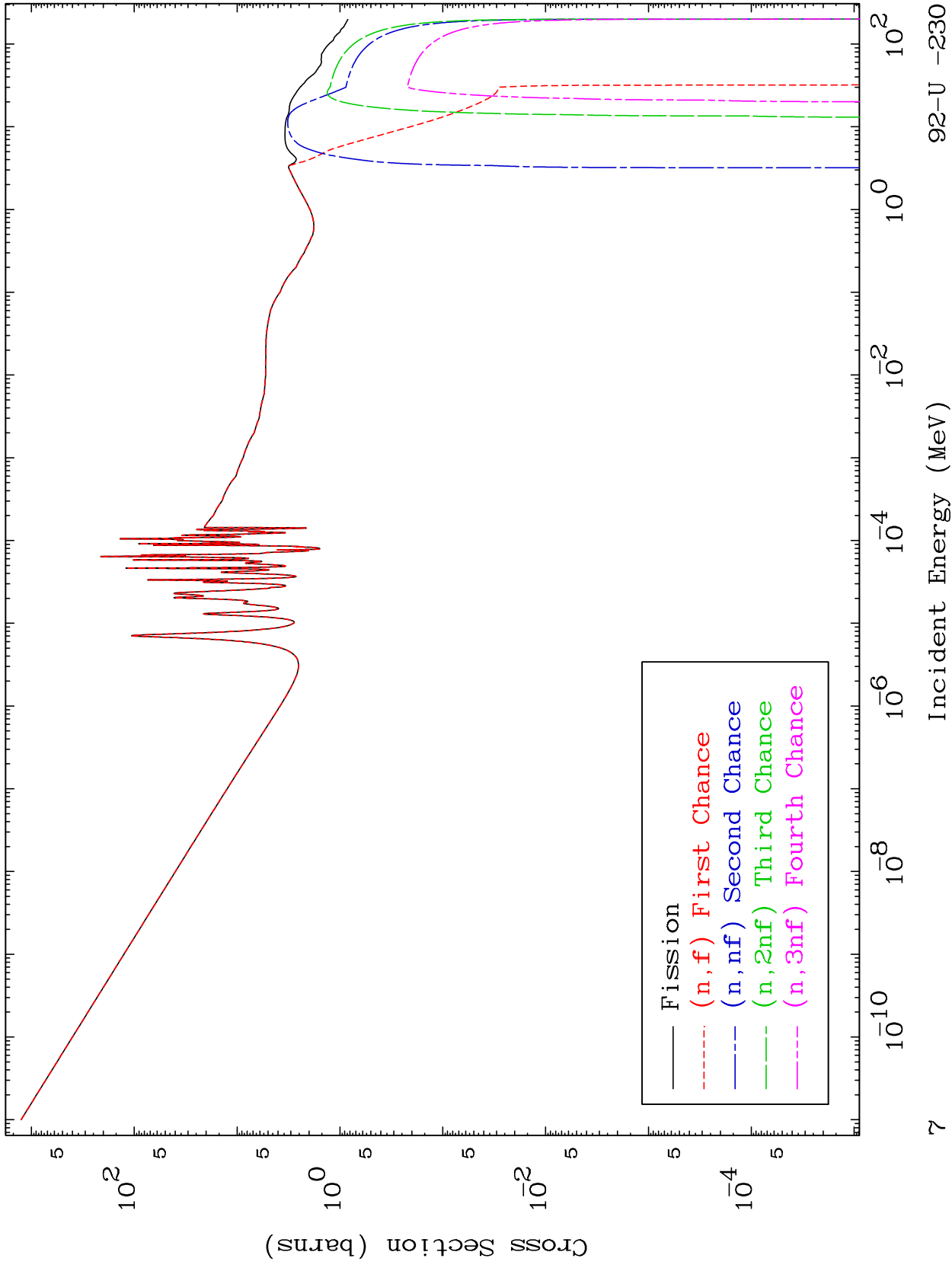
92-U -230



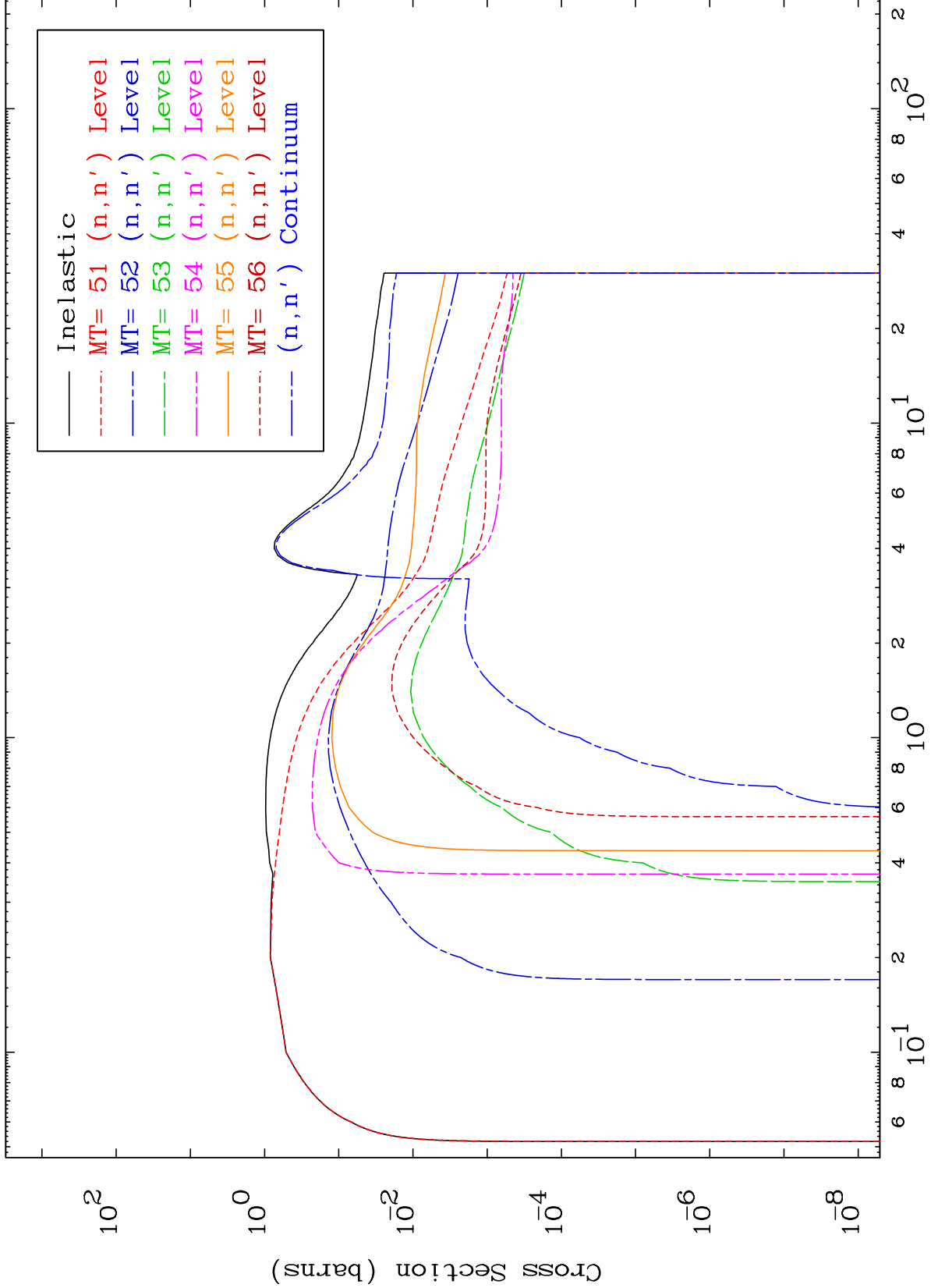
MAT 9213

Fission  
293 Kelvin Cross Sections

92-U -230



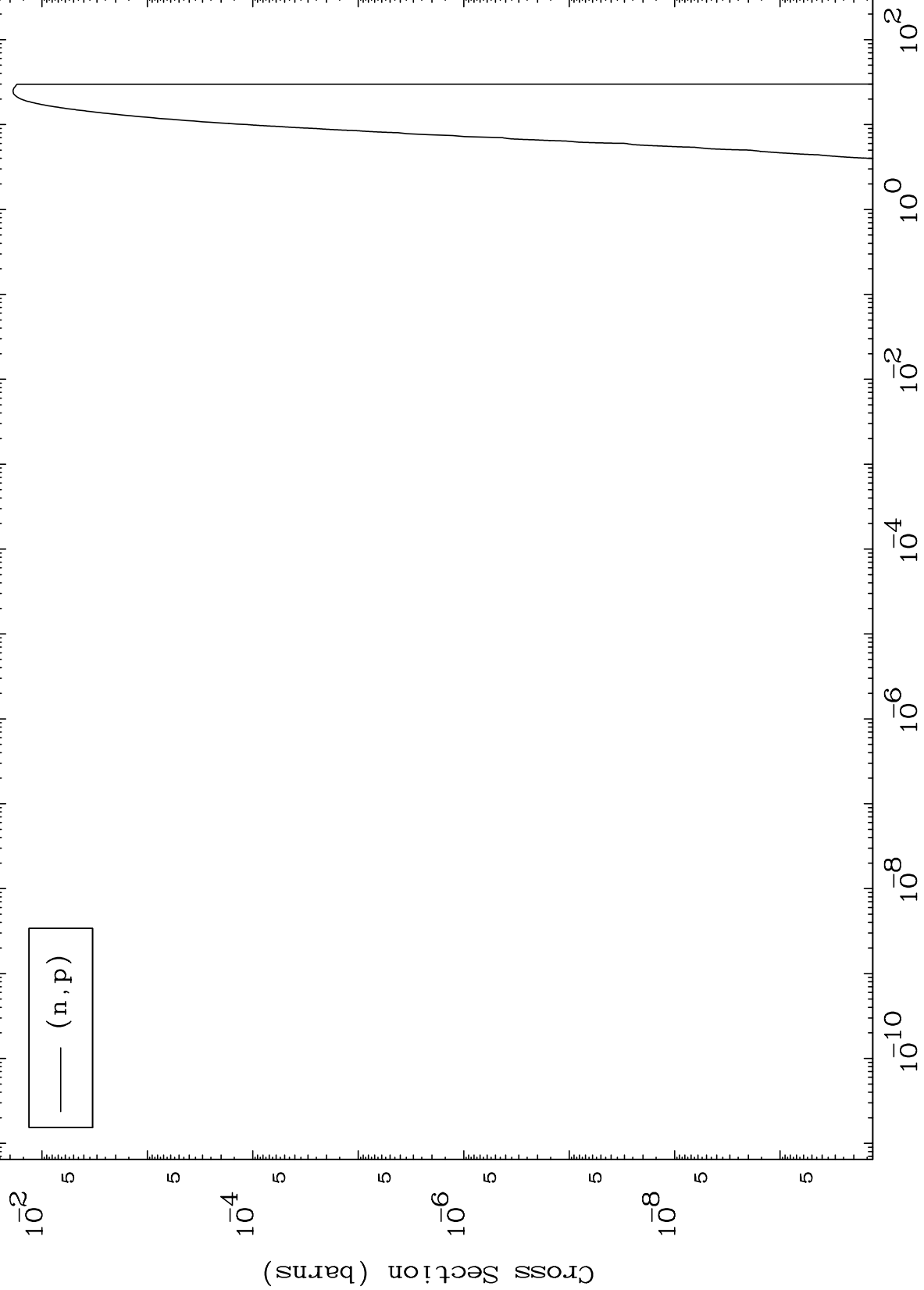
293 Kelvin Cross Sections



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(n,p) Levels  
293 Kelvin Cross Sections

