

Program EVALPLOT  
(Version 2021-1)

by

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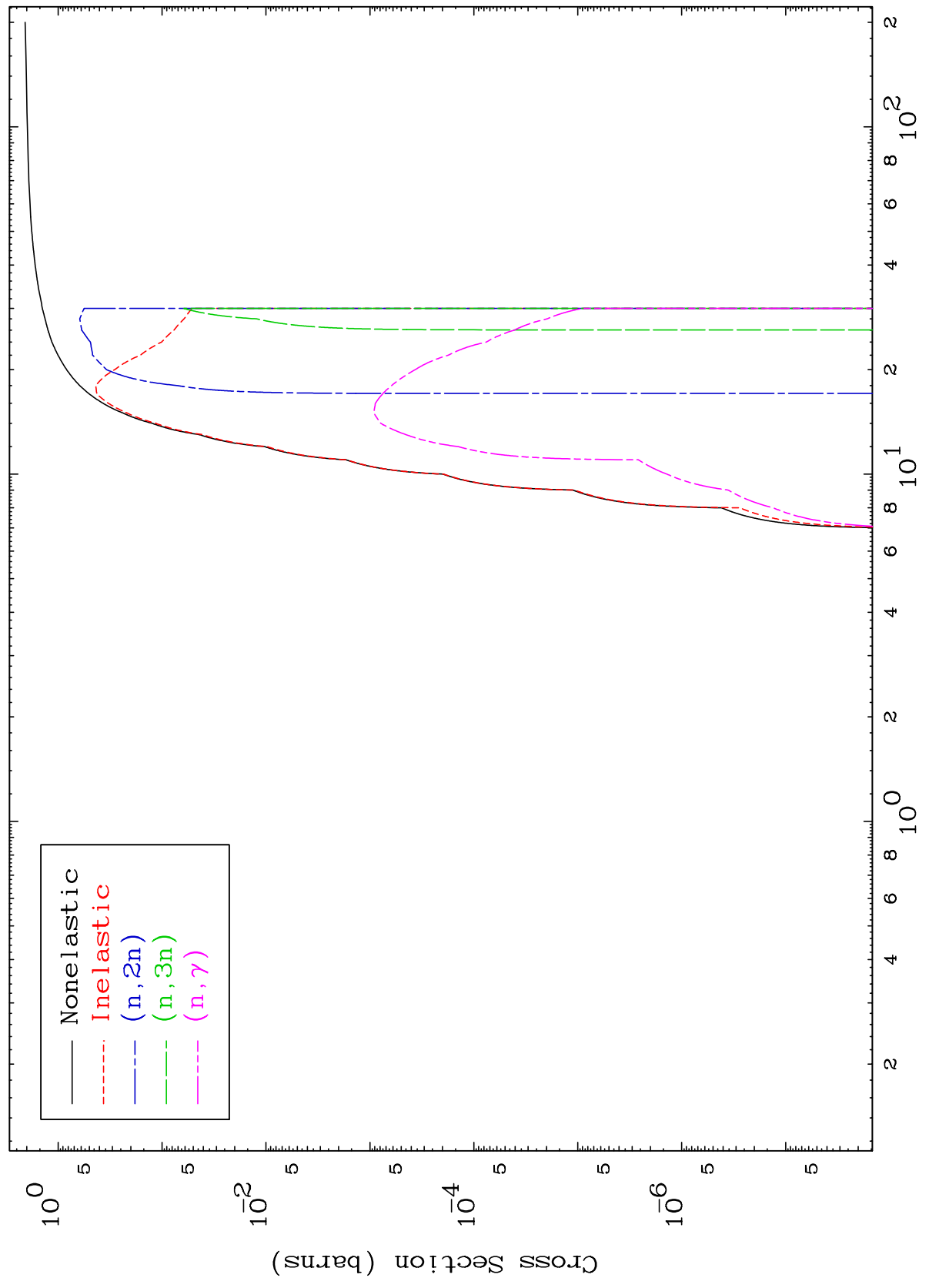
Press Mouse Button to Start

