

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

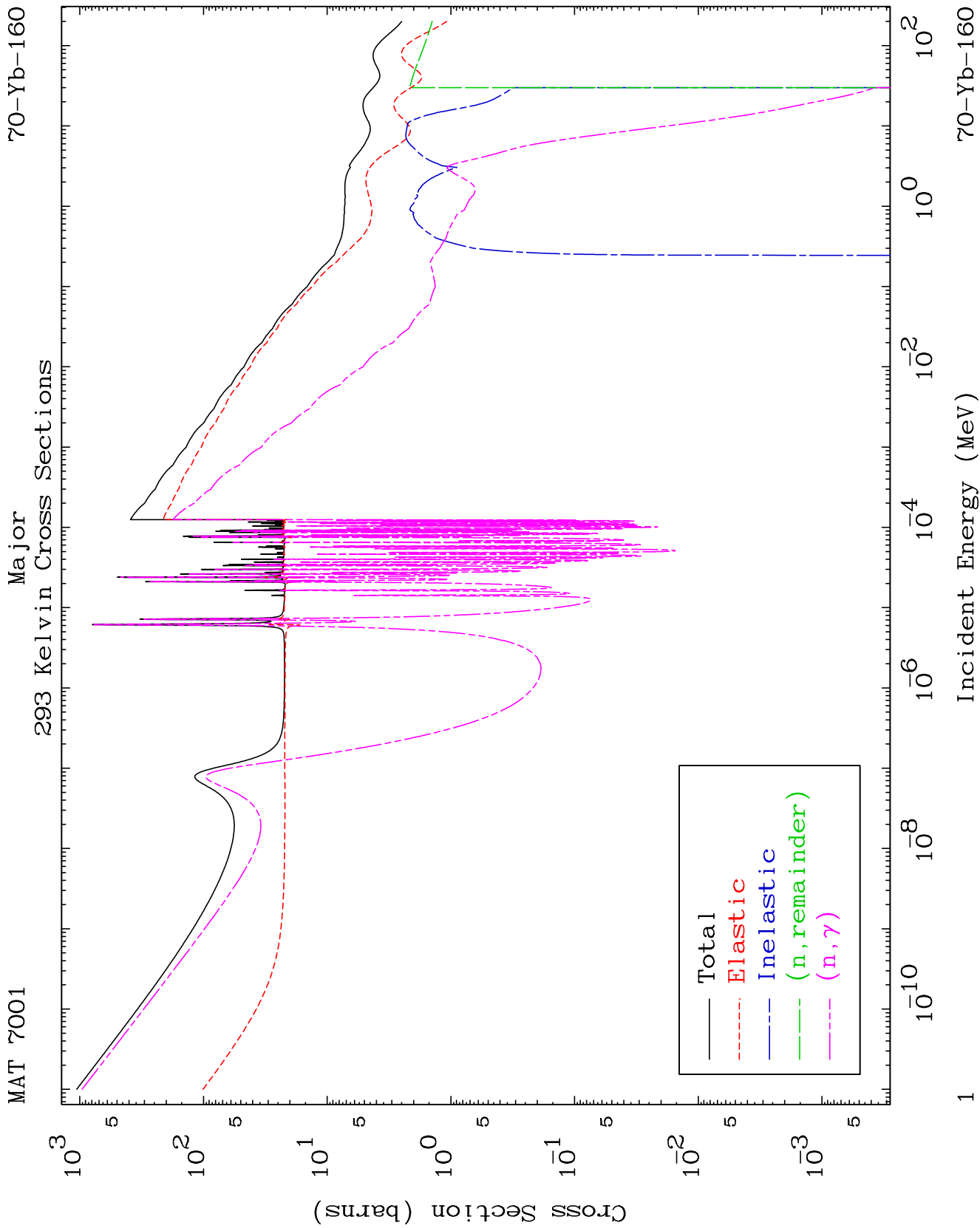
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Web:redcullen1.net/HOMEPAGE.NEW

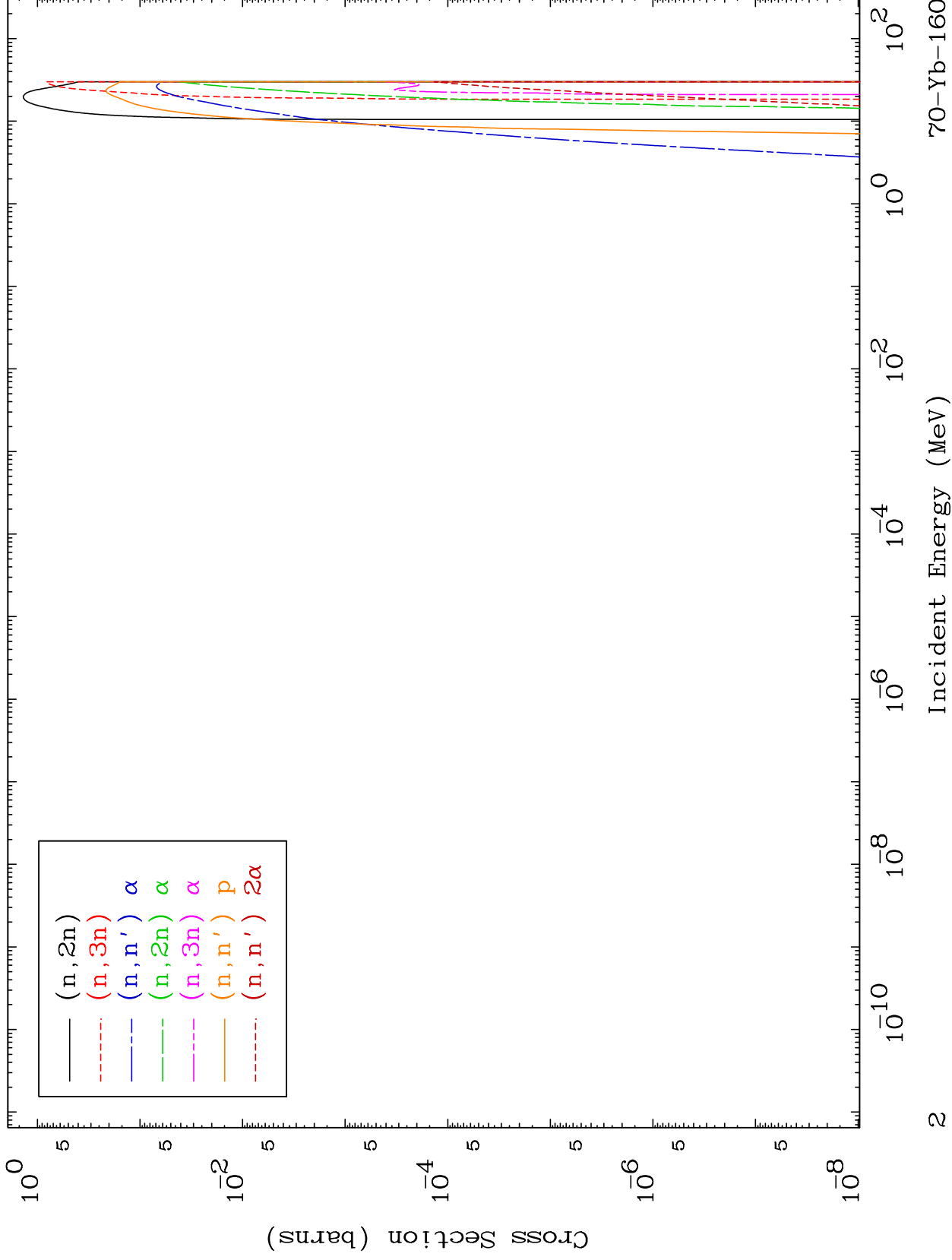
Press Mouse Button to Start



MAT 7001

Neutron Production
293 Kelvin Cross Sections

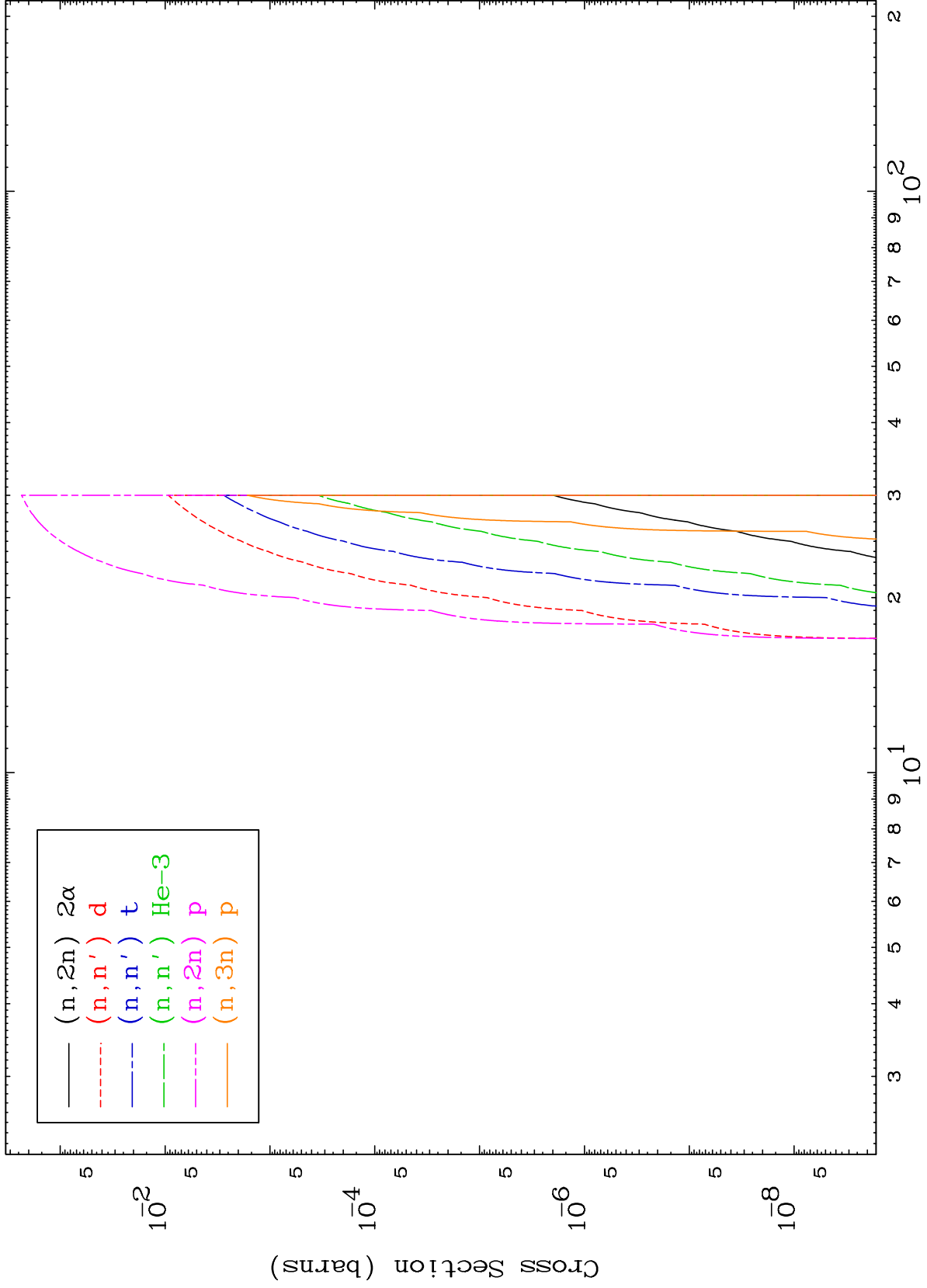
70-Yb-160



MAT 7001

Neutron Production
293 Kelvin Cross Sections

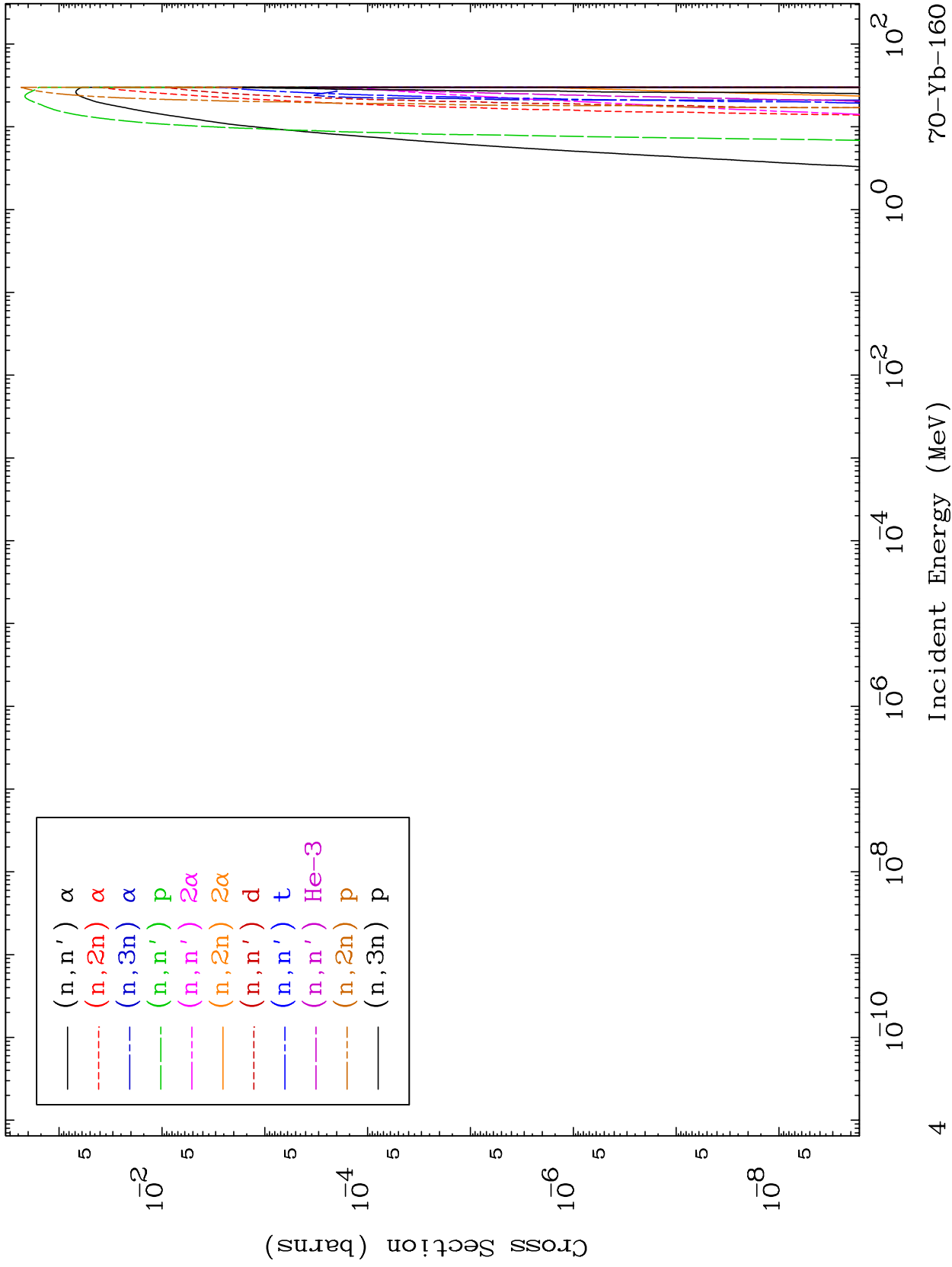
70-Yb-160



MAT 7001

Charged Particle
293 Kelvin Cross Sections

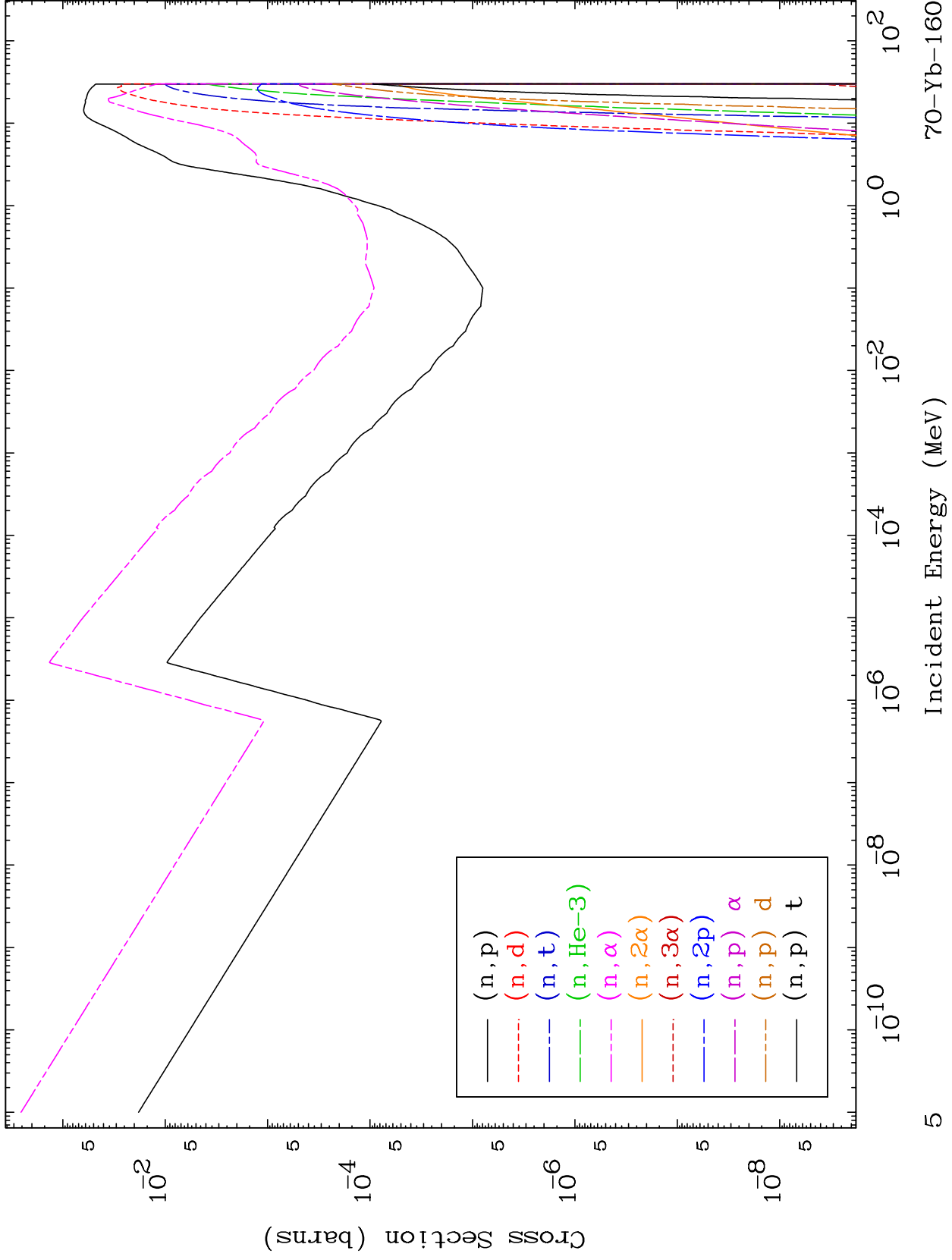
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MAT 7001

Charged Particle
293 Kelvin Cross Sections

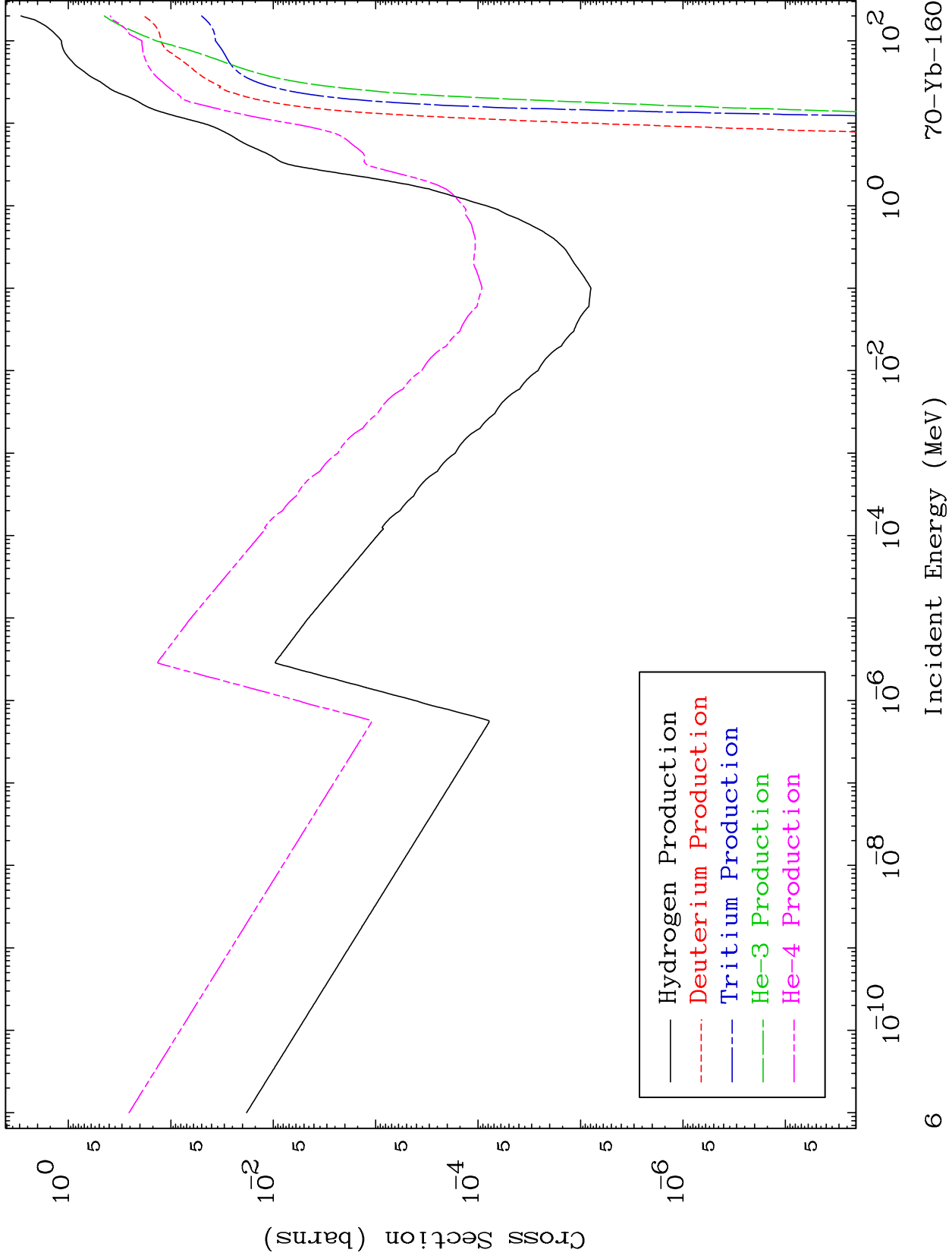
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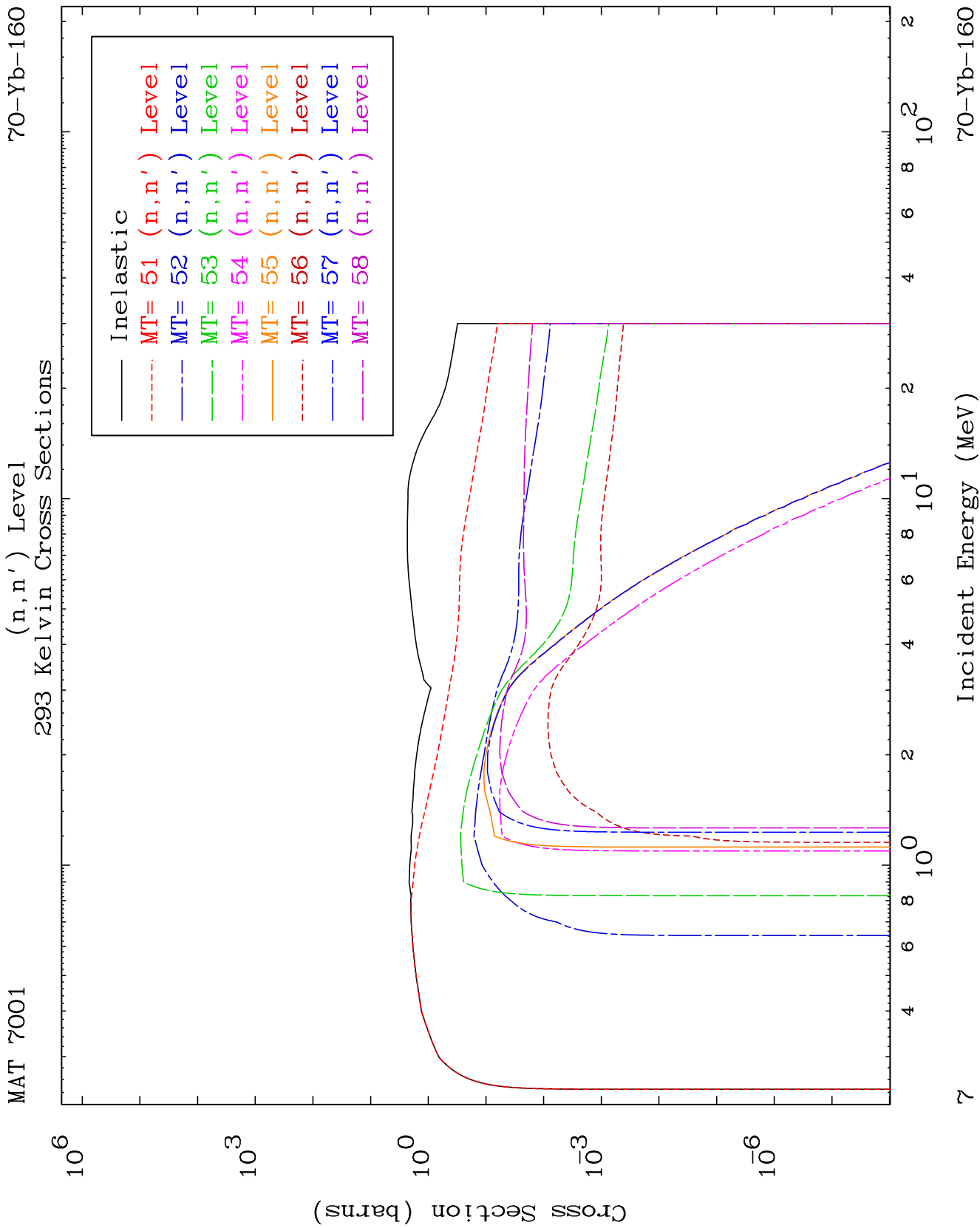


MAT 7001

Particle Production
293 Kelvin Cross Sections

70-Yb-160



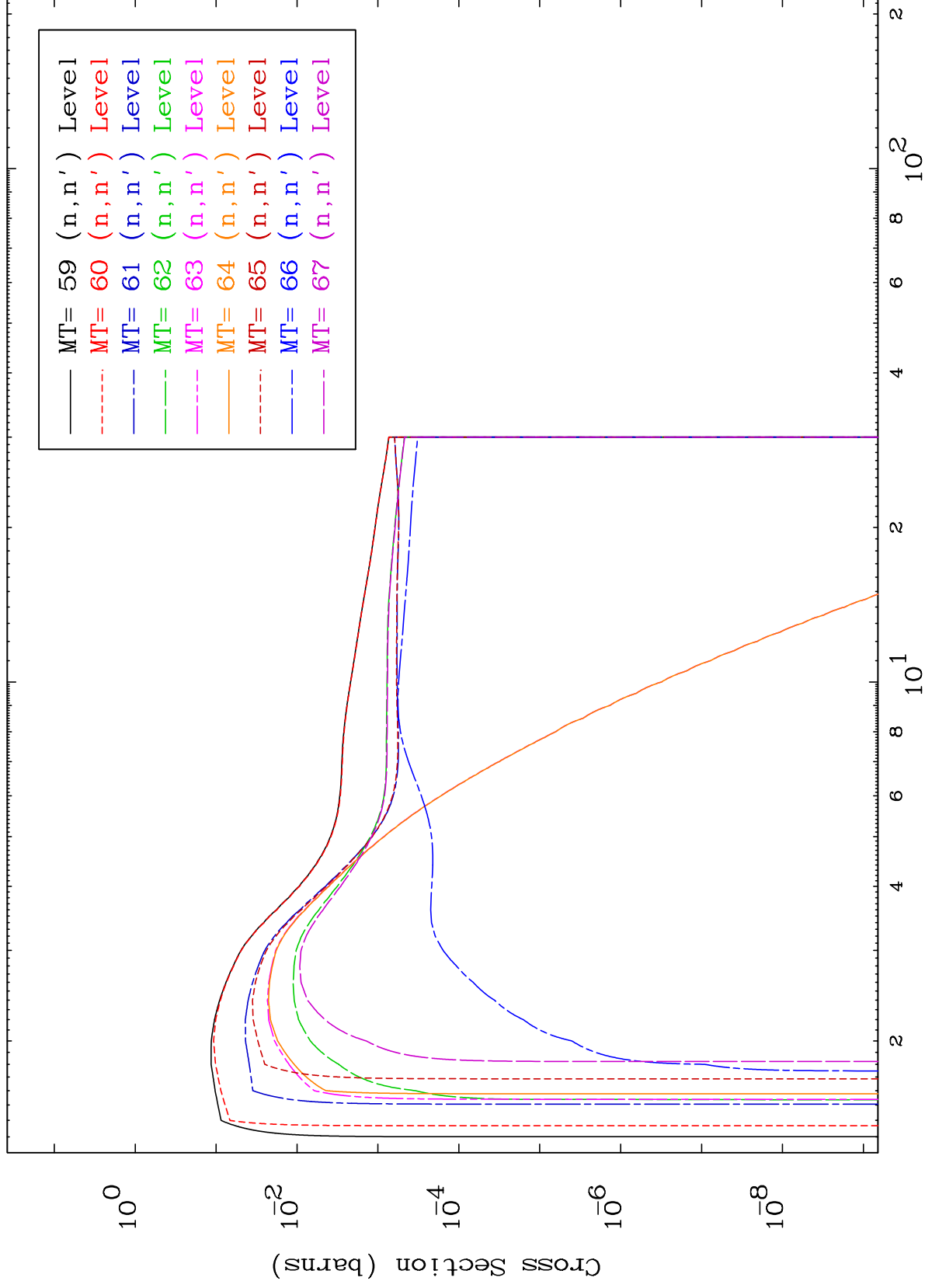


MAT 7001

(n,n') Level

70-Yb-160

293 Kelvin Cross Sections



8

Incident Energy (MeV)

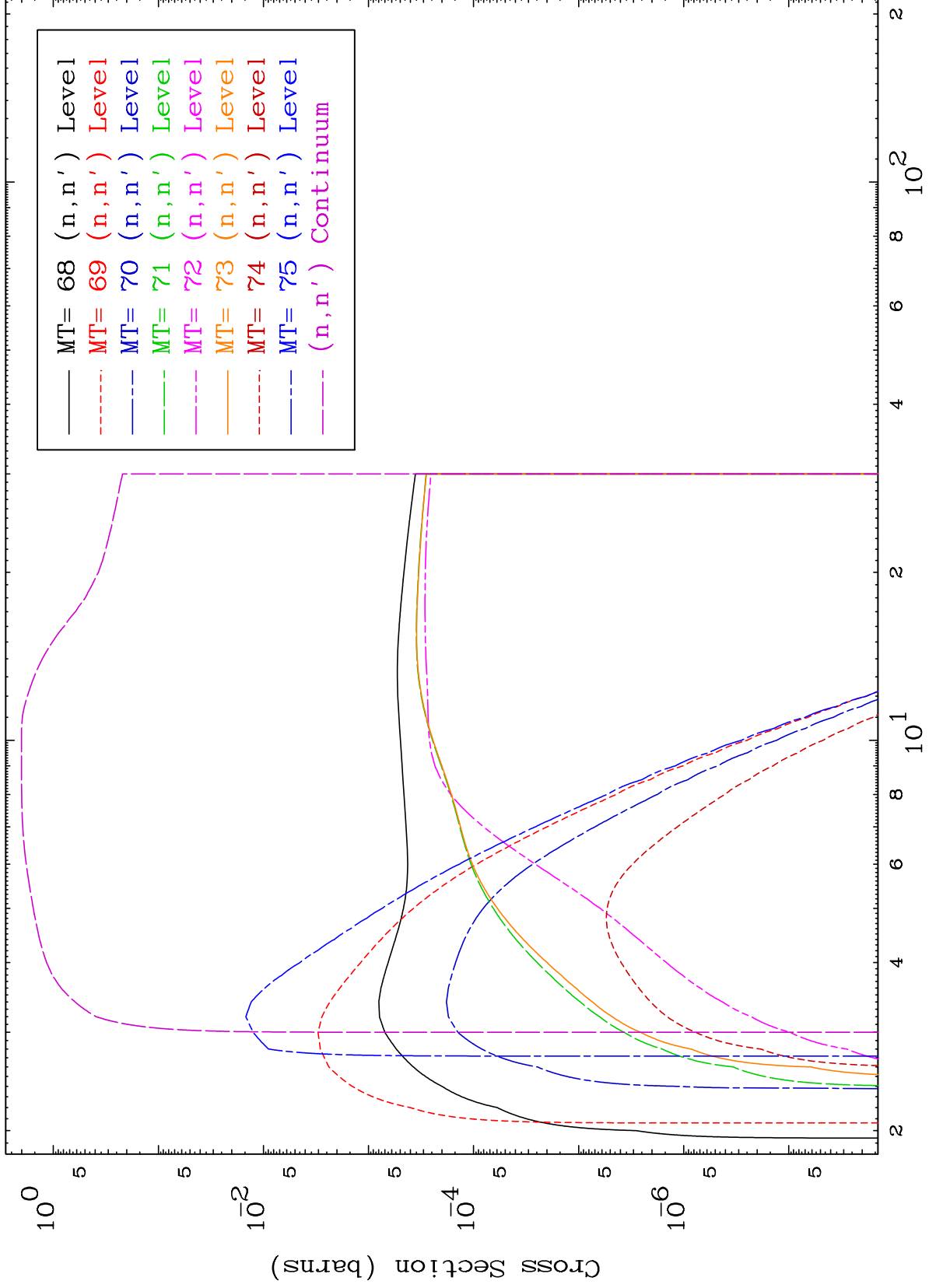
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MAT 7001

(n,n') Level

70-Yb-160

293 Kelvin Cross Sections



9

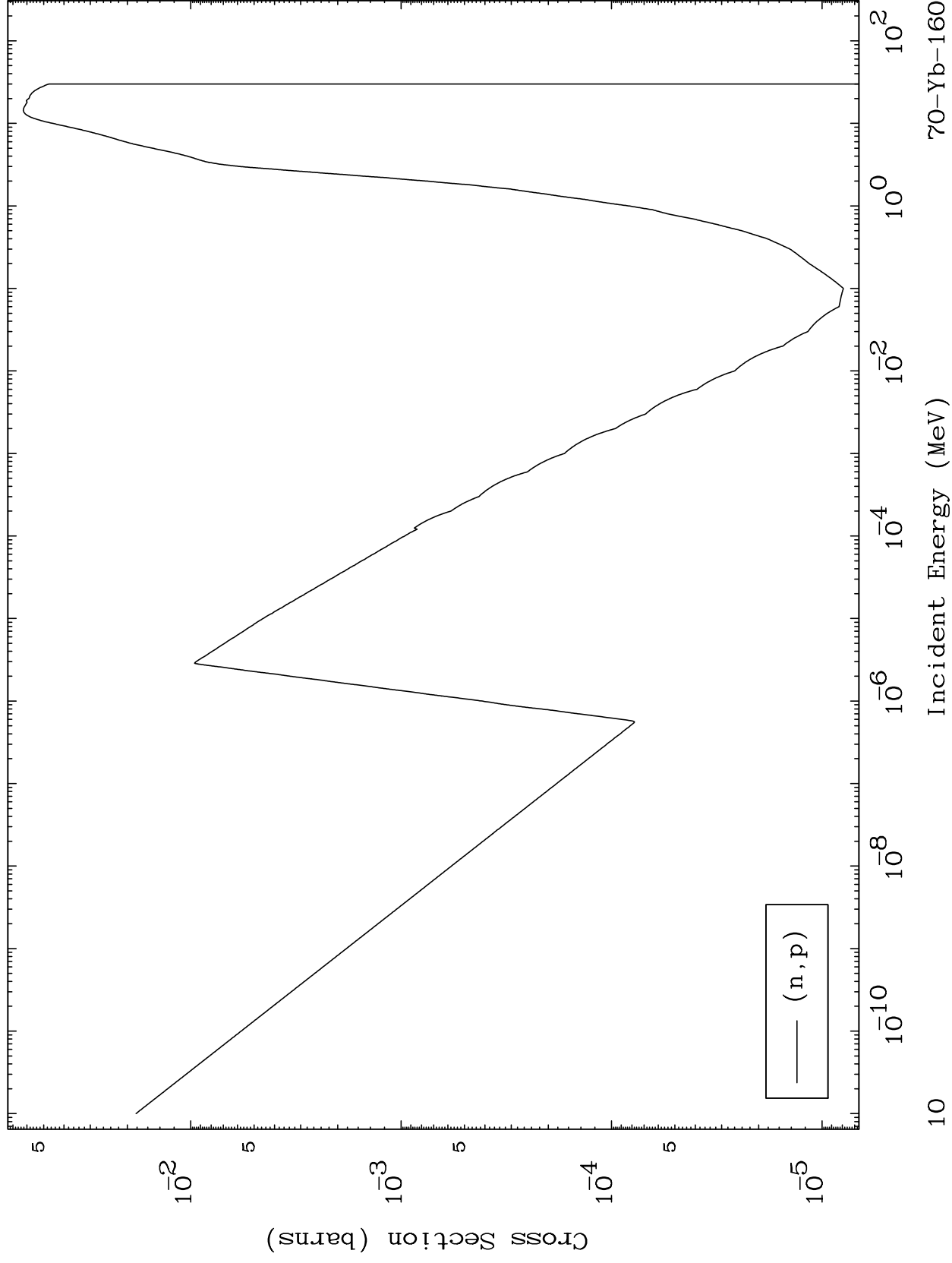
Incident Energy (MeV)

70-Yb-160

MAT 7001

(n,p) Levels
293 Kelvin Cross Sections

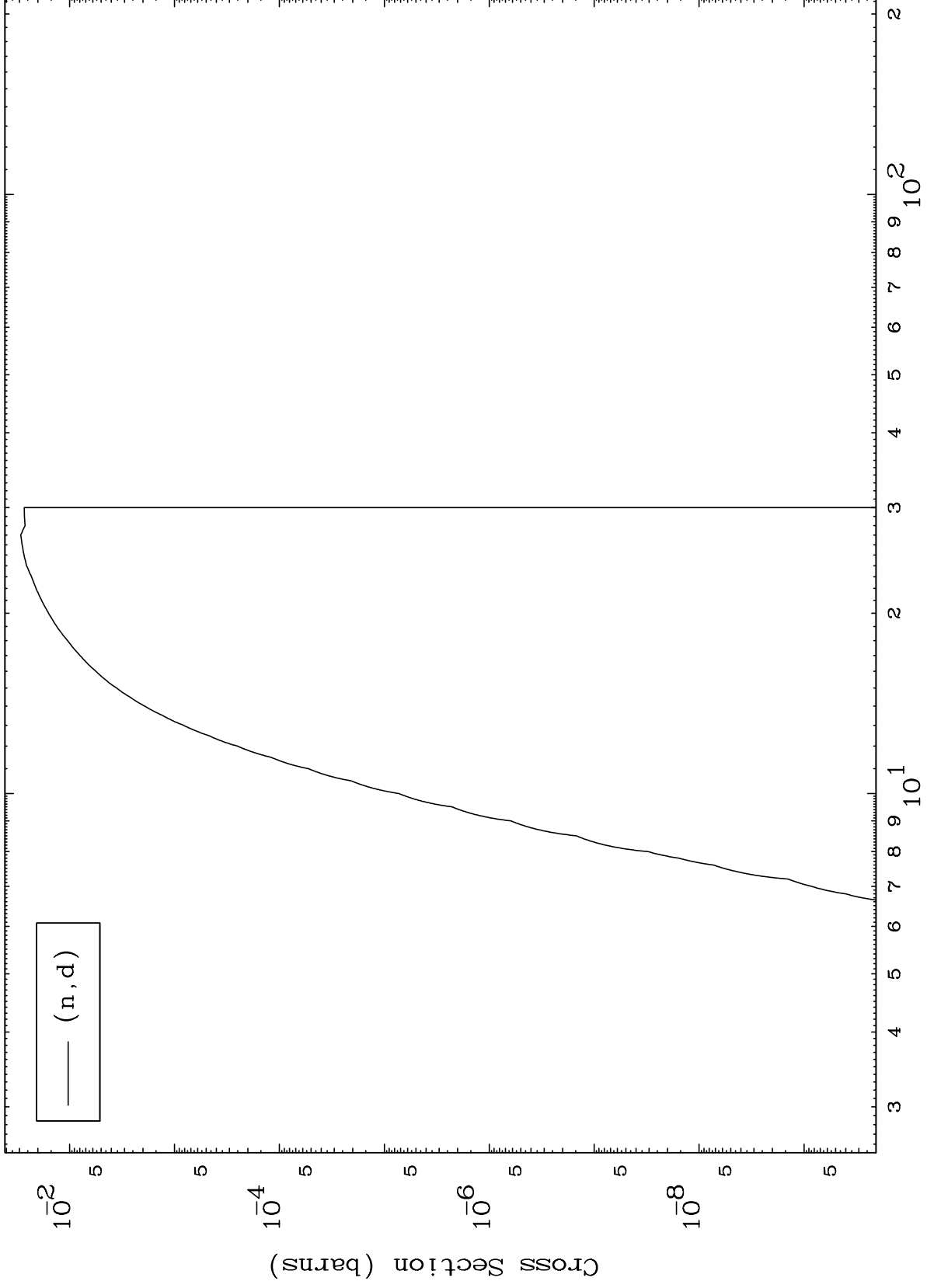
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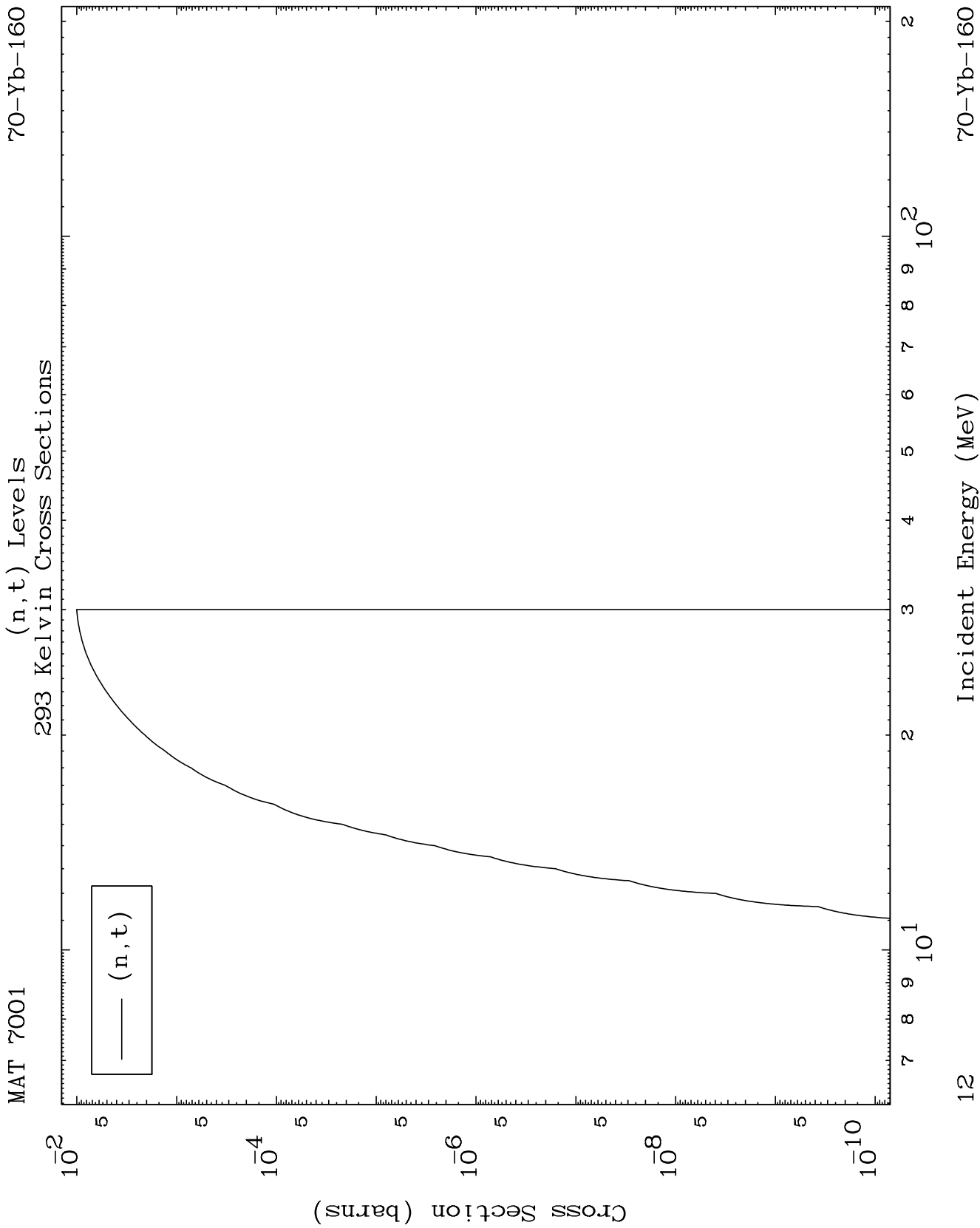