92-U -230

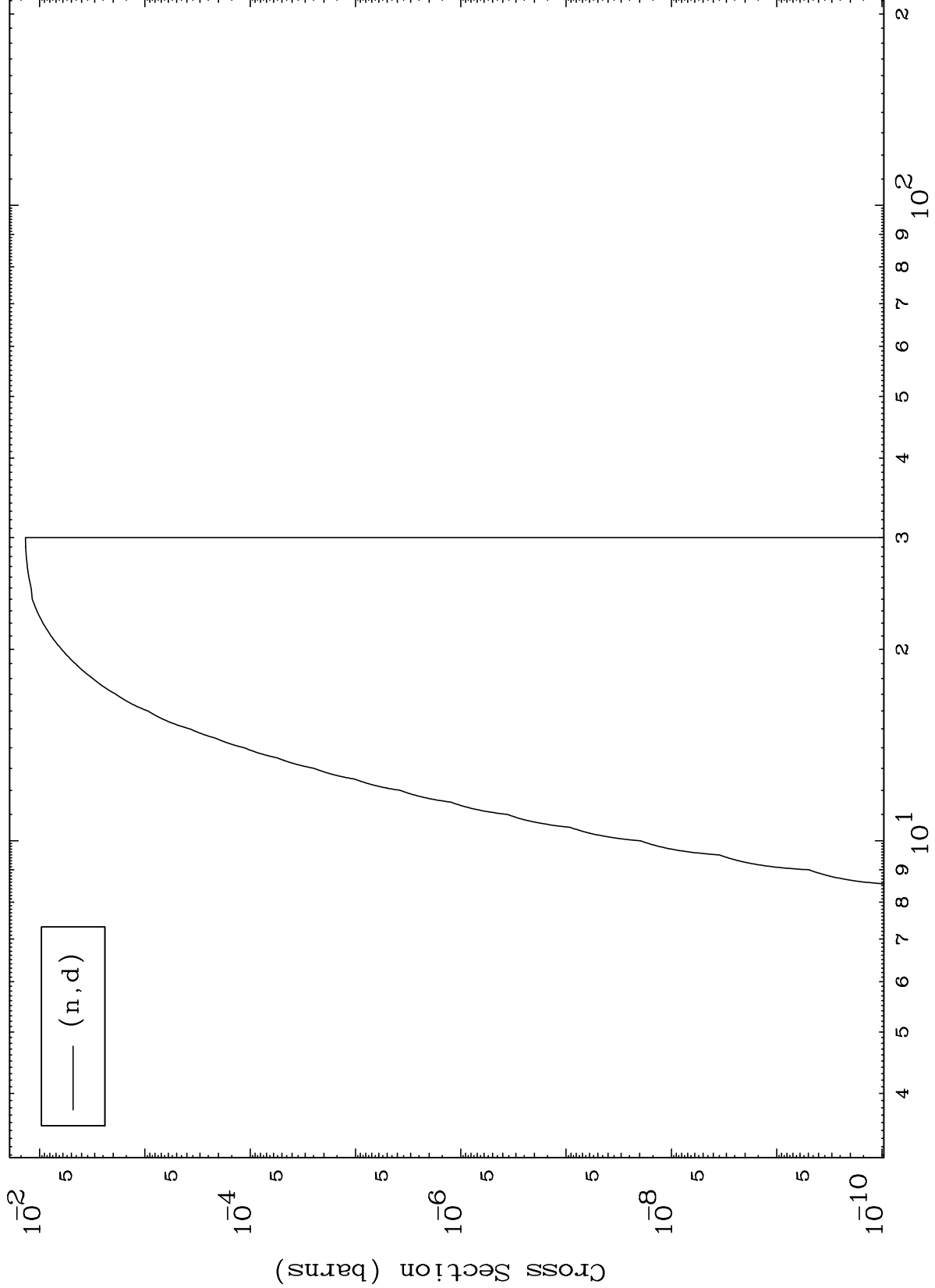


92-U -230

MAT 9213

(n,d) Levels  
293 Kelvin Cross Sections

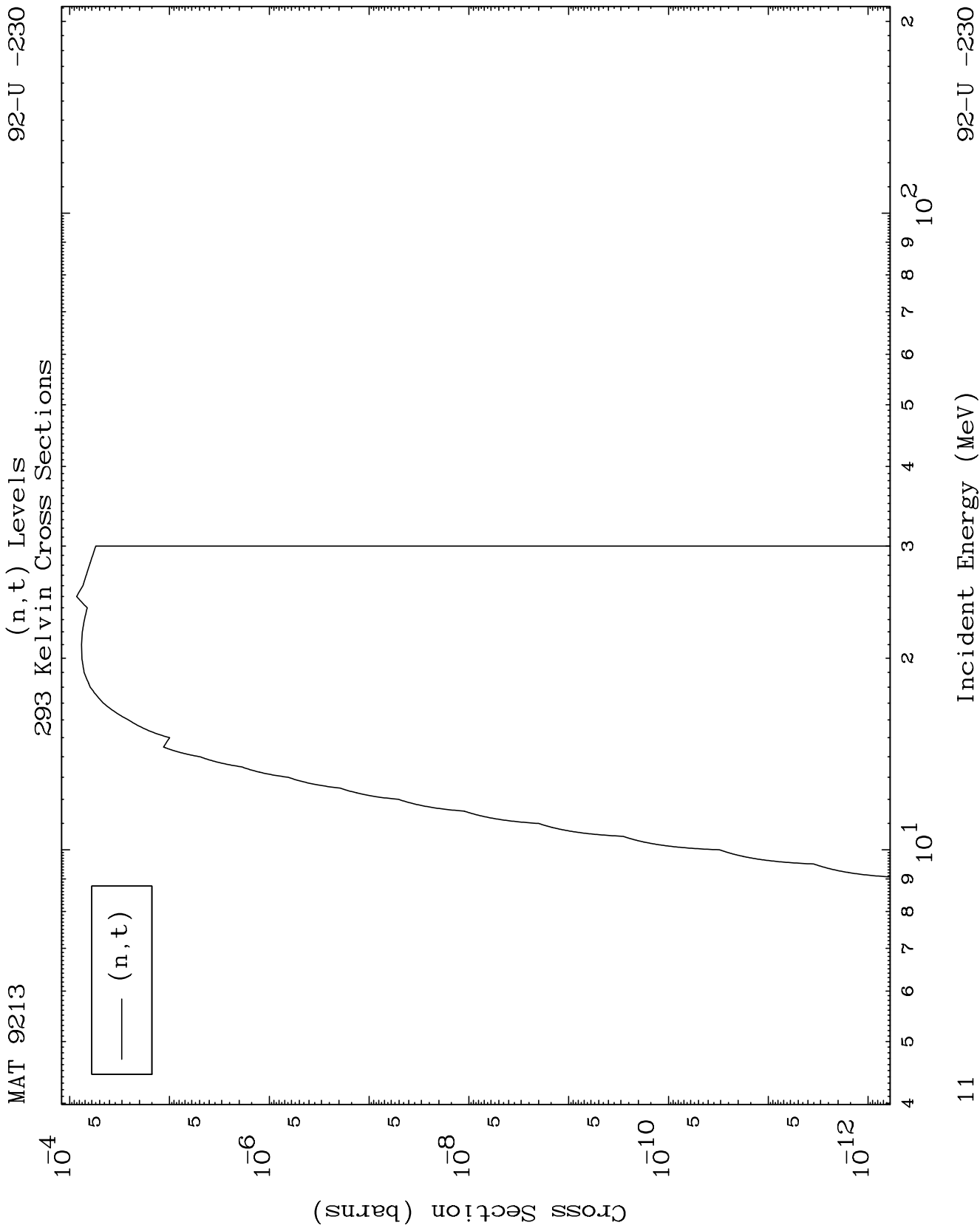
92-U -230



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Incident Energy (MeV)

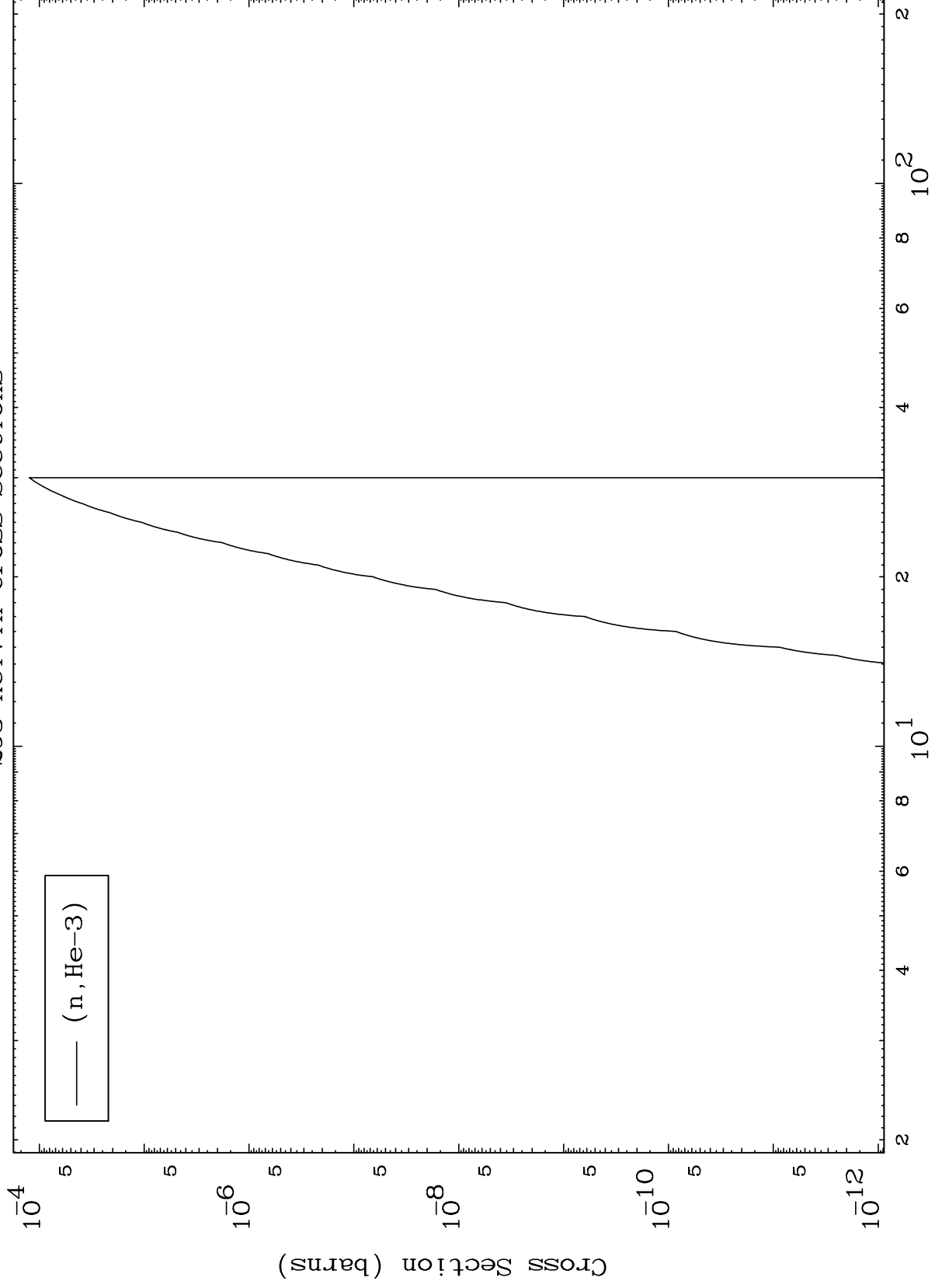
92-U -230



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(n,He3) Levels  
293 Kelvin Cross Sections