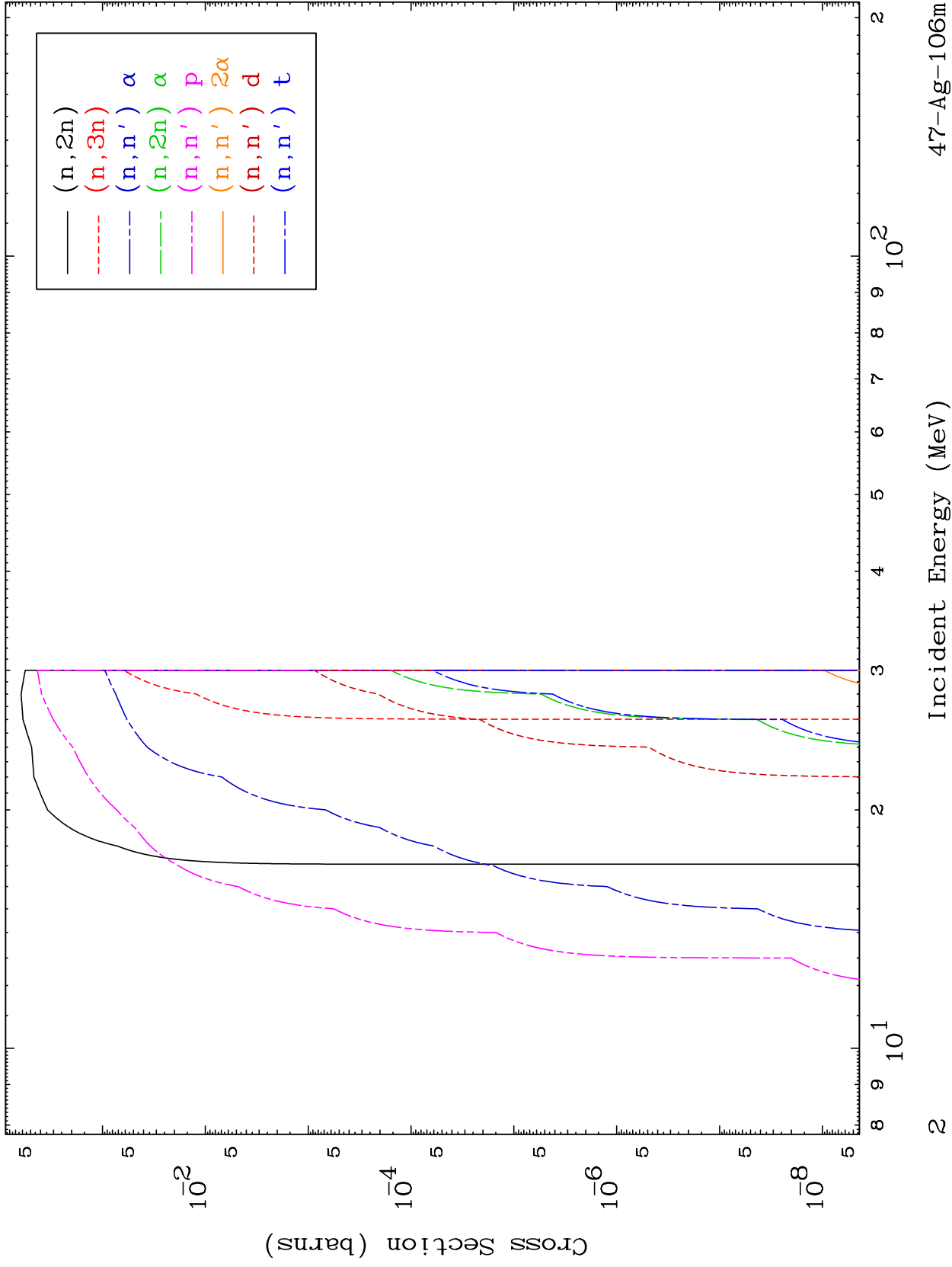
MAT 4723

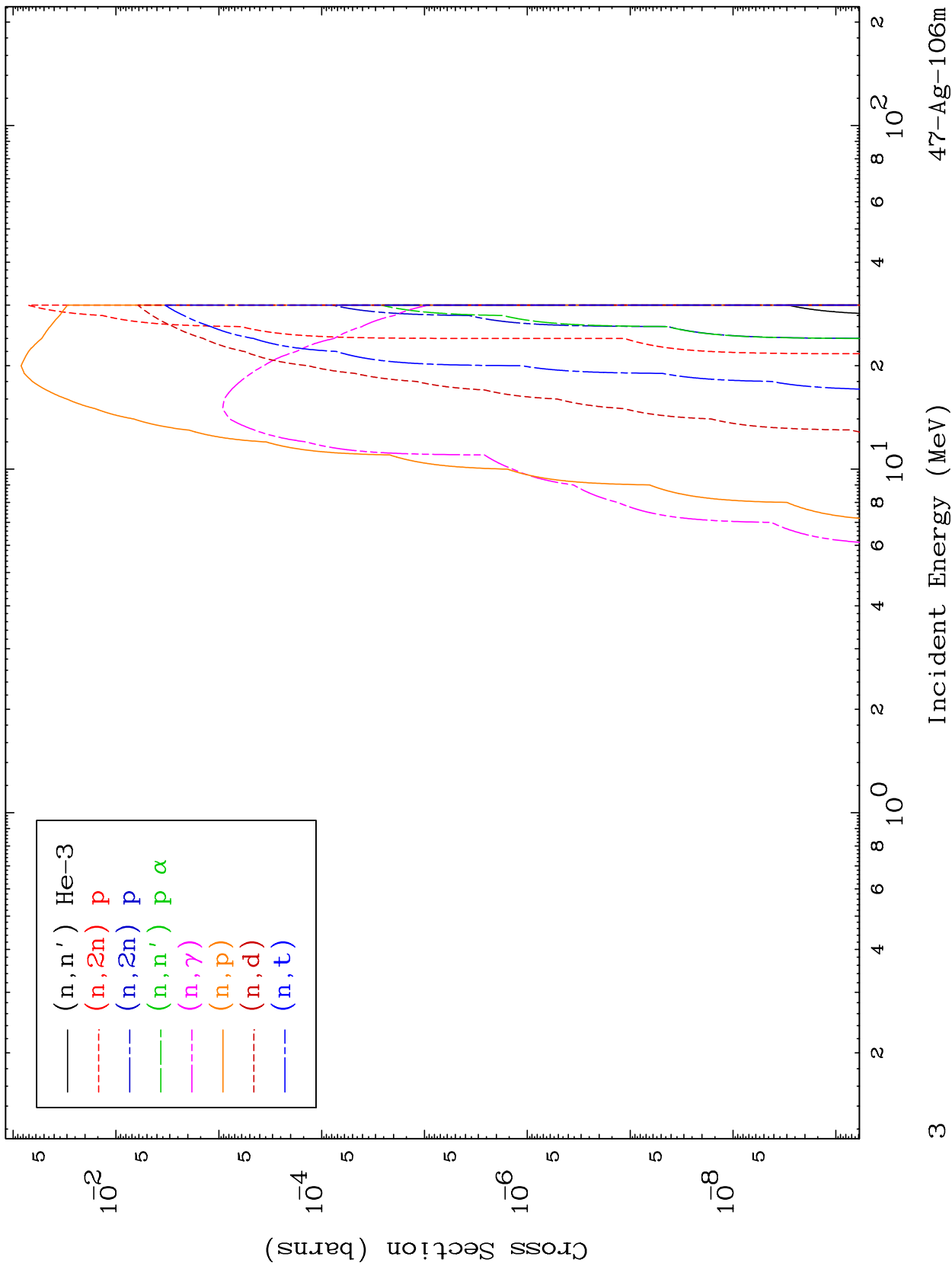
0 Kelvin