MAT 7001

(n,d) Levels
293 Kelvin Cross Sections

70-Yb-160

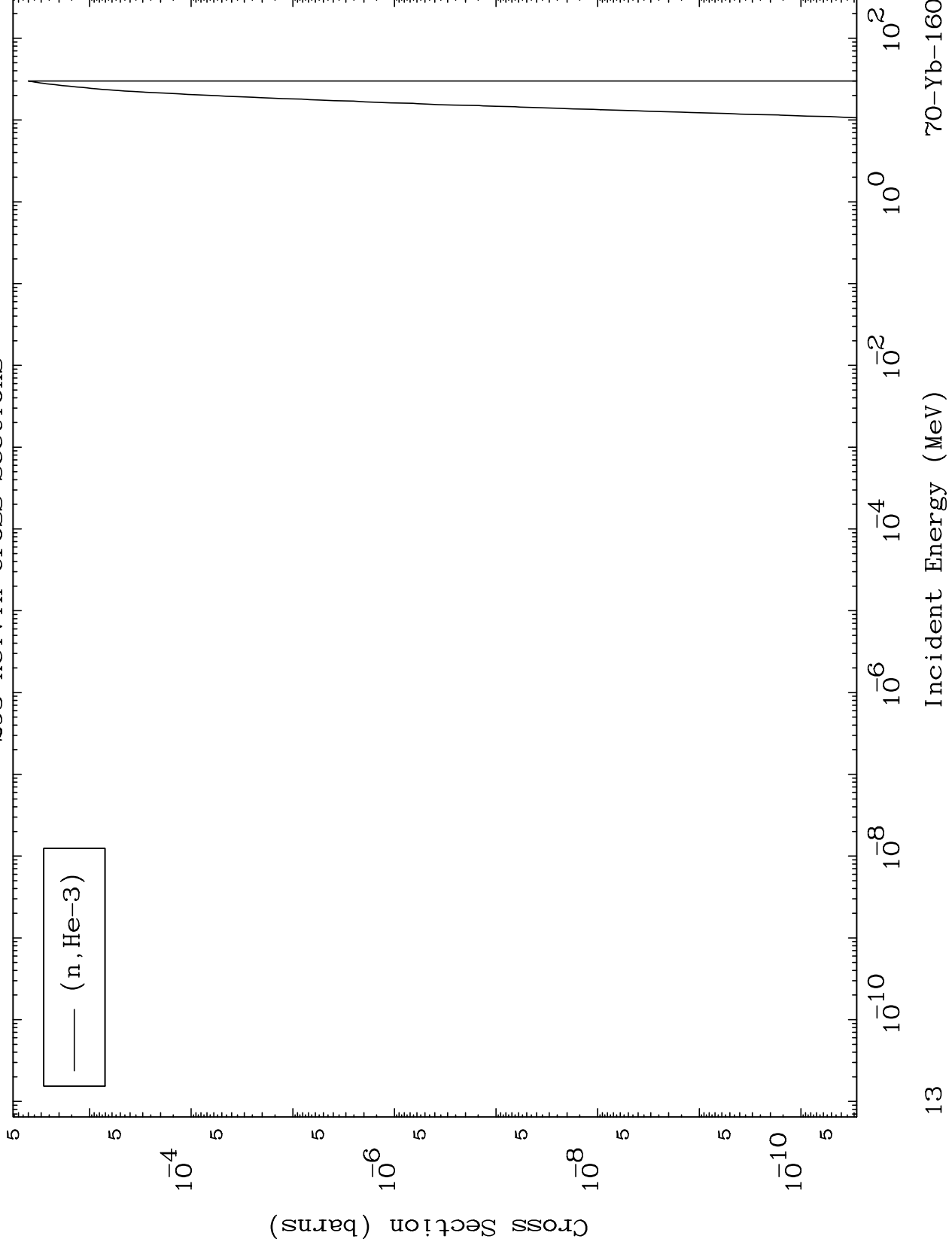




MAT 7001

(n,He3) Levels
293 Kelvin Cross Sections

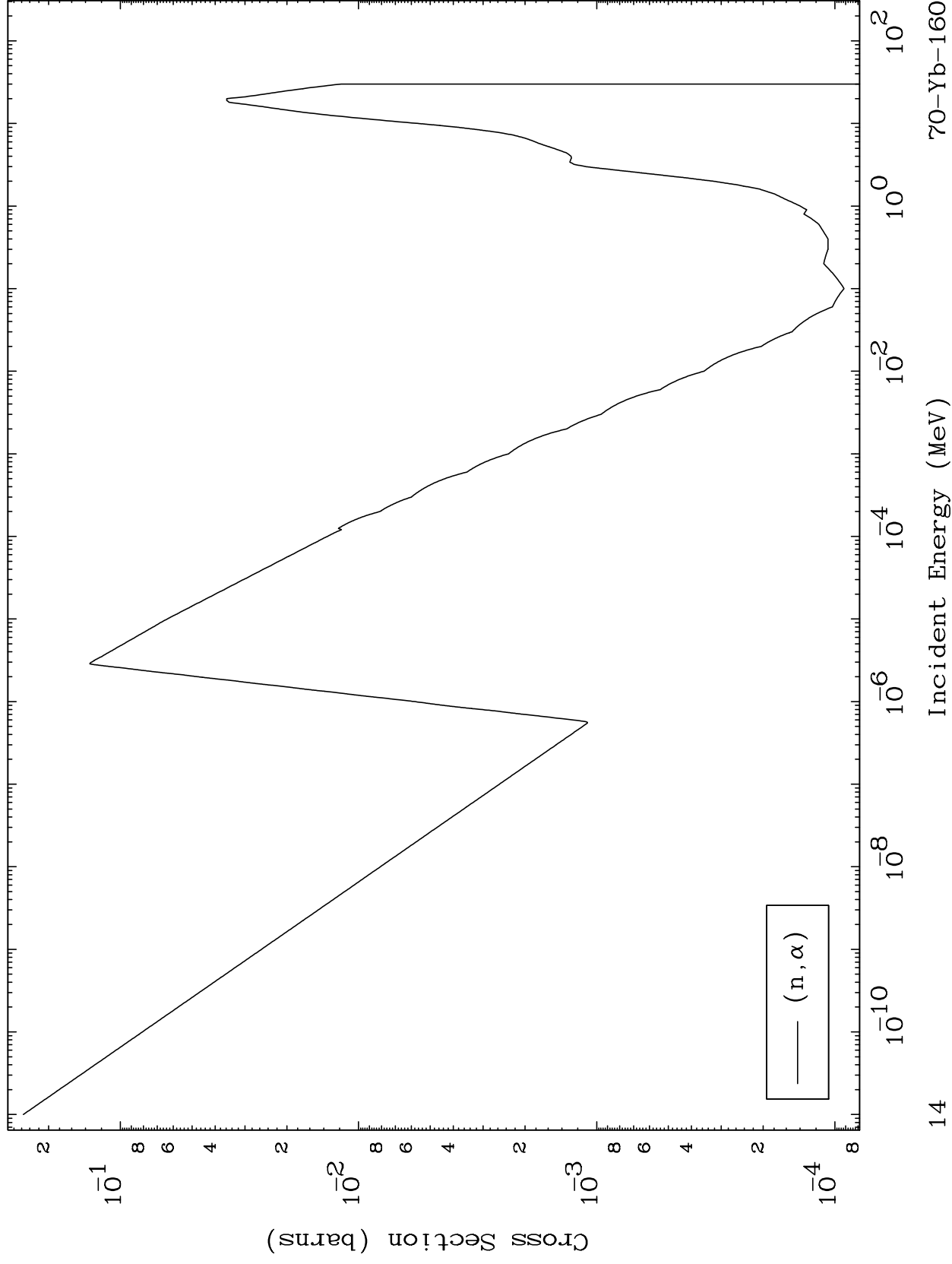
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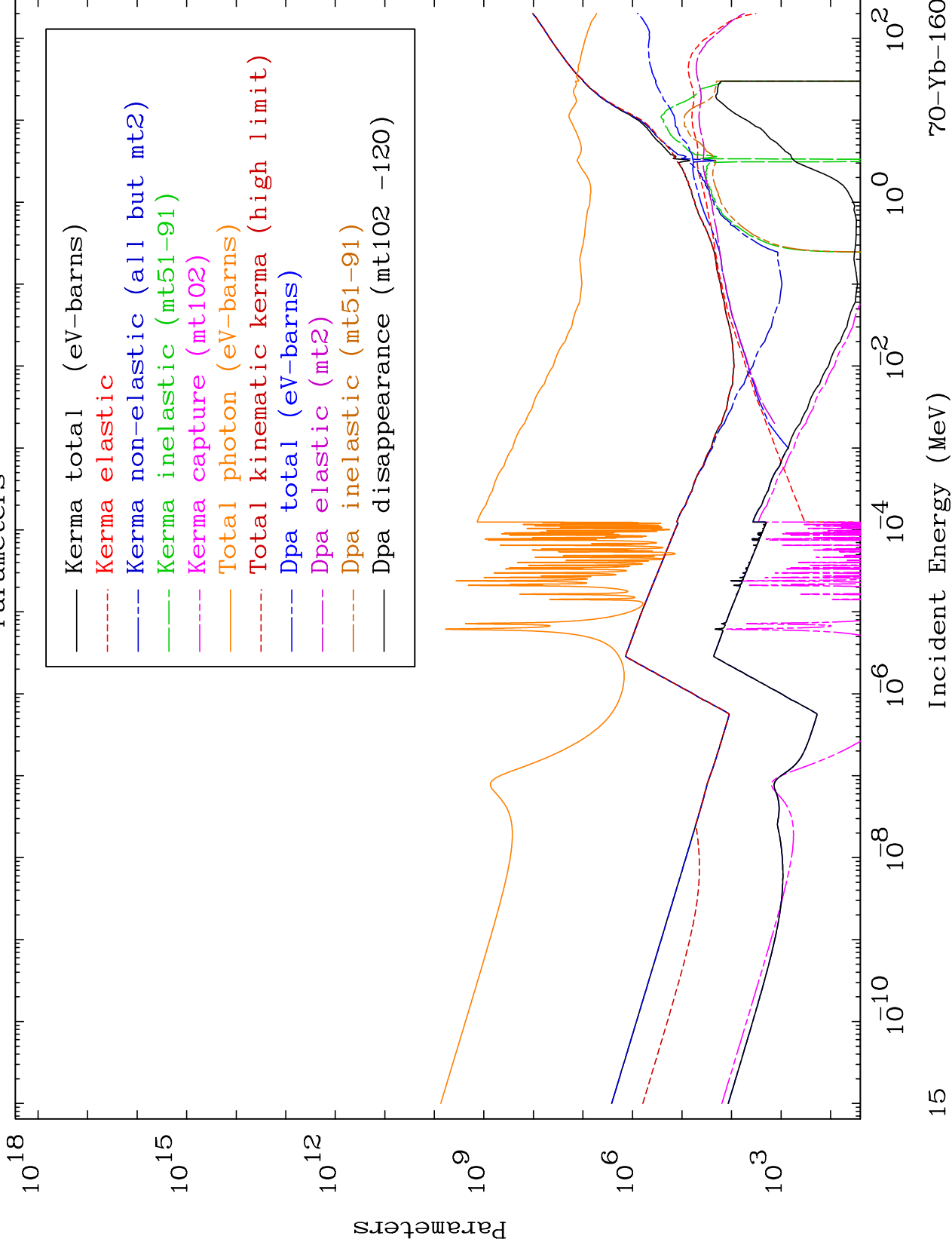


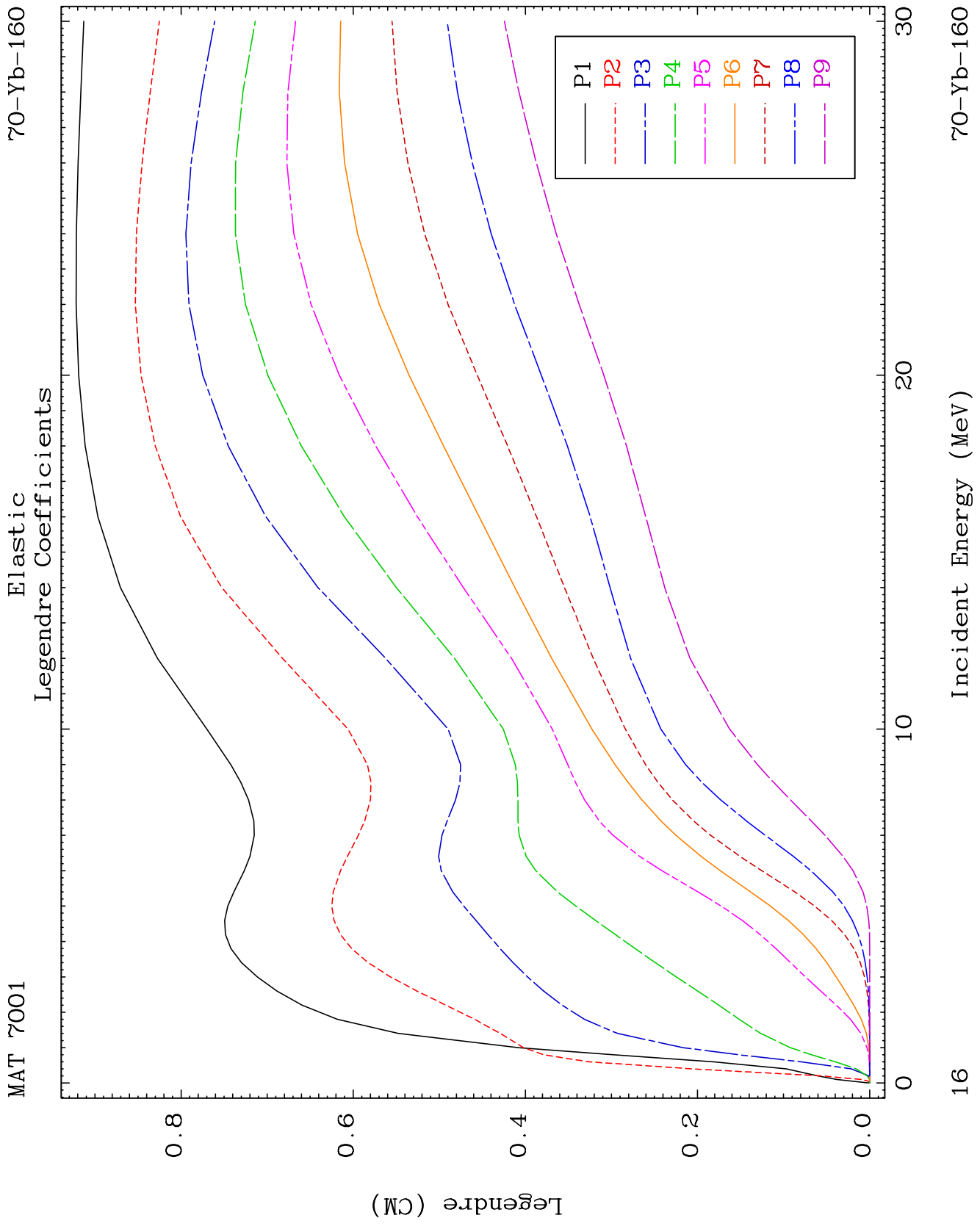
MAT 7001

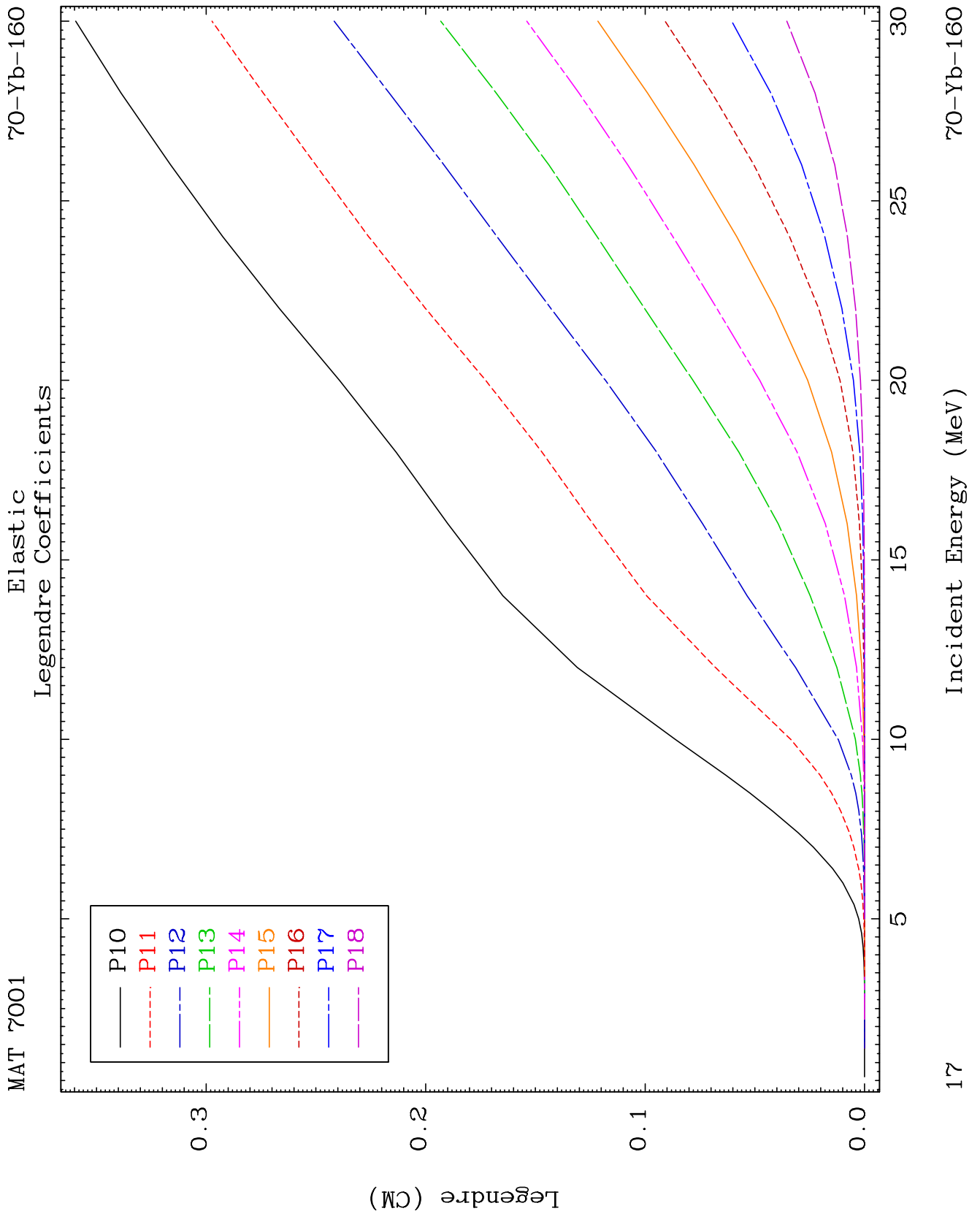
(n, α) Levels
293 Kelvin Cross Sections

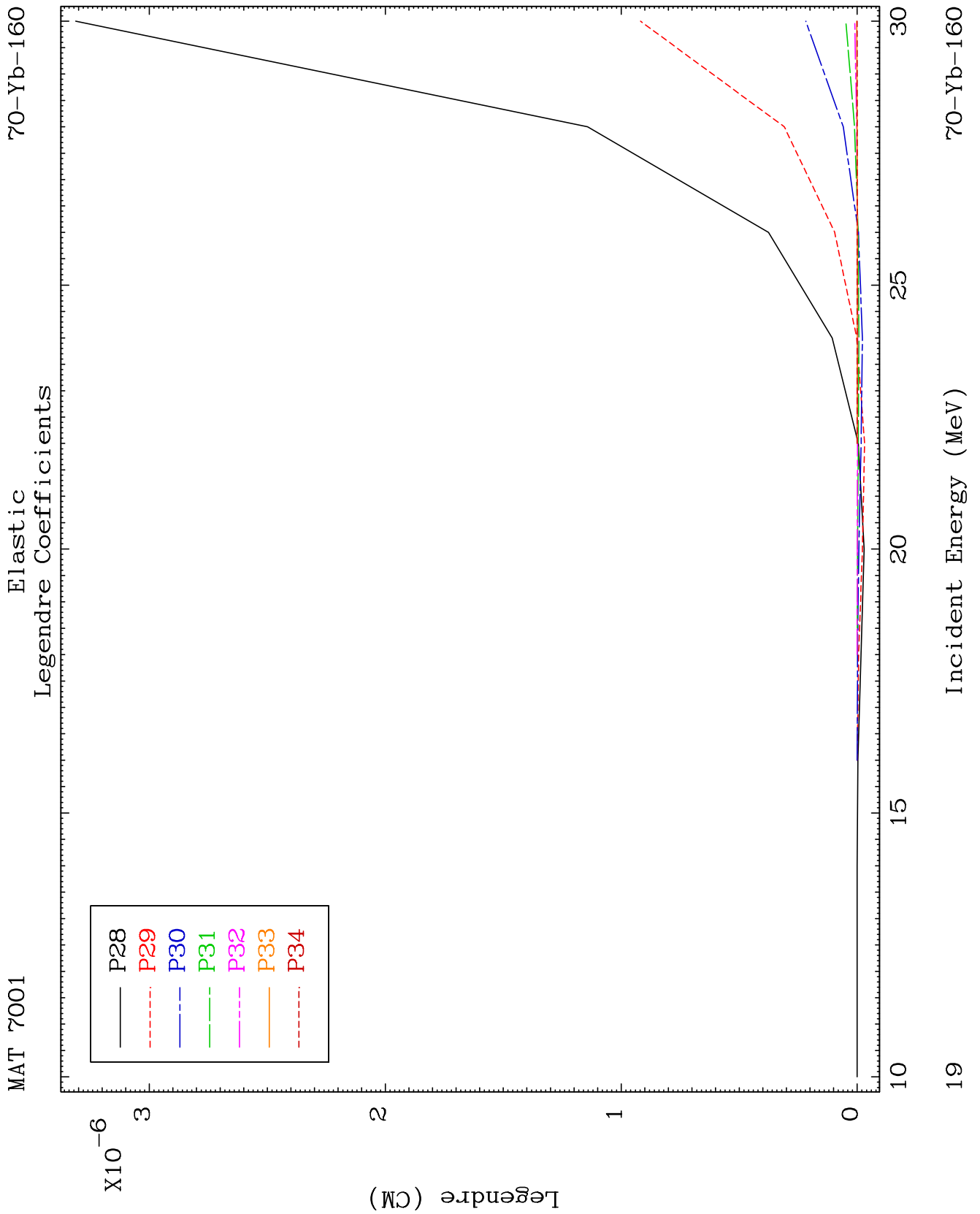
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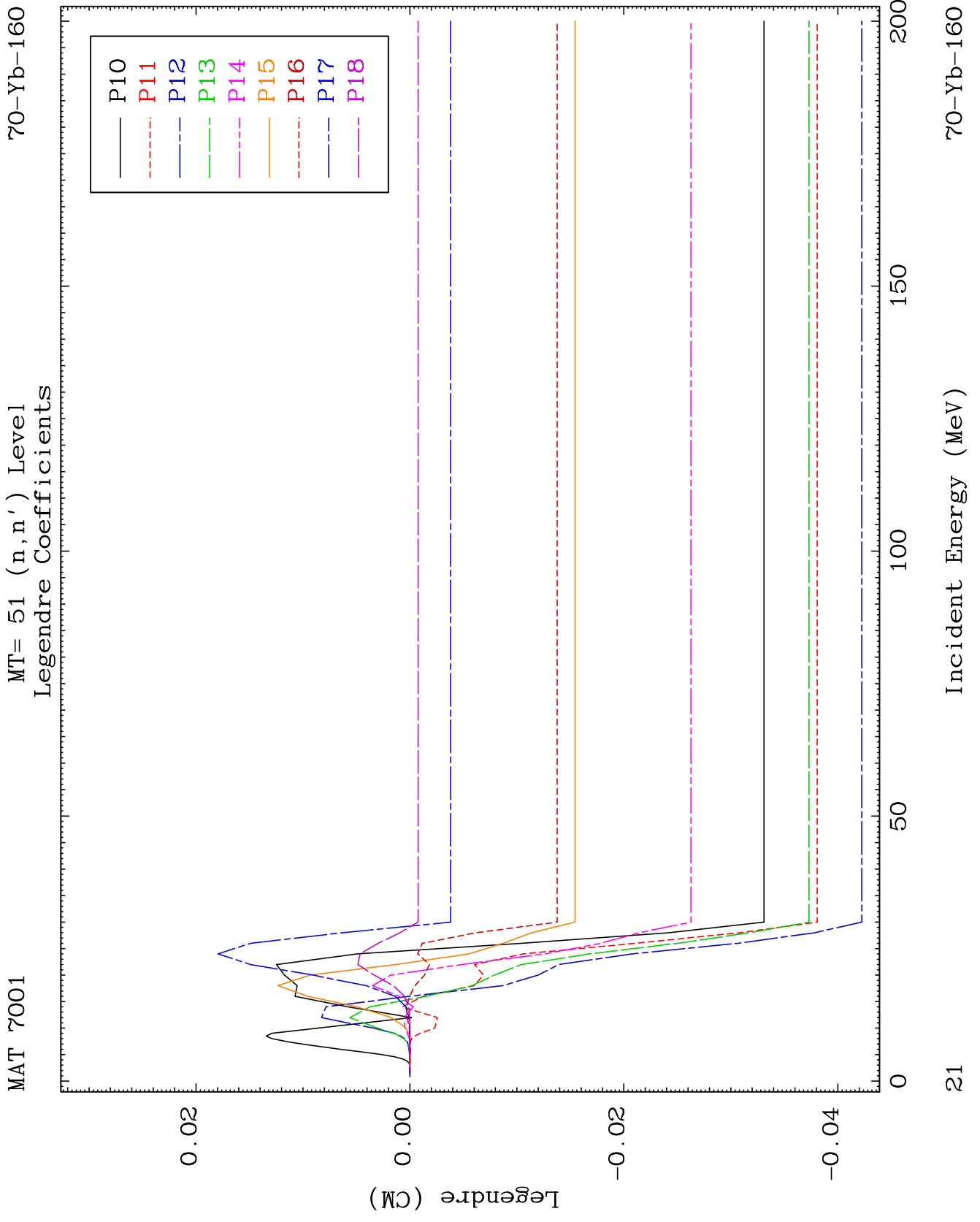


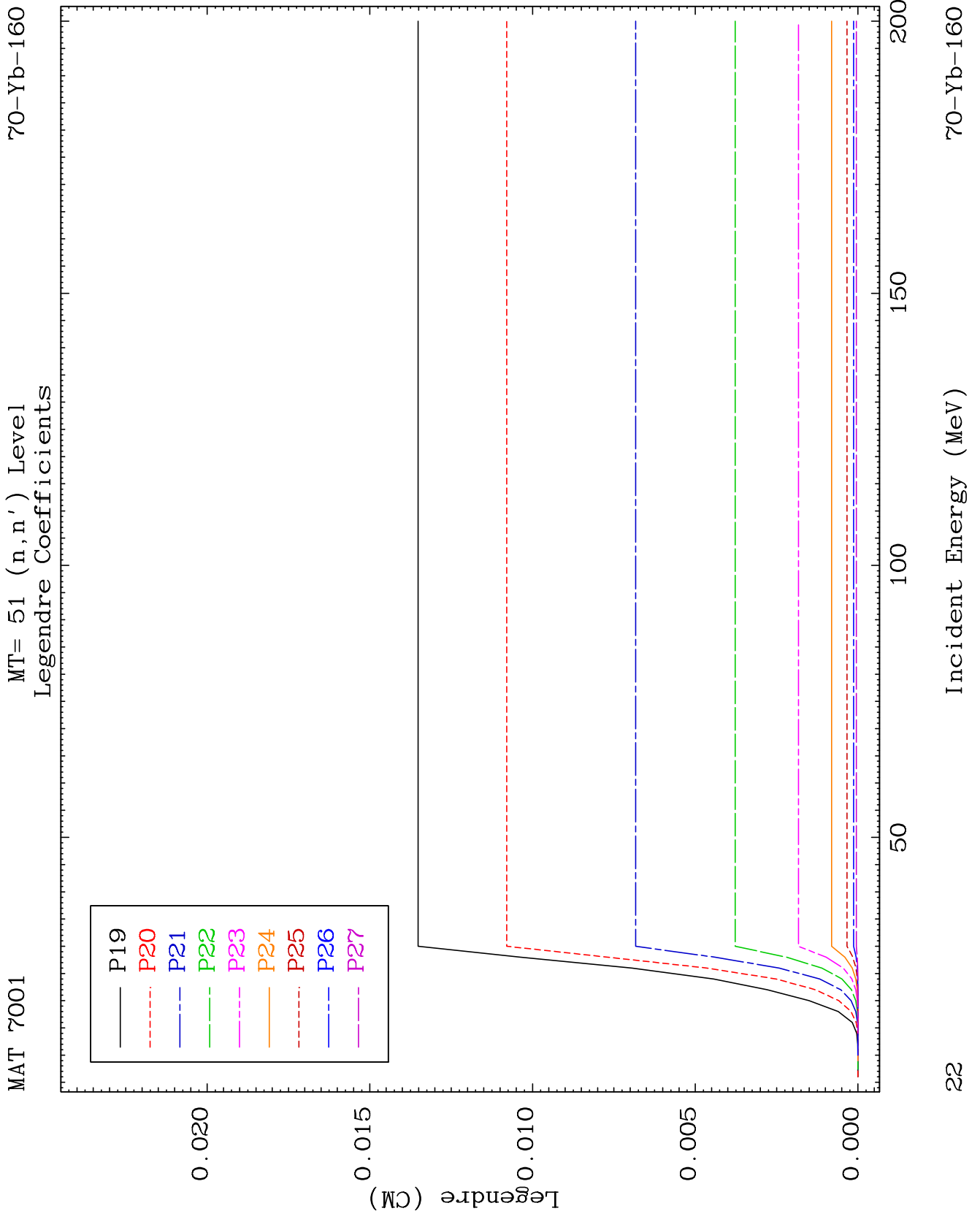


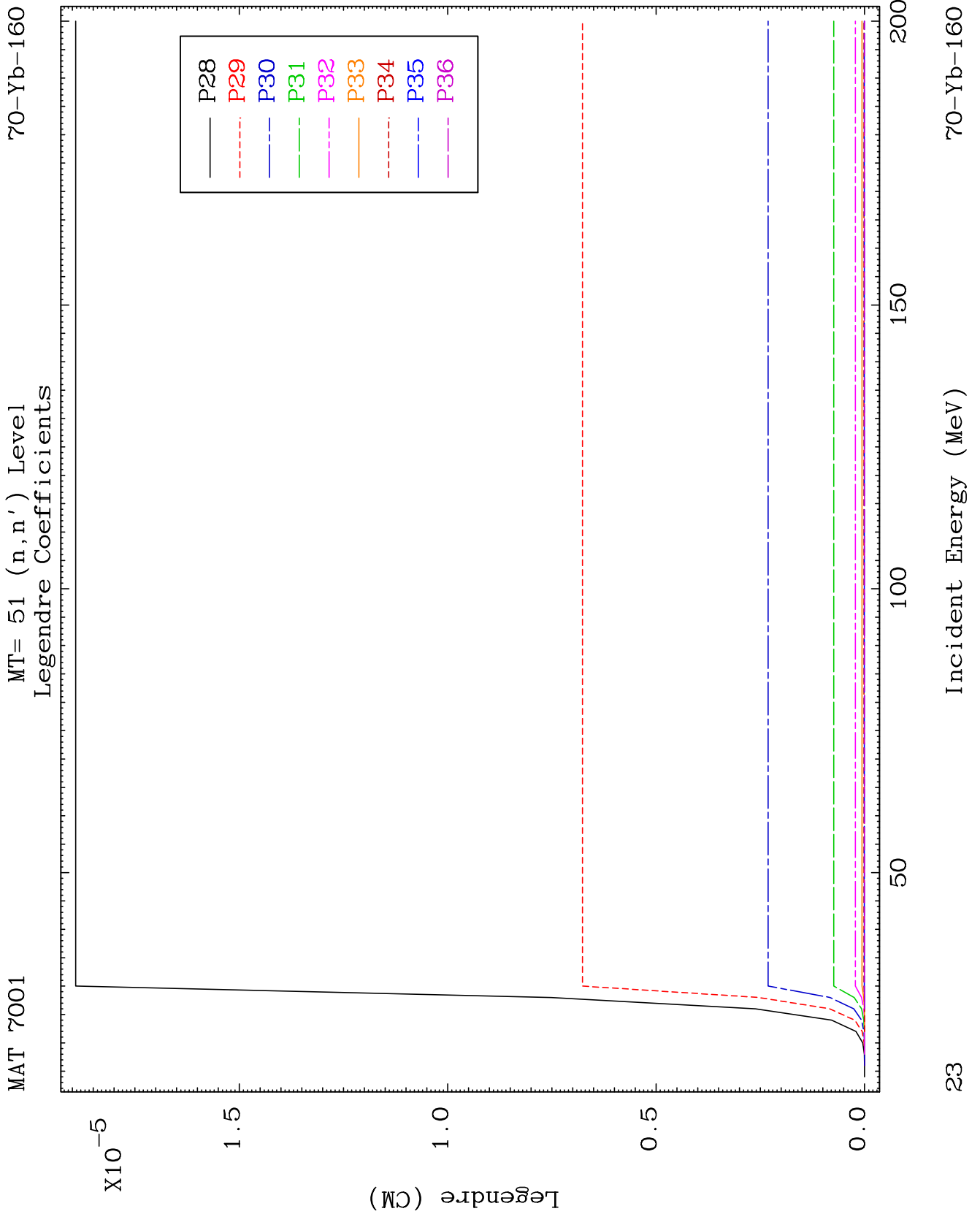


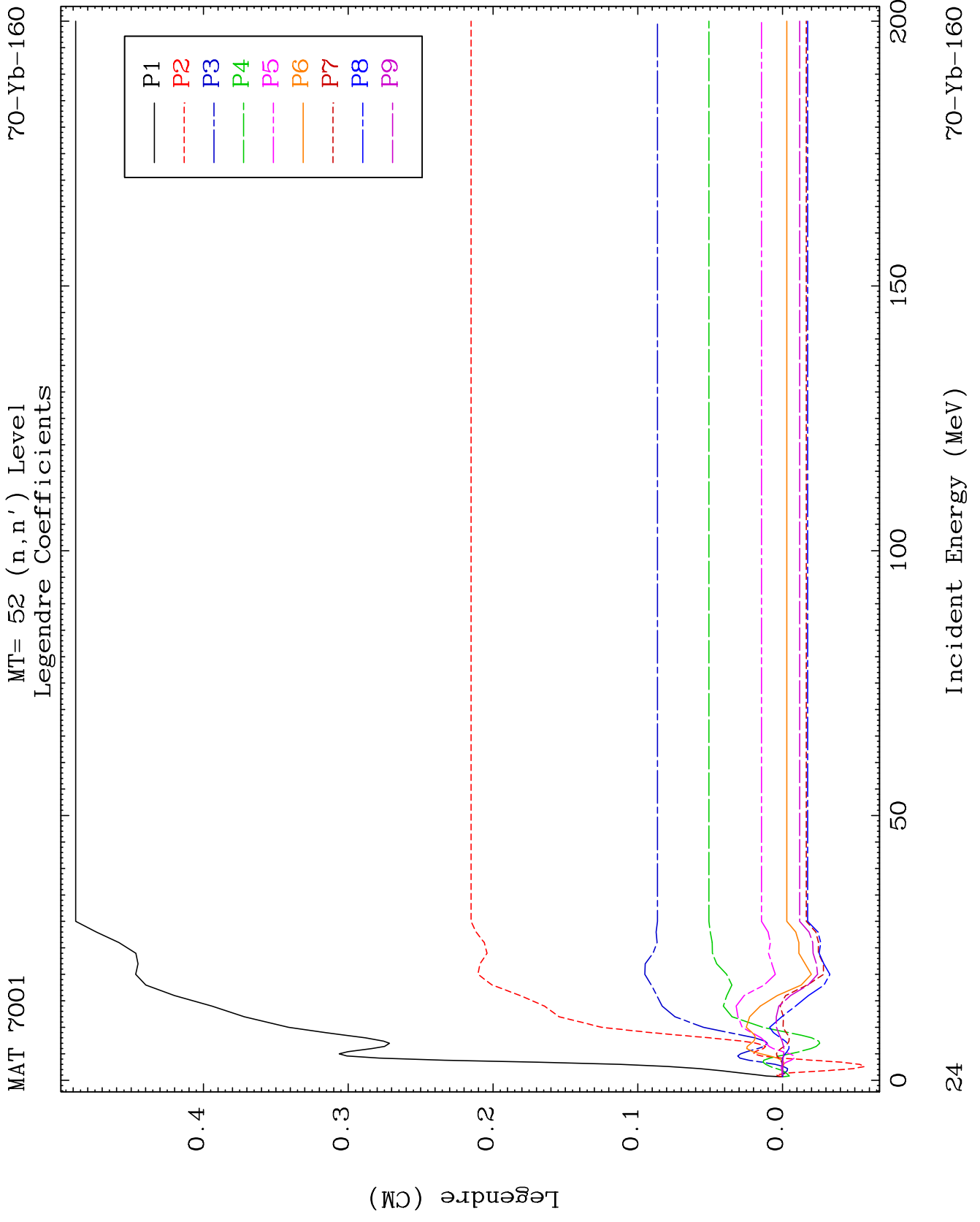








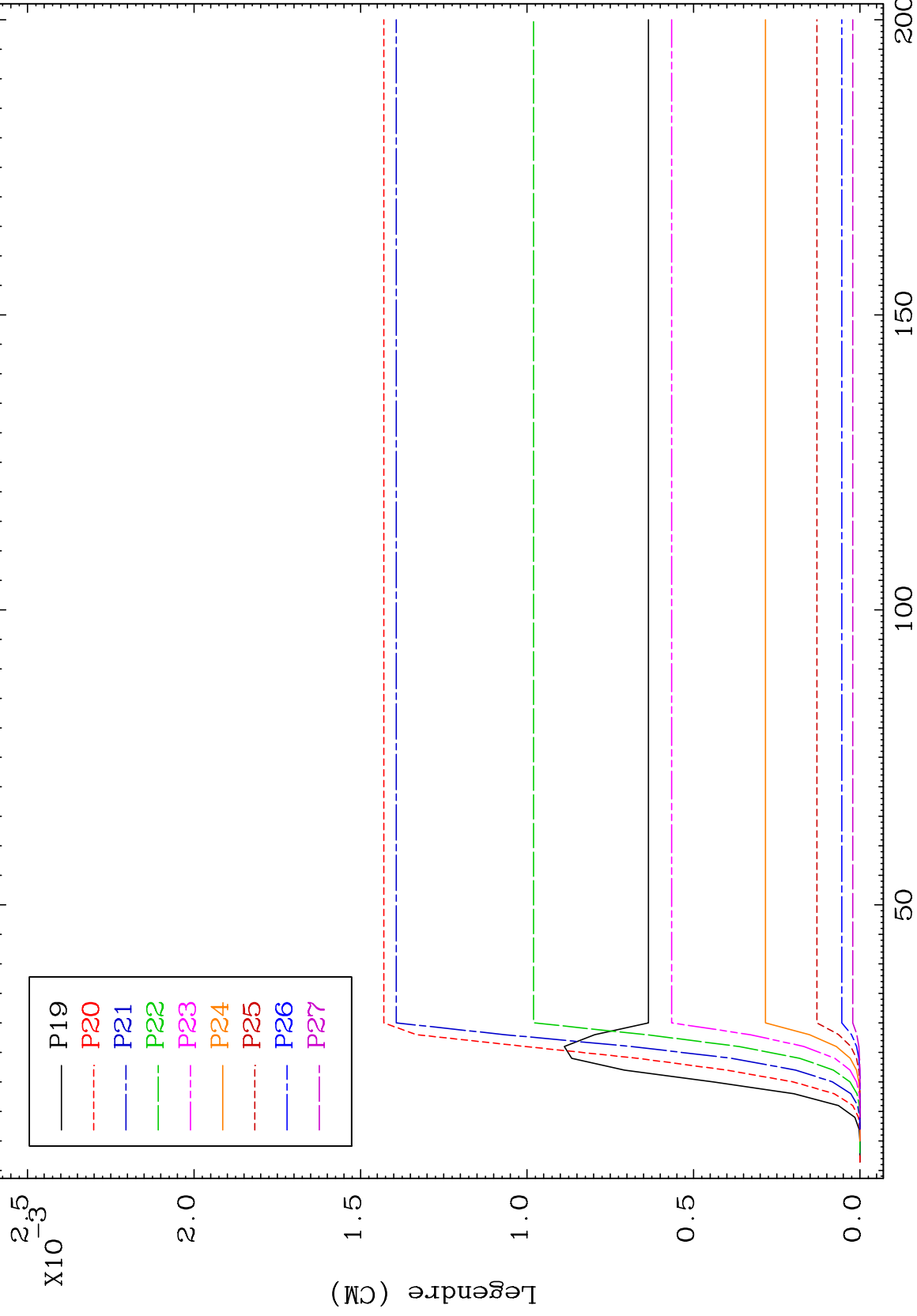




MAT 7001

MT= 52 (n,n') Level
Legendre Coefficients

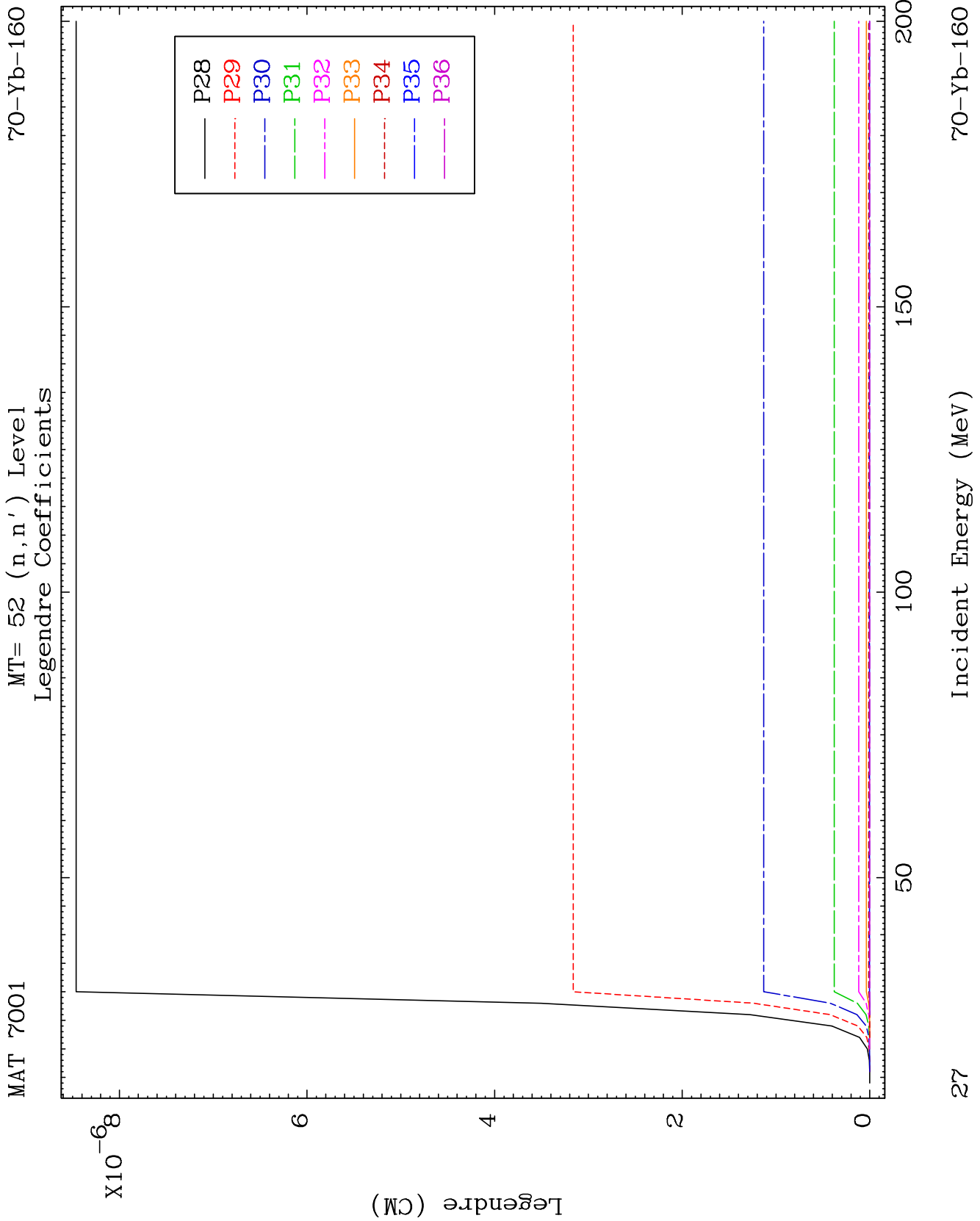
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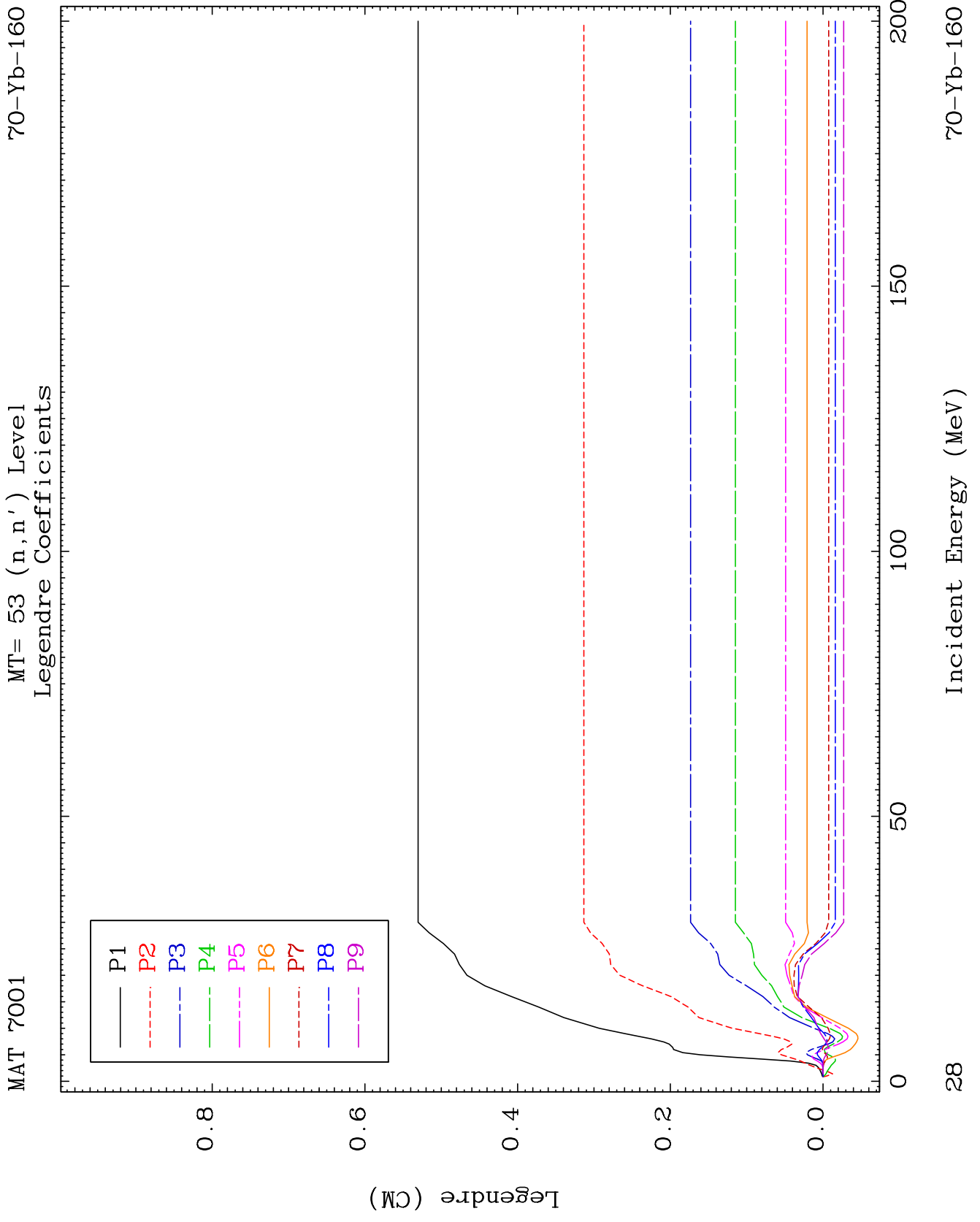