92-U -230



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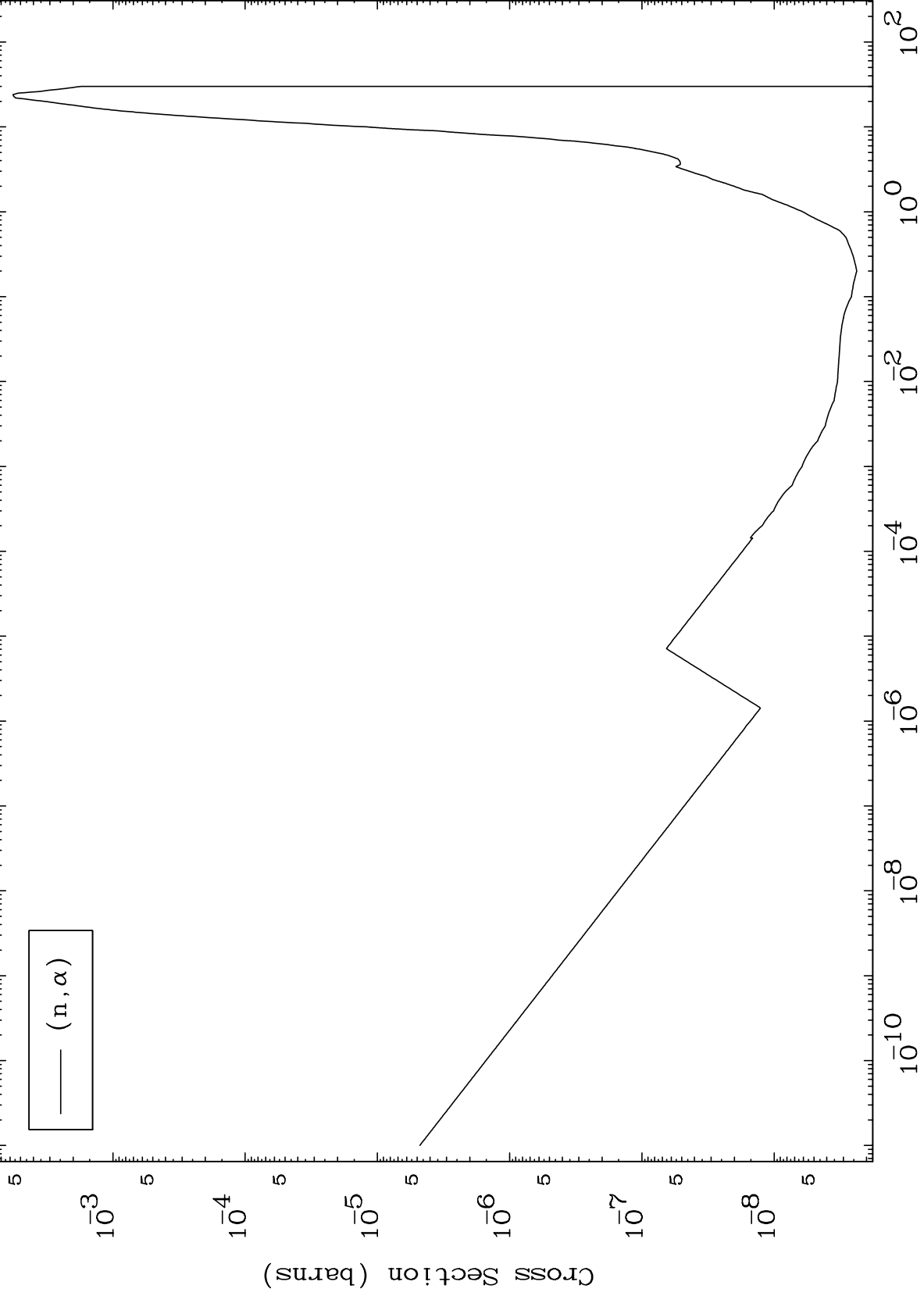
Incident Energy (MeV)