$\alpha$  Major Cross Sections

47-Ag-106m



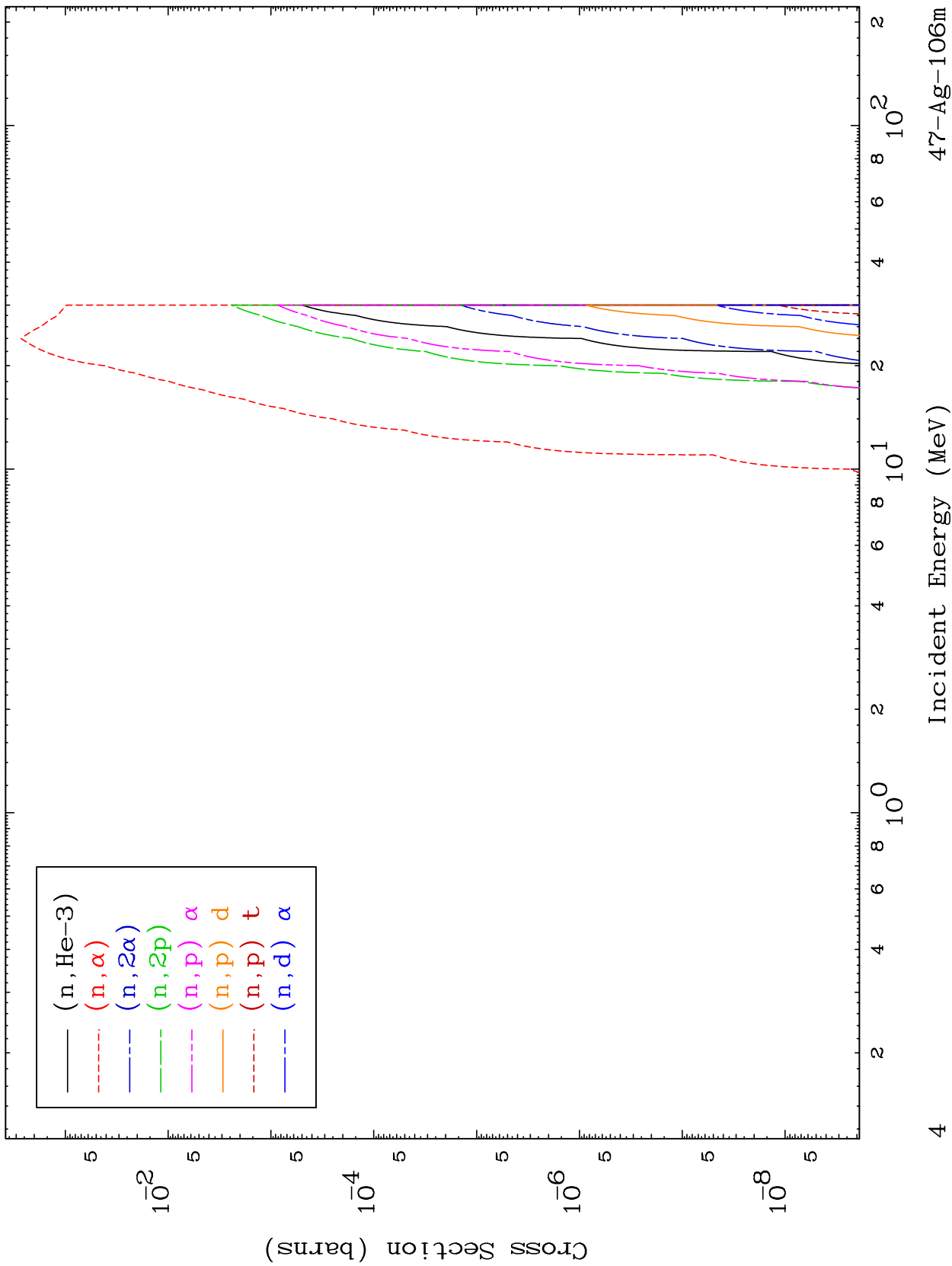


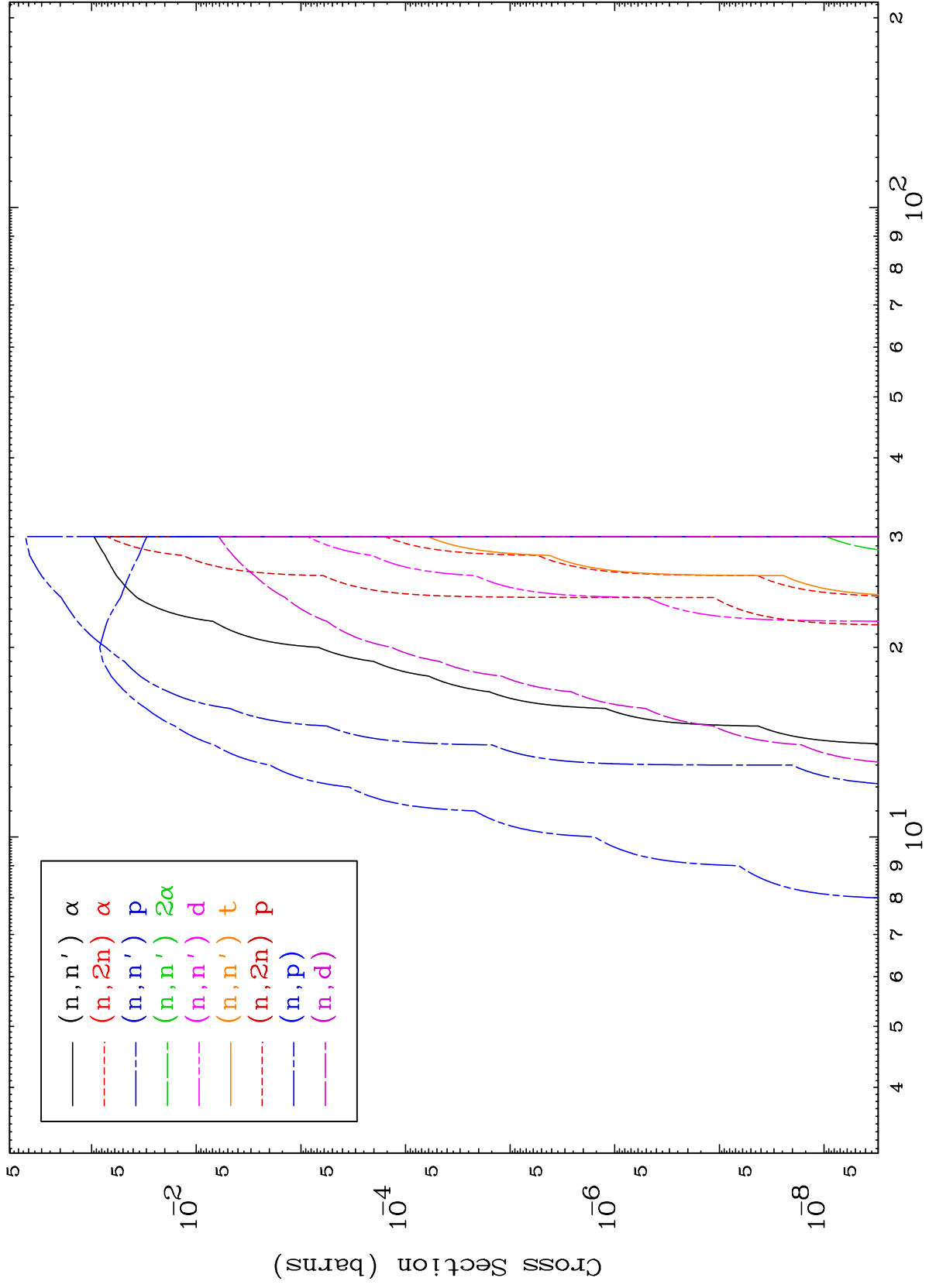