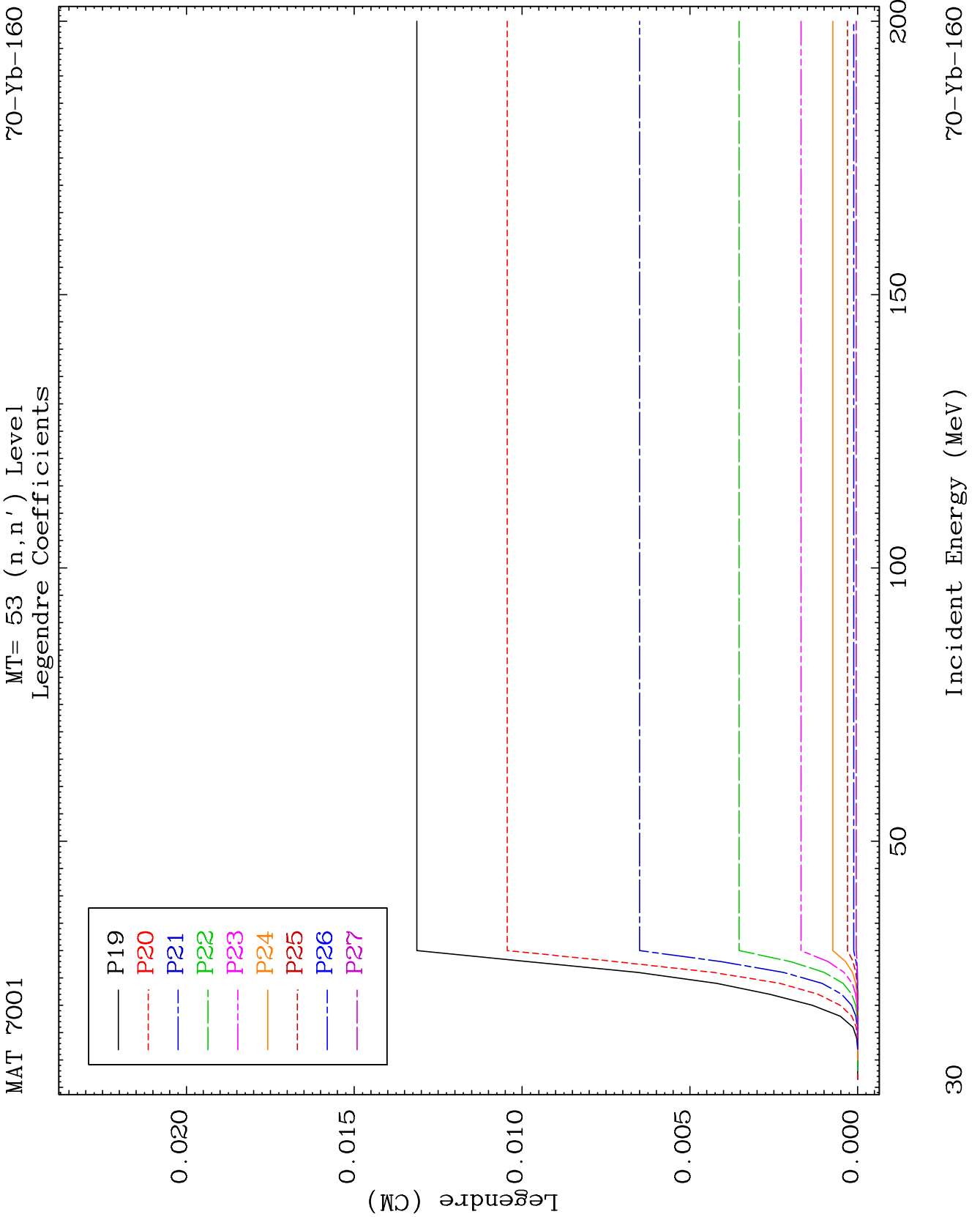
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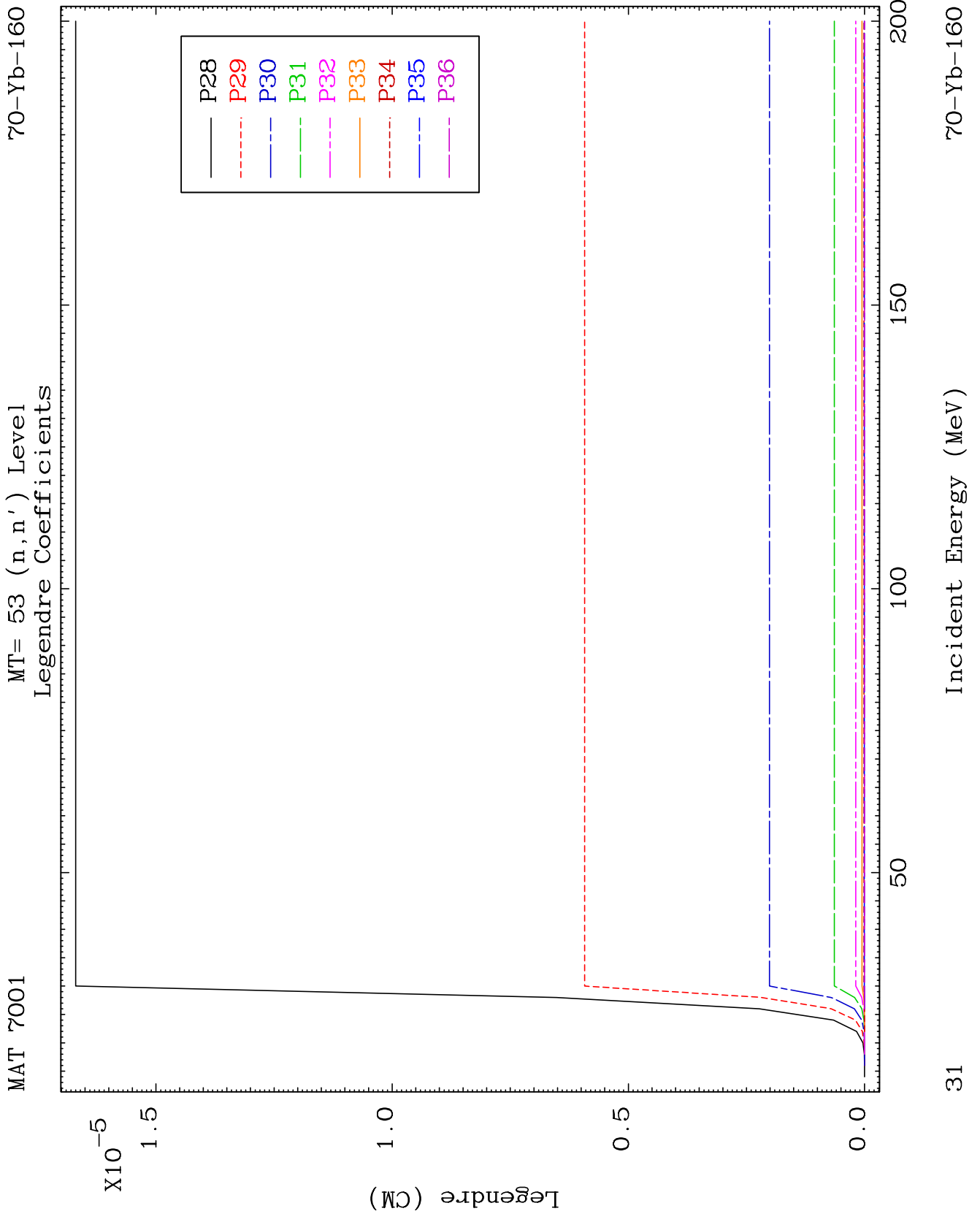
Incident Energy (MeV)

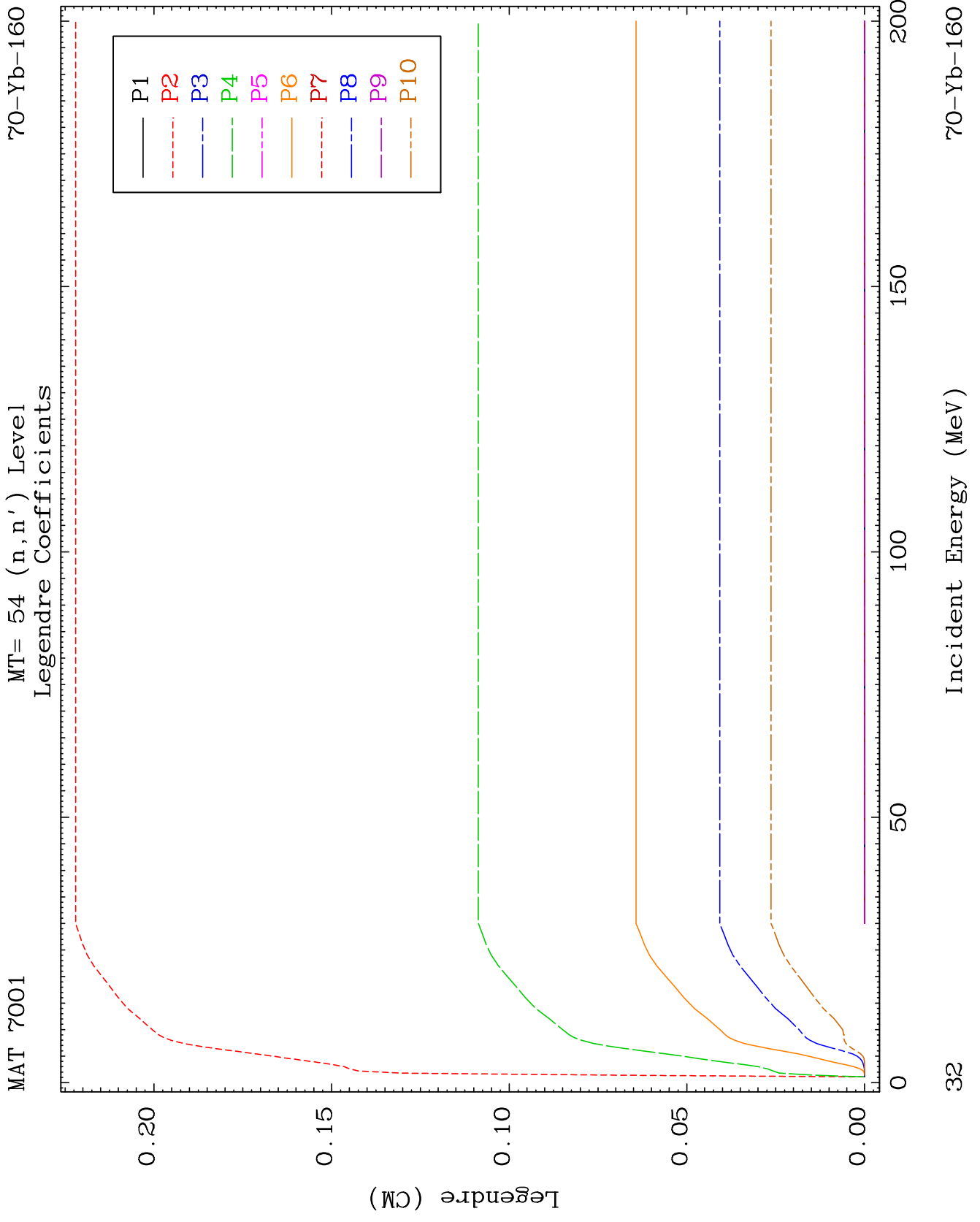
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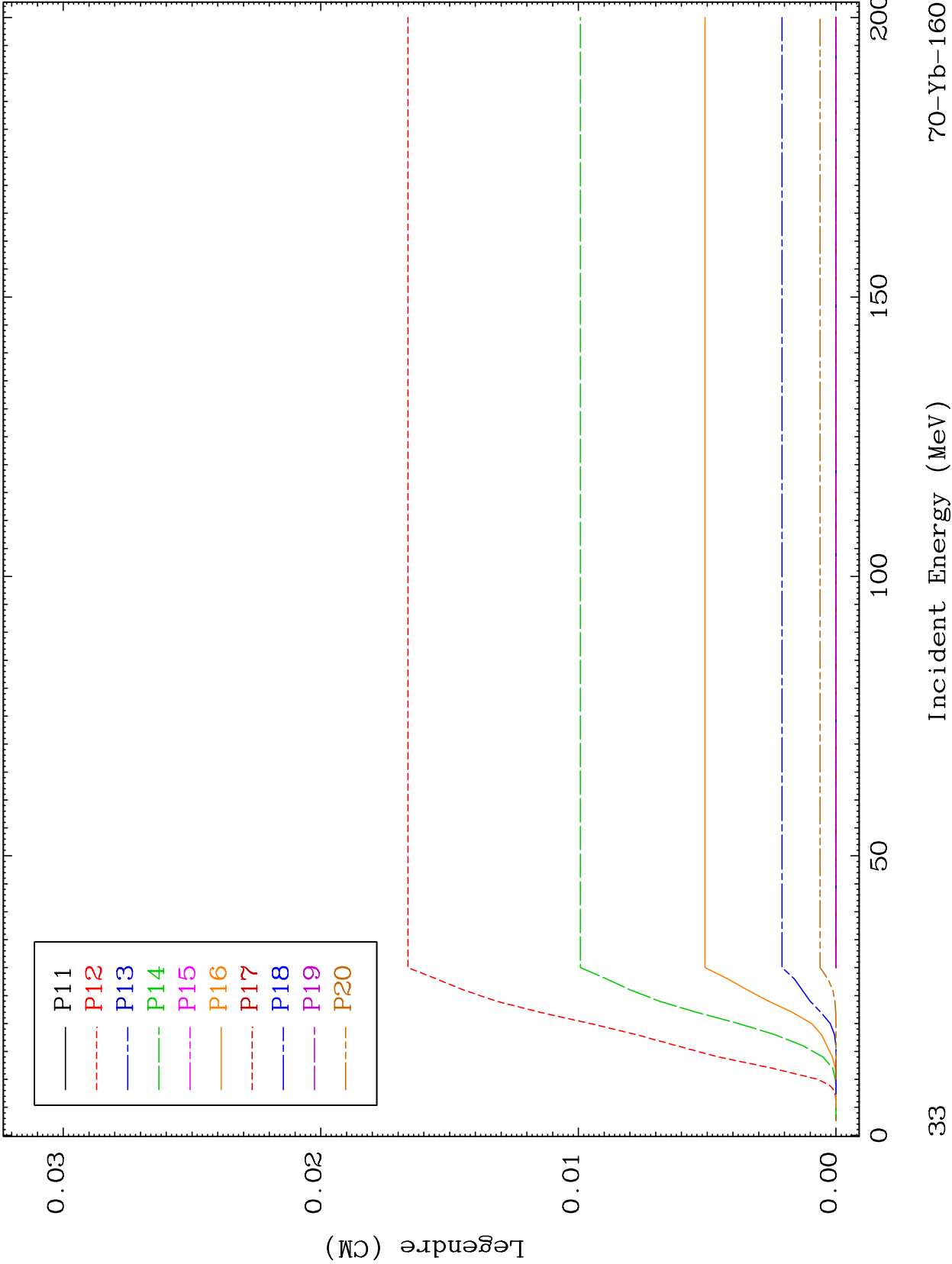