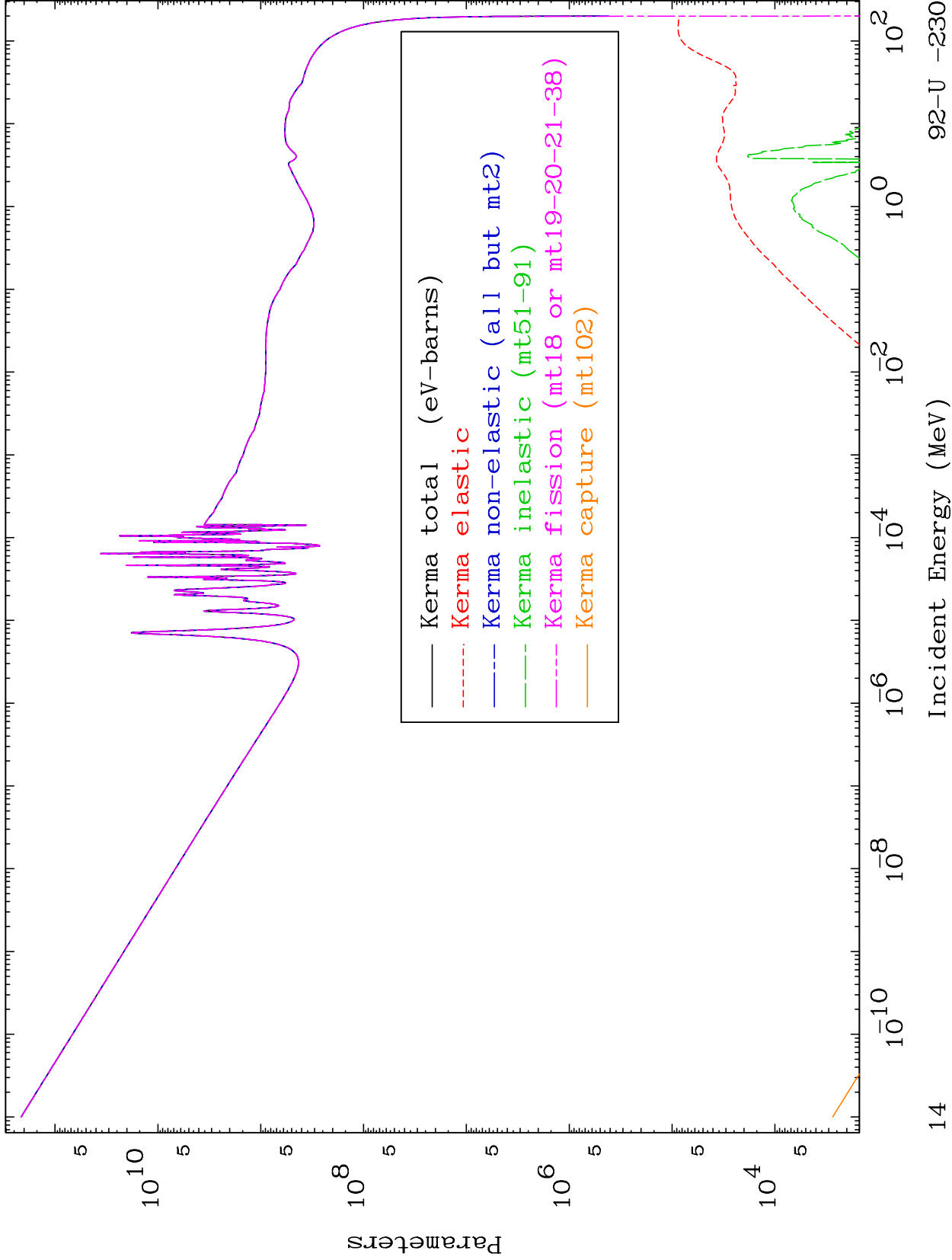
92-U -230

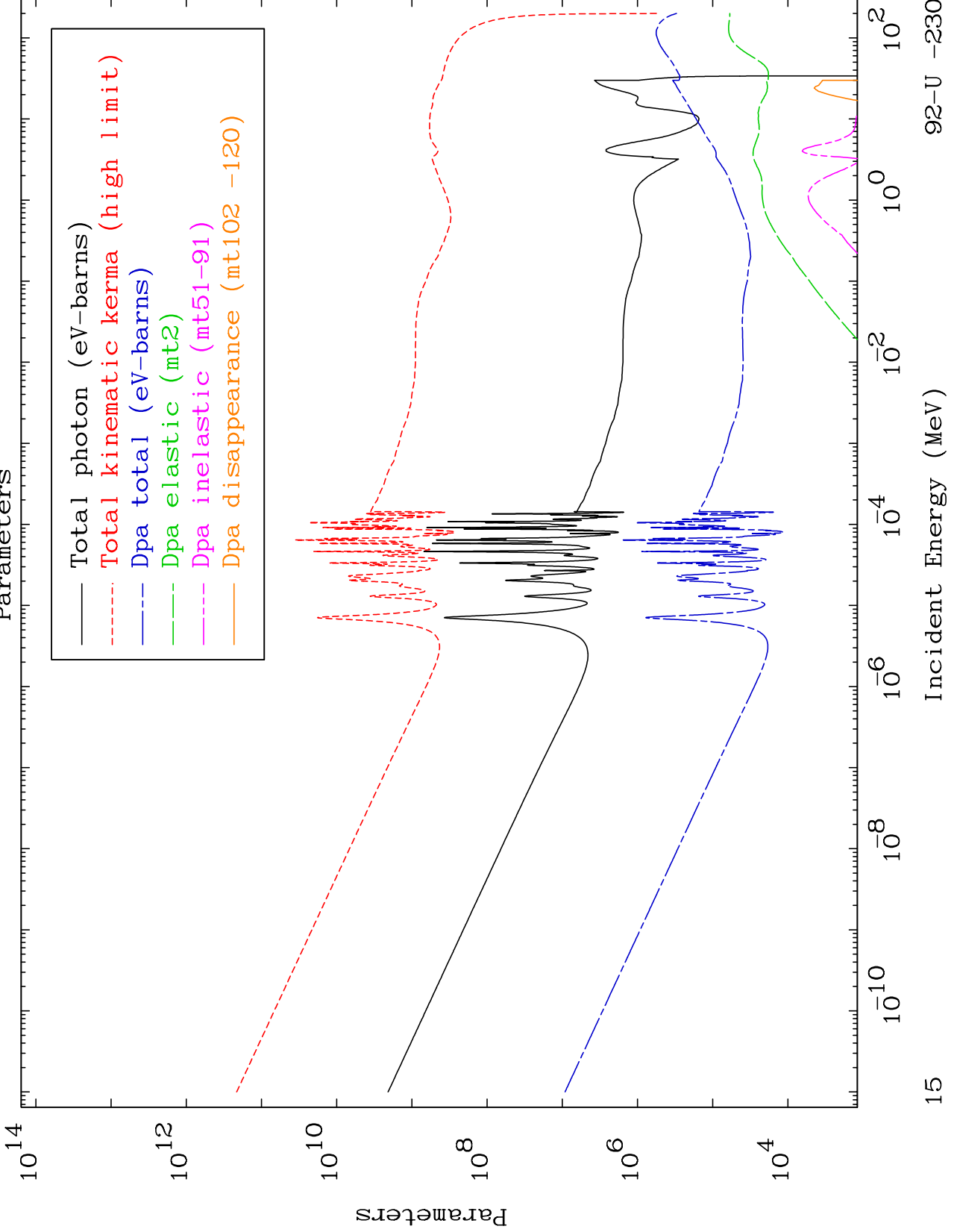
MAT 9213

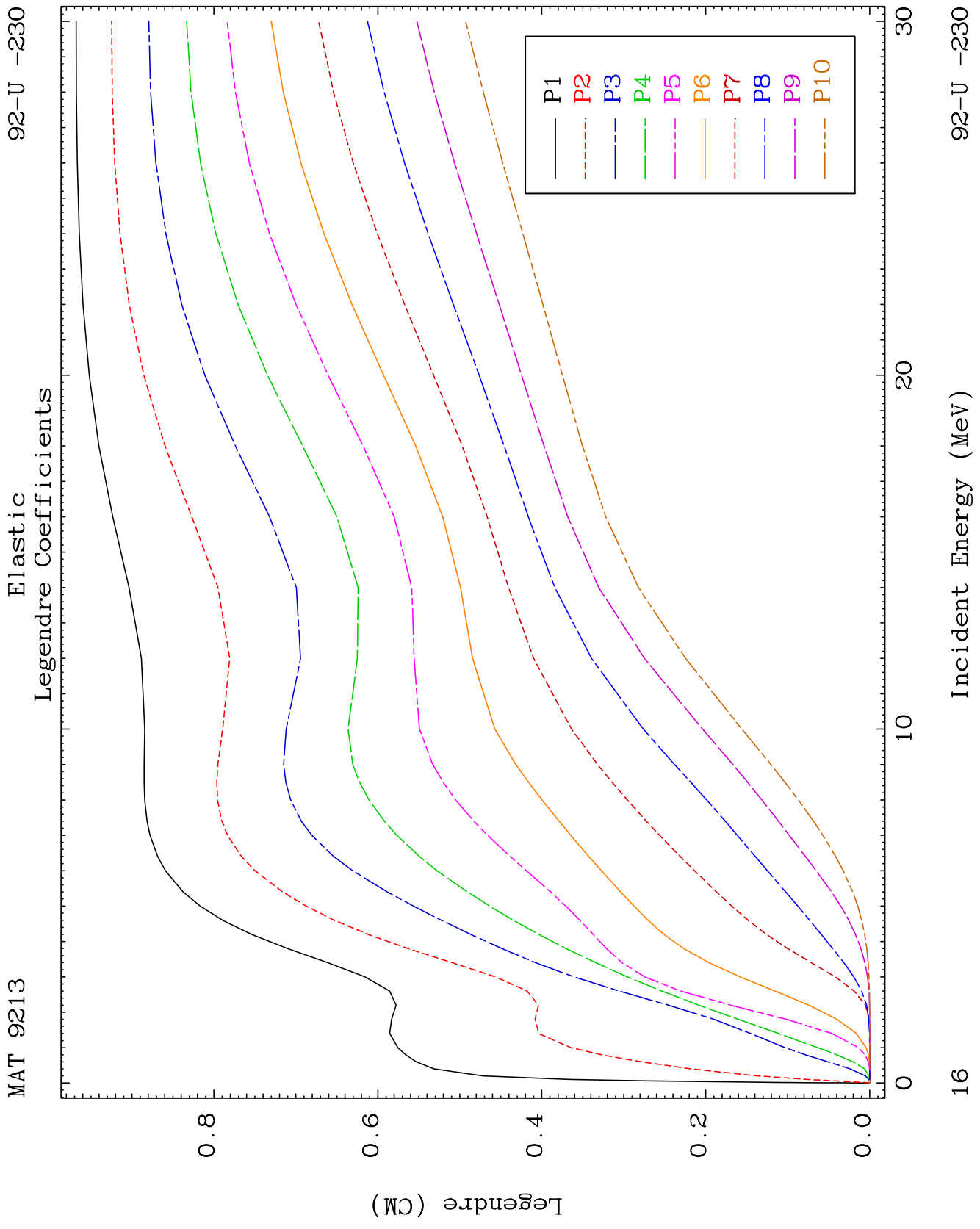
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