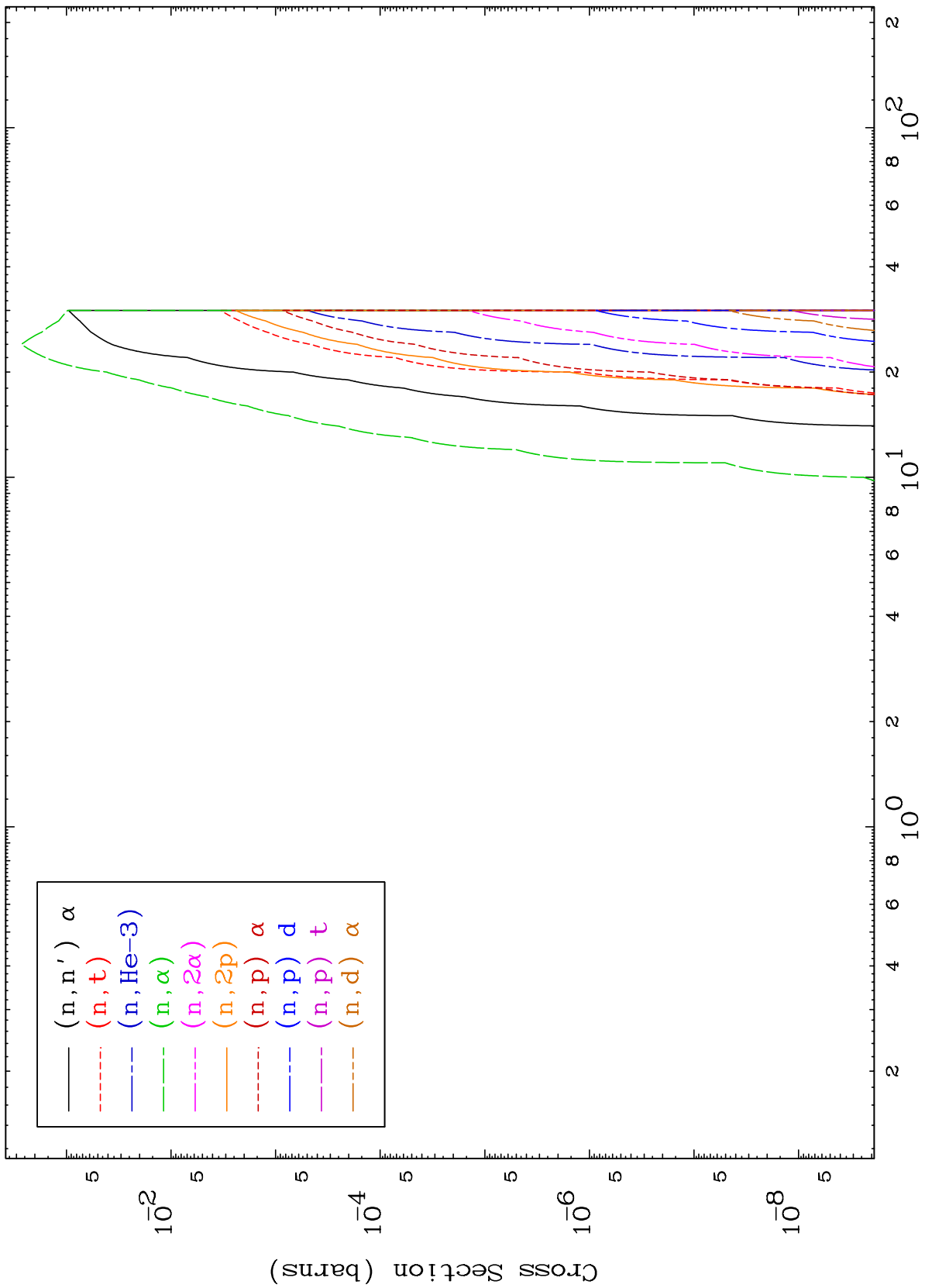
MAT 4723

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