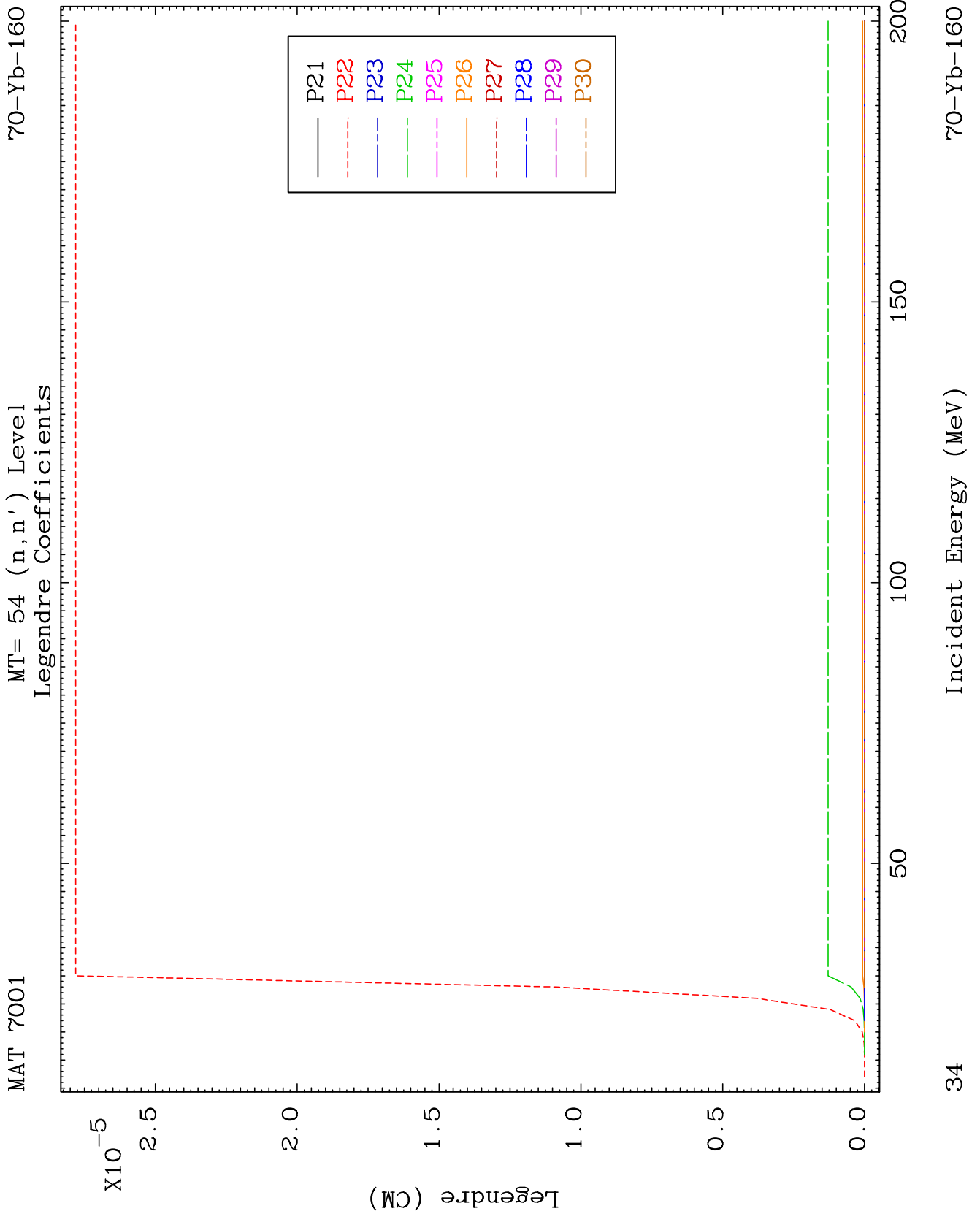


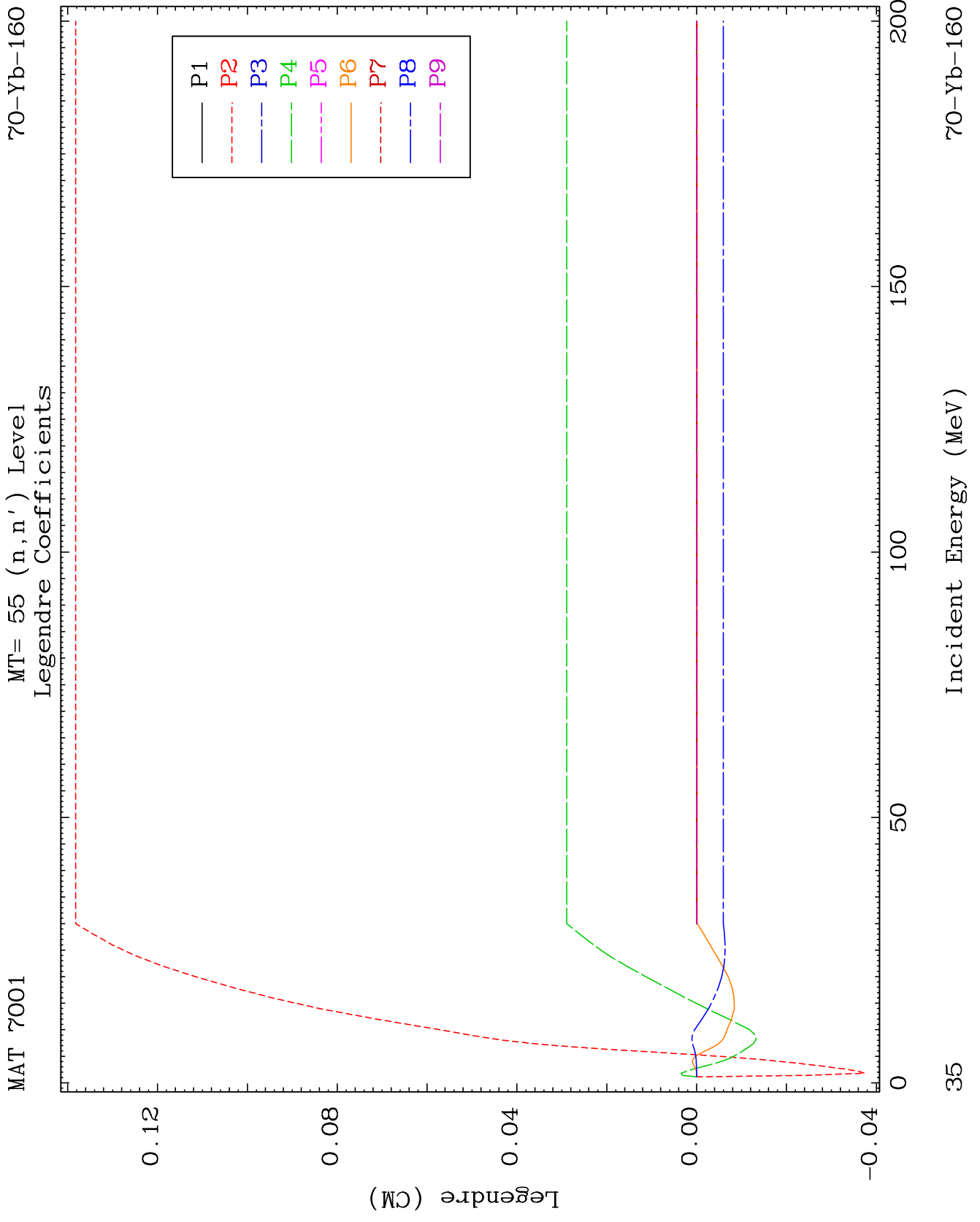


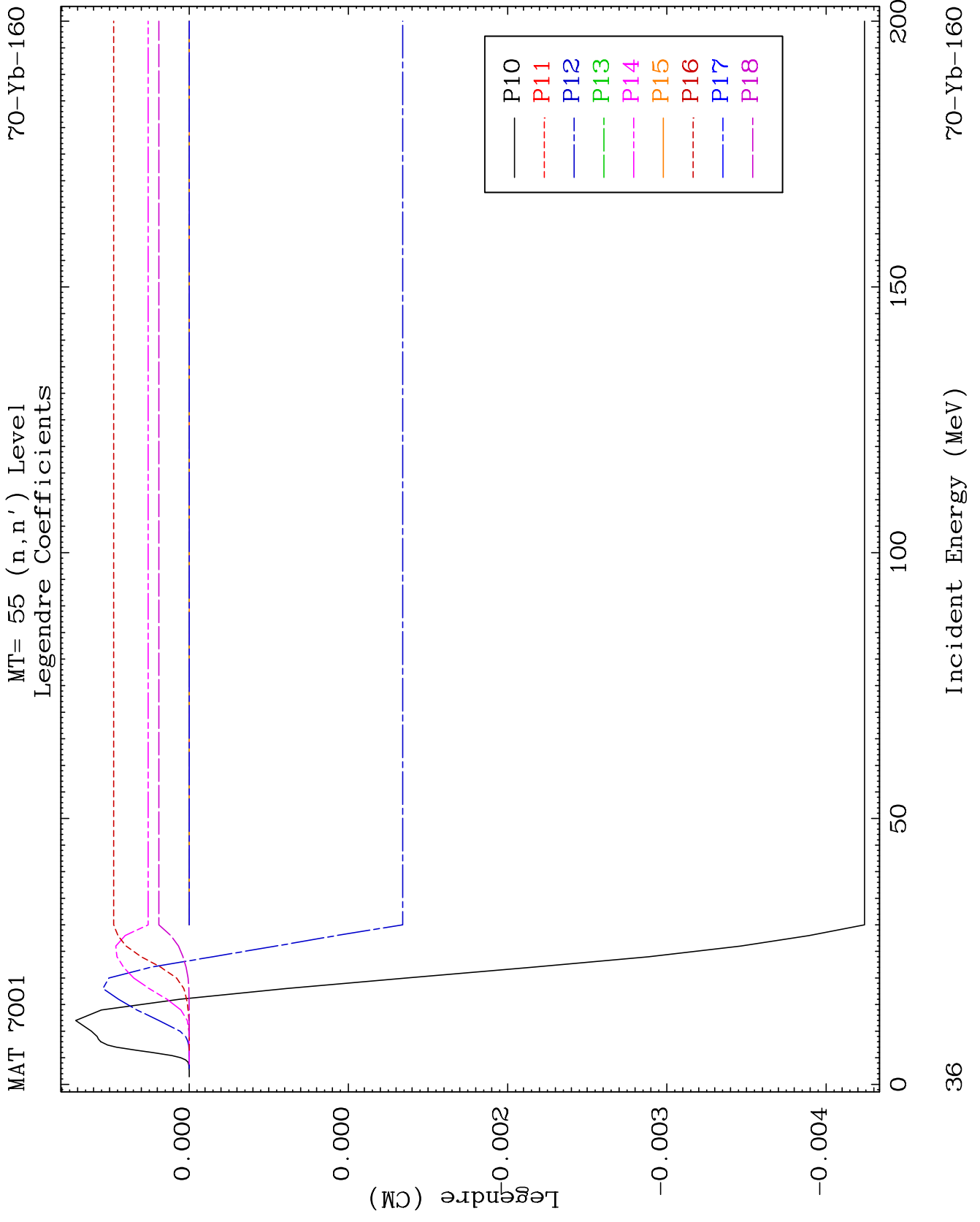


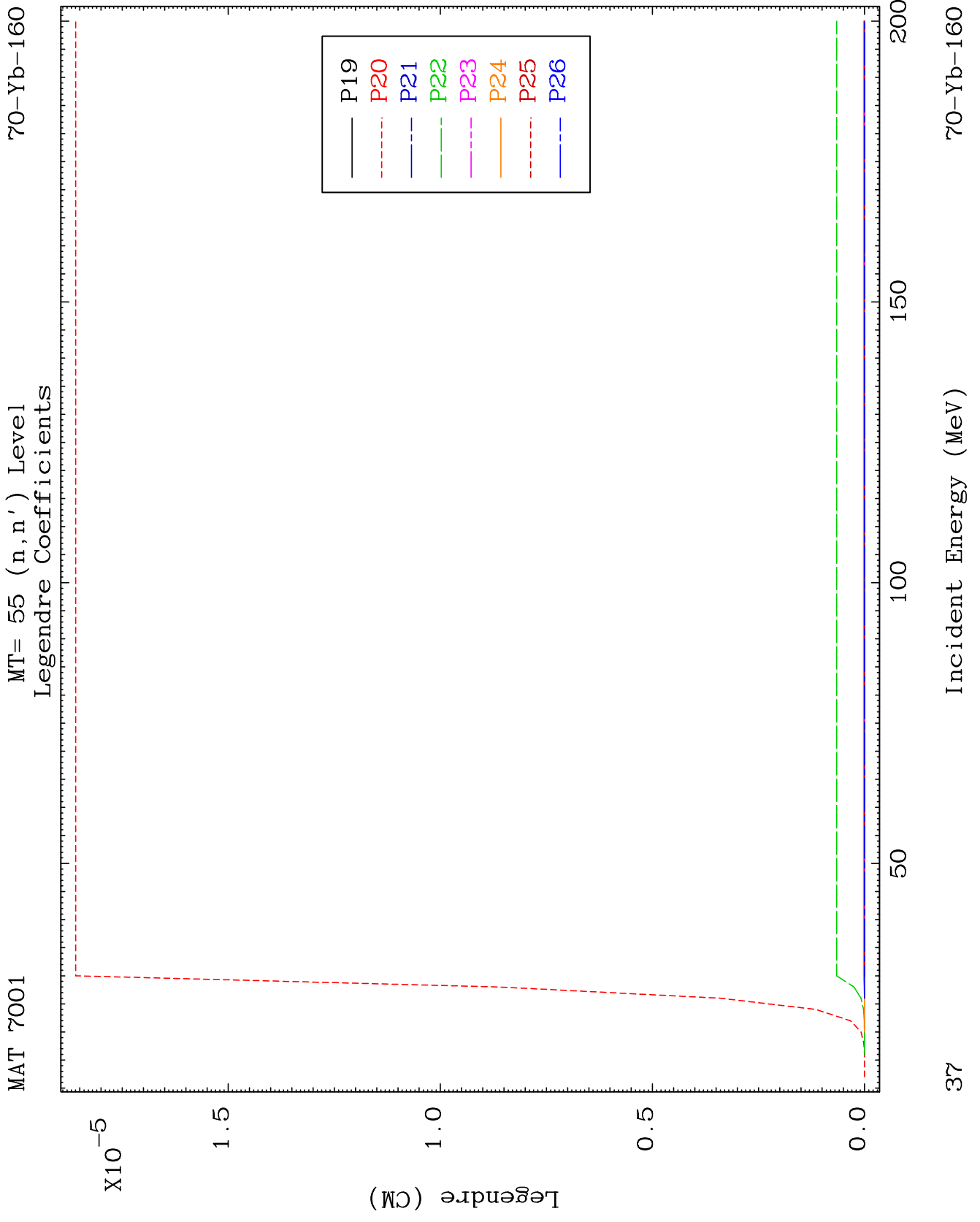


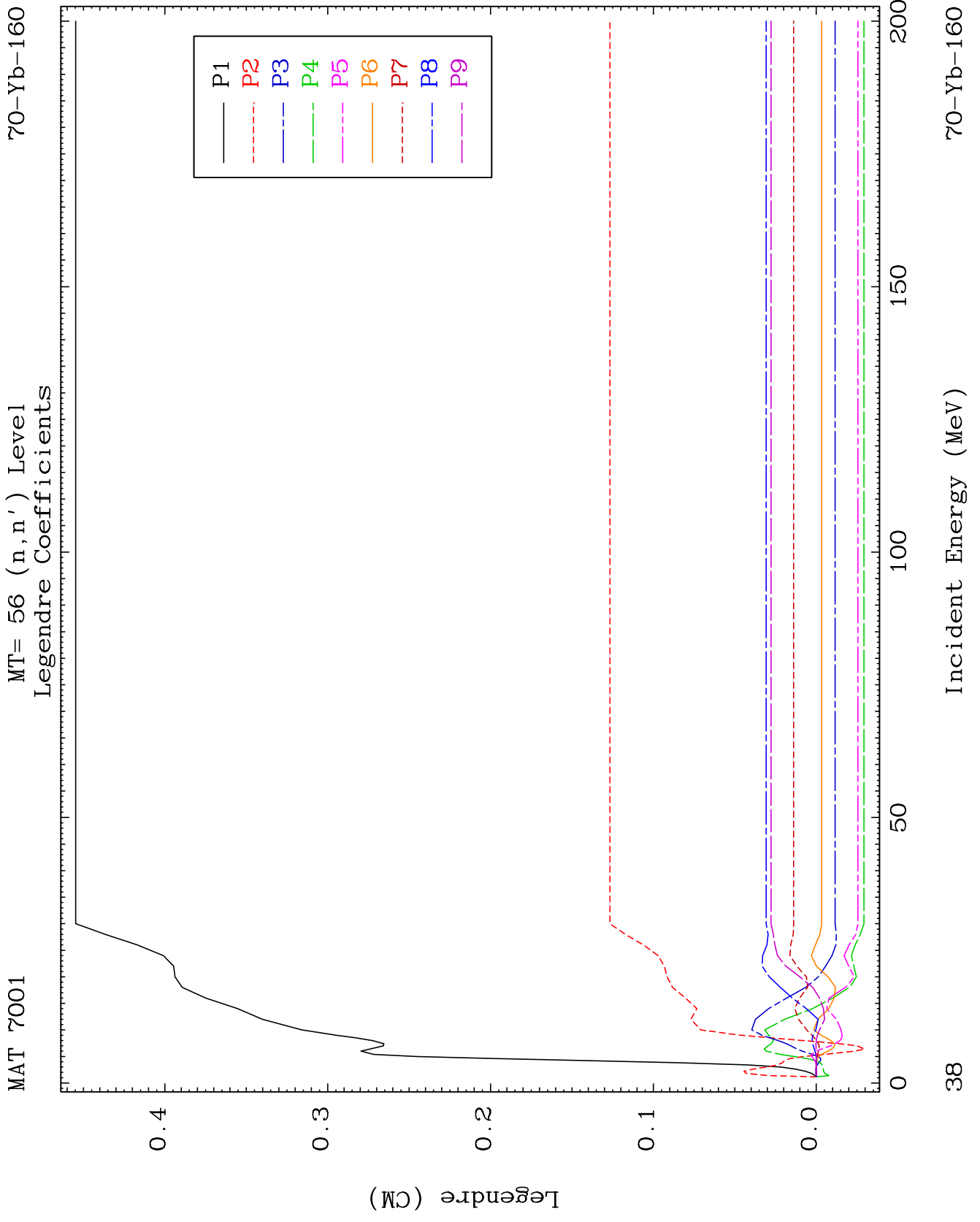


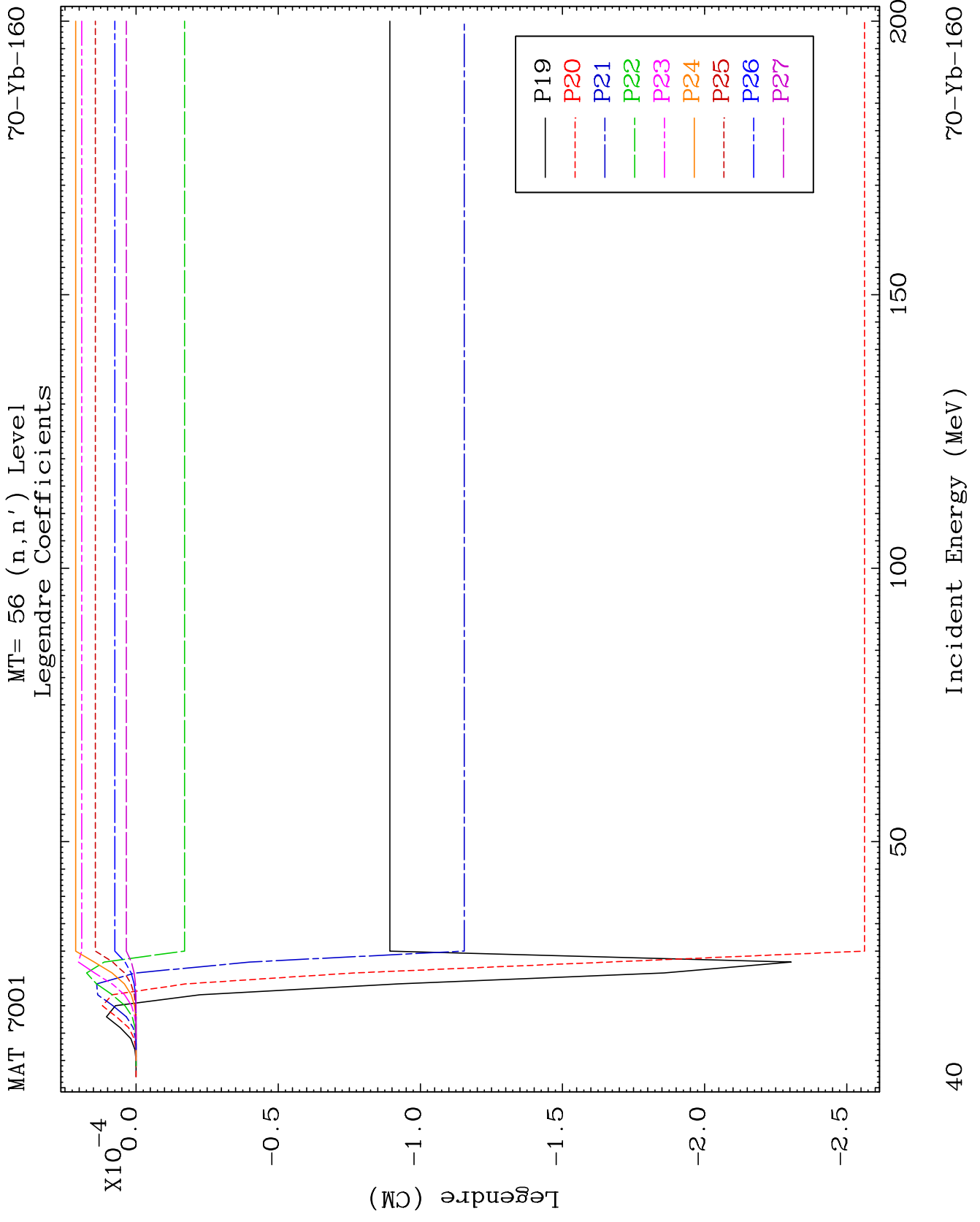


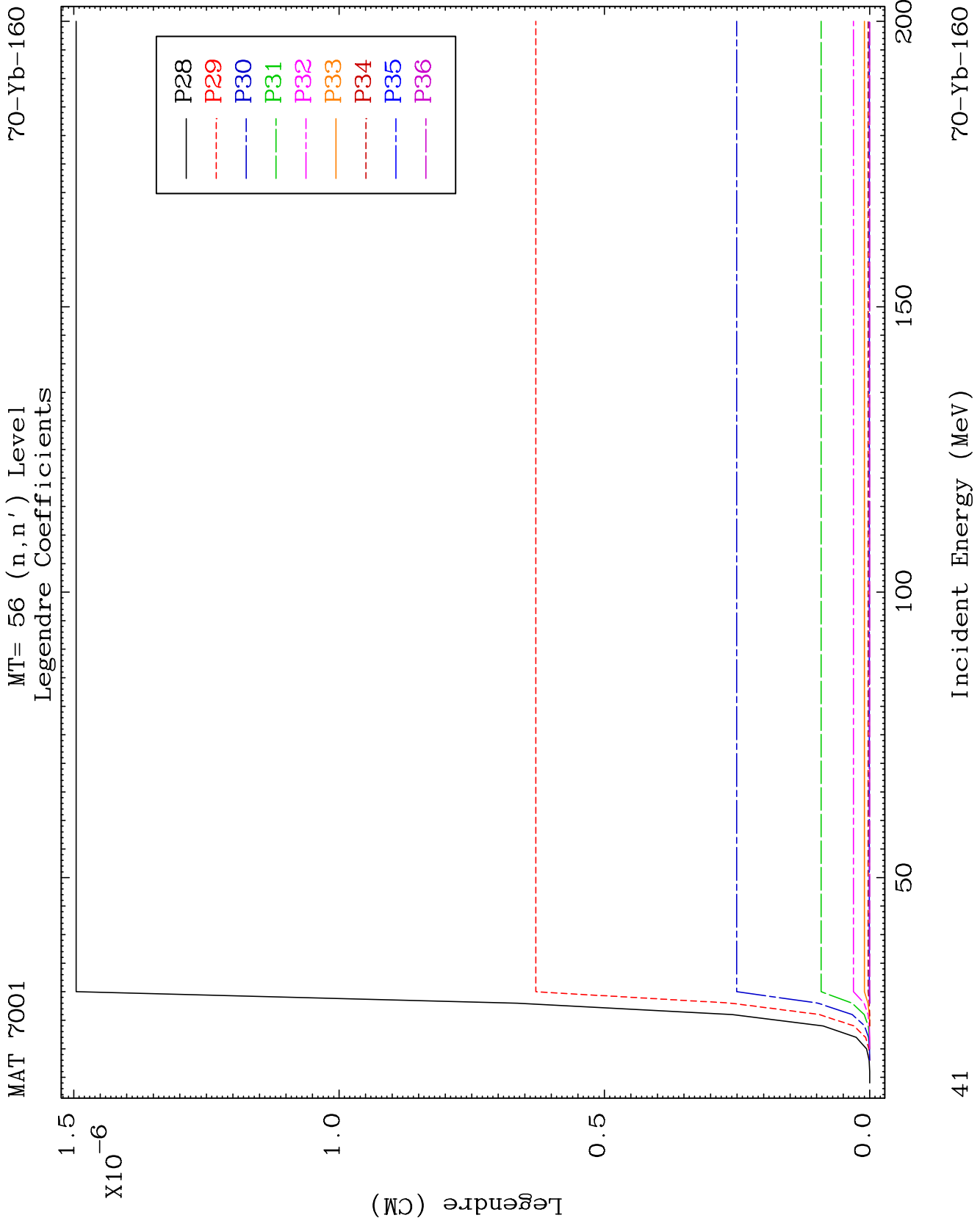


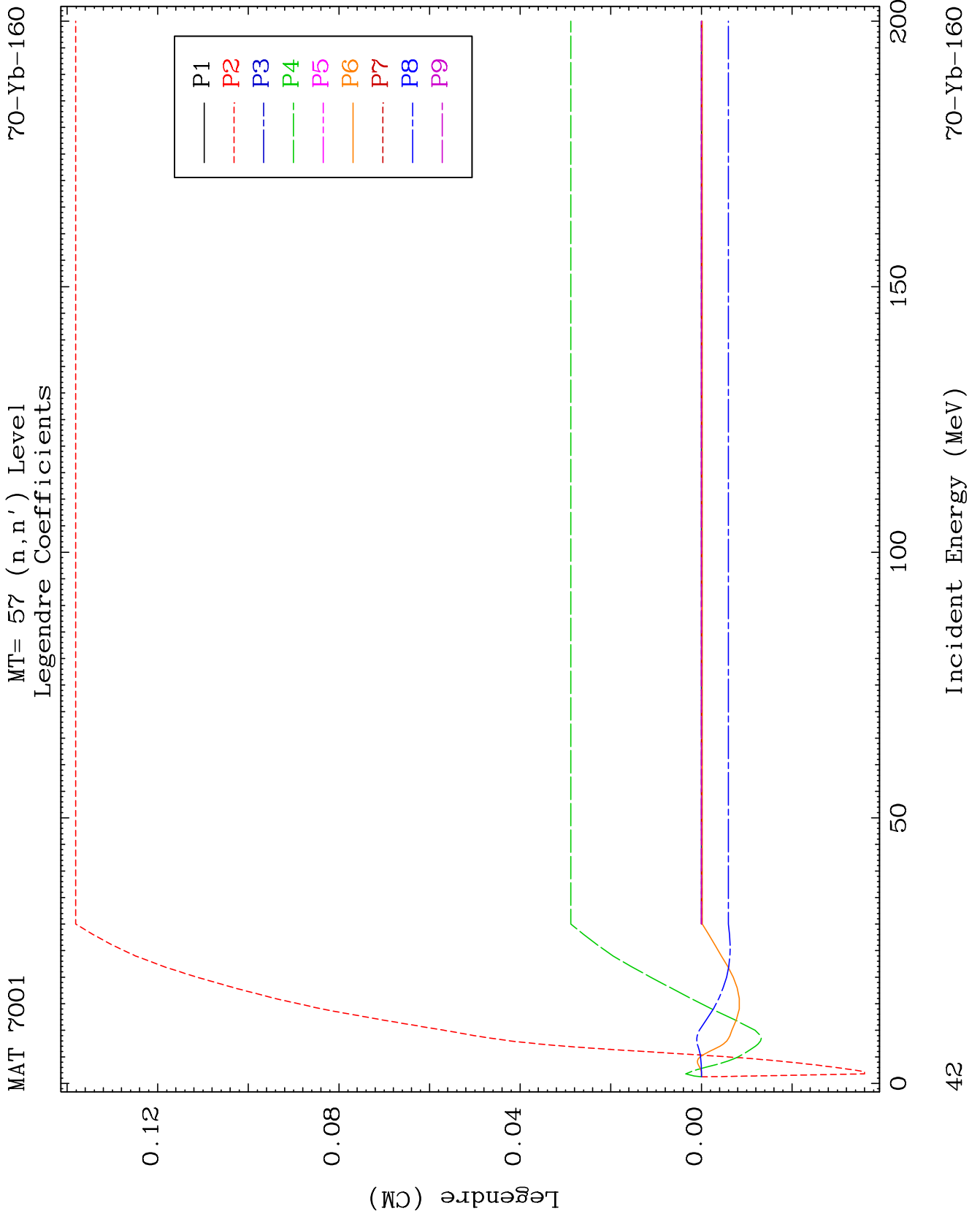


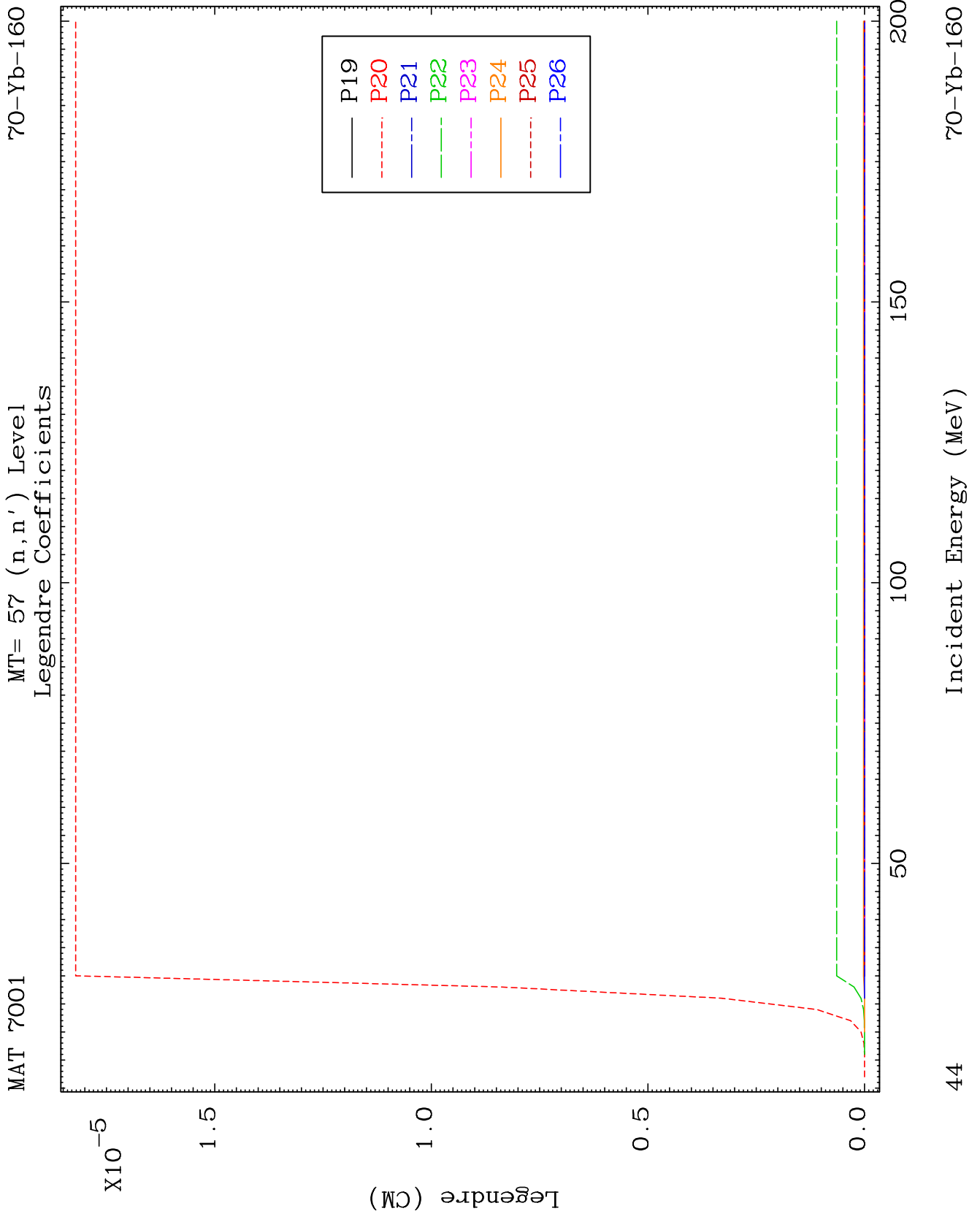


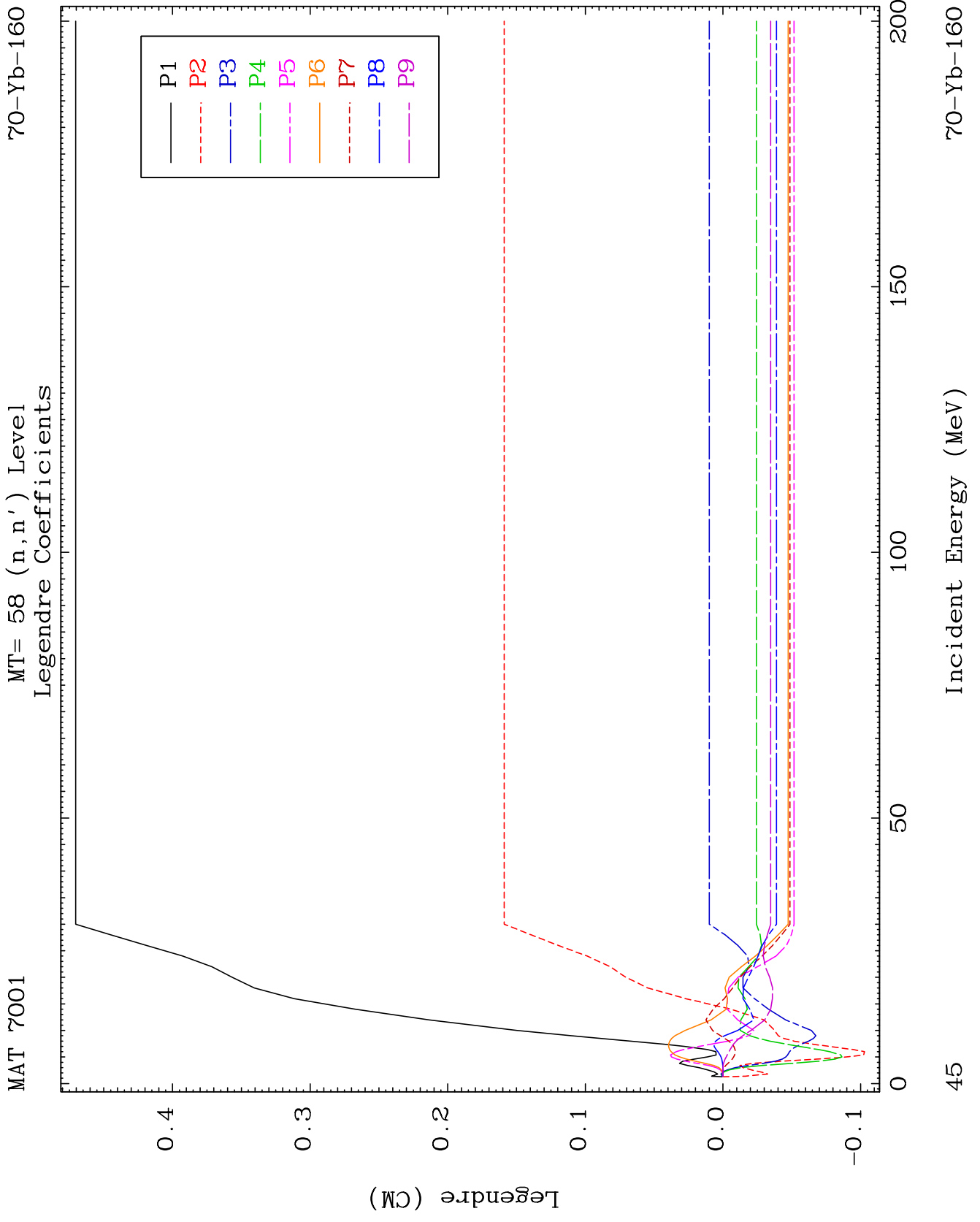


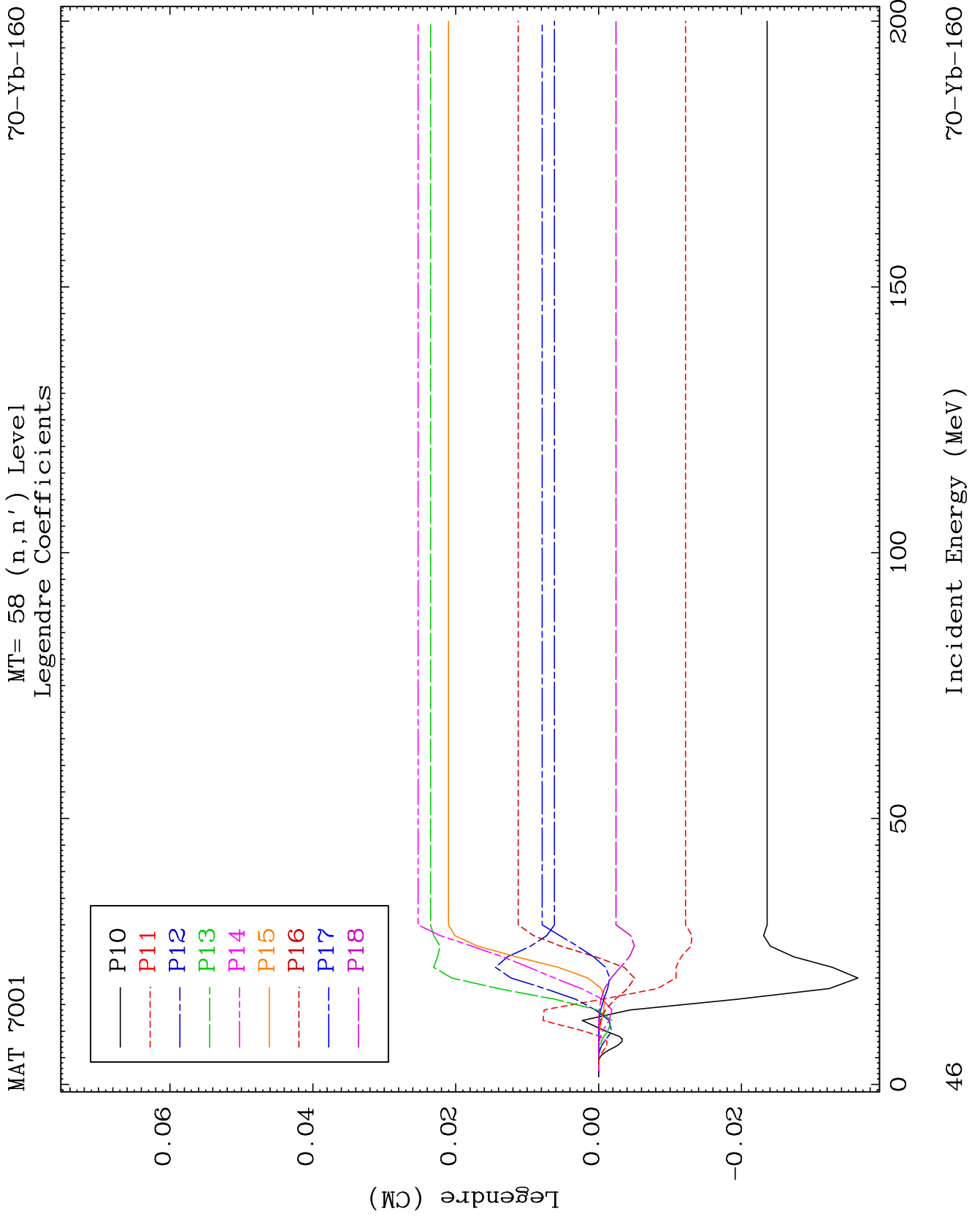


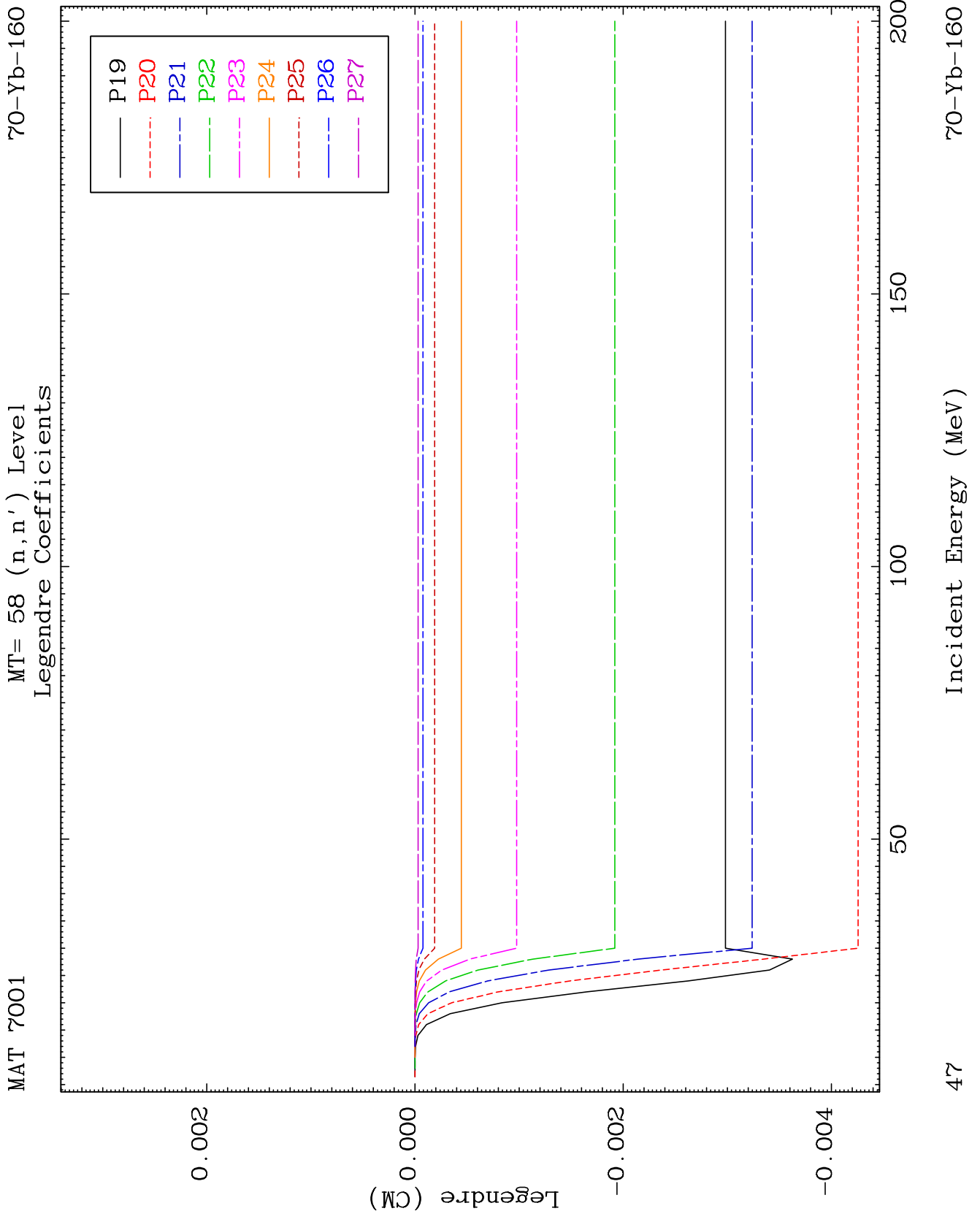


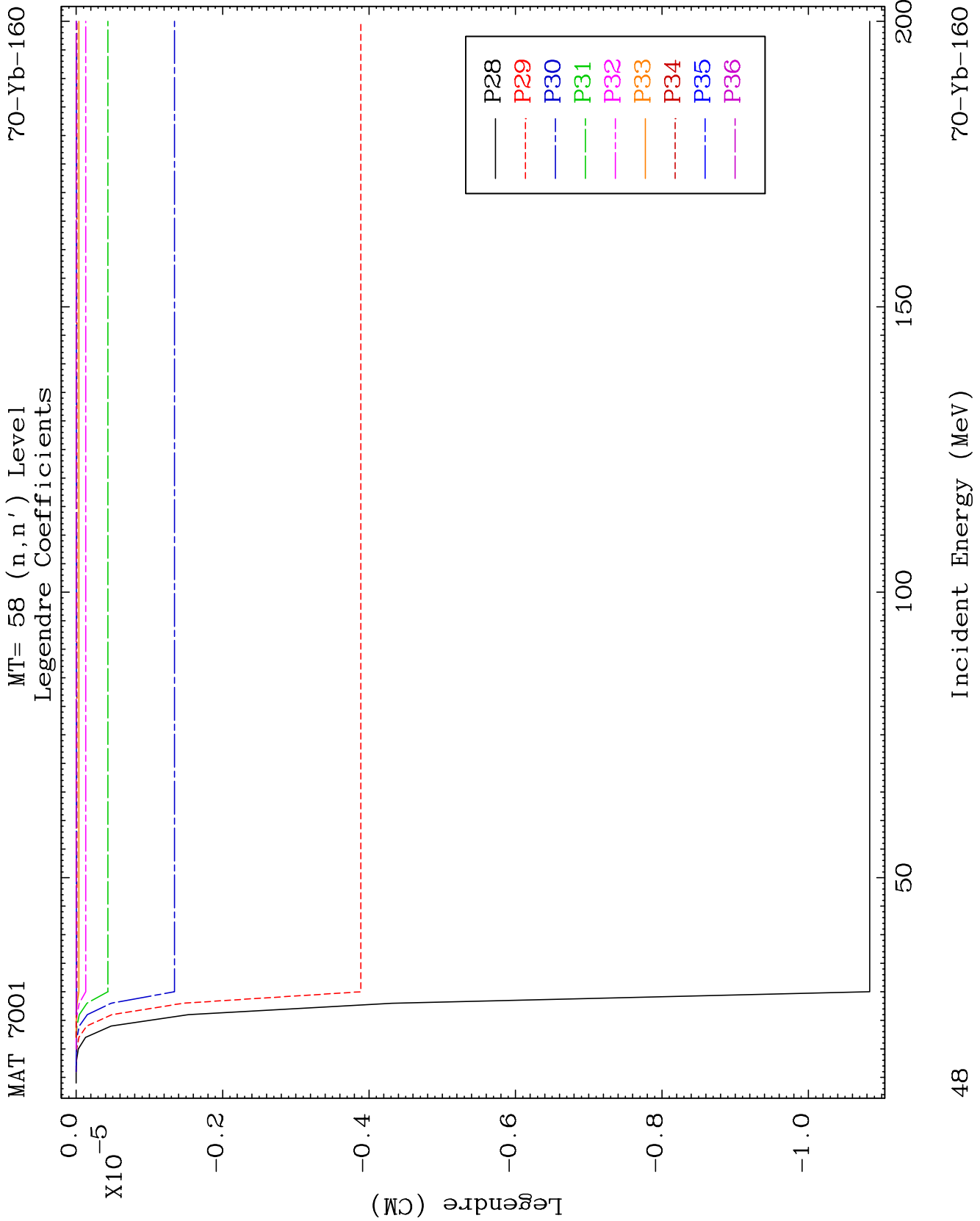


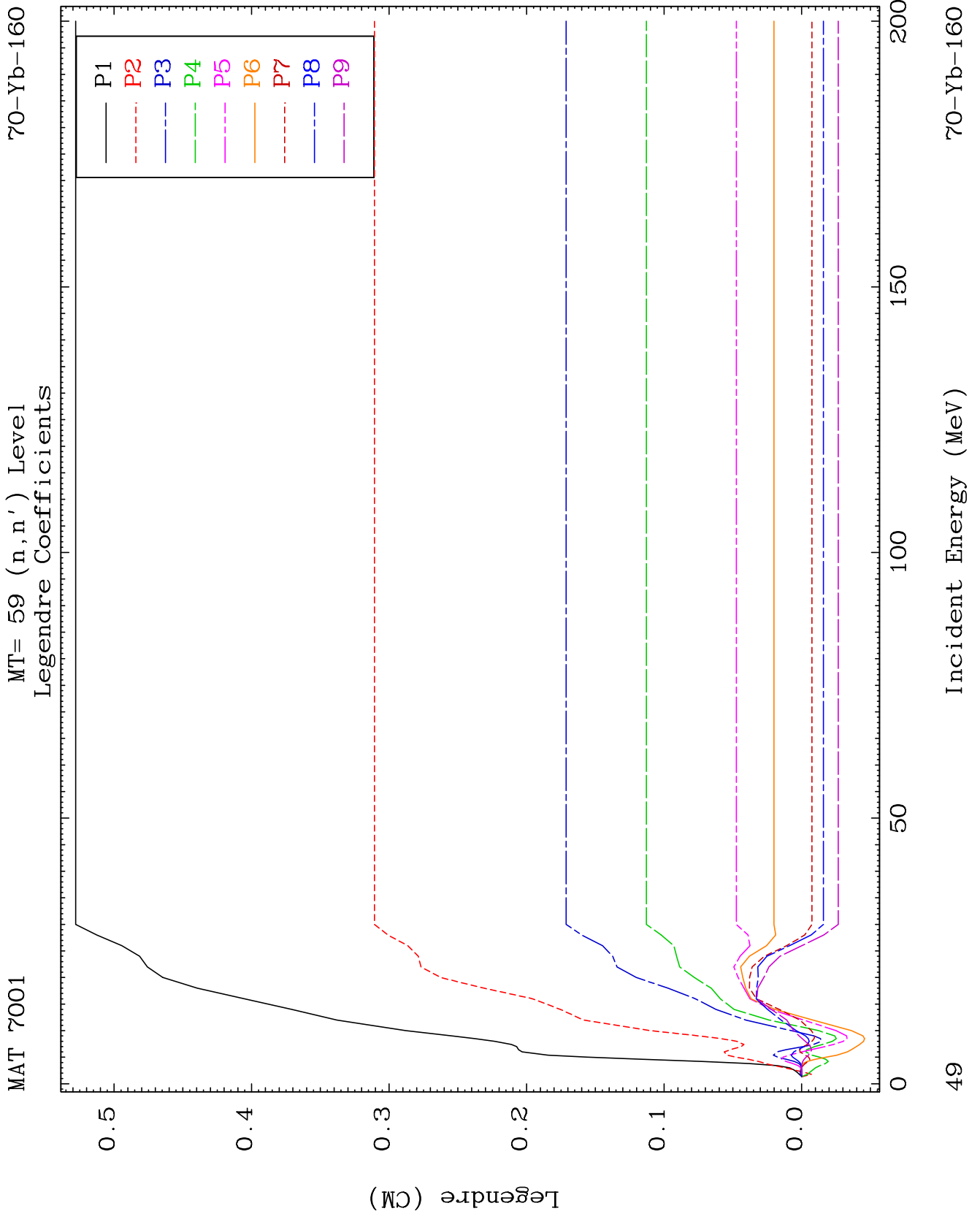


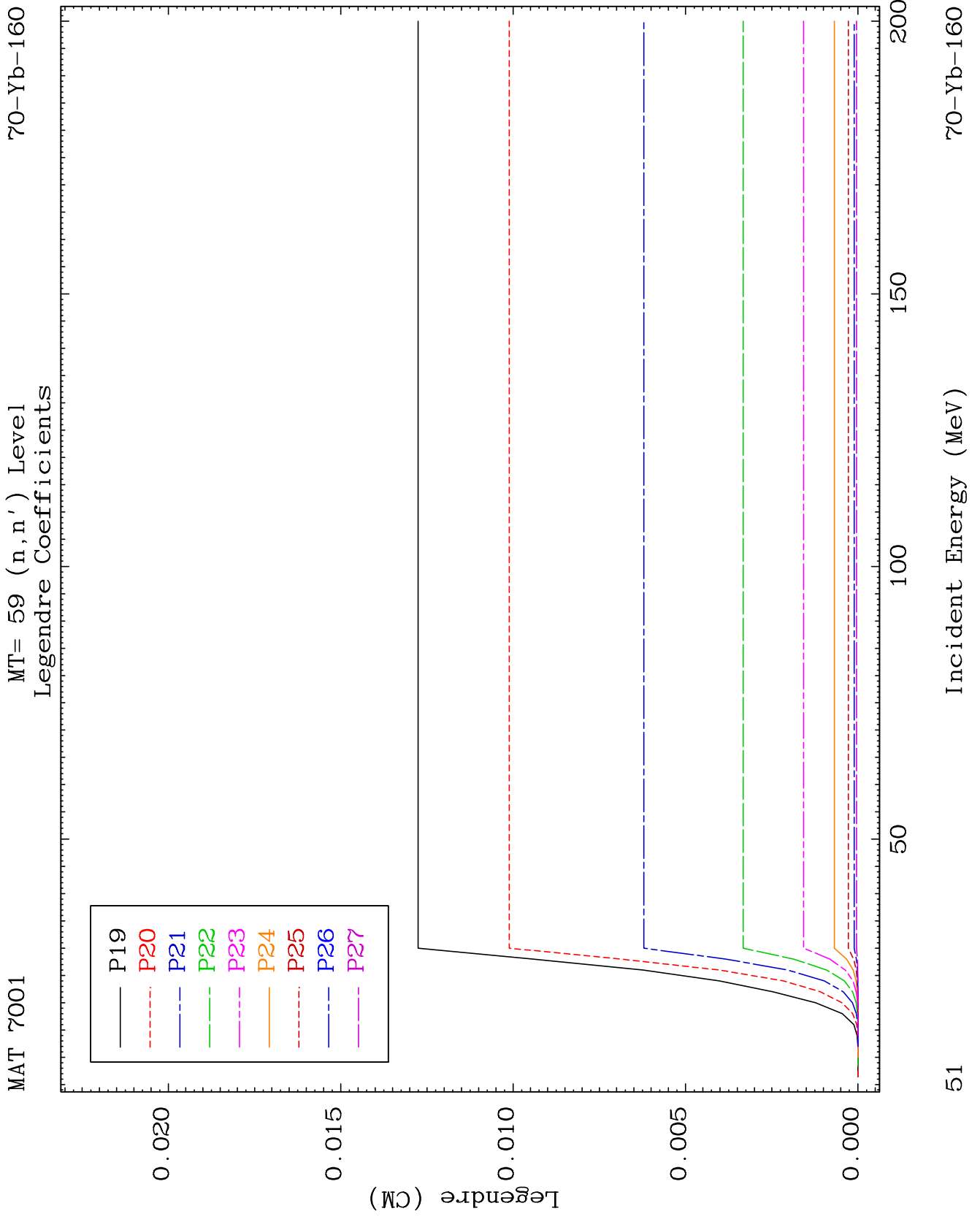


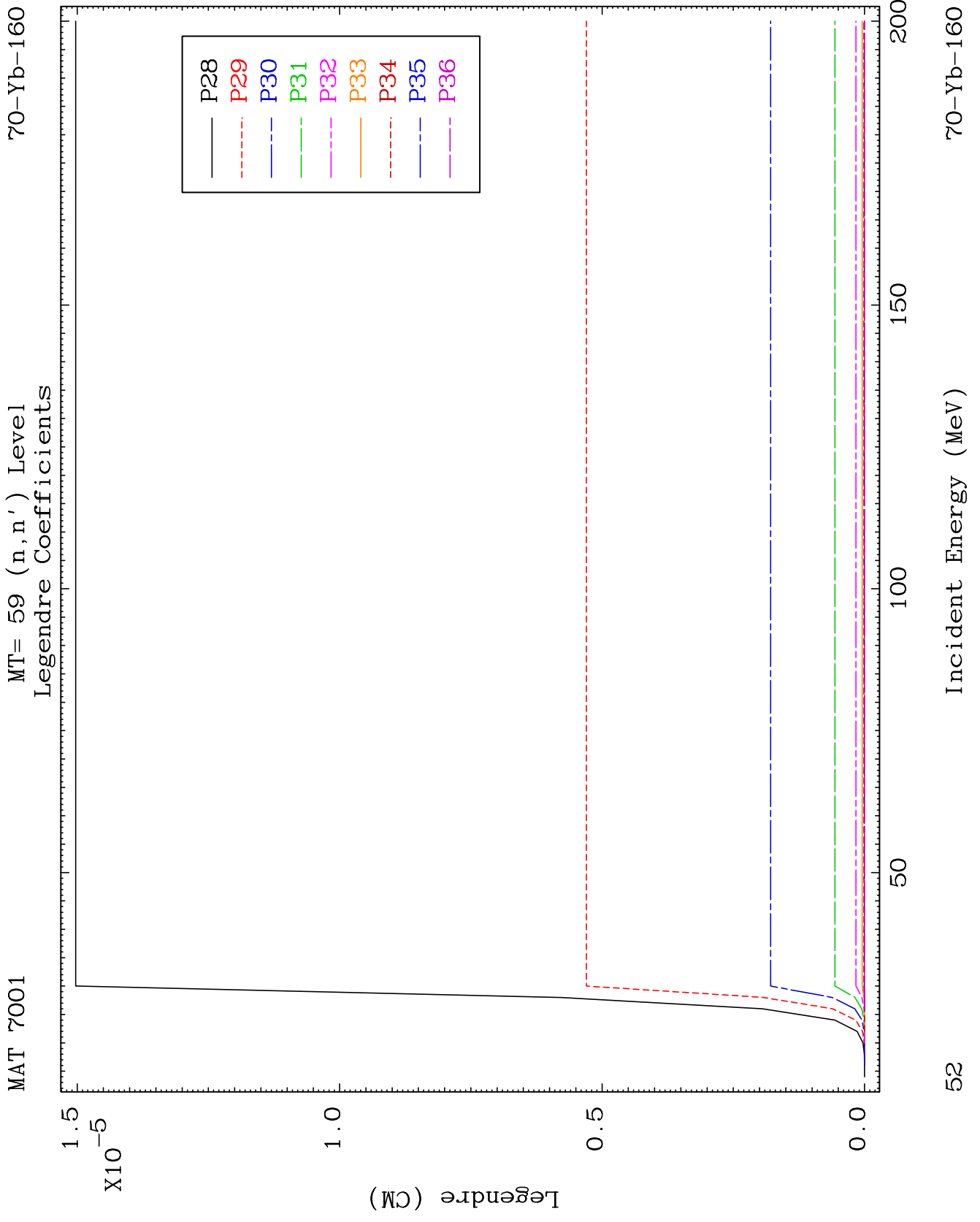