92-U -230







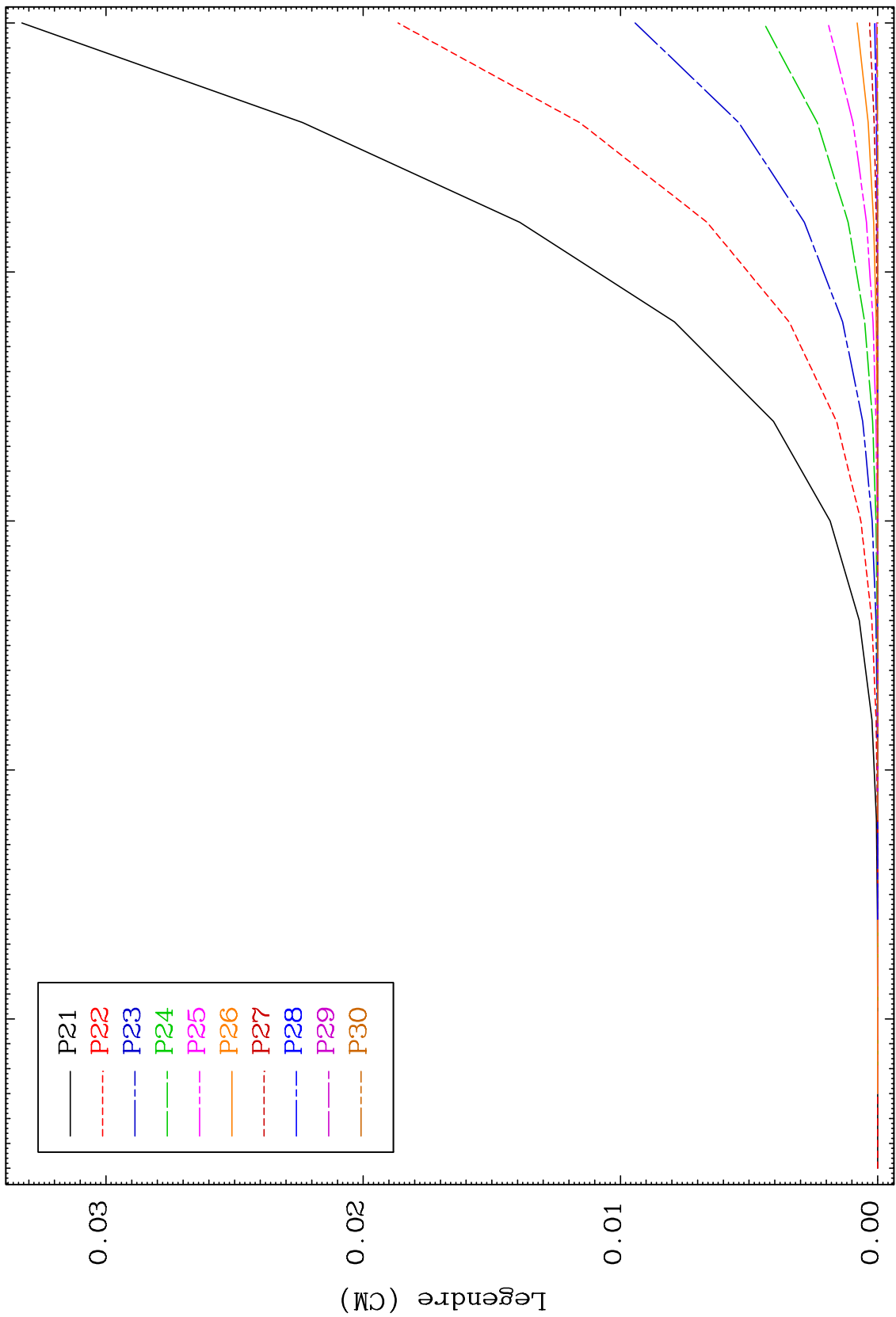




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Elastic Legendre Coefficients

92-U -230



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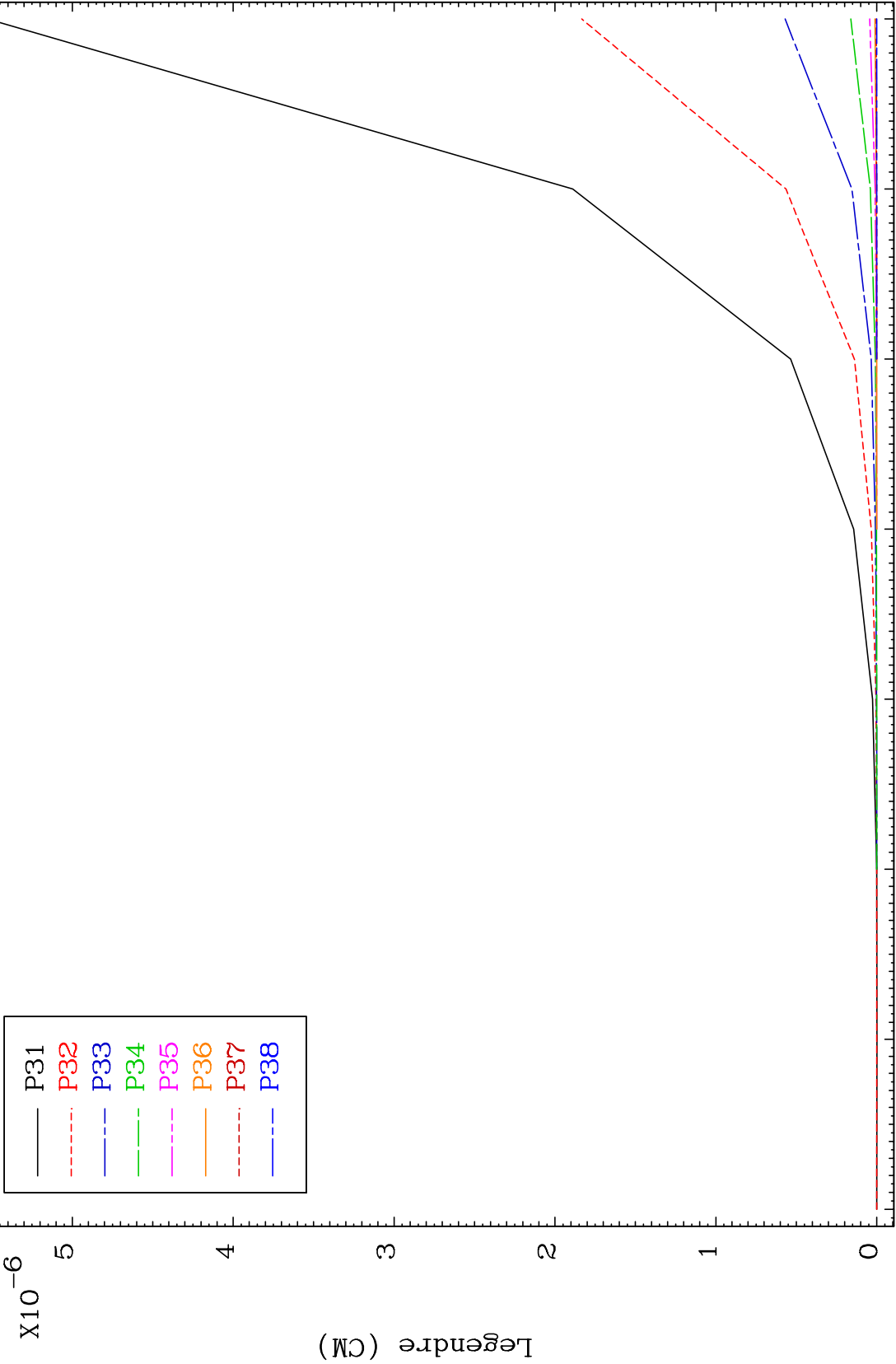
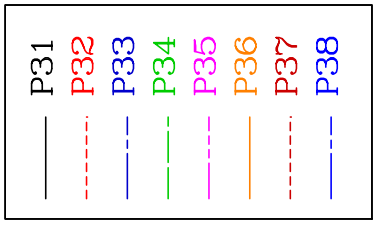
Incident Energy (MeV)