47-Ag-106m





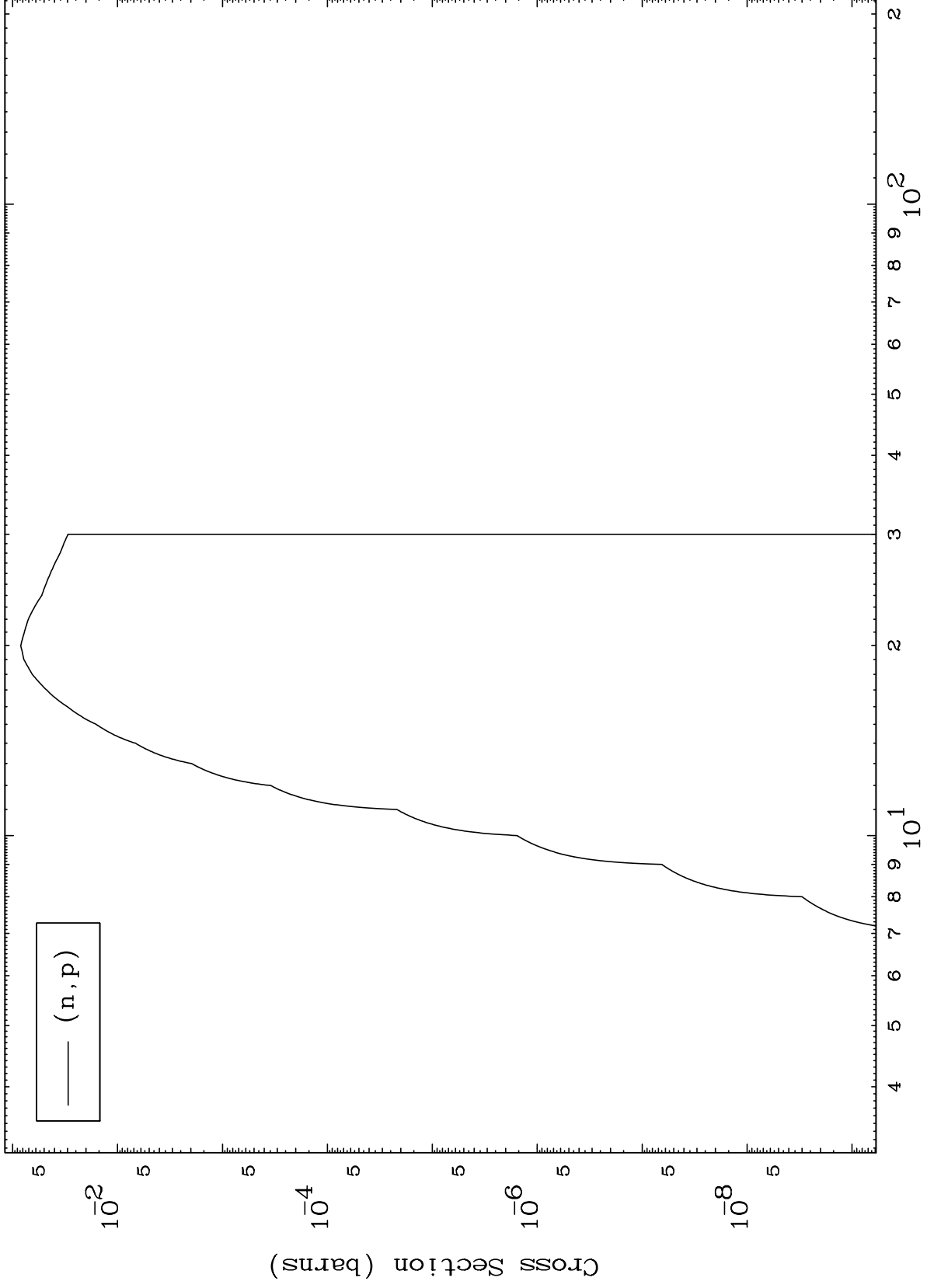


MAT 4723

( $\alpha, p$ ) Levels