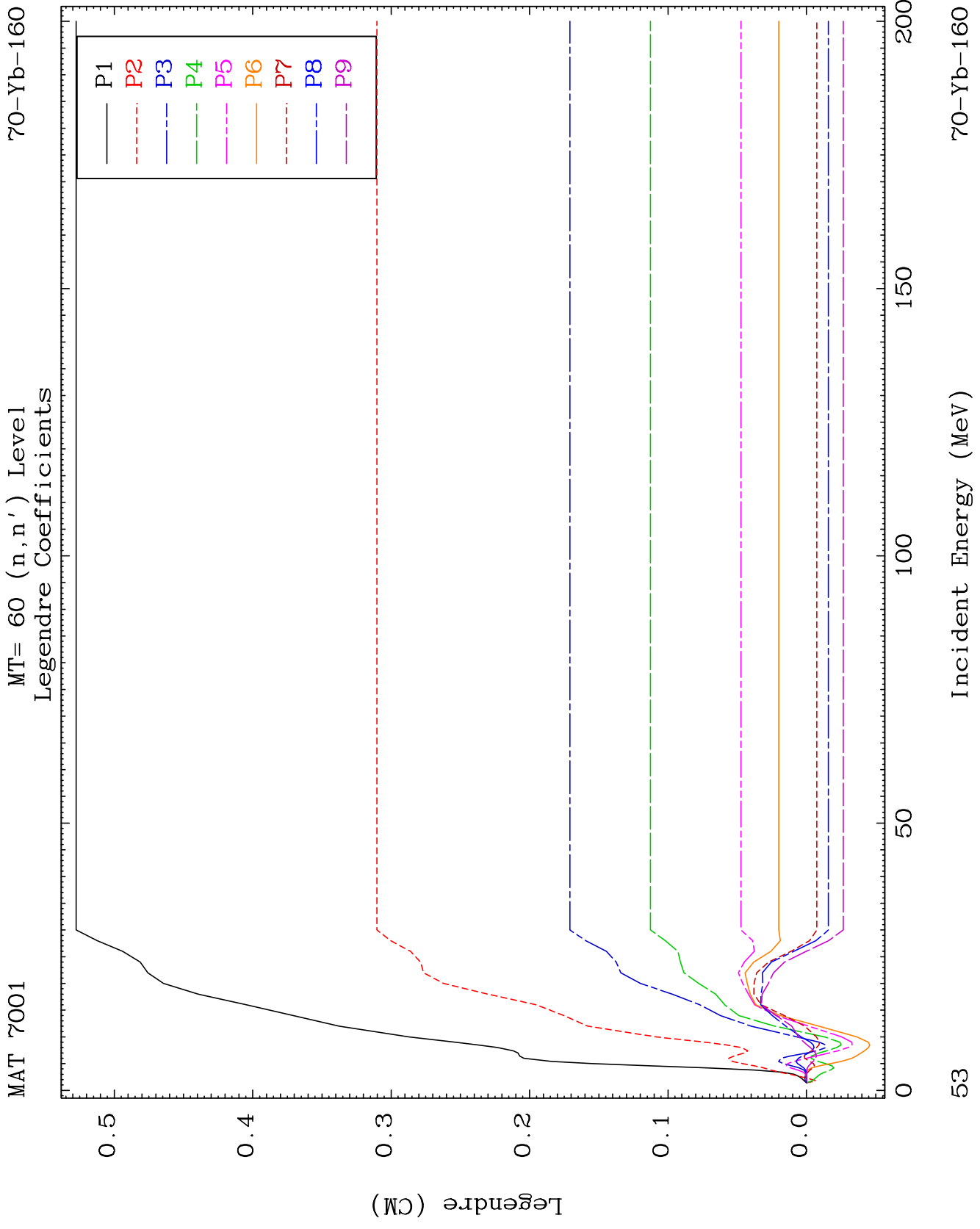


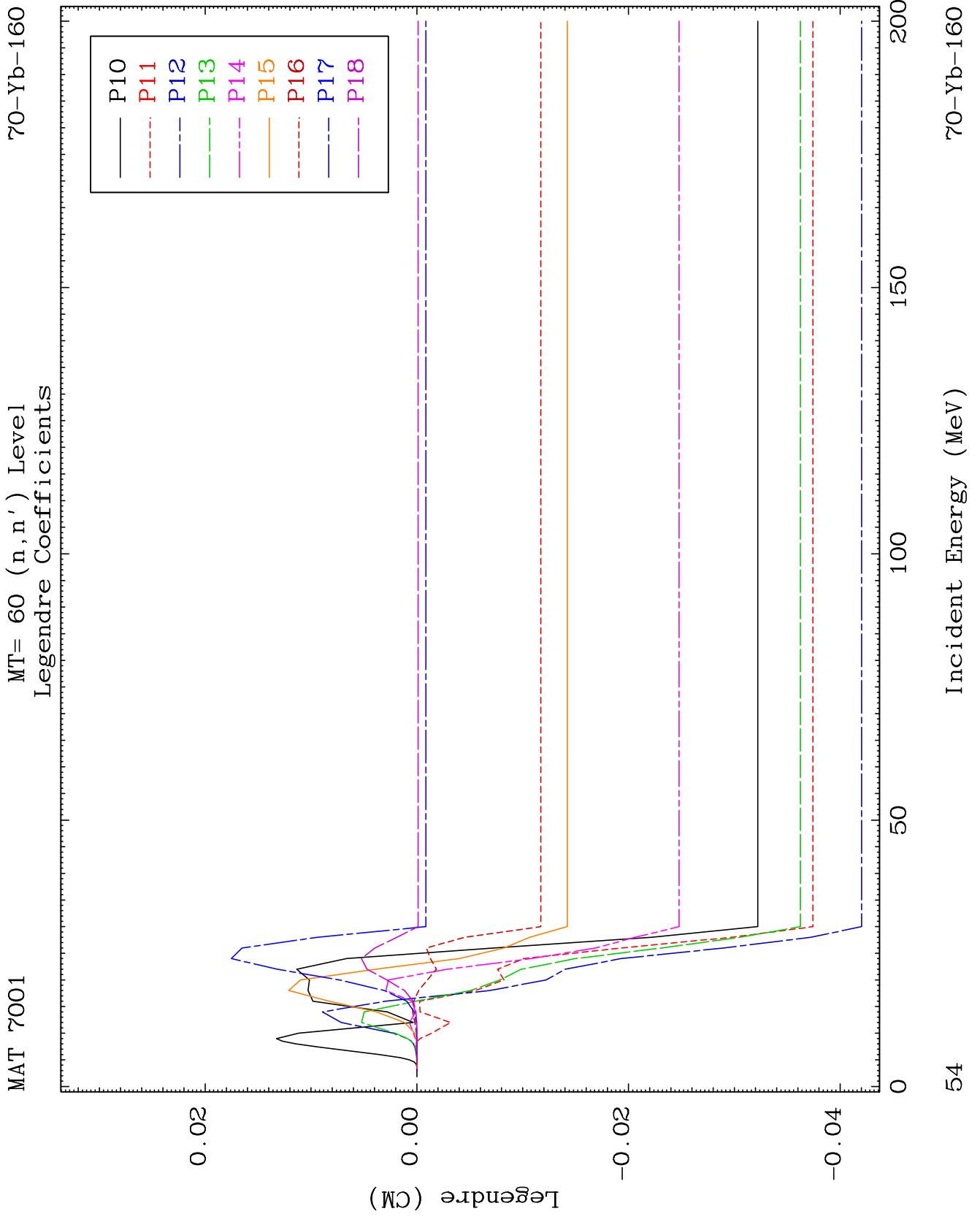


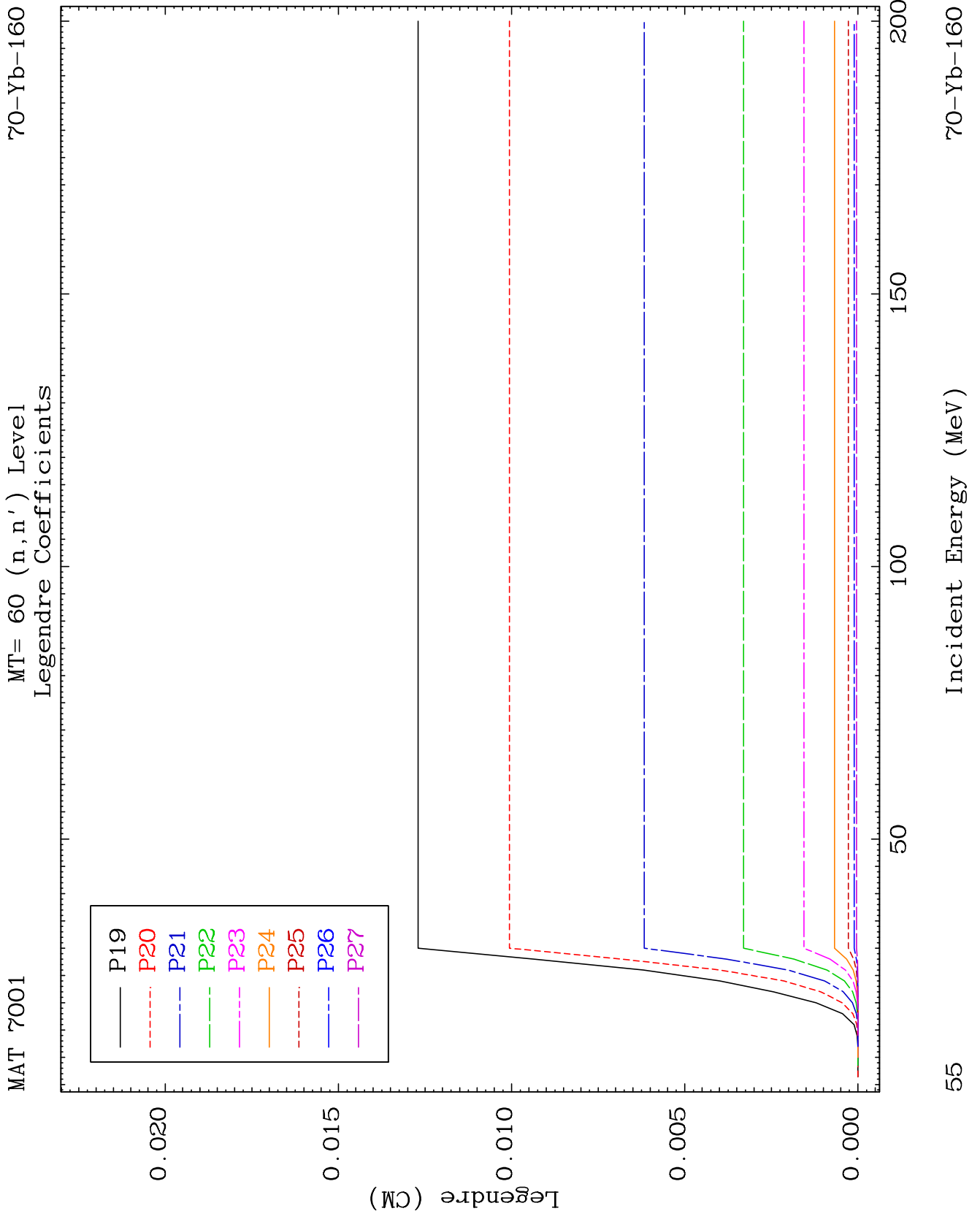


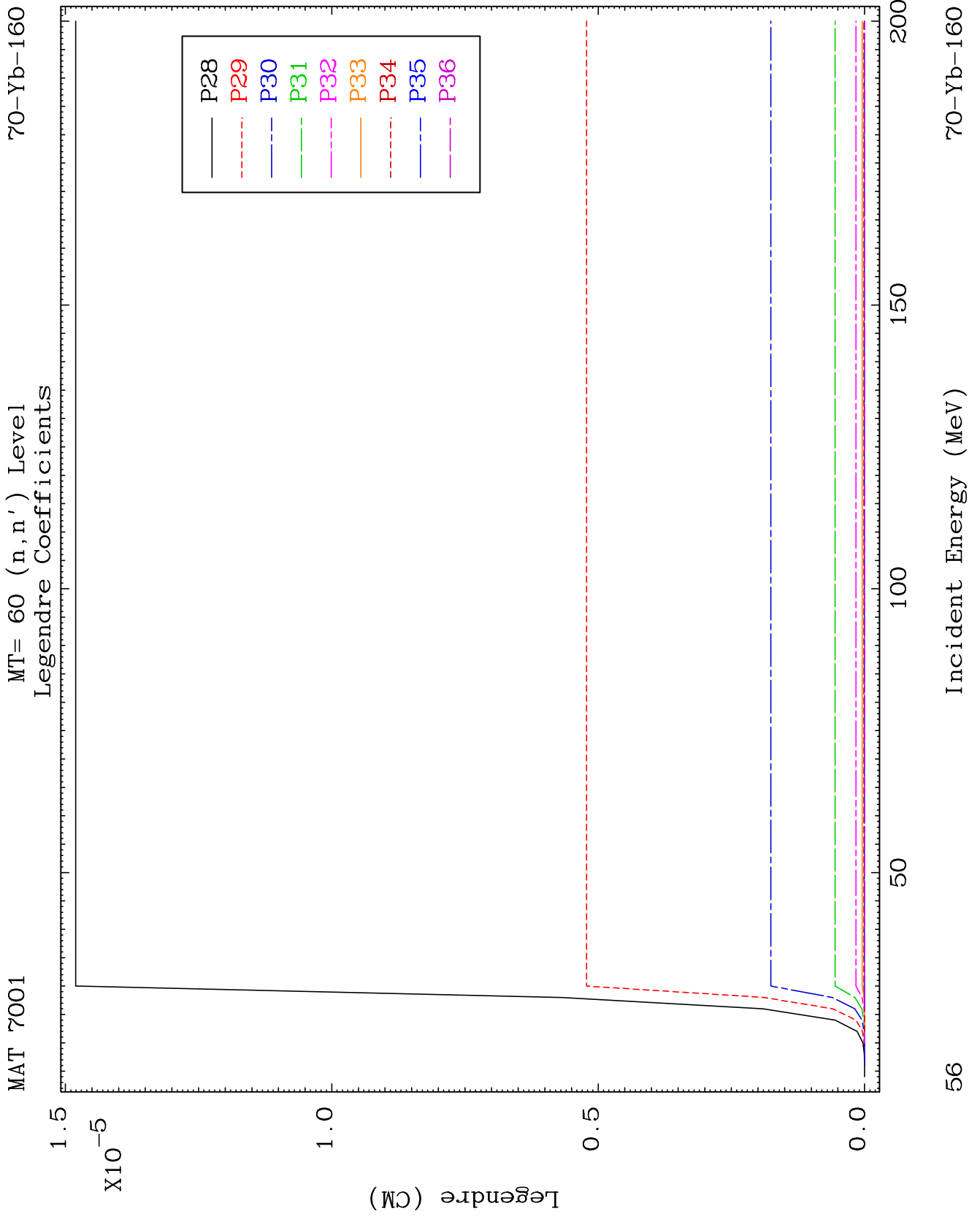


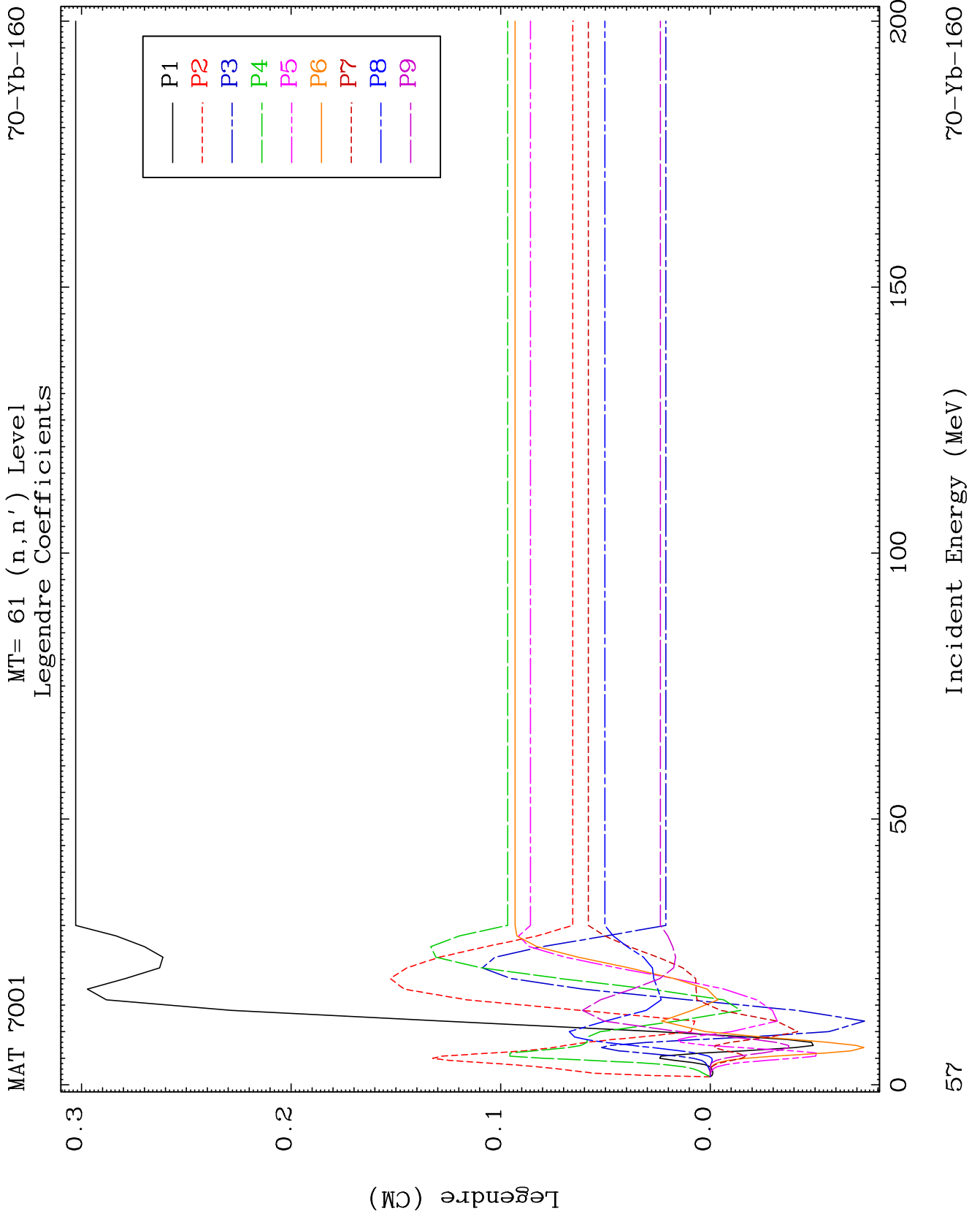


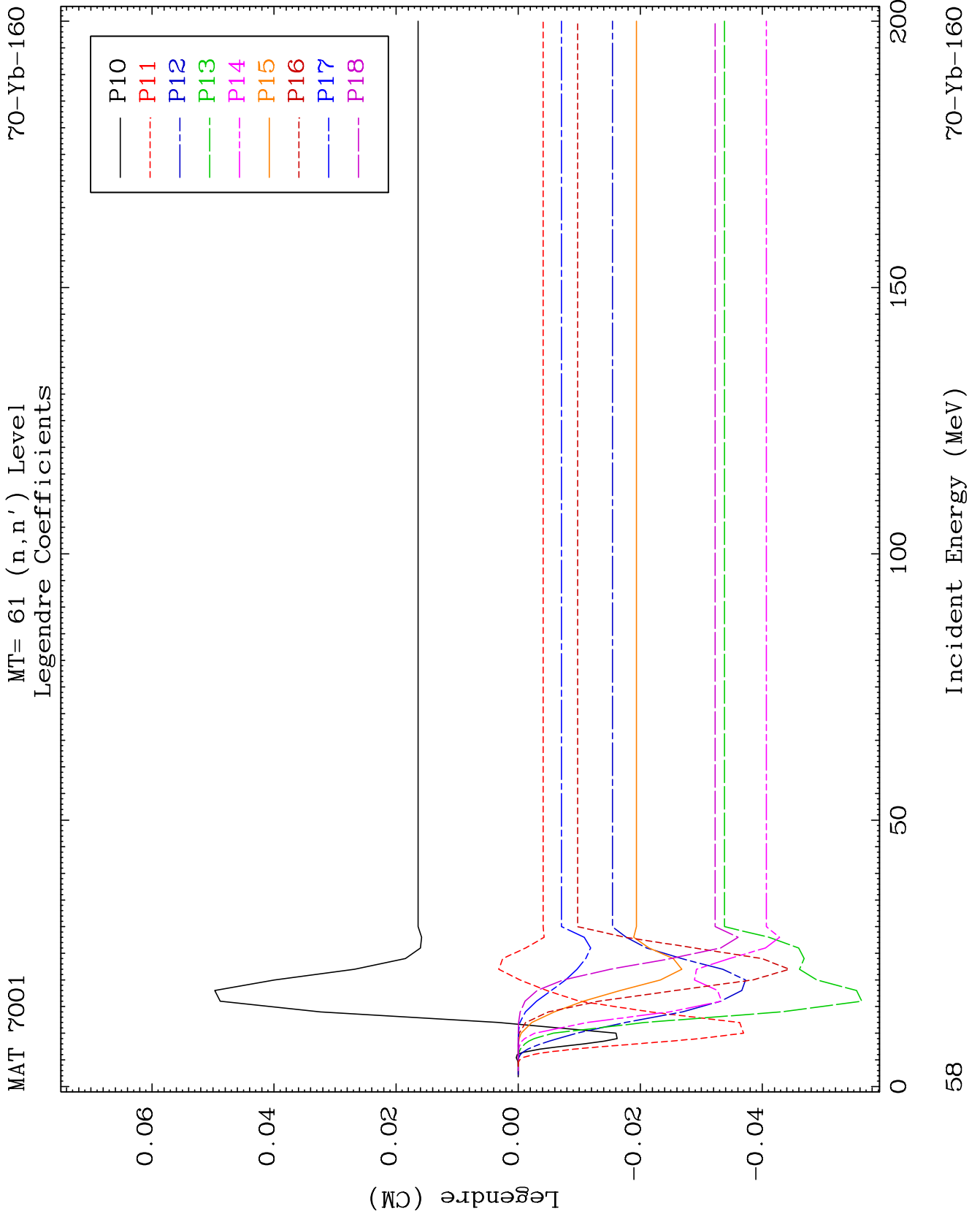


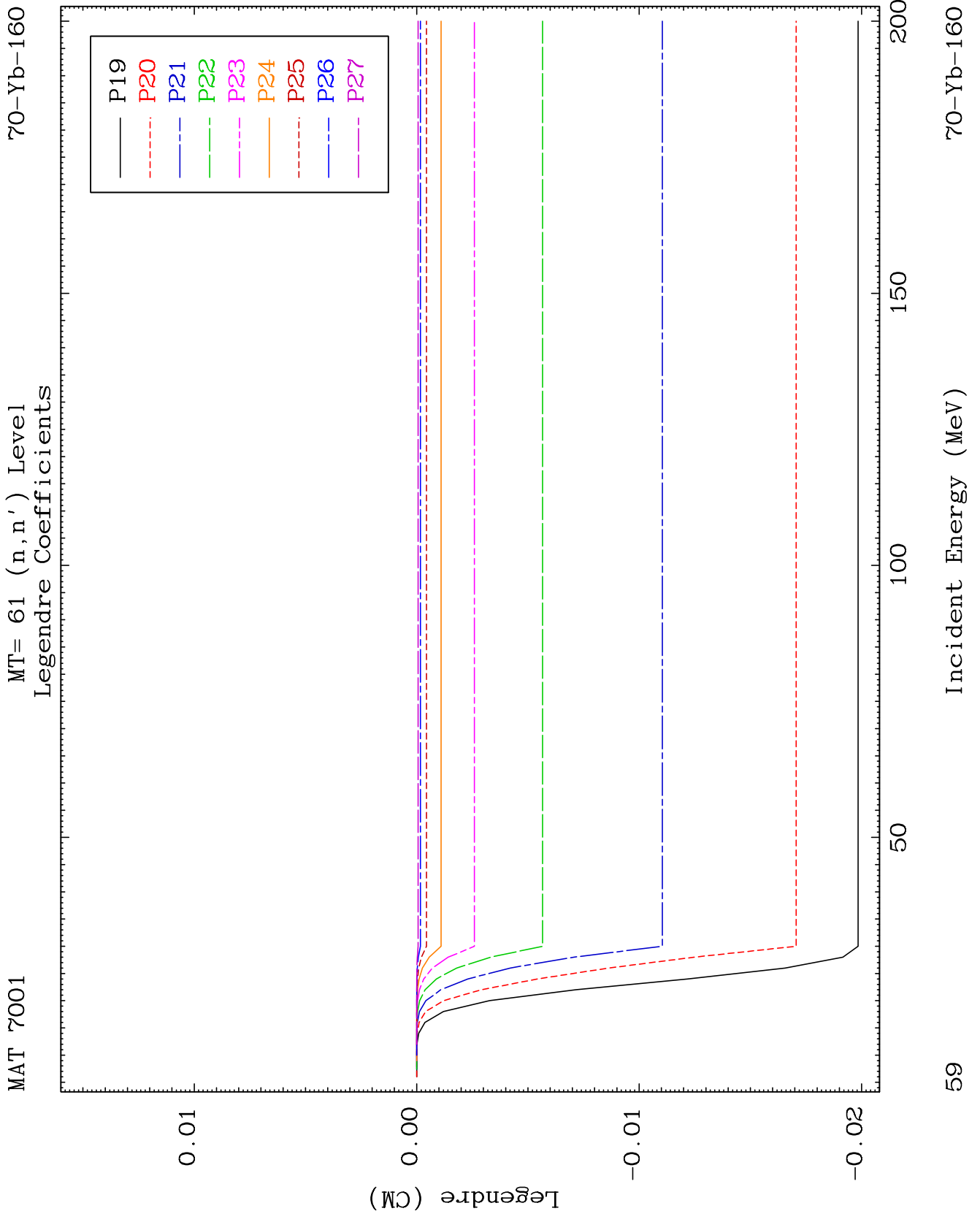


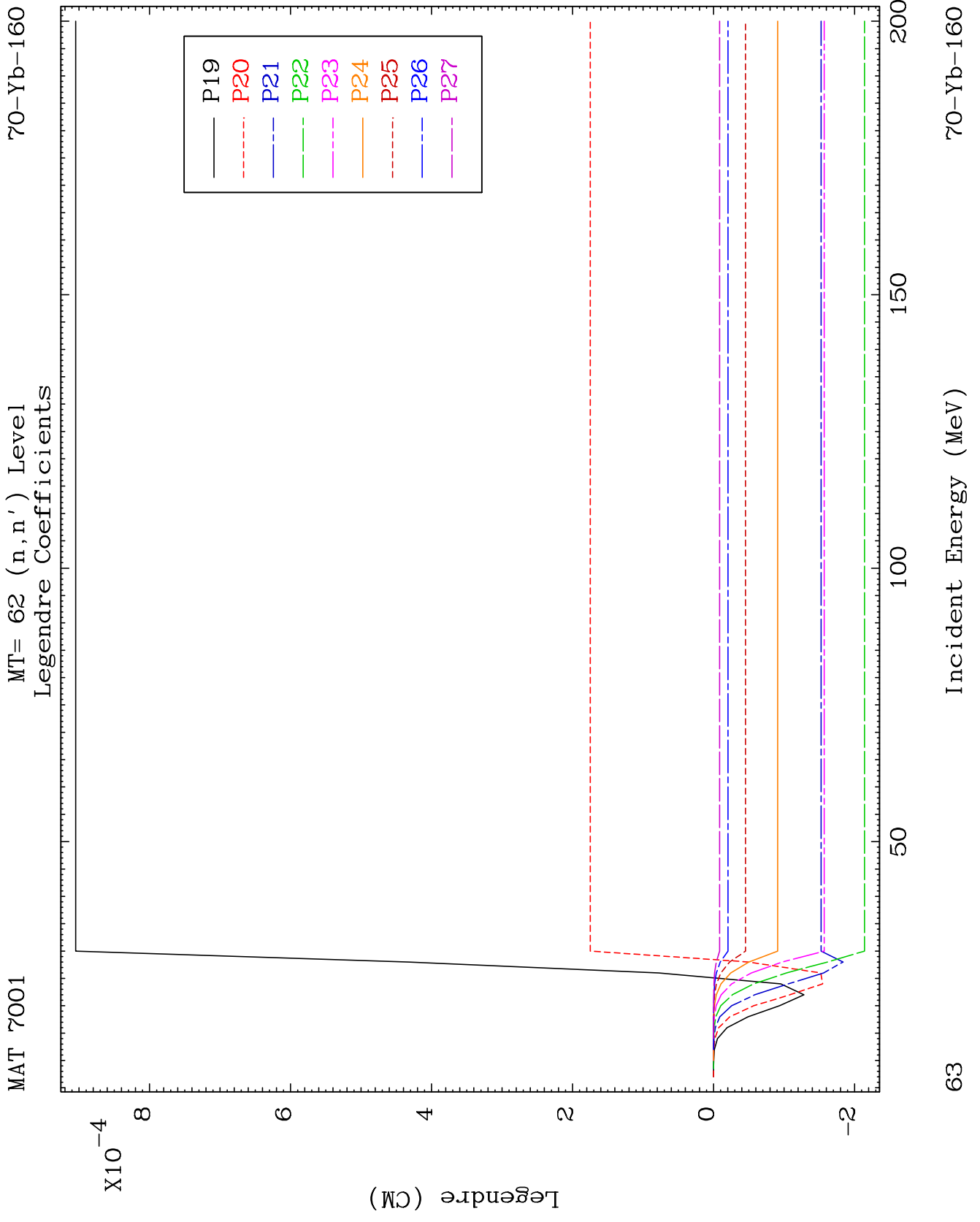


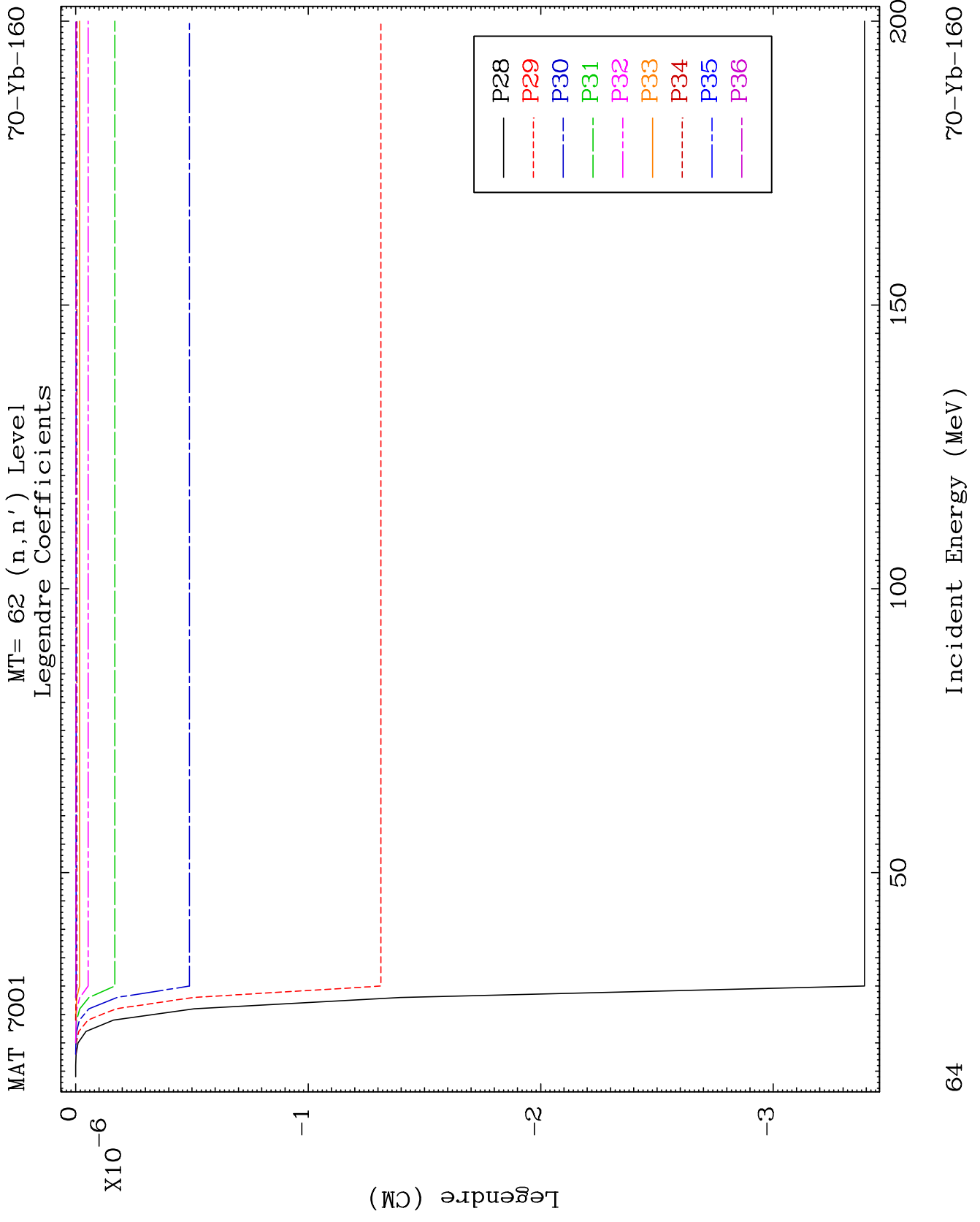


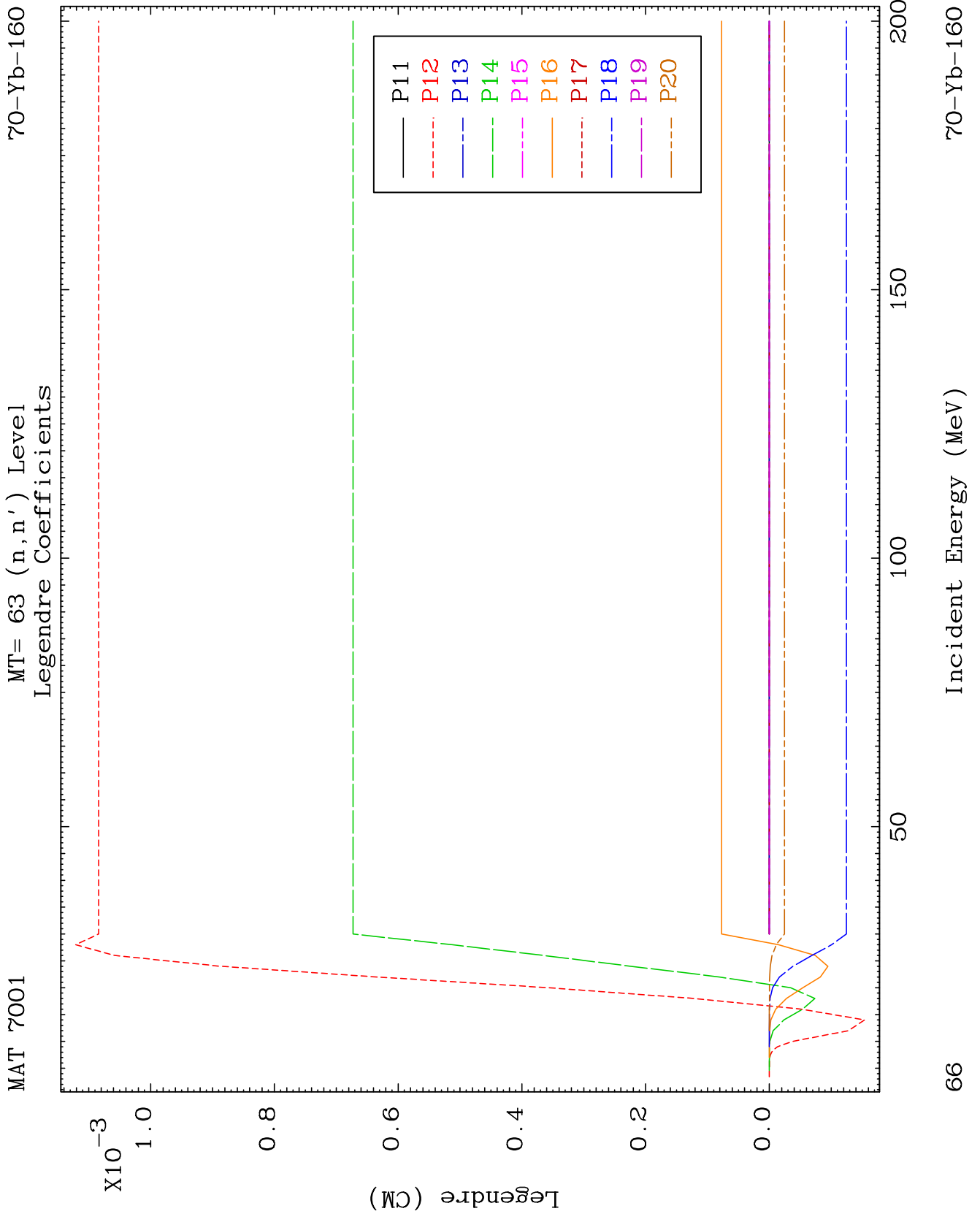


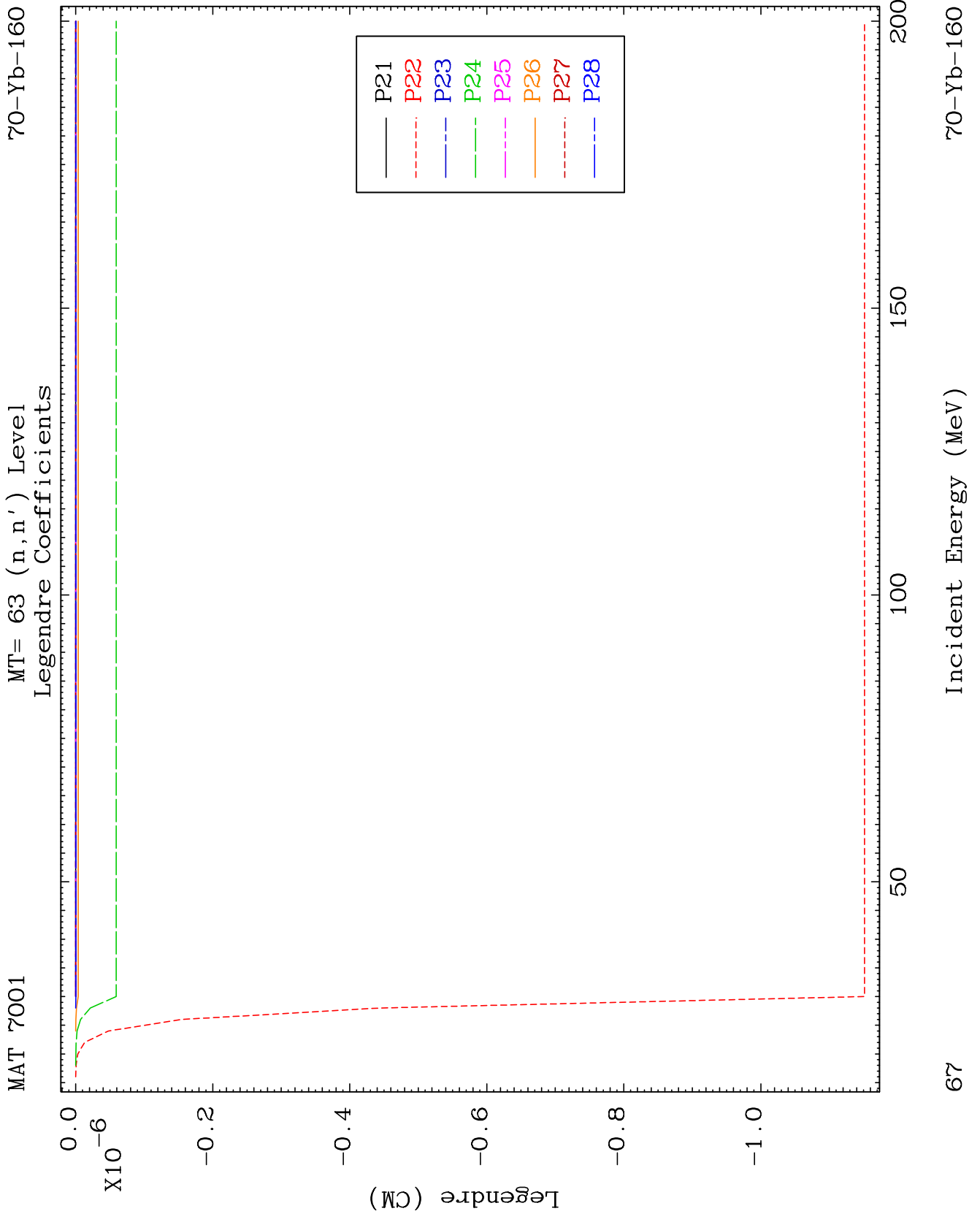


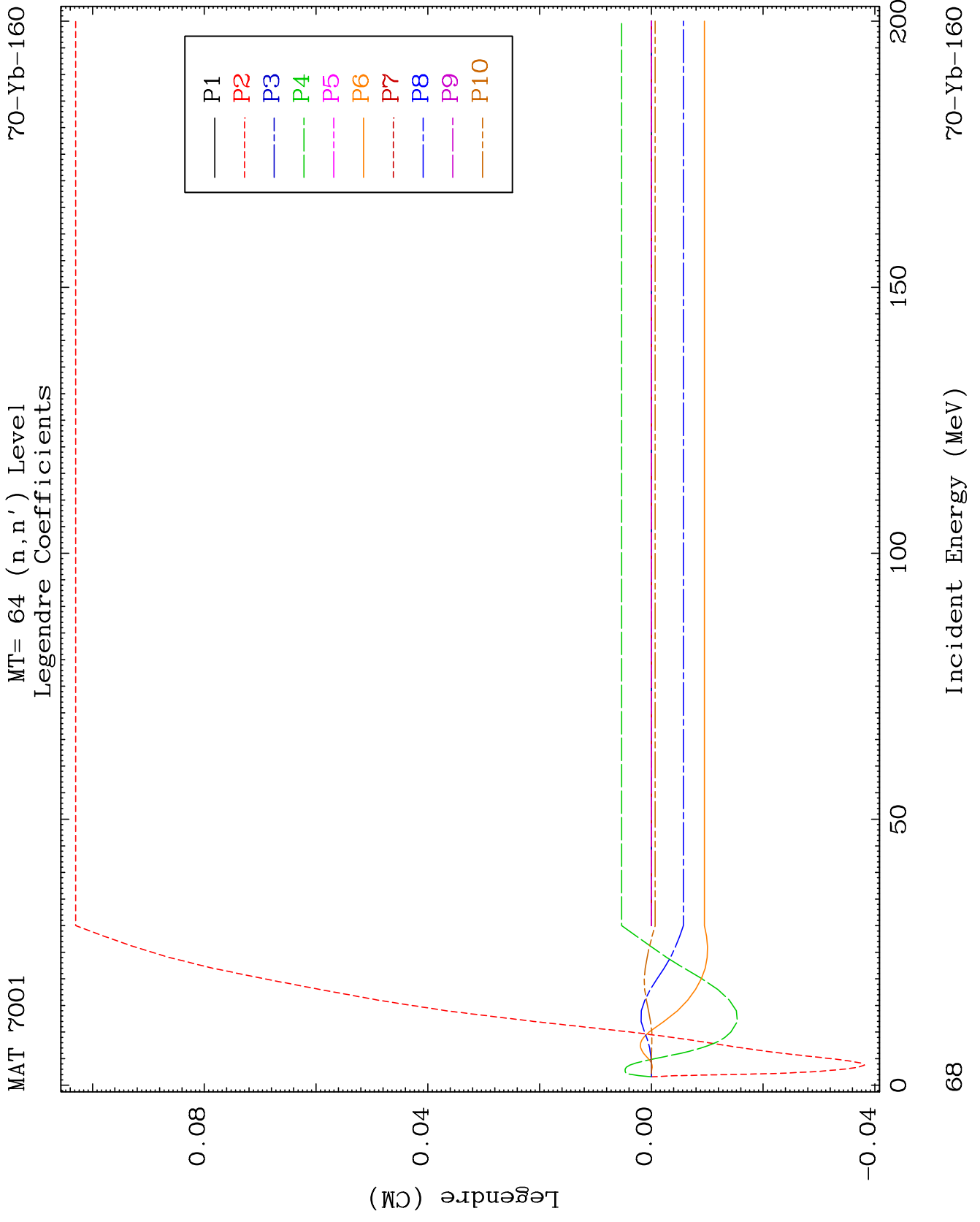


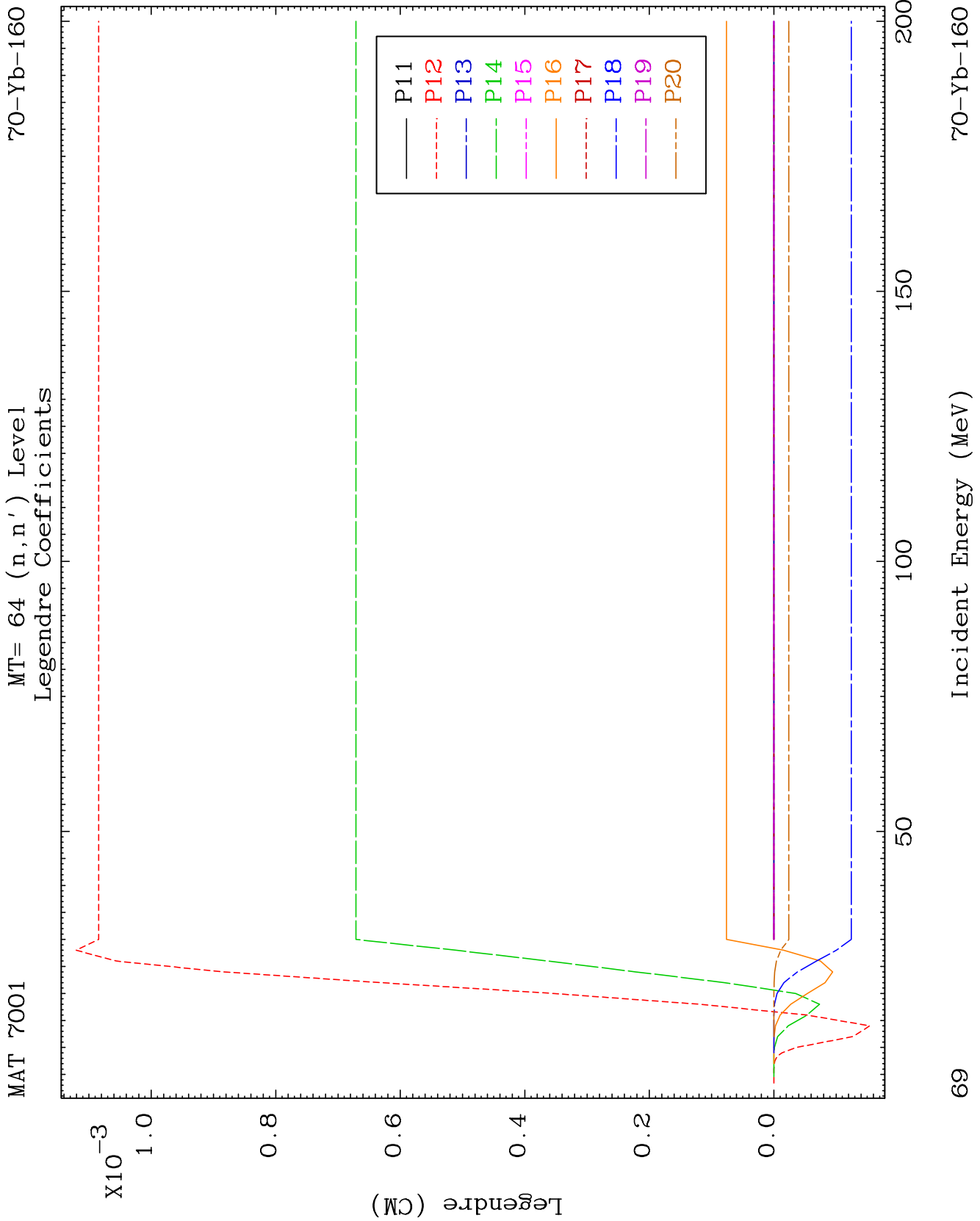


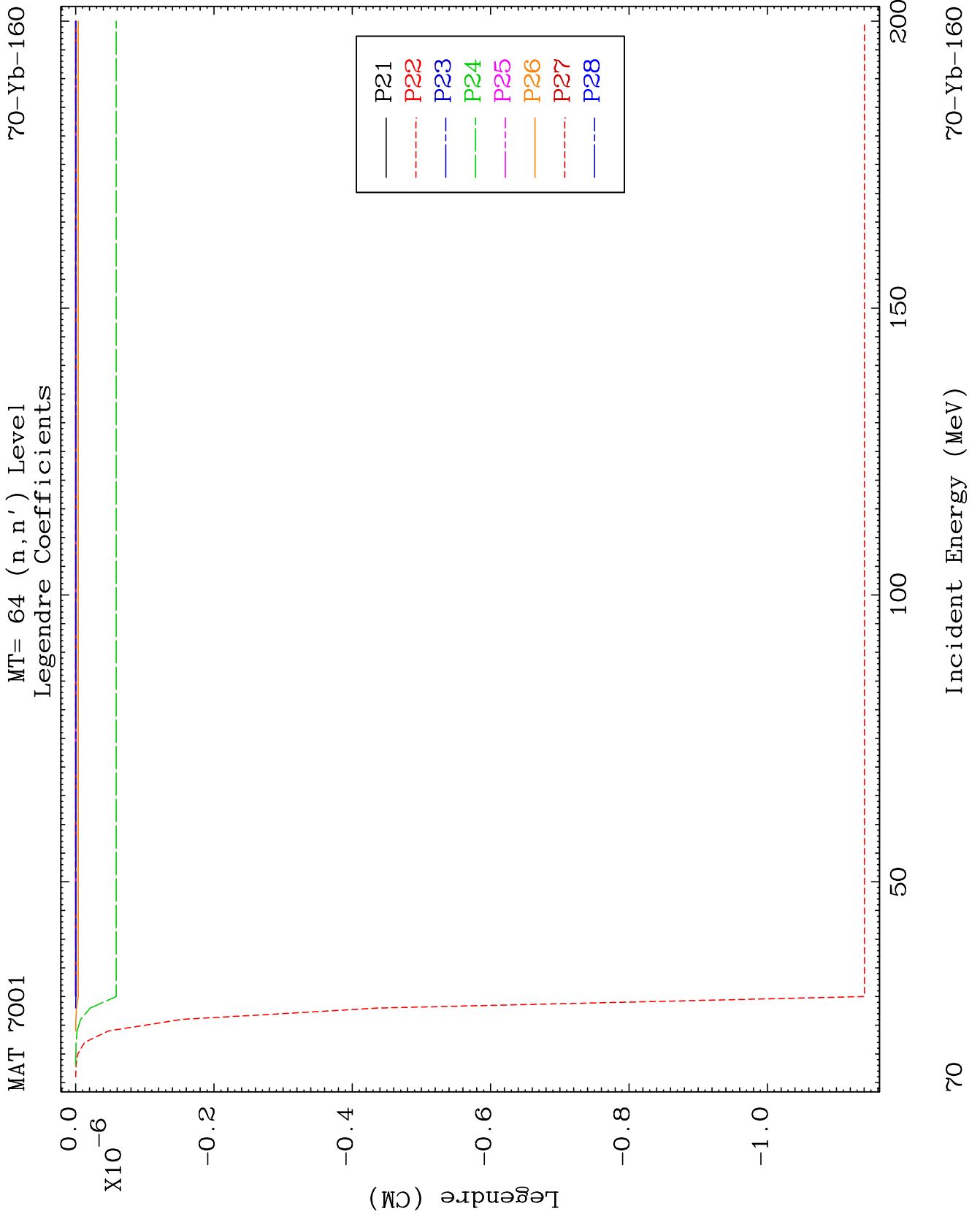


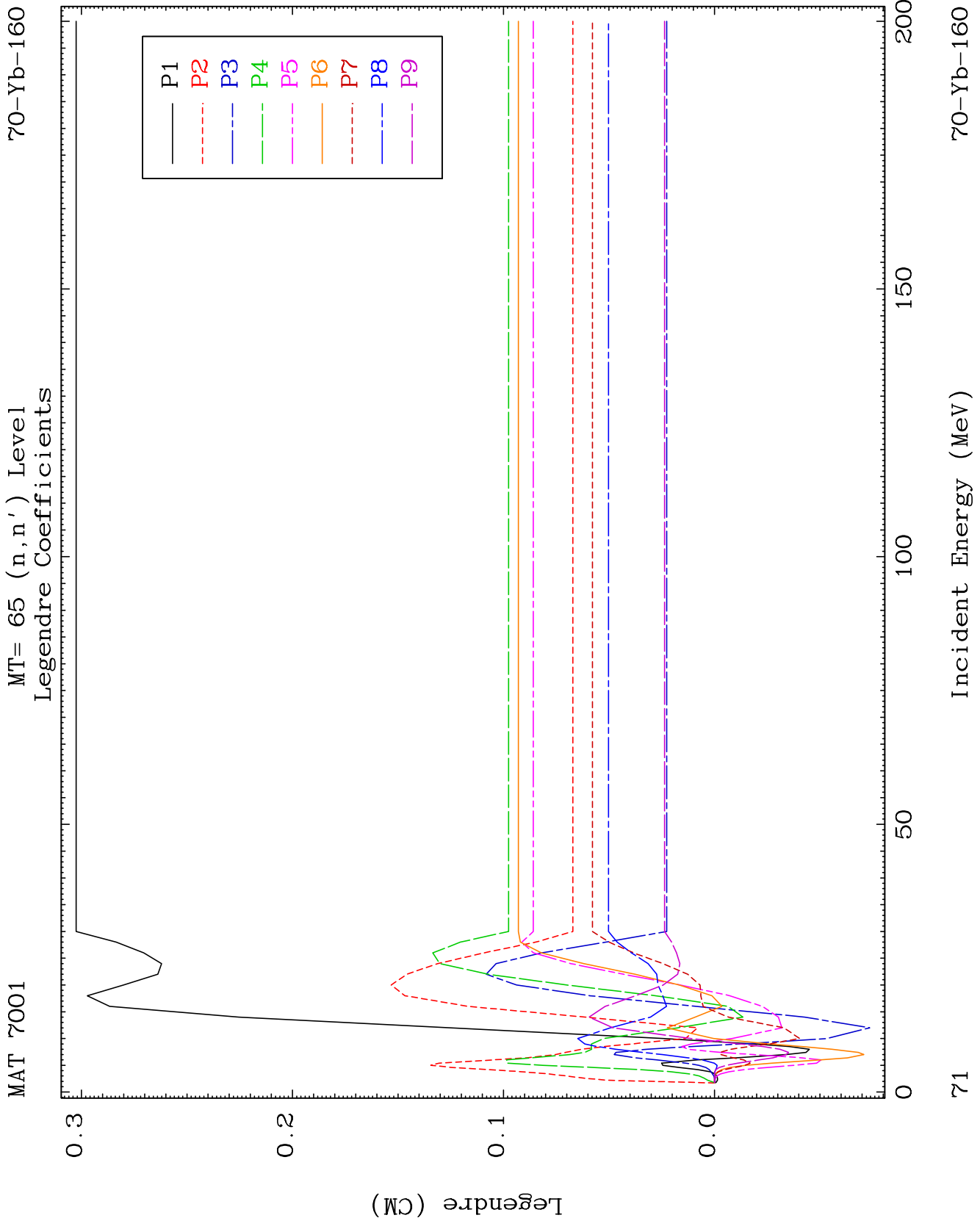


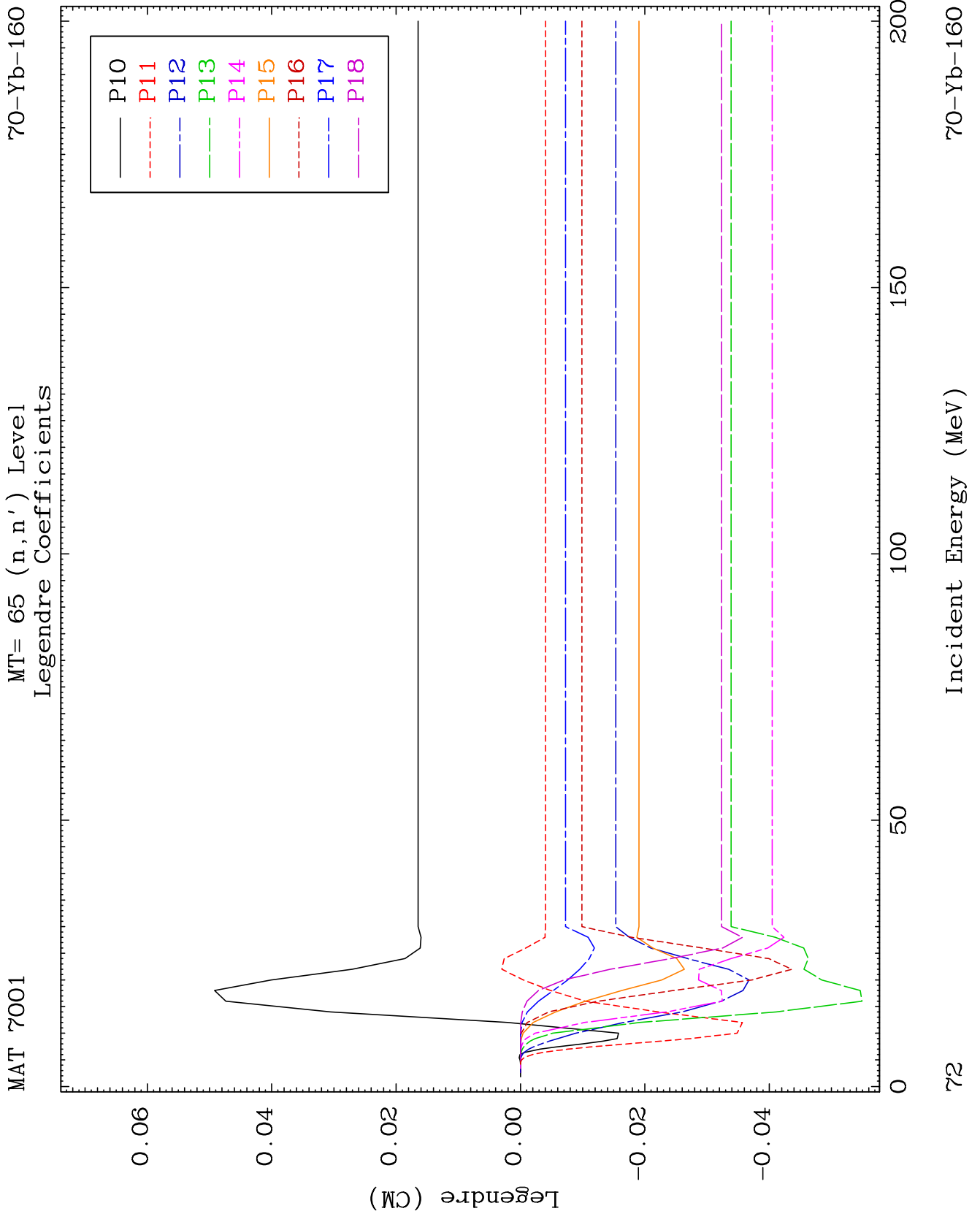