92-U -230

MAT 9213

Elastic Legendre Coefficients

92-U -230



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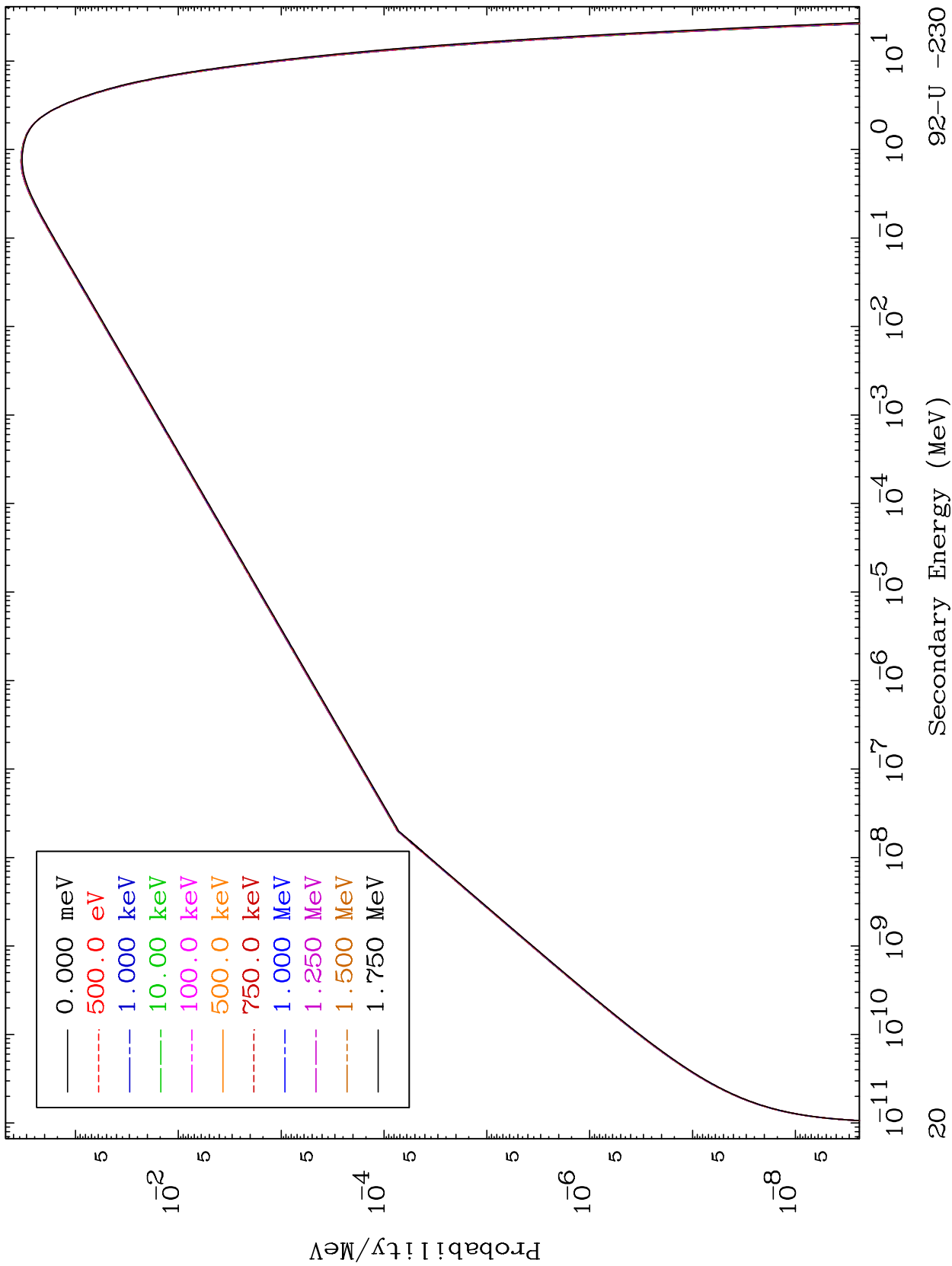
Incident Energy (MeV)