47-Ag-106m

0 Kelvin Cross Sections



7

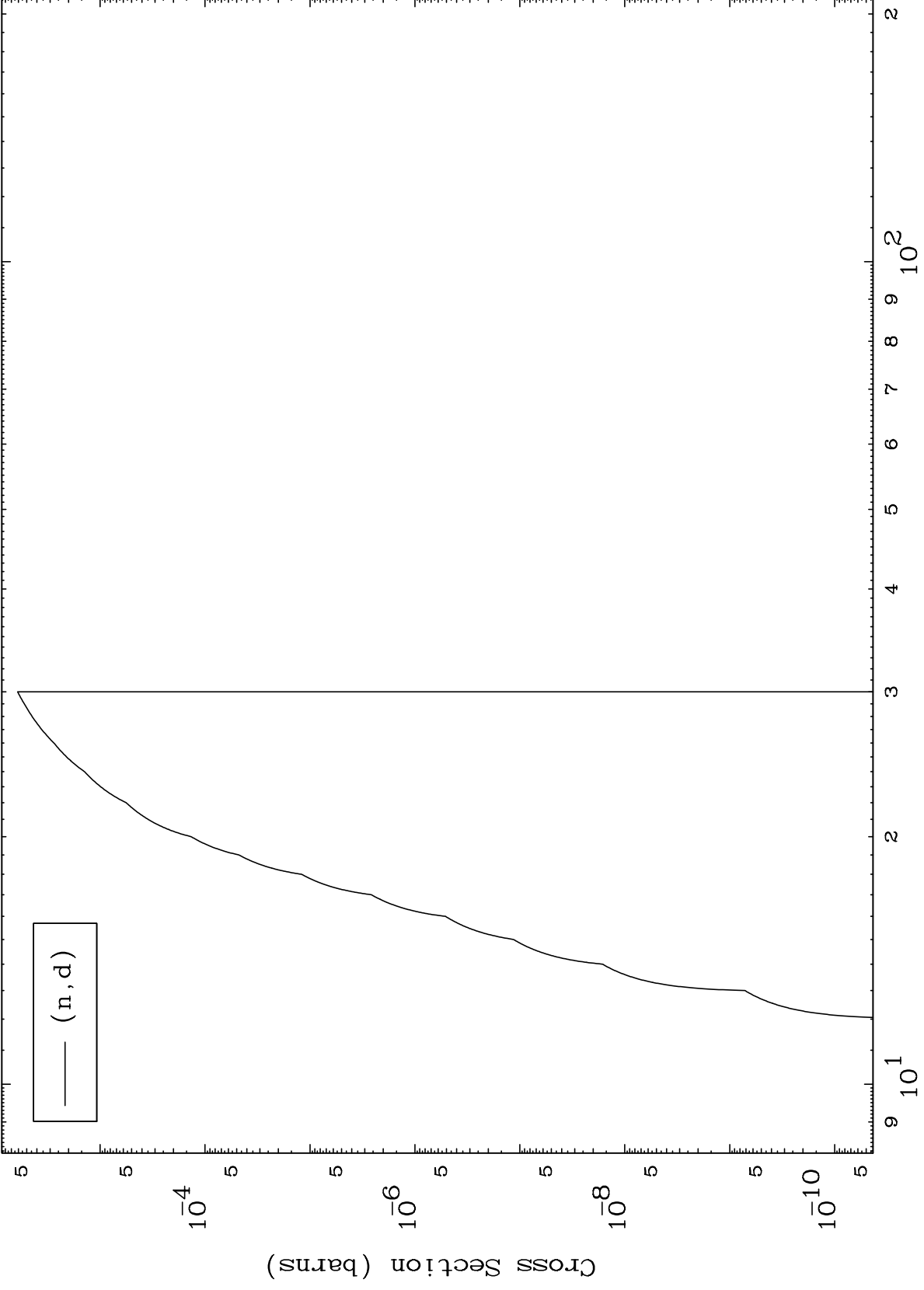
Incident Energy (MeV)