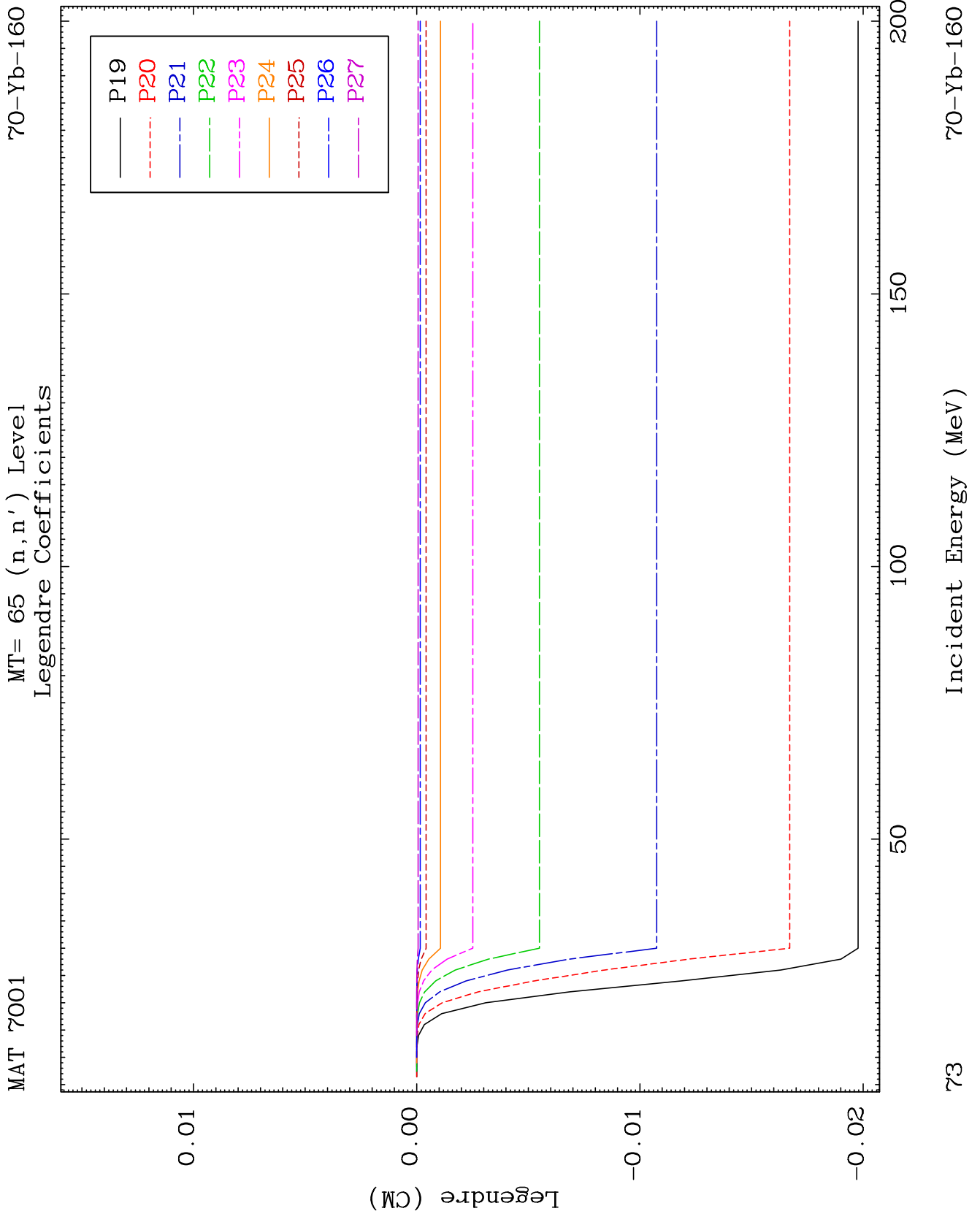


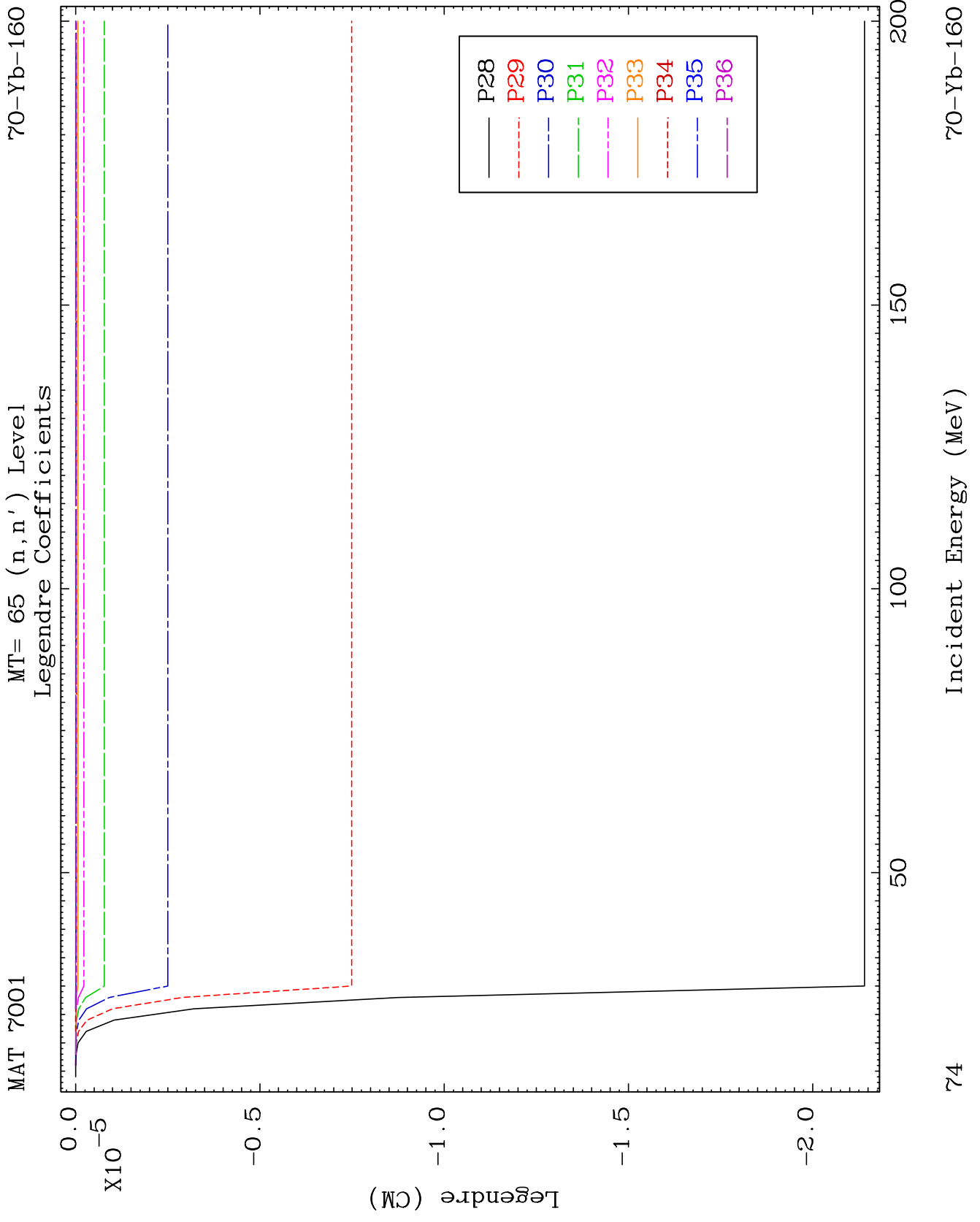


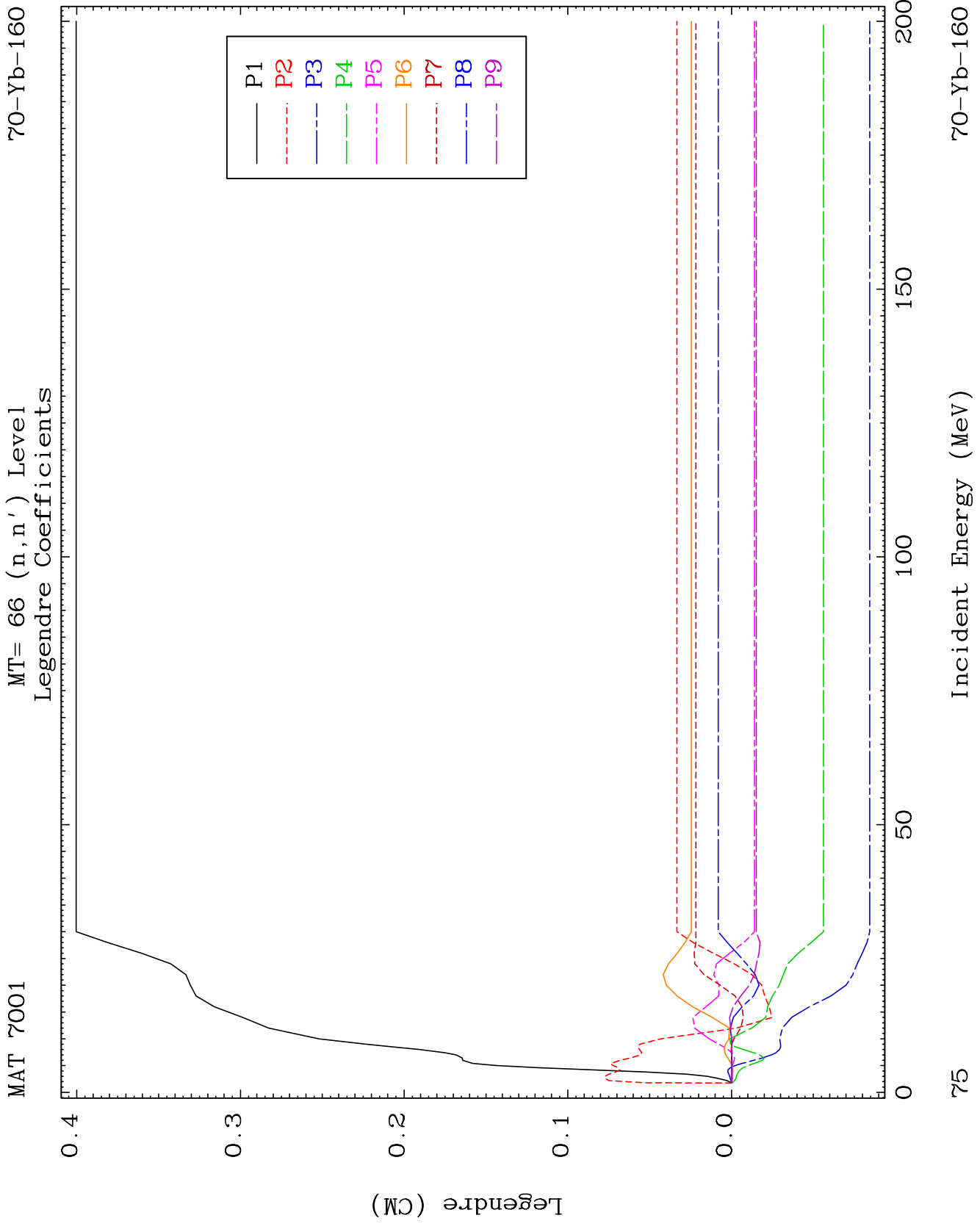


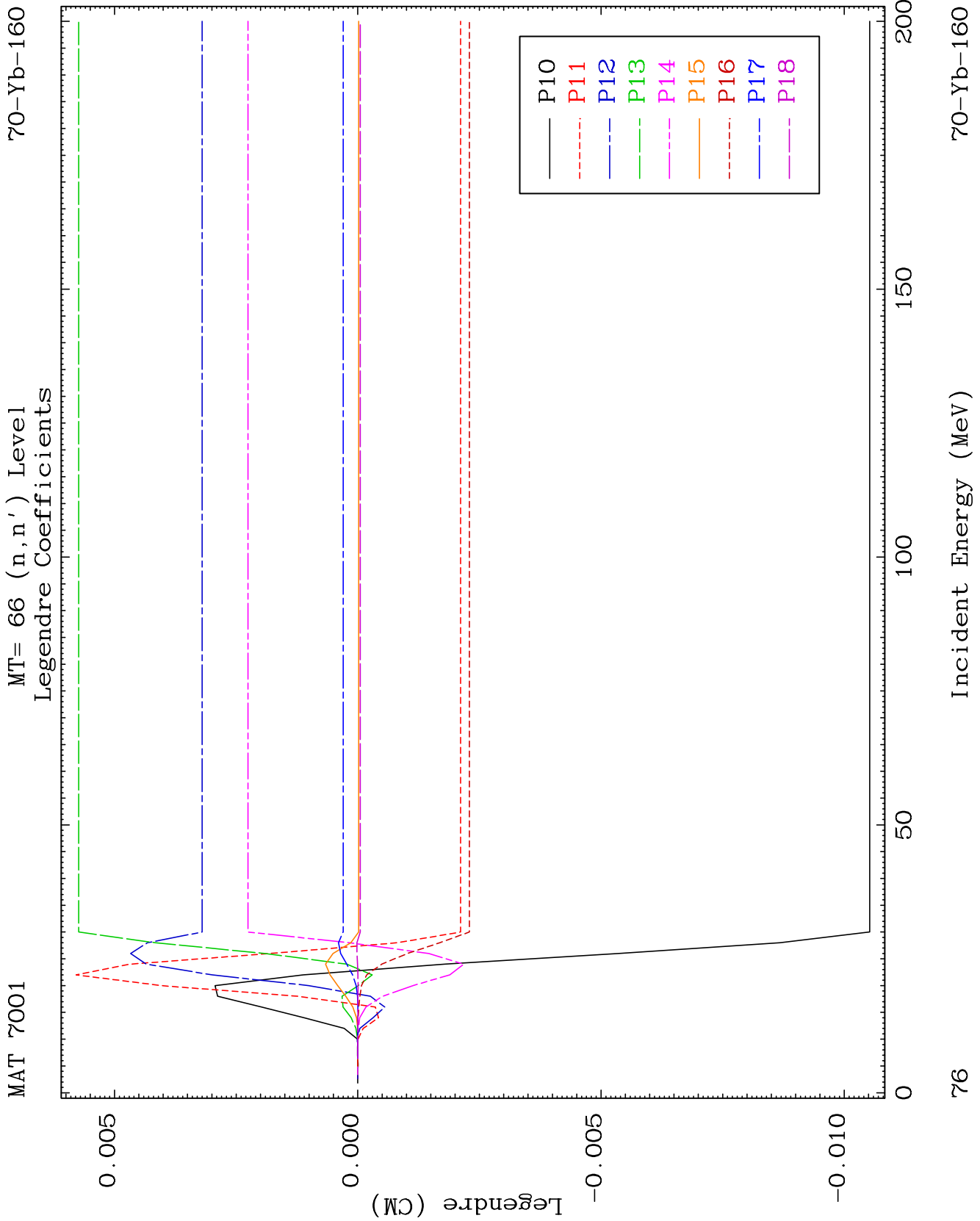








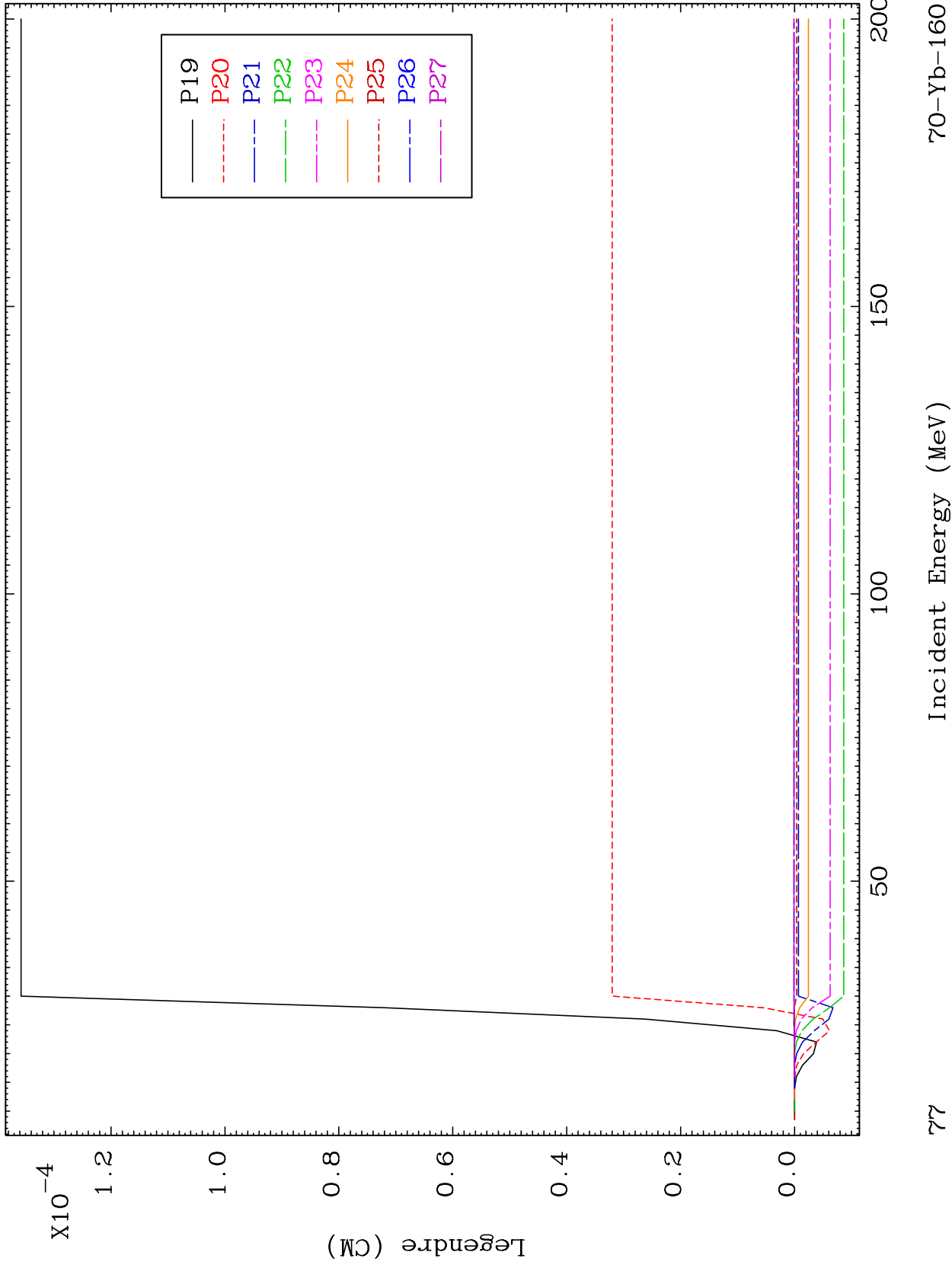




MAT 7001

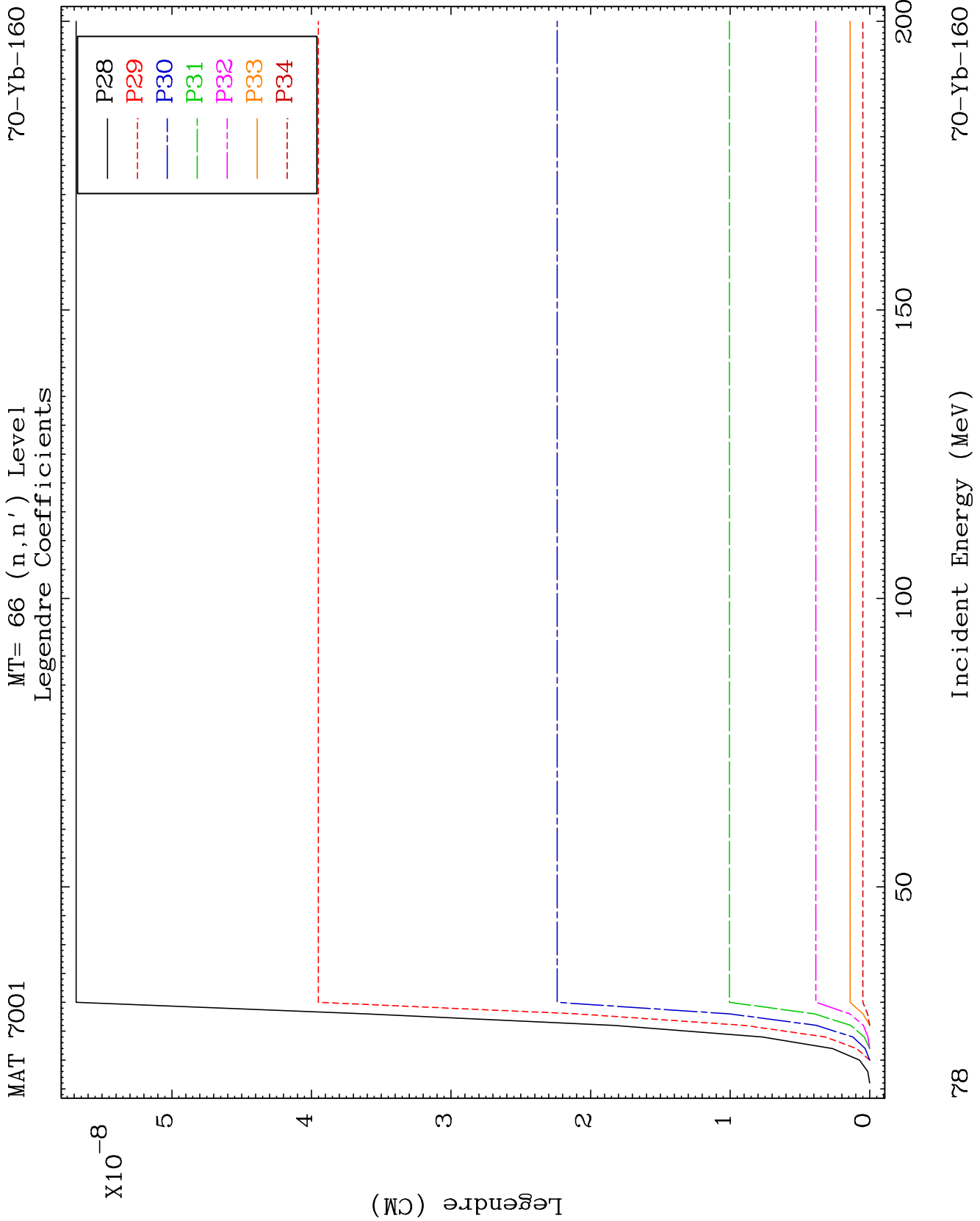
MT= 66 (n,n') Level
Legendre Coefficients

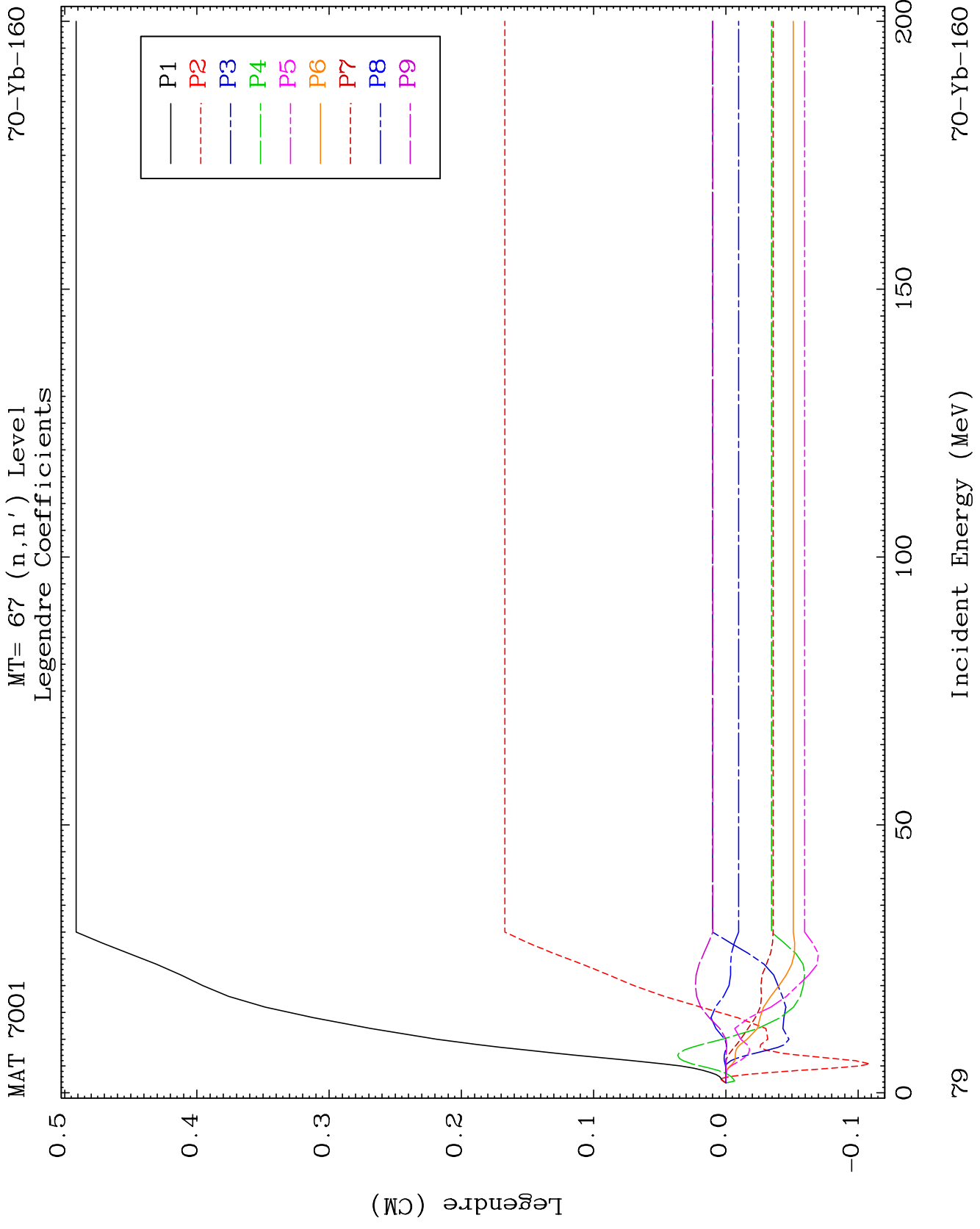
70-Yb-160

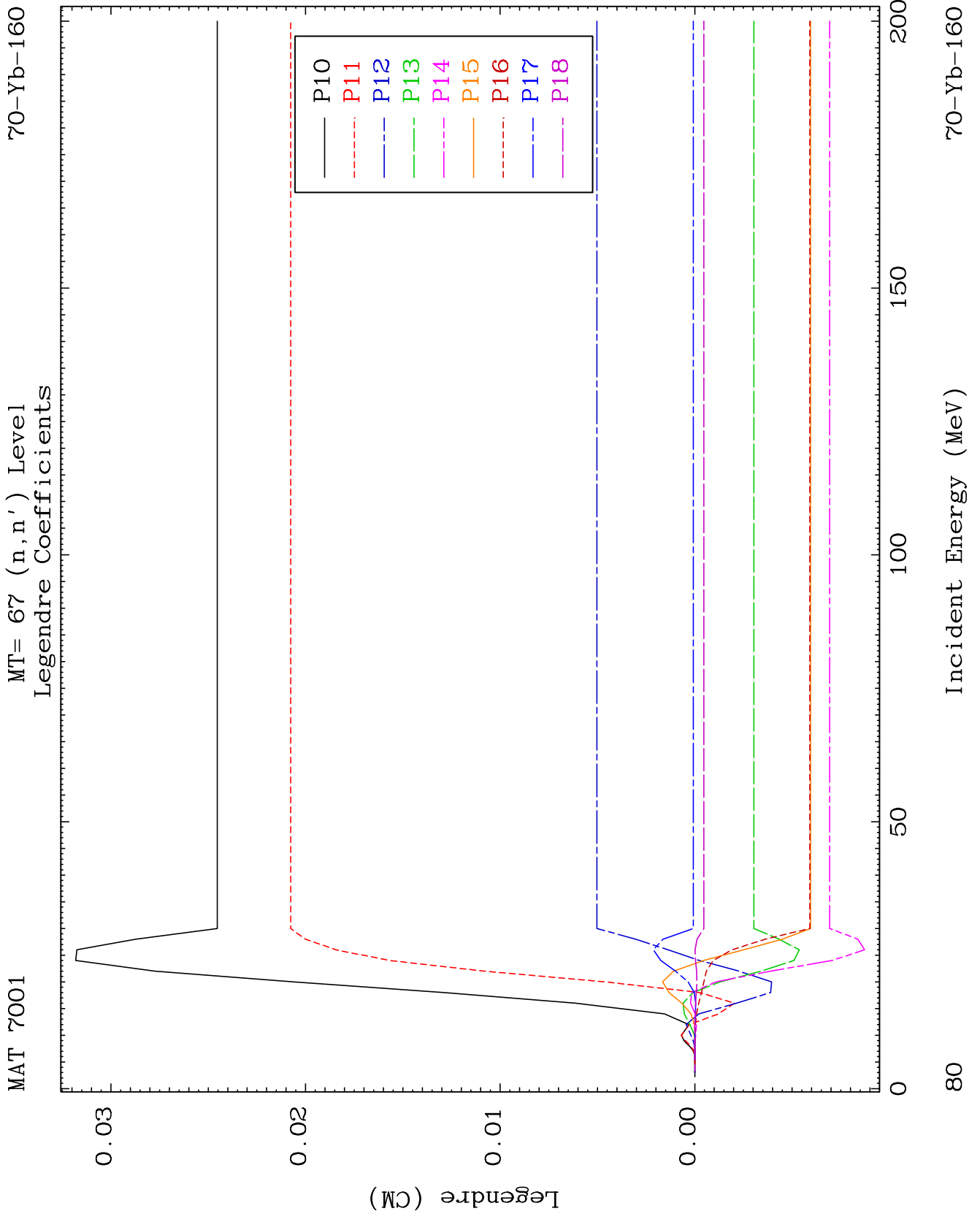


77

70-Yb-160



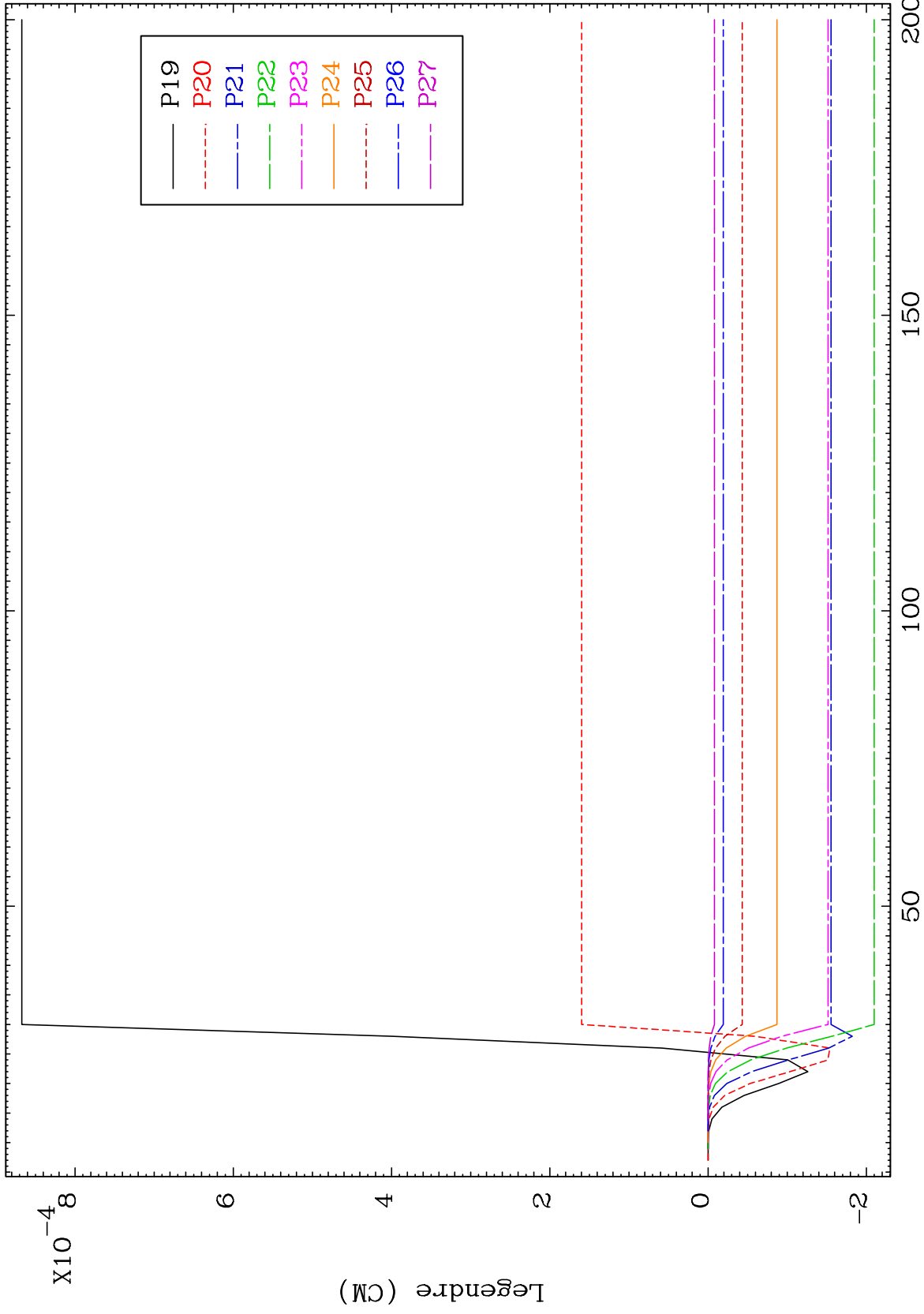




MAT 7001

MT= 67 (n,n') Level
Legendre Coefficients

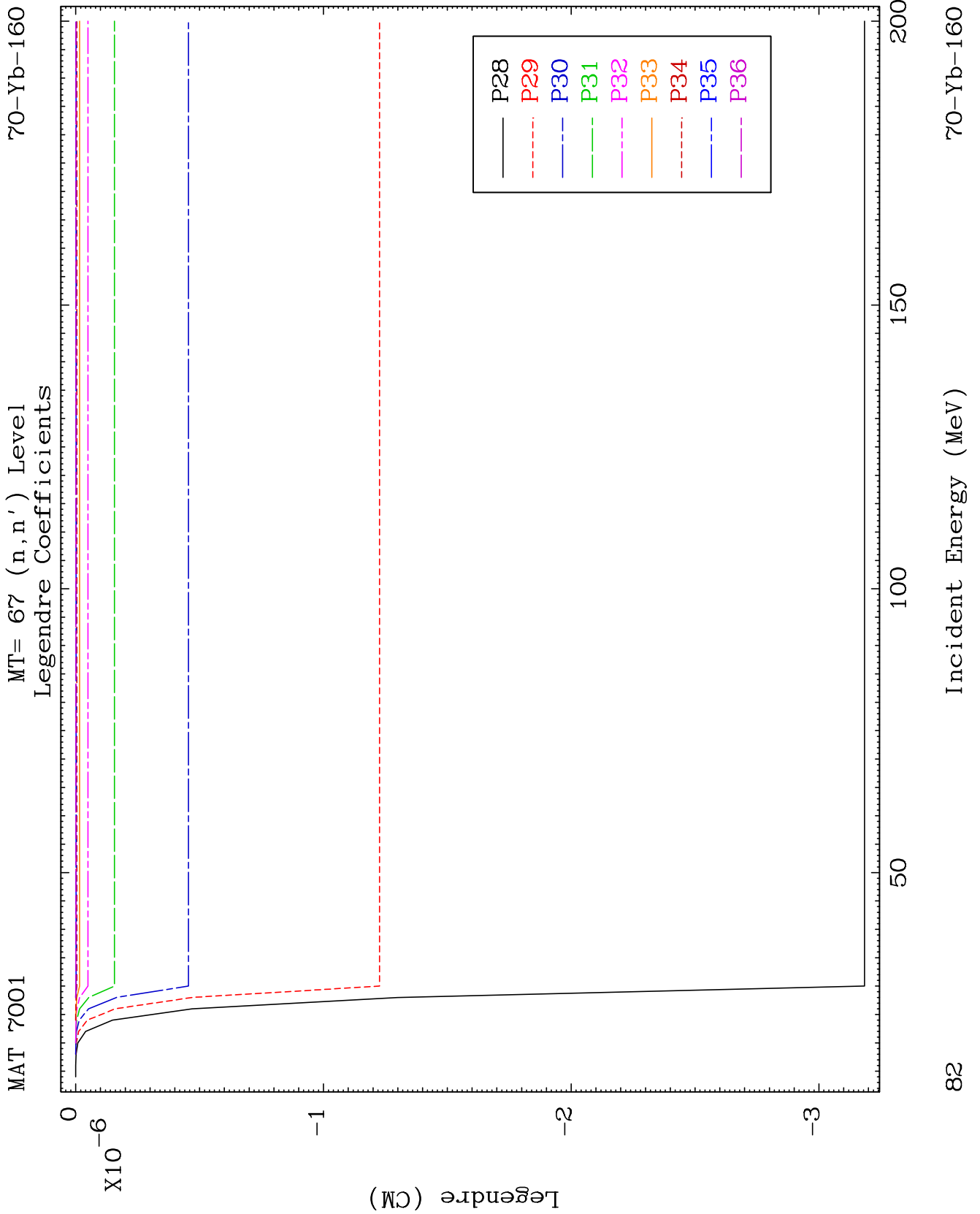
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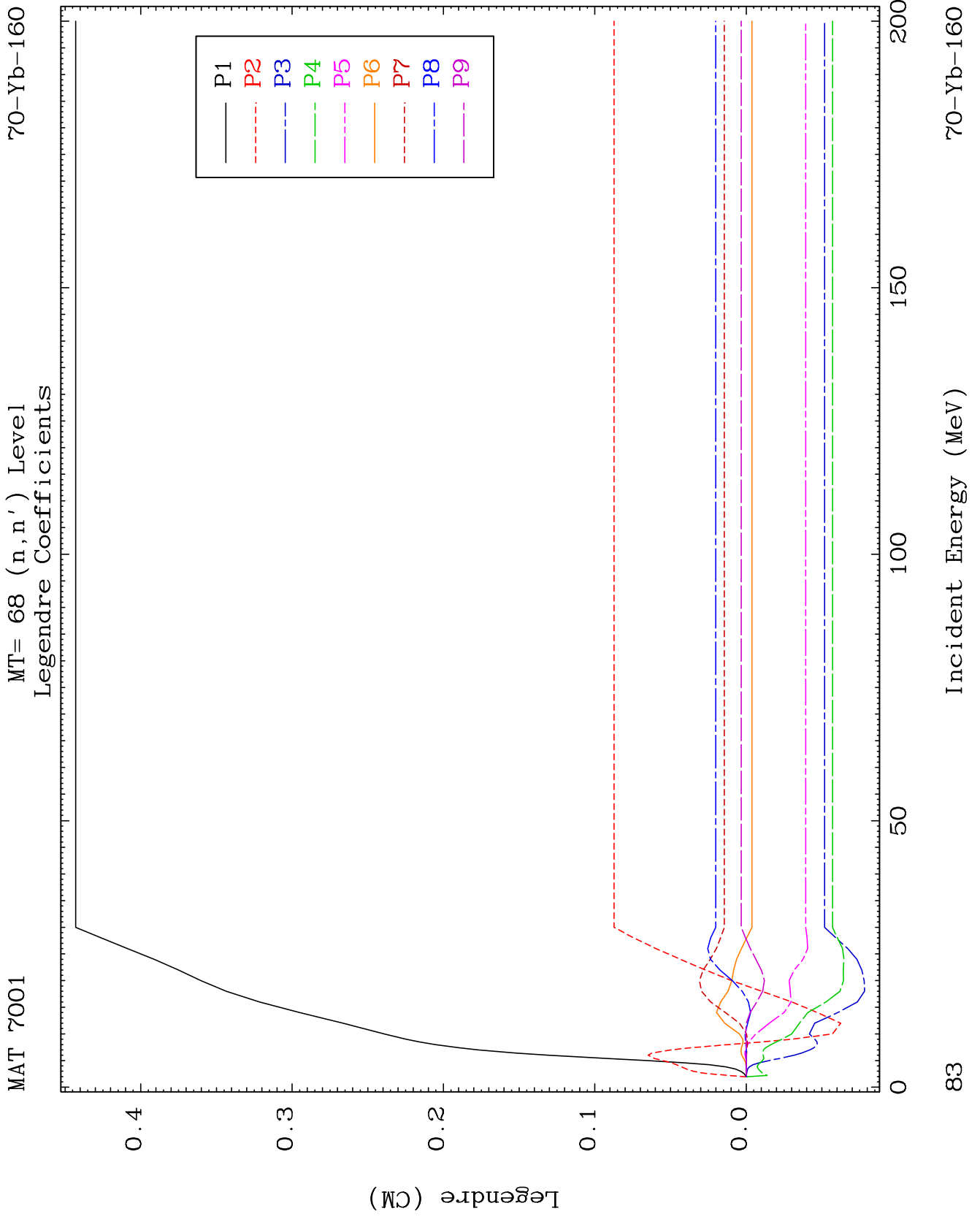