92-U -230

MAT 9213

### Fission Energy Distribution

92-U -230



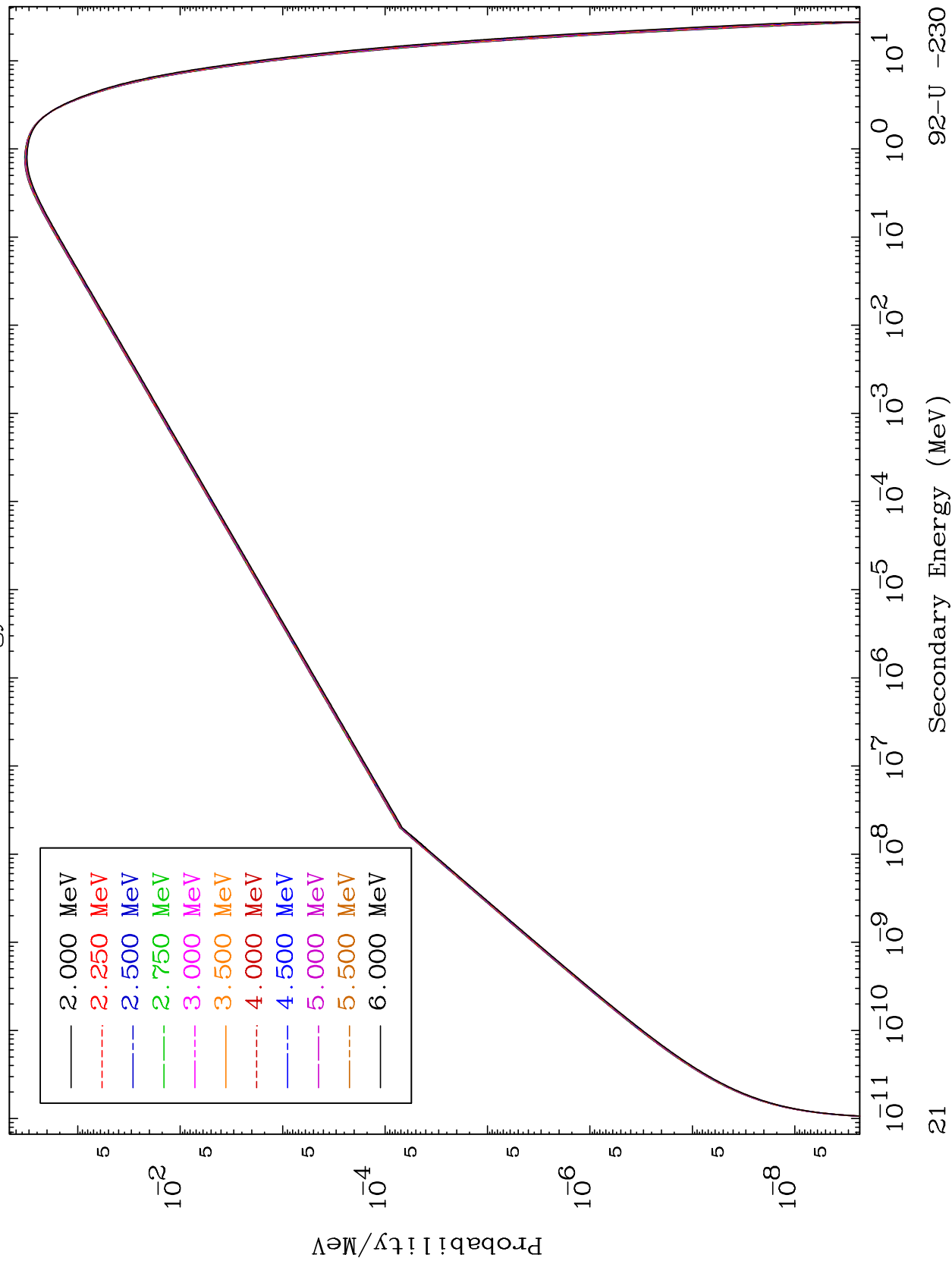
20

92-U -230

MAT 9213

### Fission Energy Distribution

92-U -230

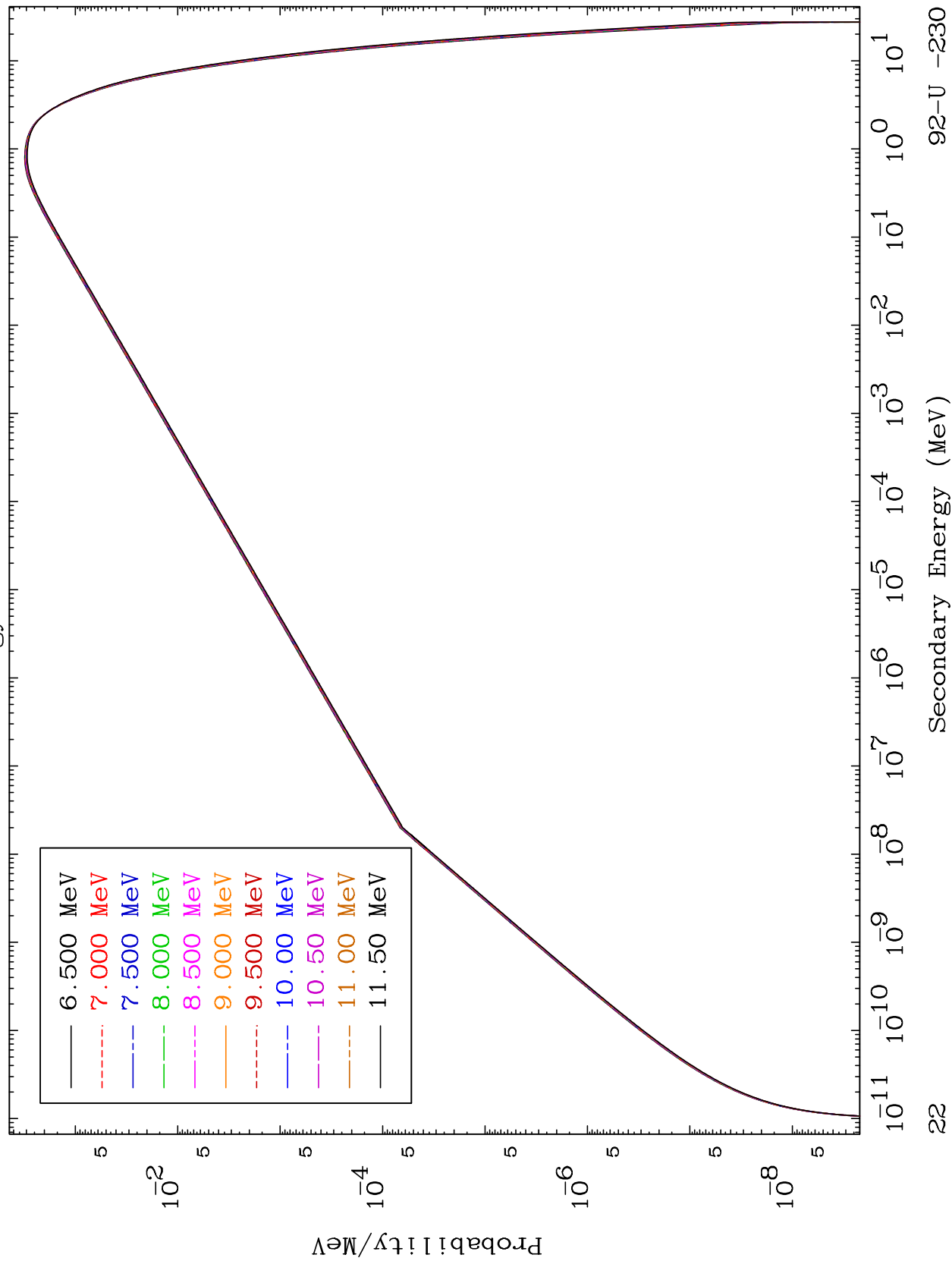


92-U -230