47-Ag-106m

MAT 4723

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

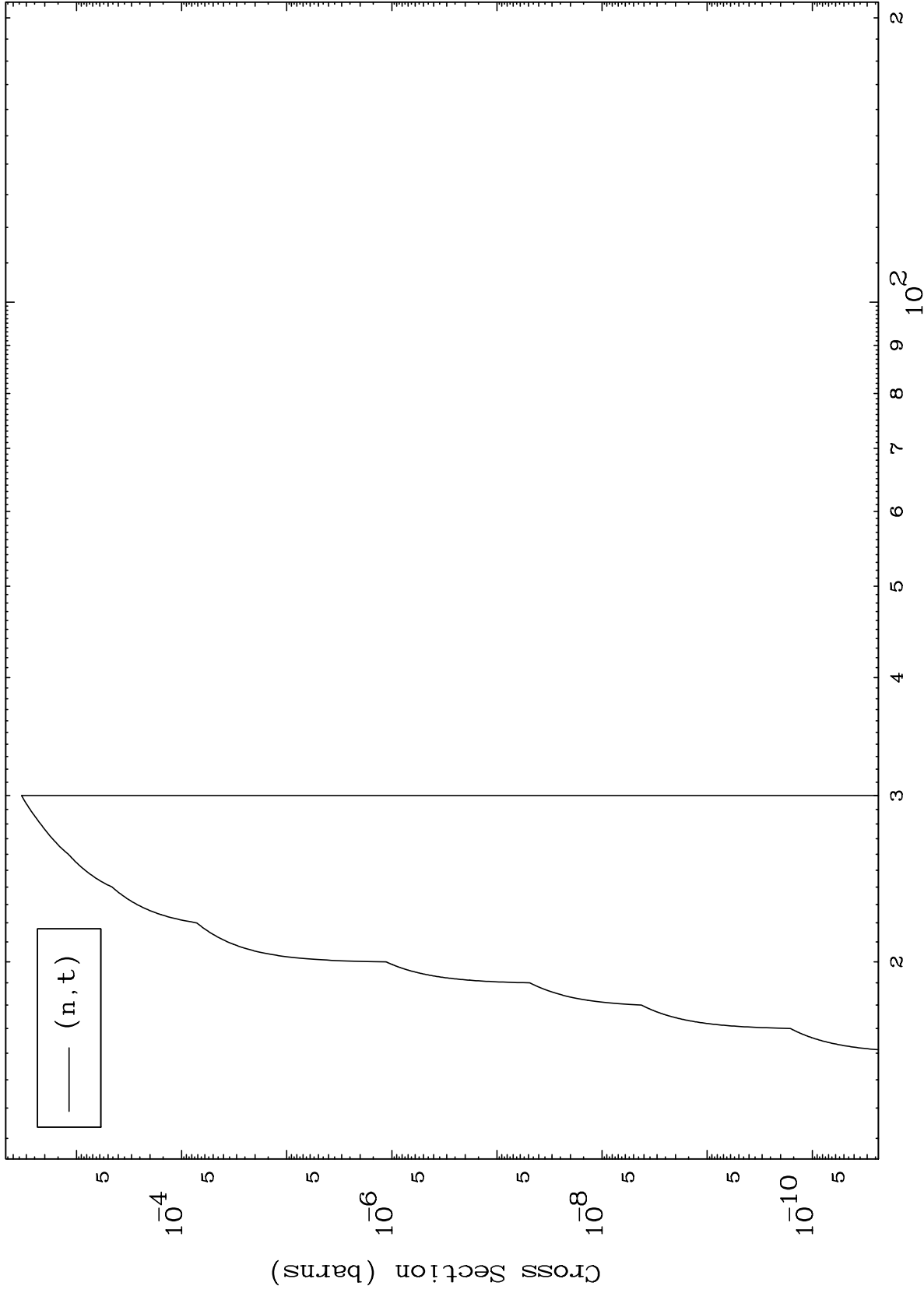
47-Ag-106m



Incident Energy (MeV)

47-Ag-106m

8