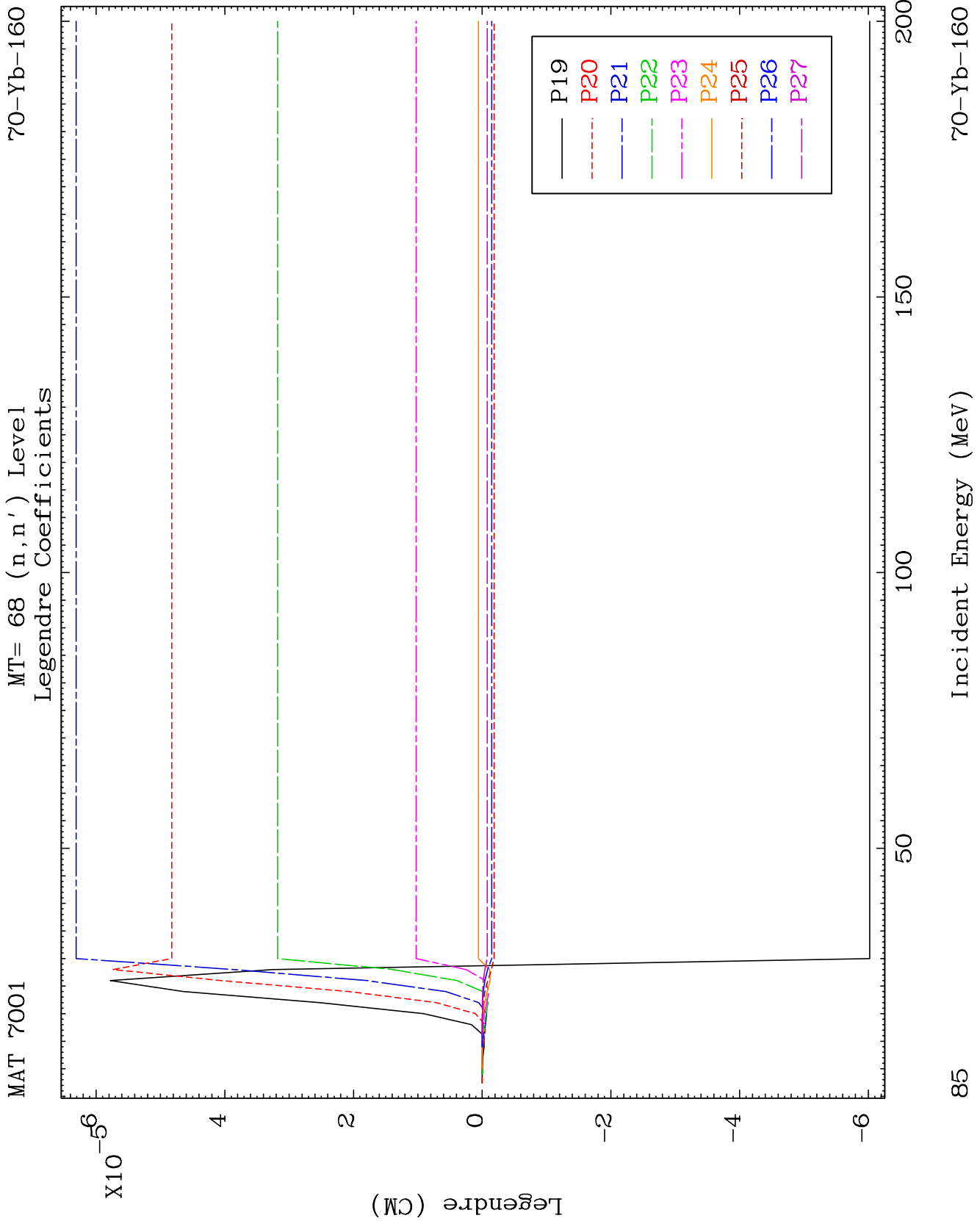
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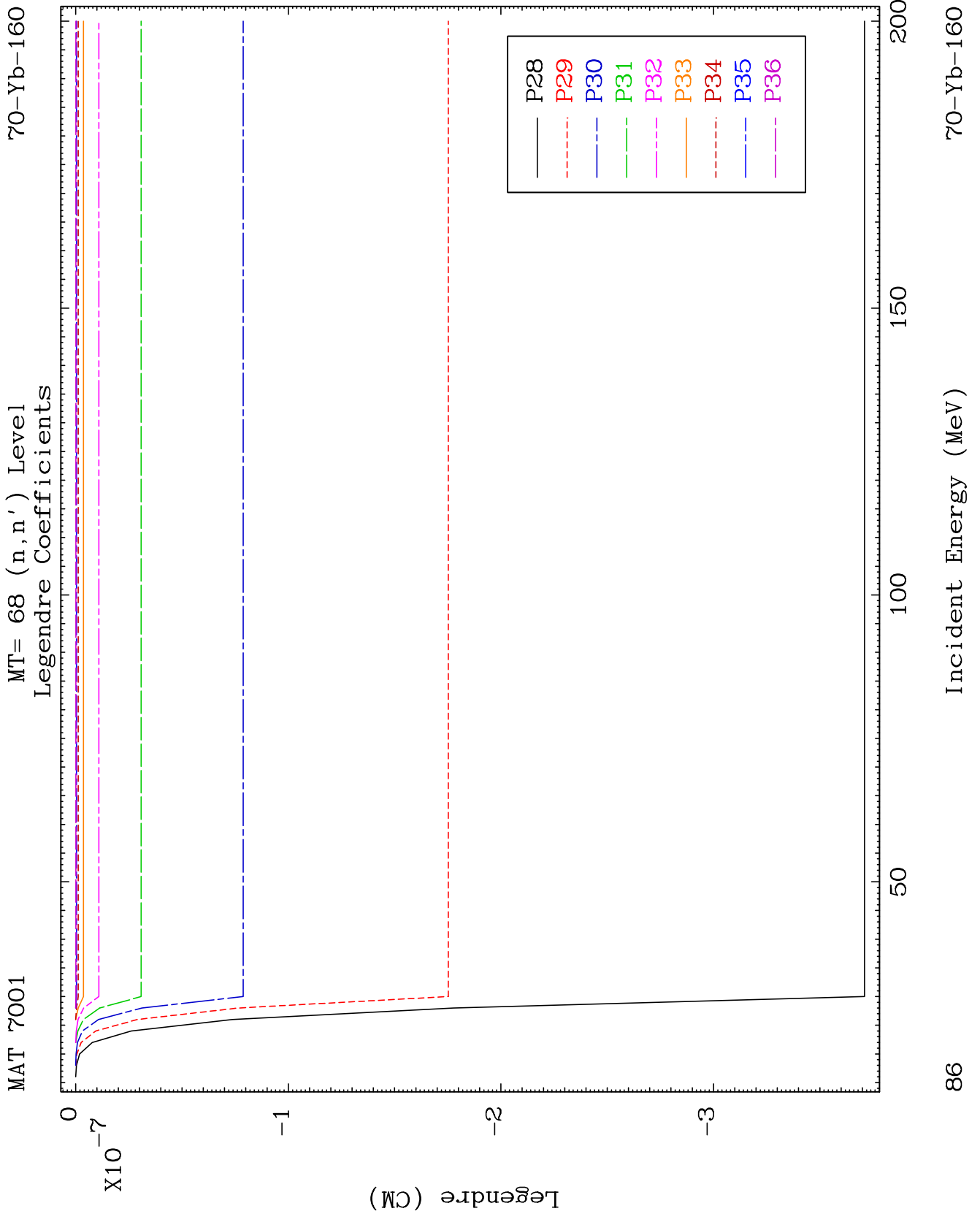
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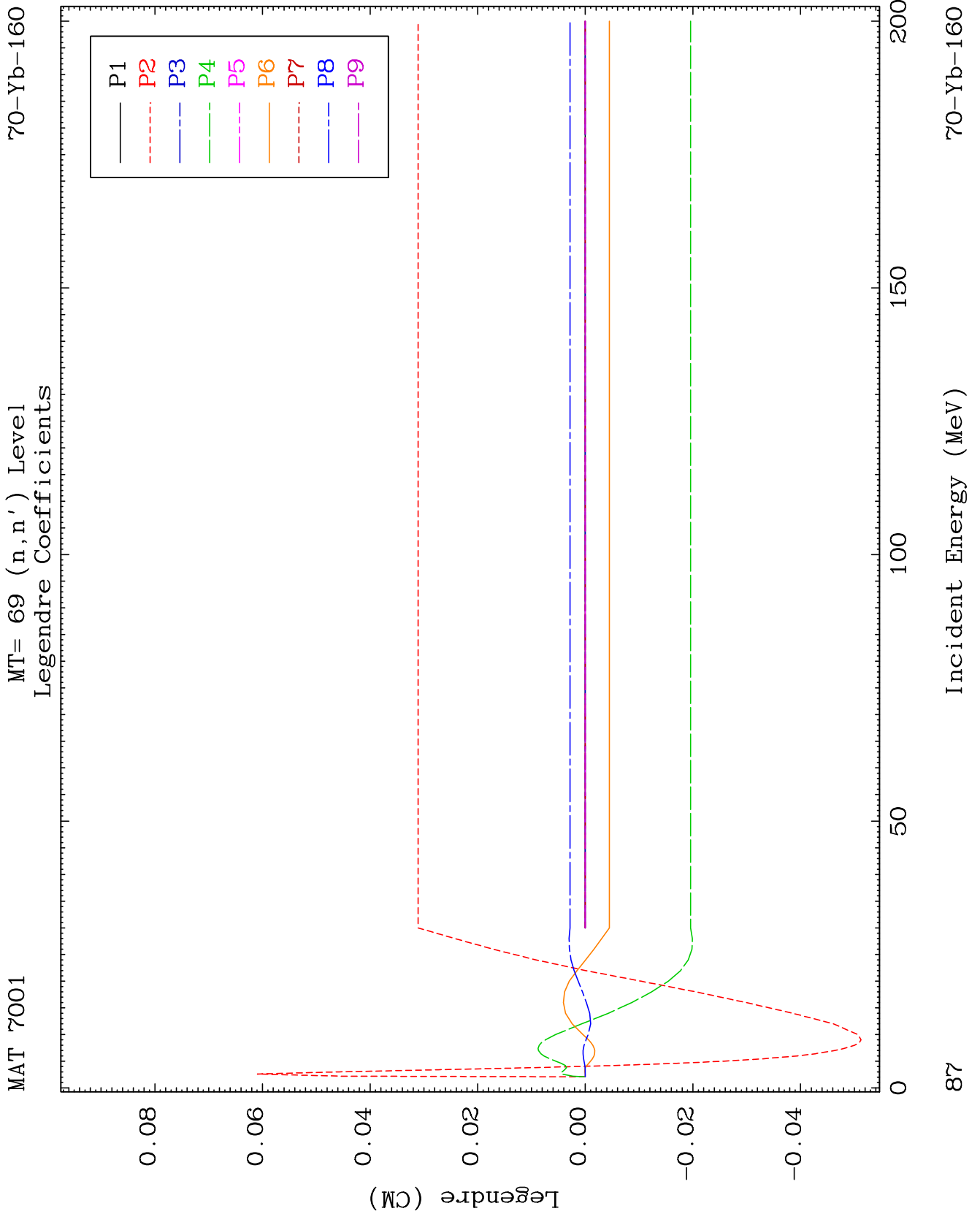
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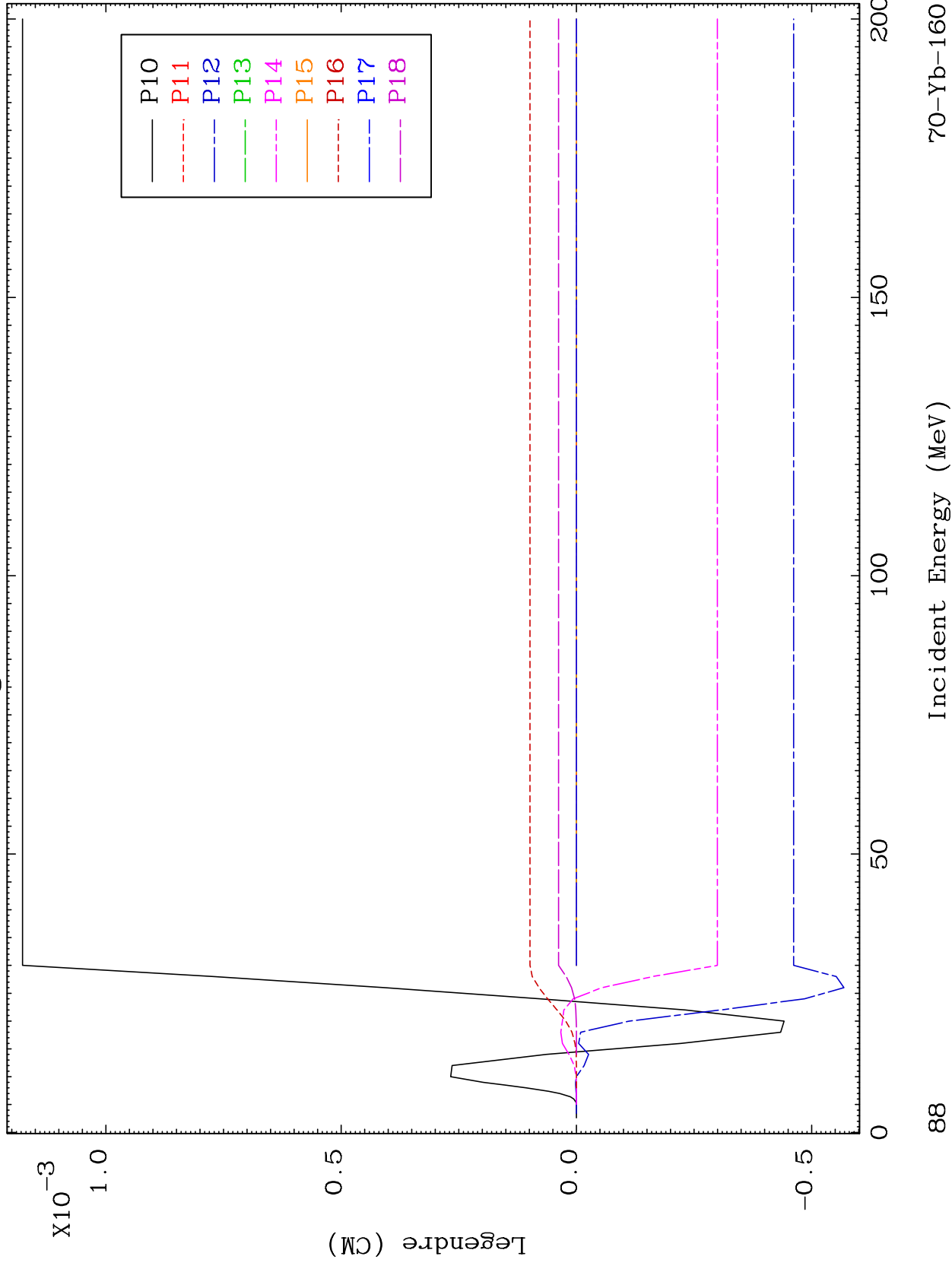




MAT 7001

MT= 69 (n,n') Level
Legendre Coefficients

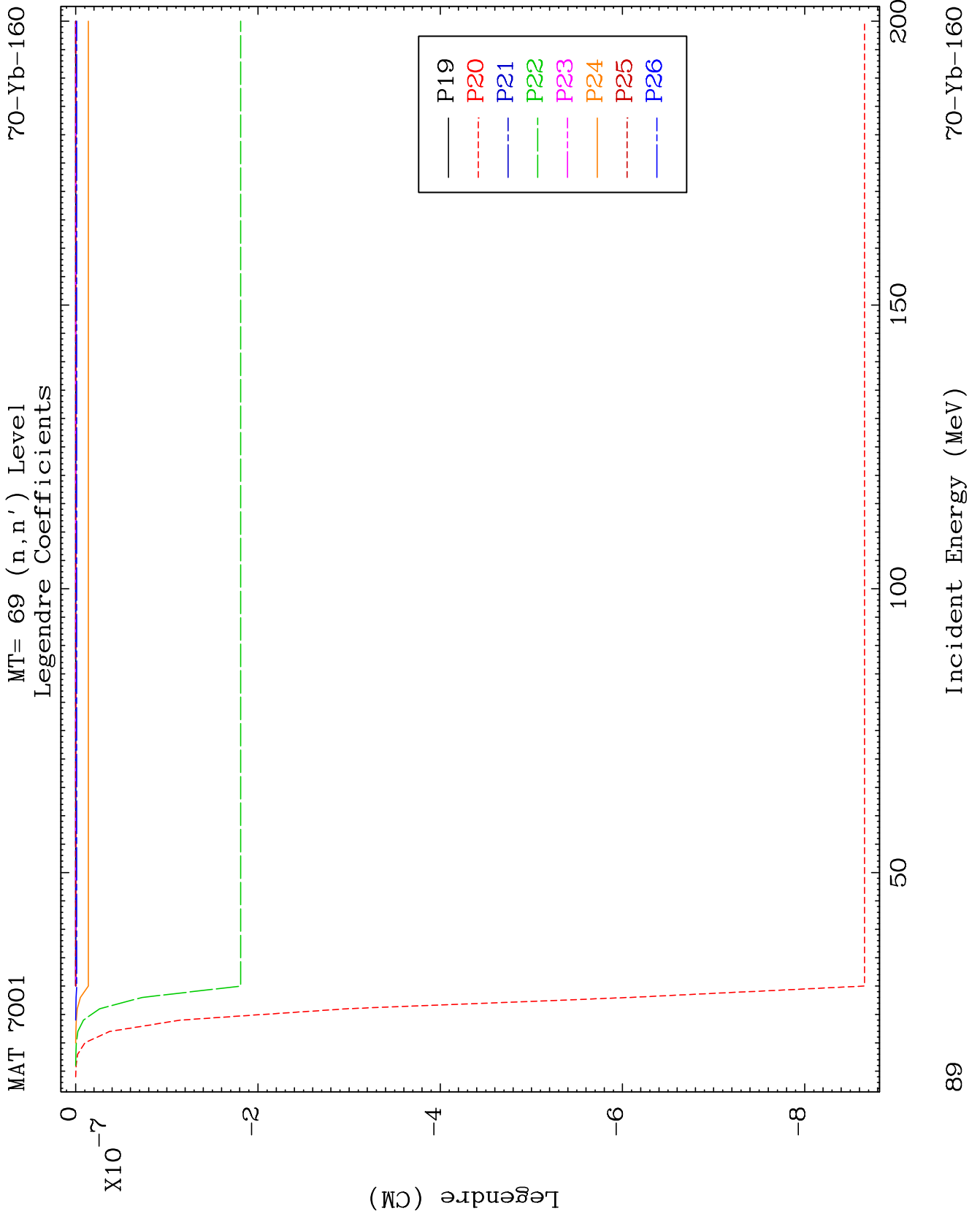
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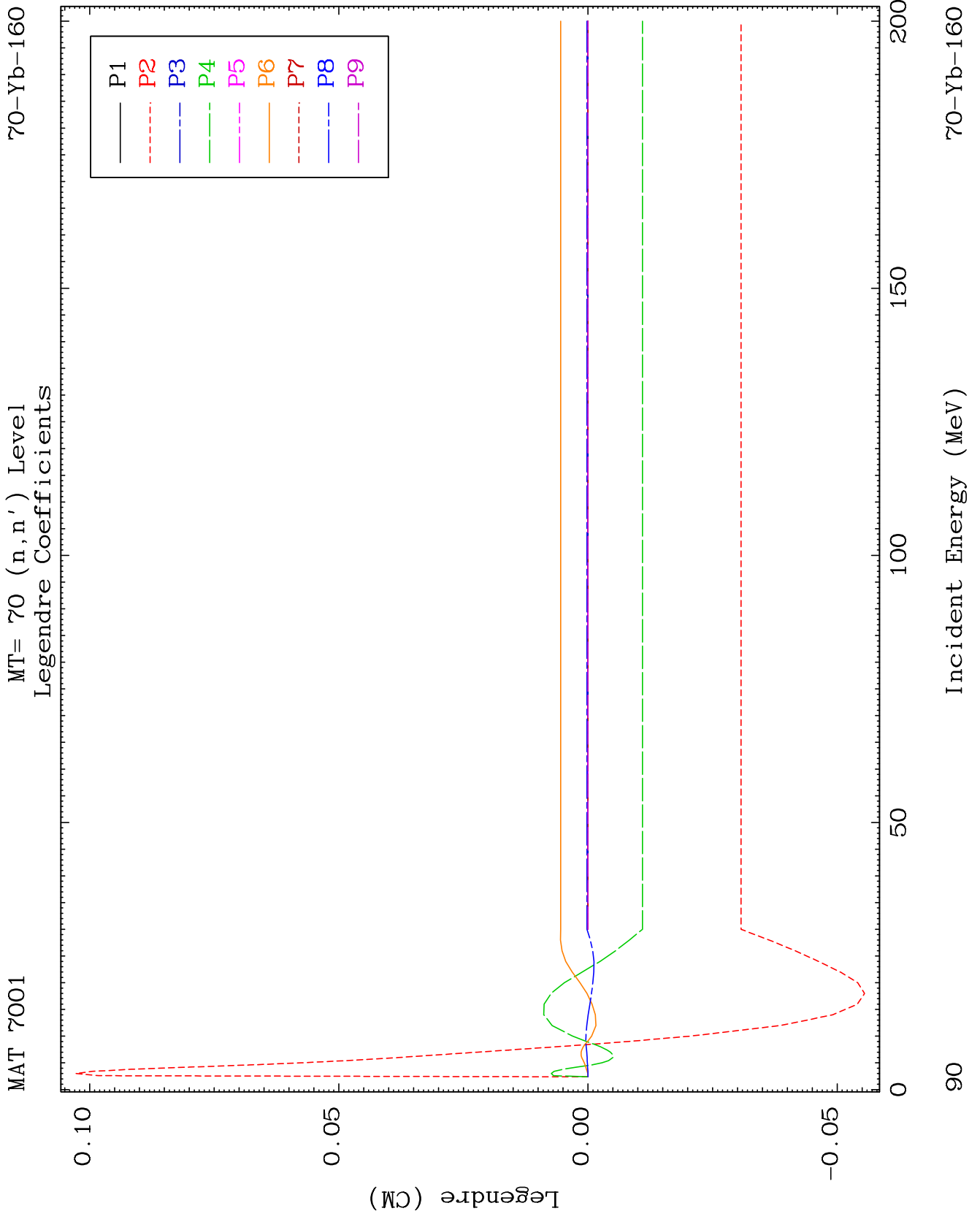


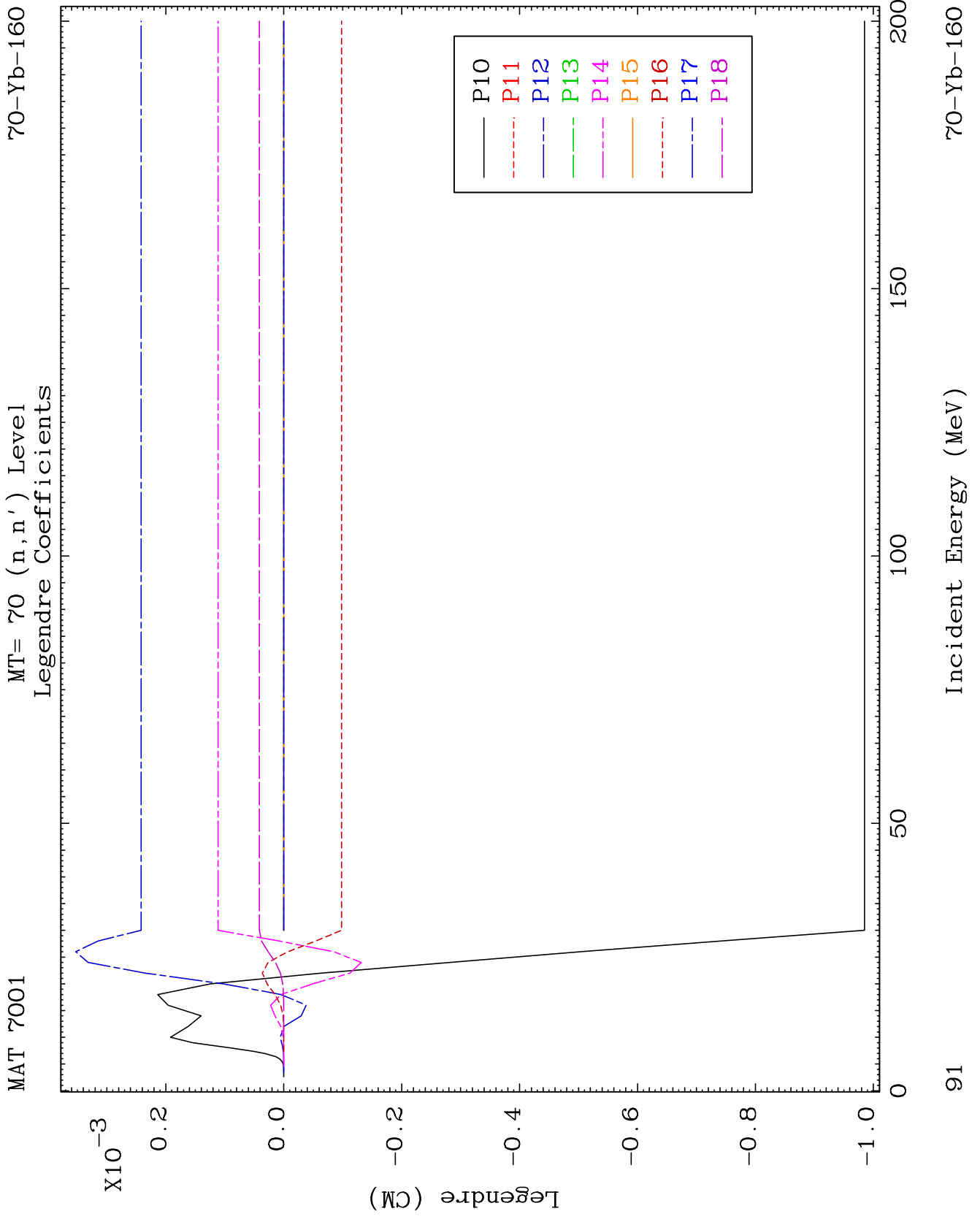
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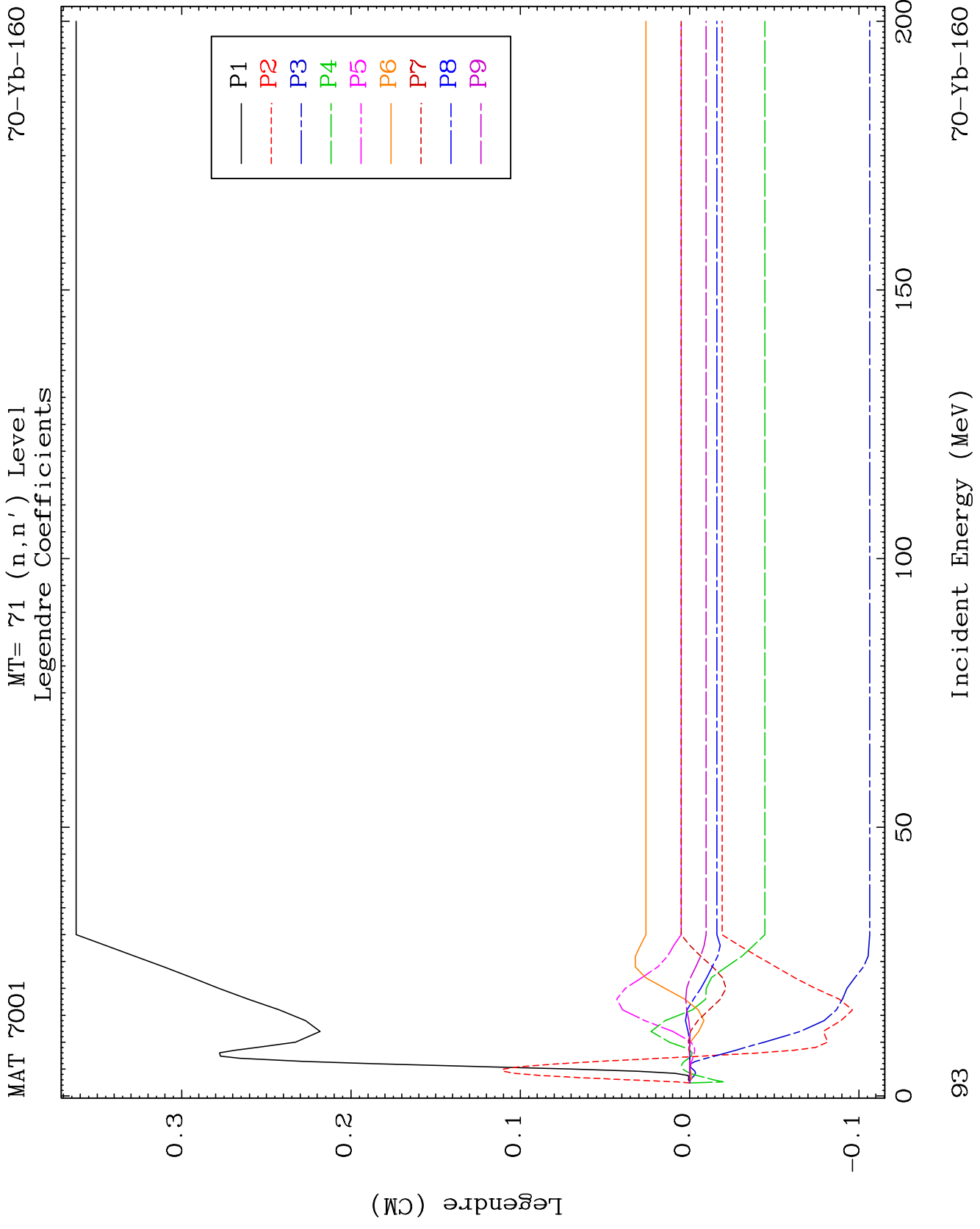
Incident Energy (MeV)

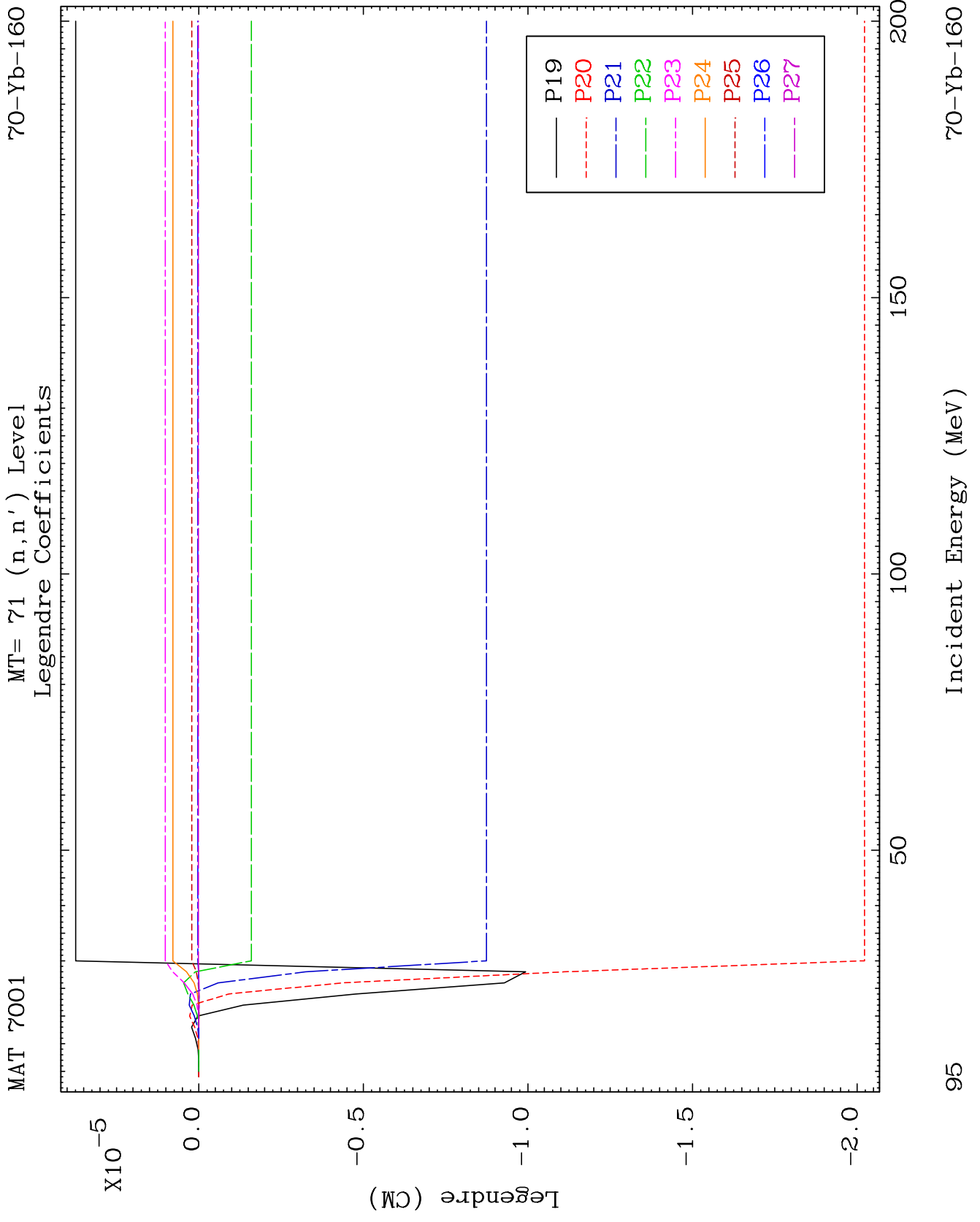
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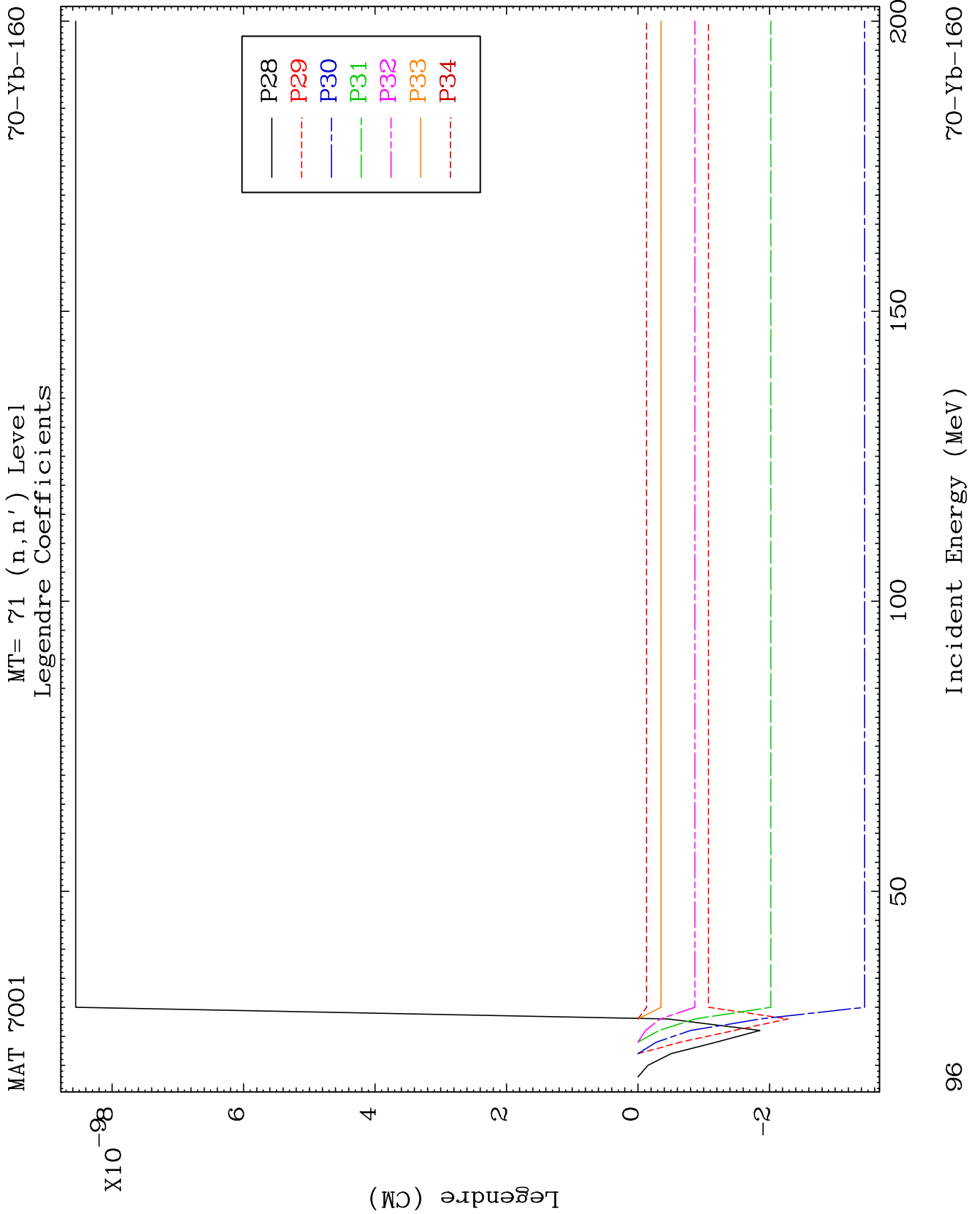