MAT 9213

### Fission Energy Distribution

92-U -230



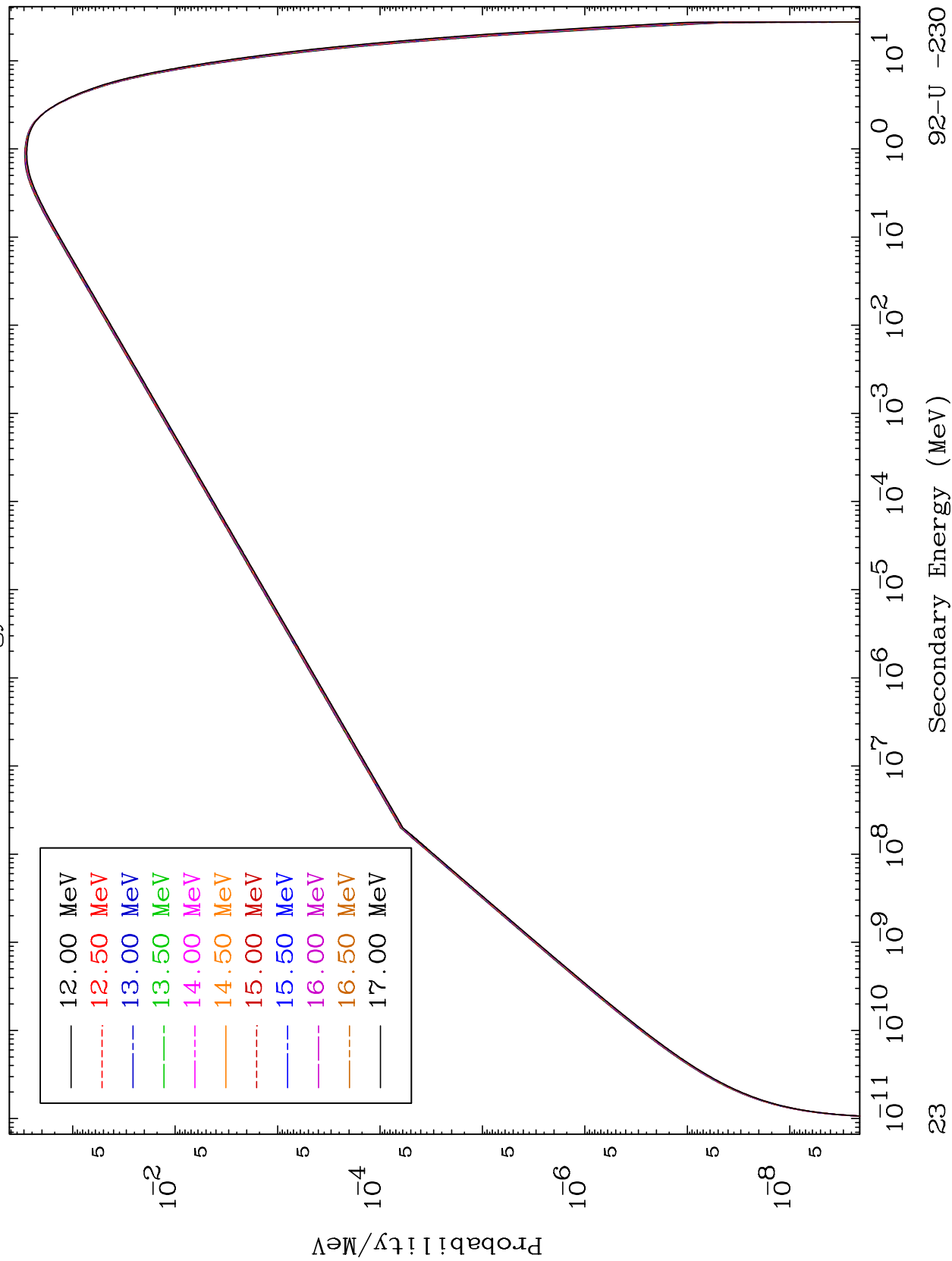
22

92-U -230

MAT 9213

### Fission Energy Distribution

92-U -230



23

92-U -230

MAT 9213

Fission Energy Distribution

92-U -230