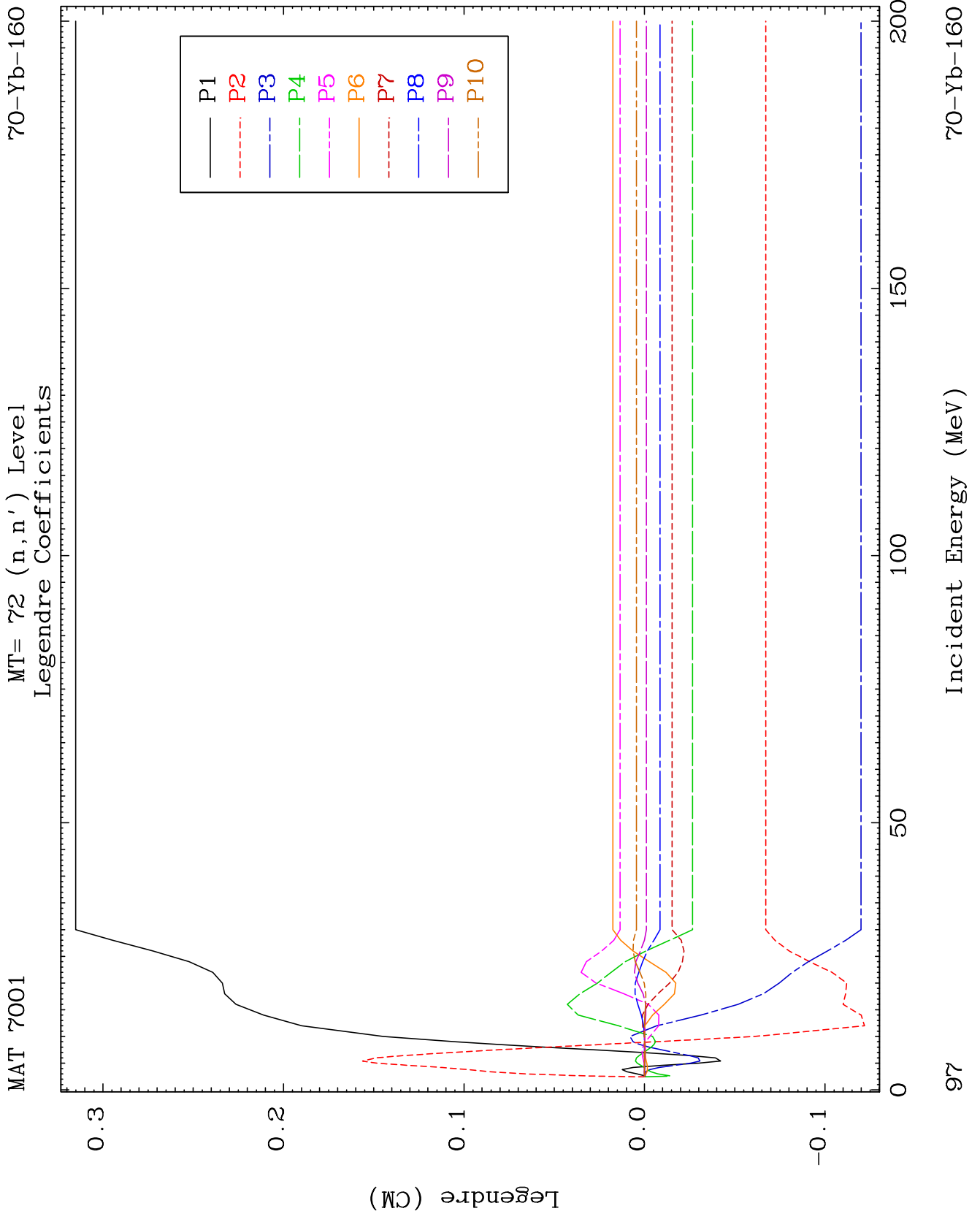


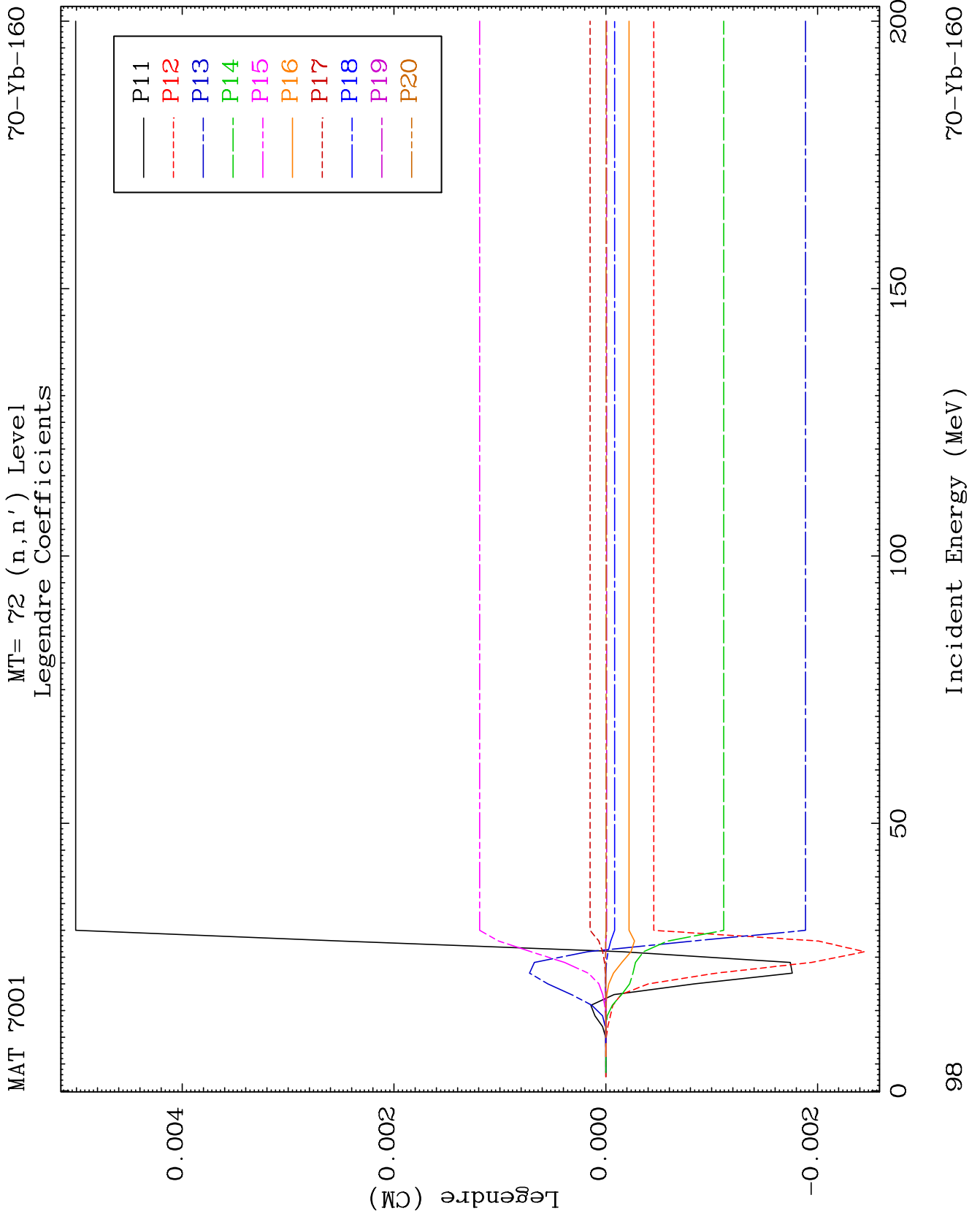


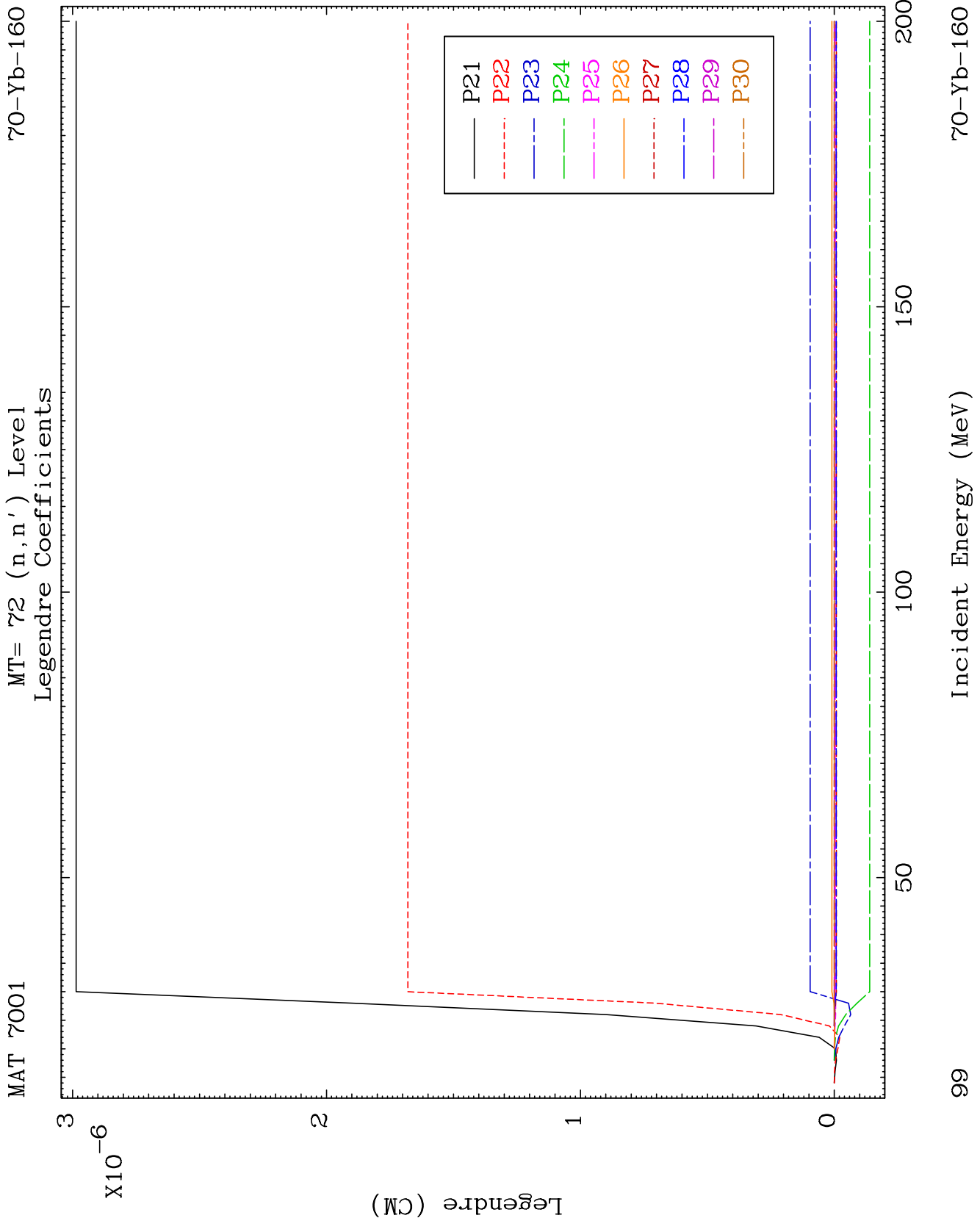


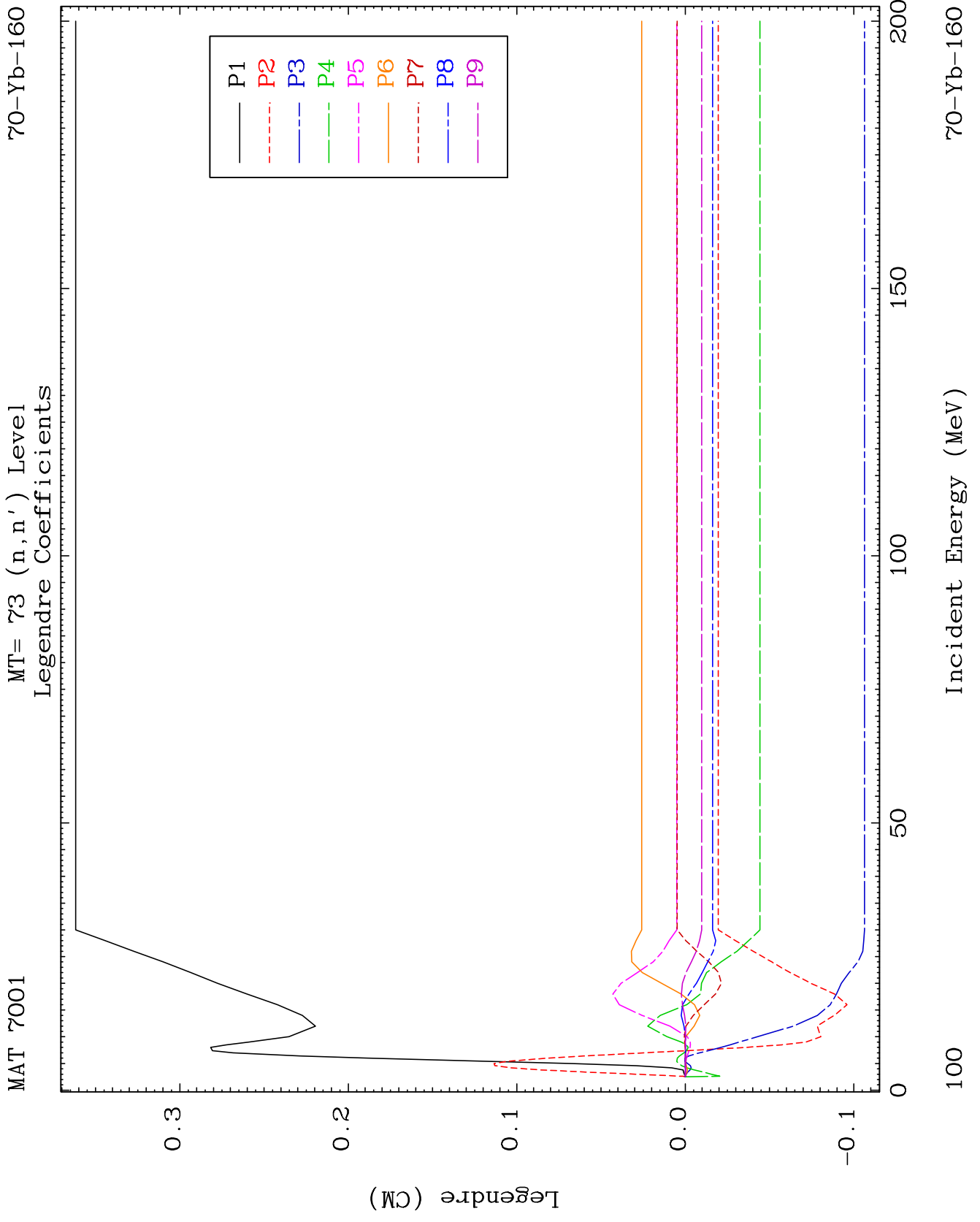


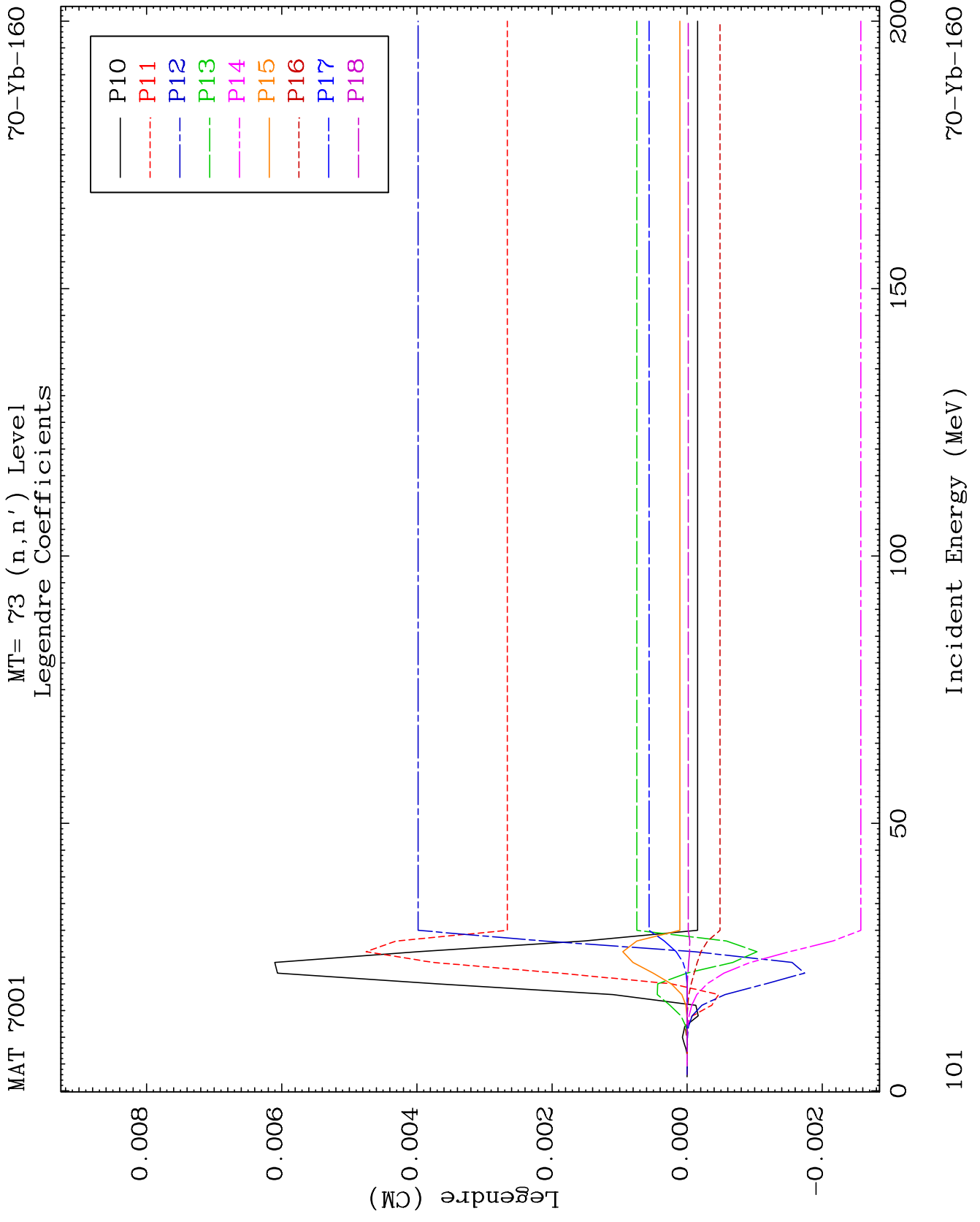


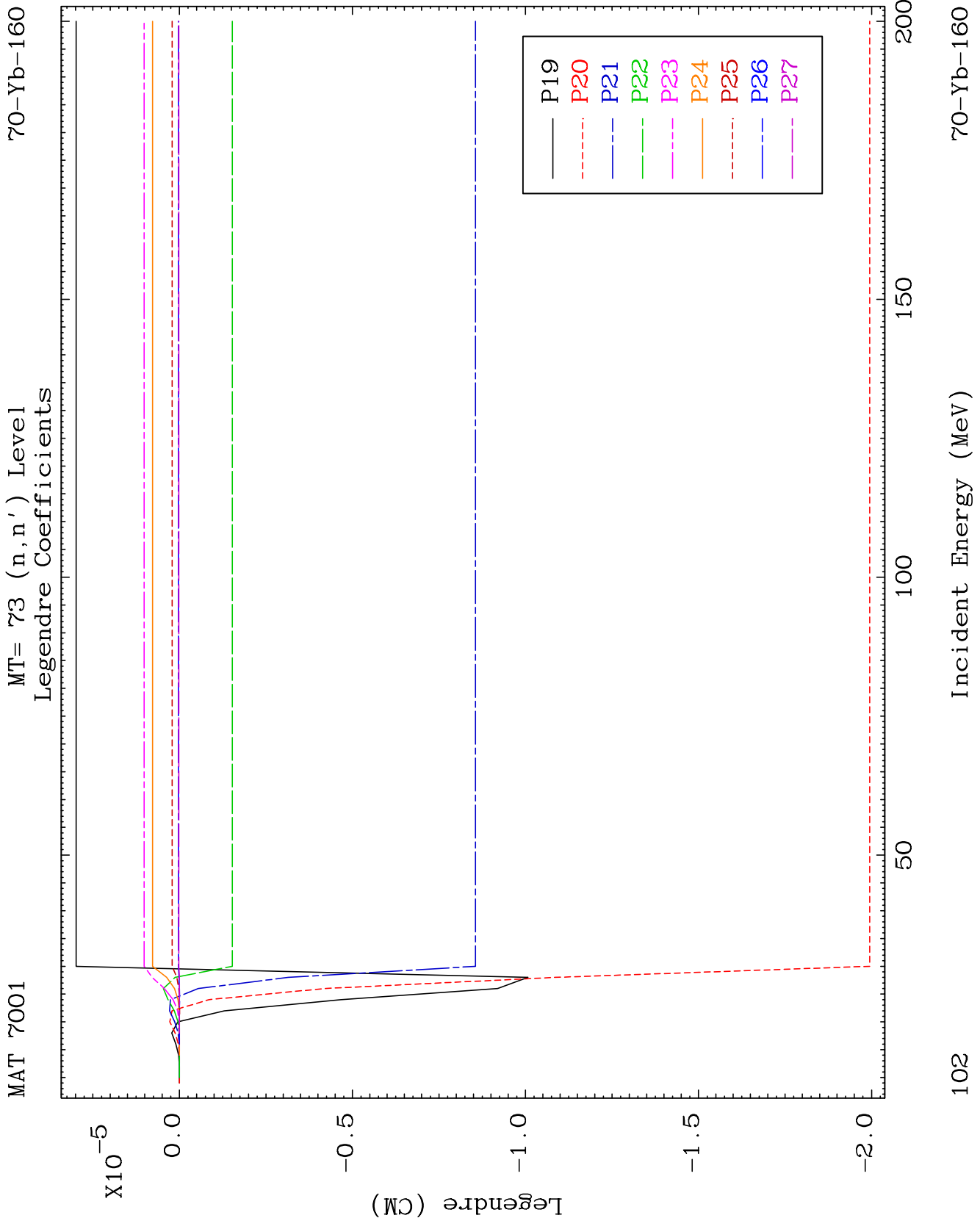


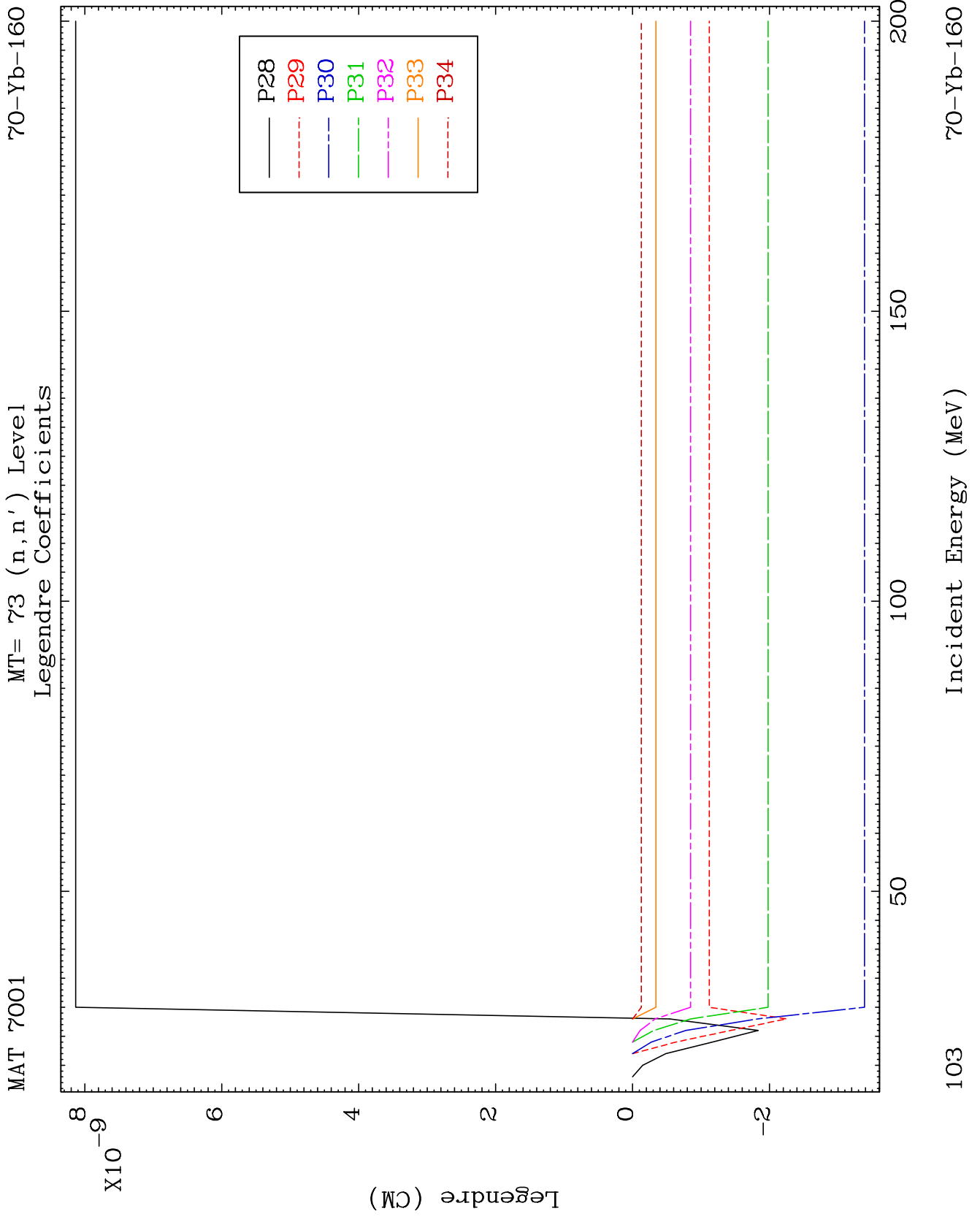


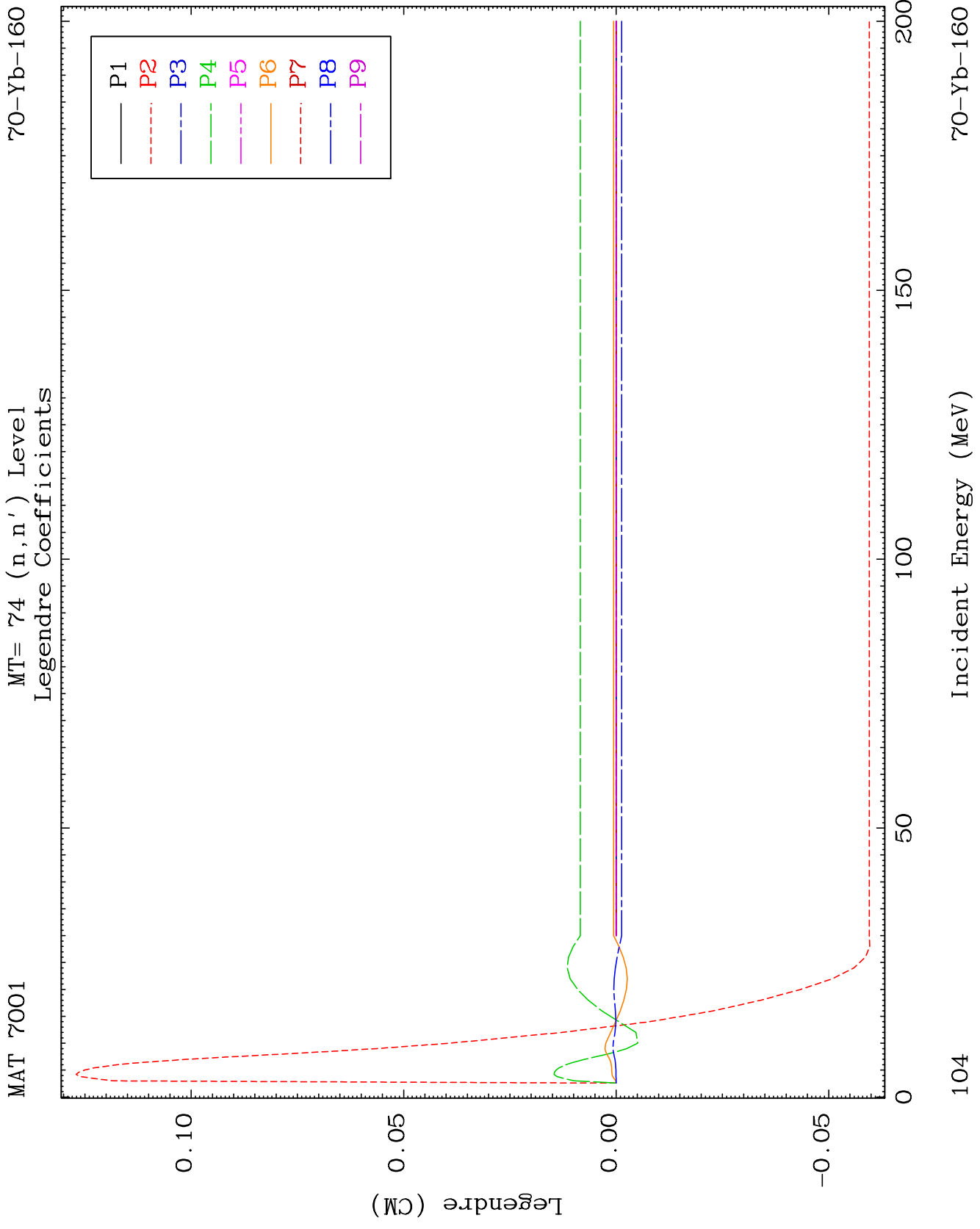


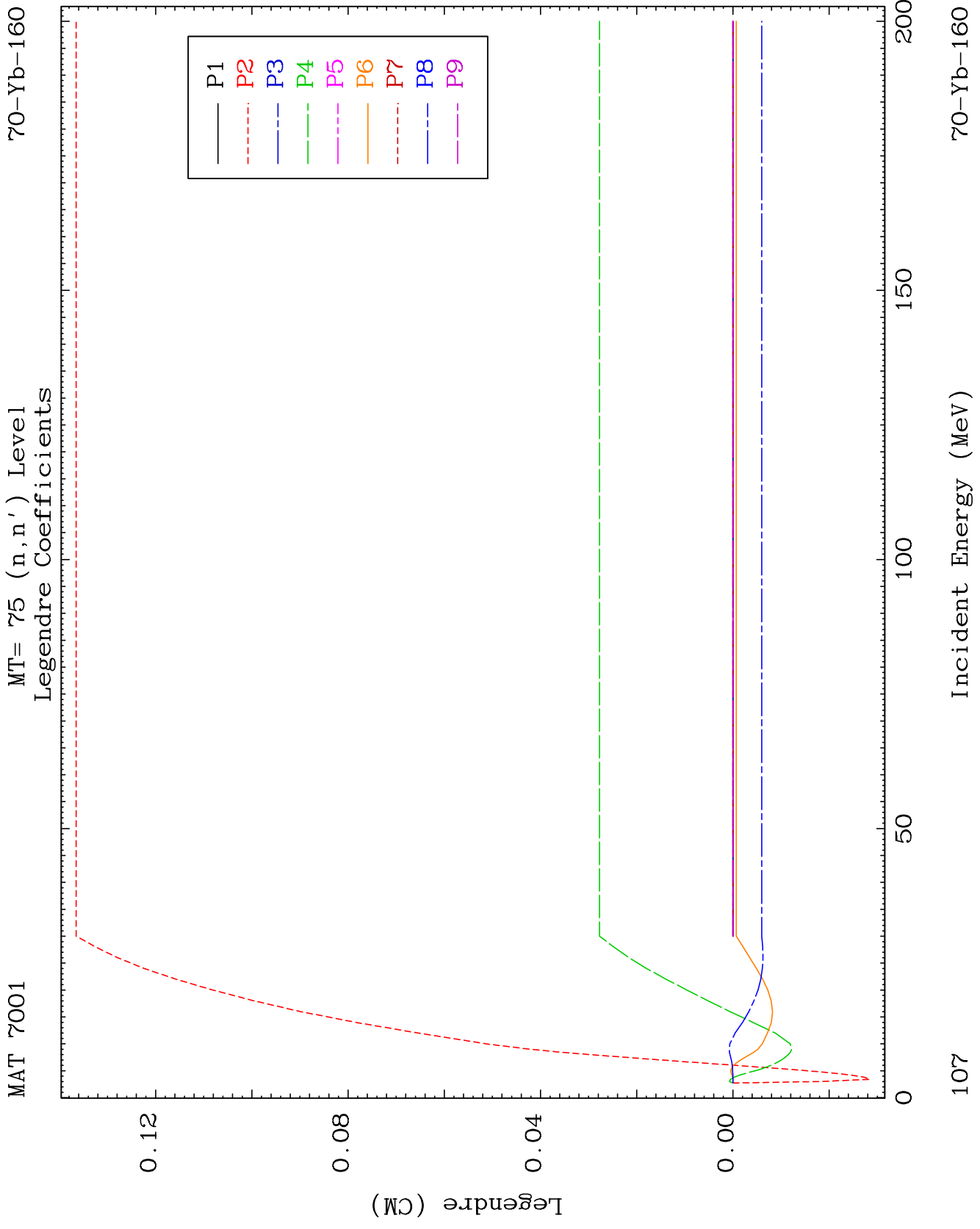


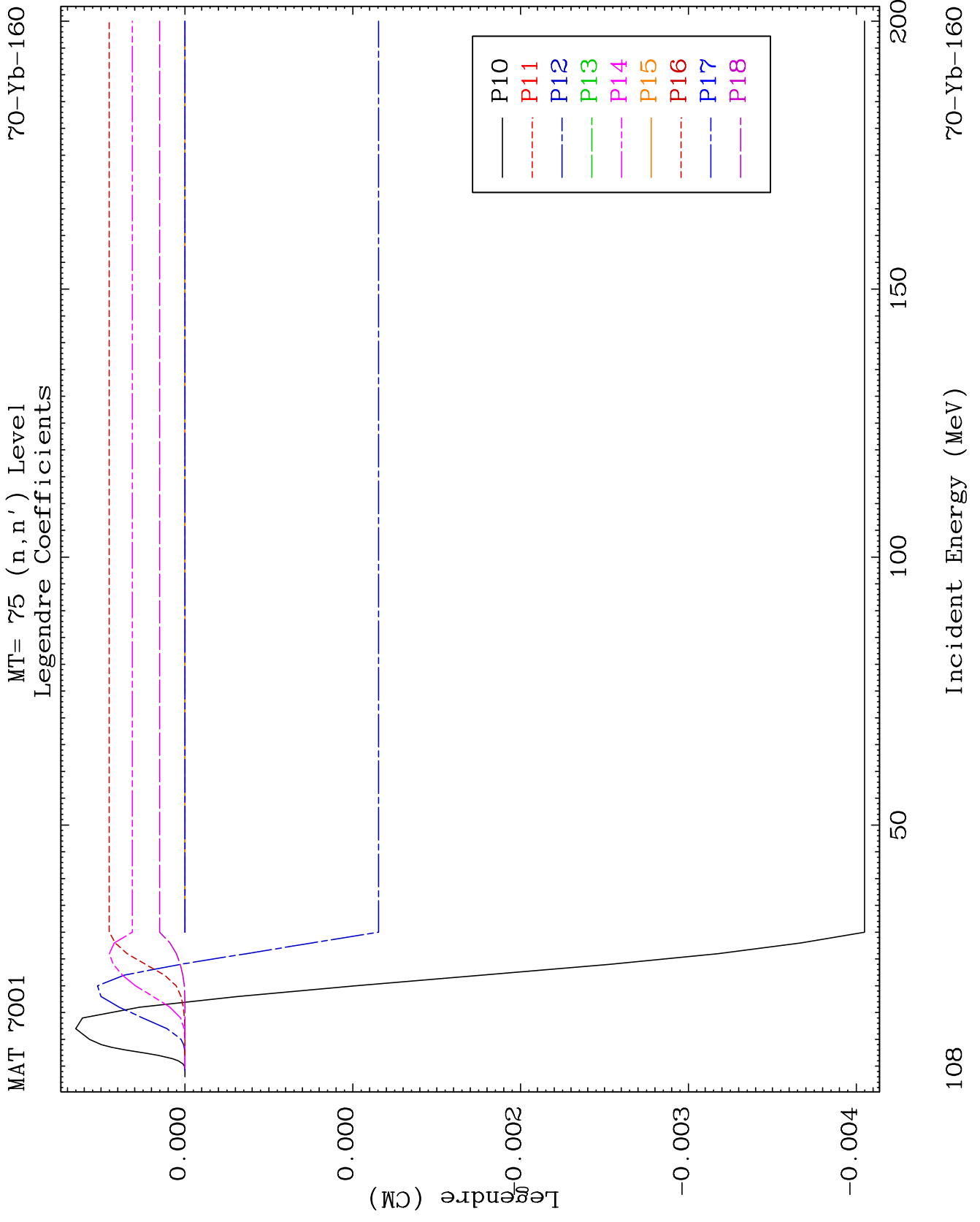


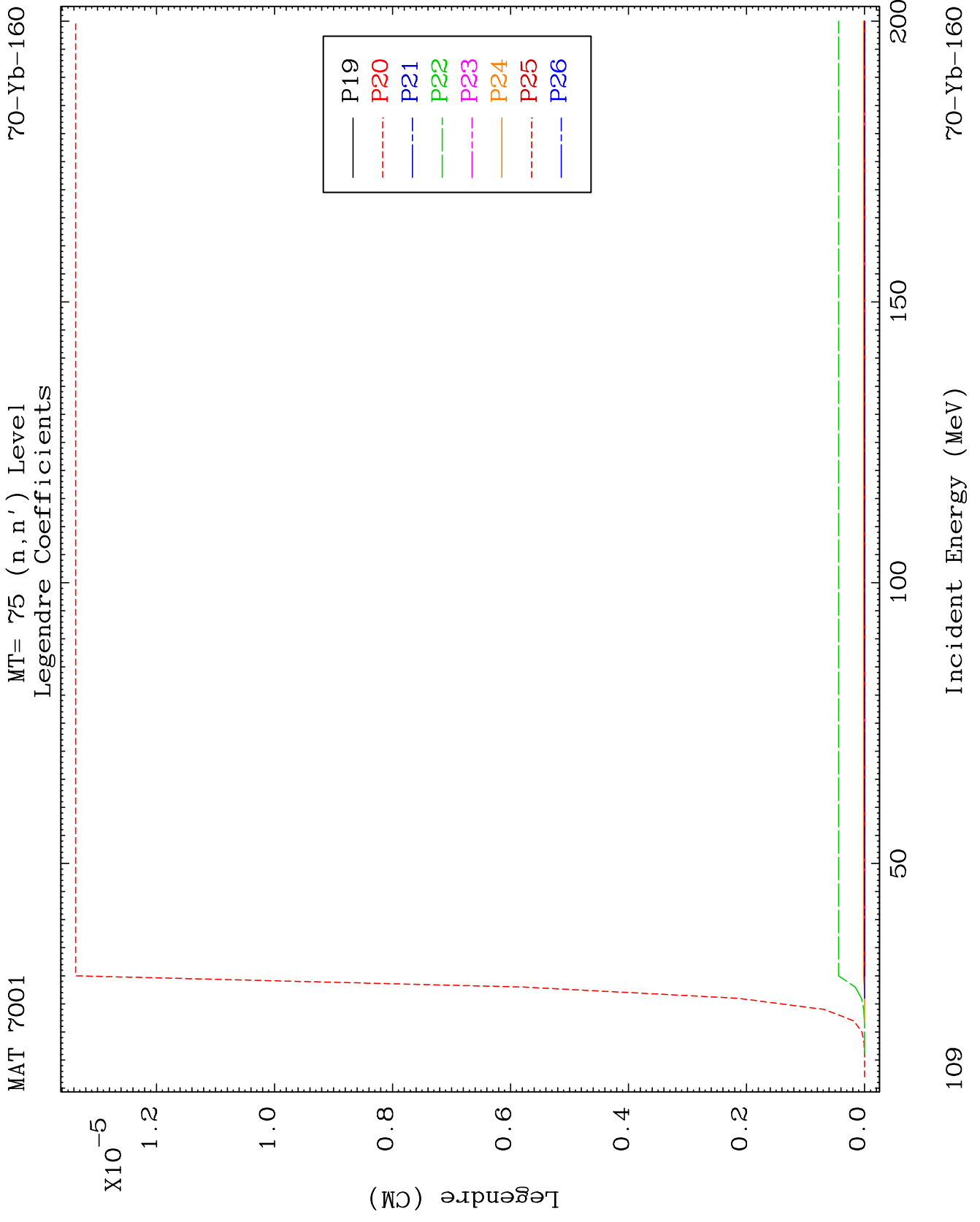










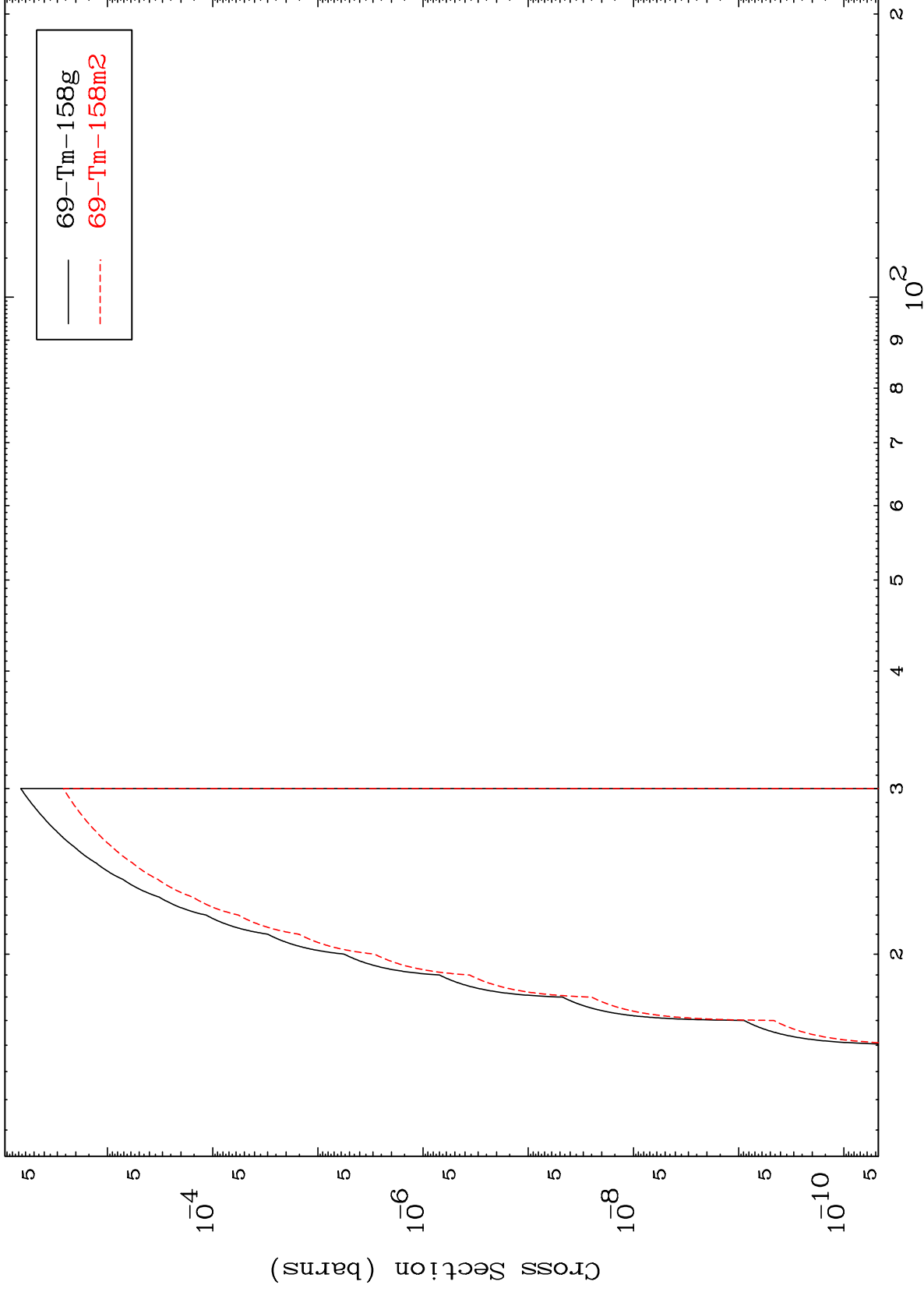


MAT 7001

(n,n') d

70-Yb-160

Radionuclide Production Cross Section



110

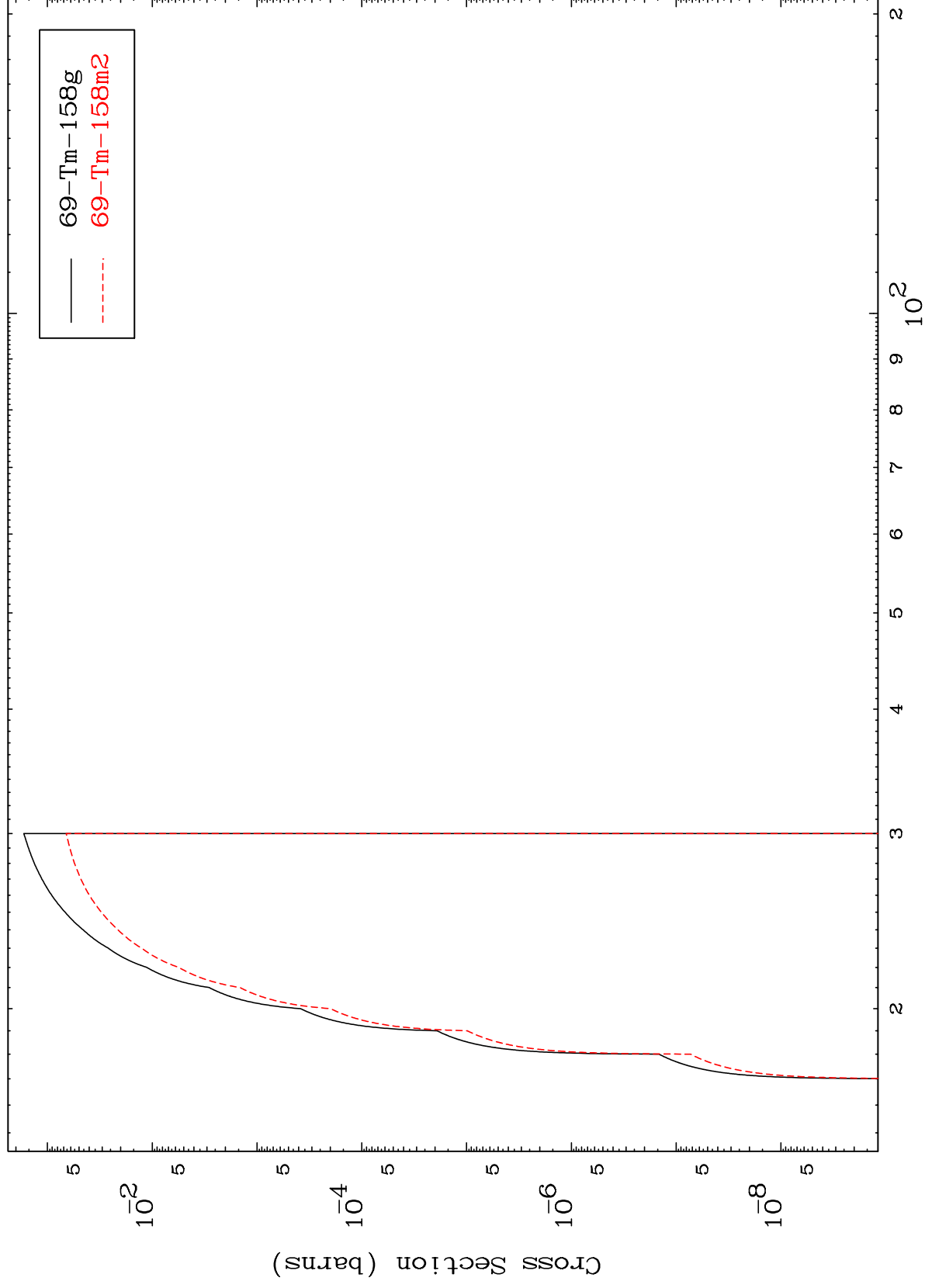
Incident Energy (MeV)

70-Yb-160

MAT 7001

70-Yb-160

(n,2n) p
Radionuclide Production Cross Section



111

70-Yb-160

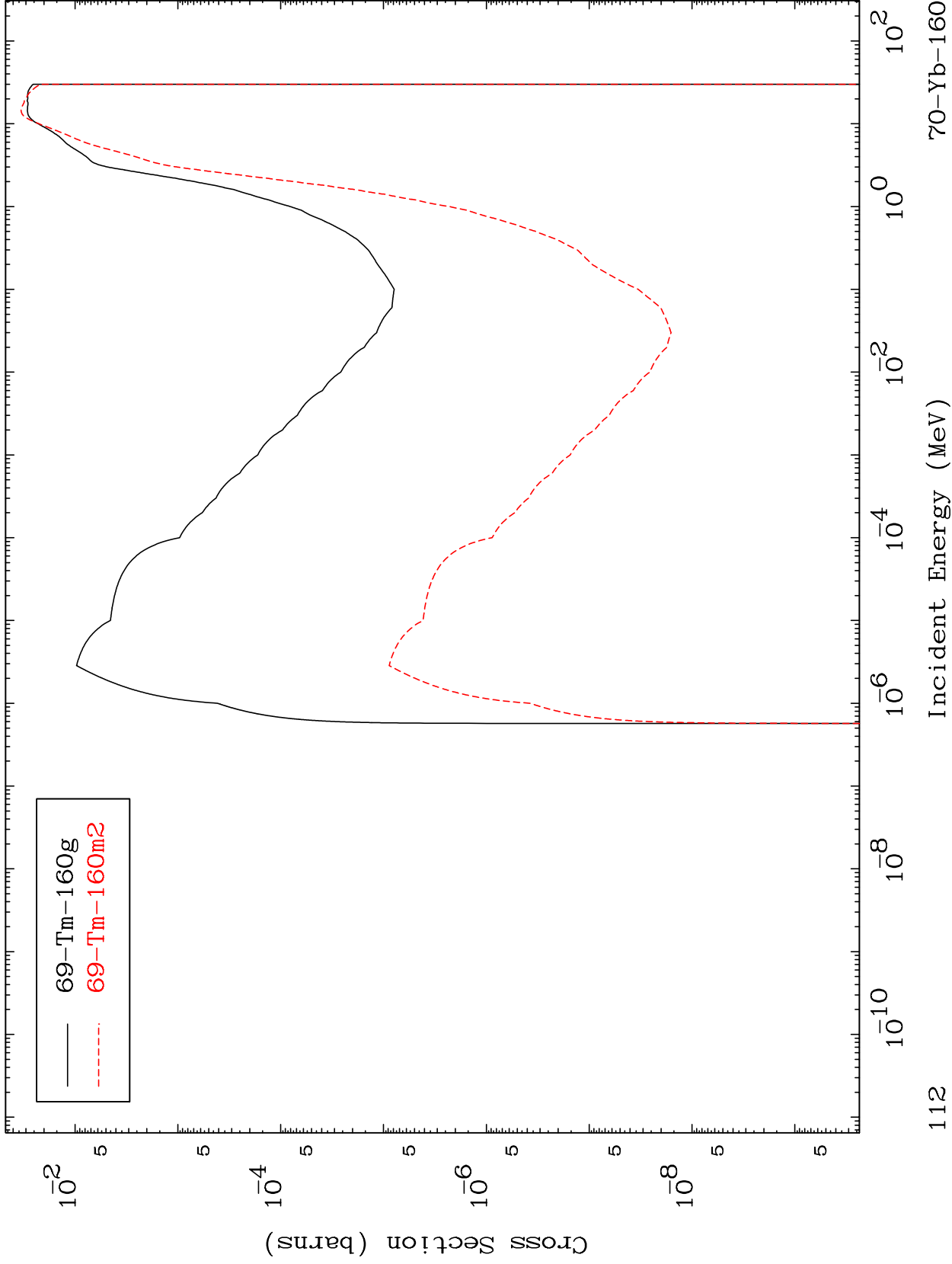
Incident Energy (MeV)

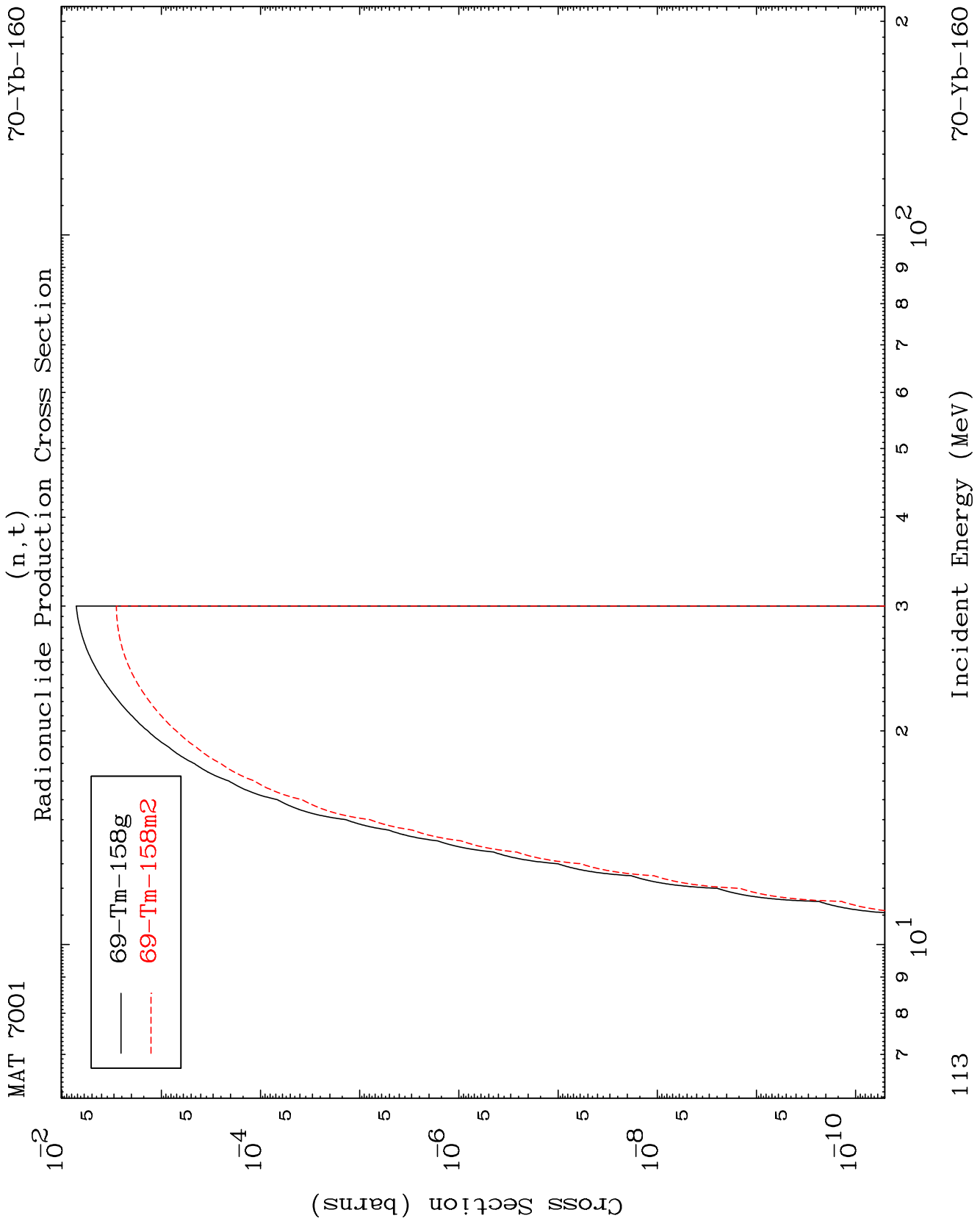
MAT 7001

(n,p)

70-Yb-160

Radionuclide Production Cross Section





MAT 7001

(n,p) α

70-Yb-160

Radionuclide Production Cross Section