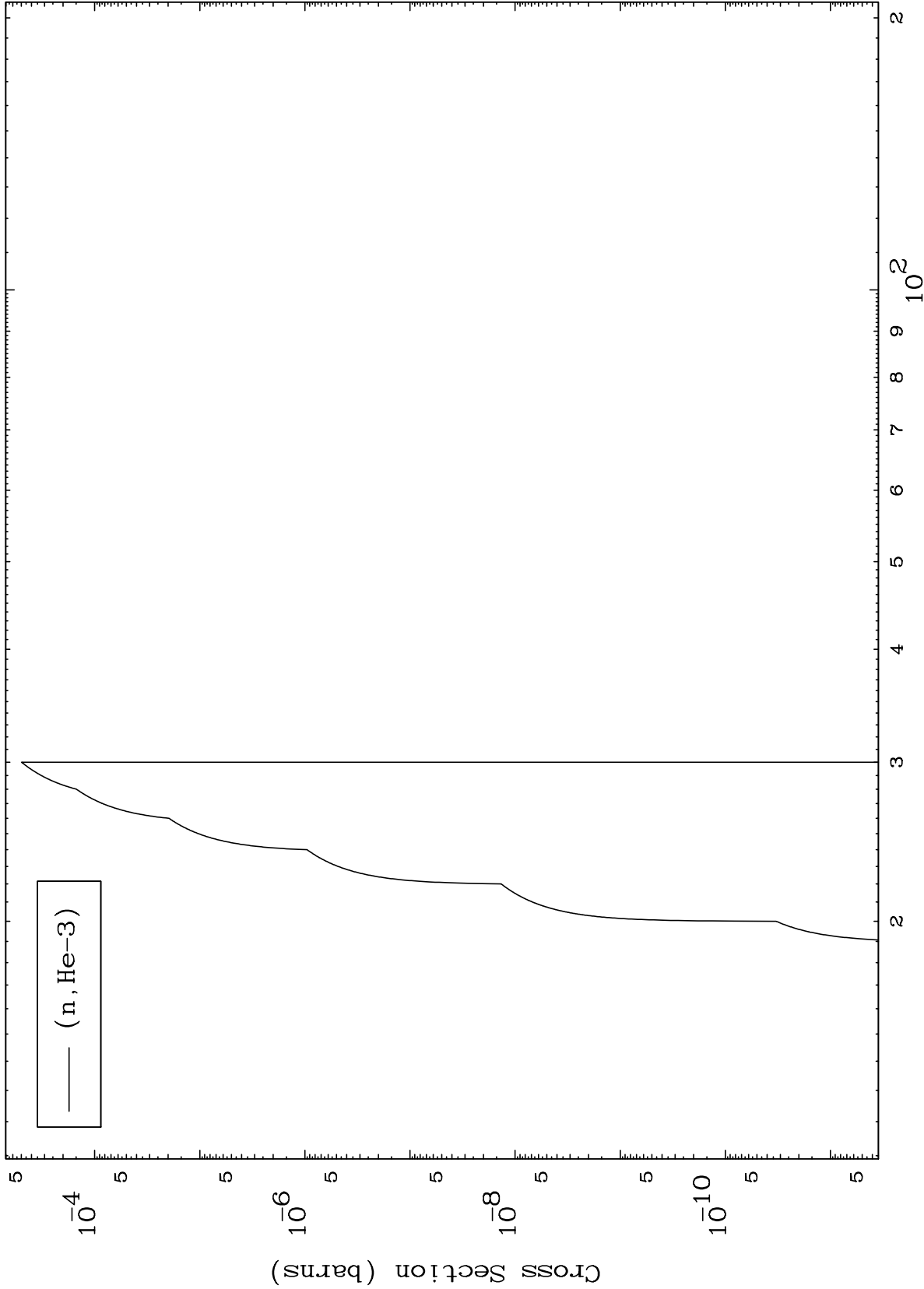


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( $\alpha, \text{He}3$ ) Levels

47-Ag-106m

0 Kelvin Cross Sections



10

Incident Energy (MeV)

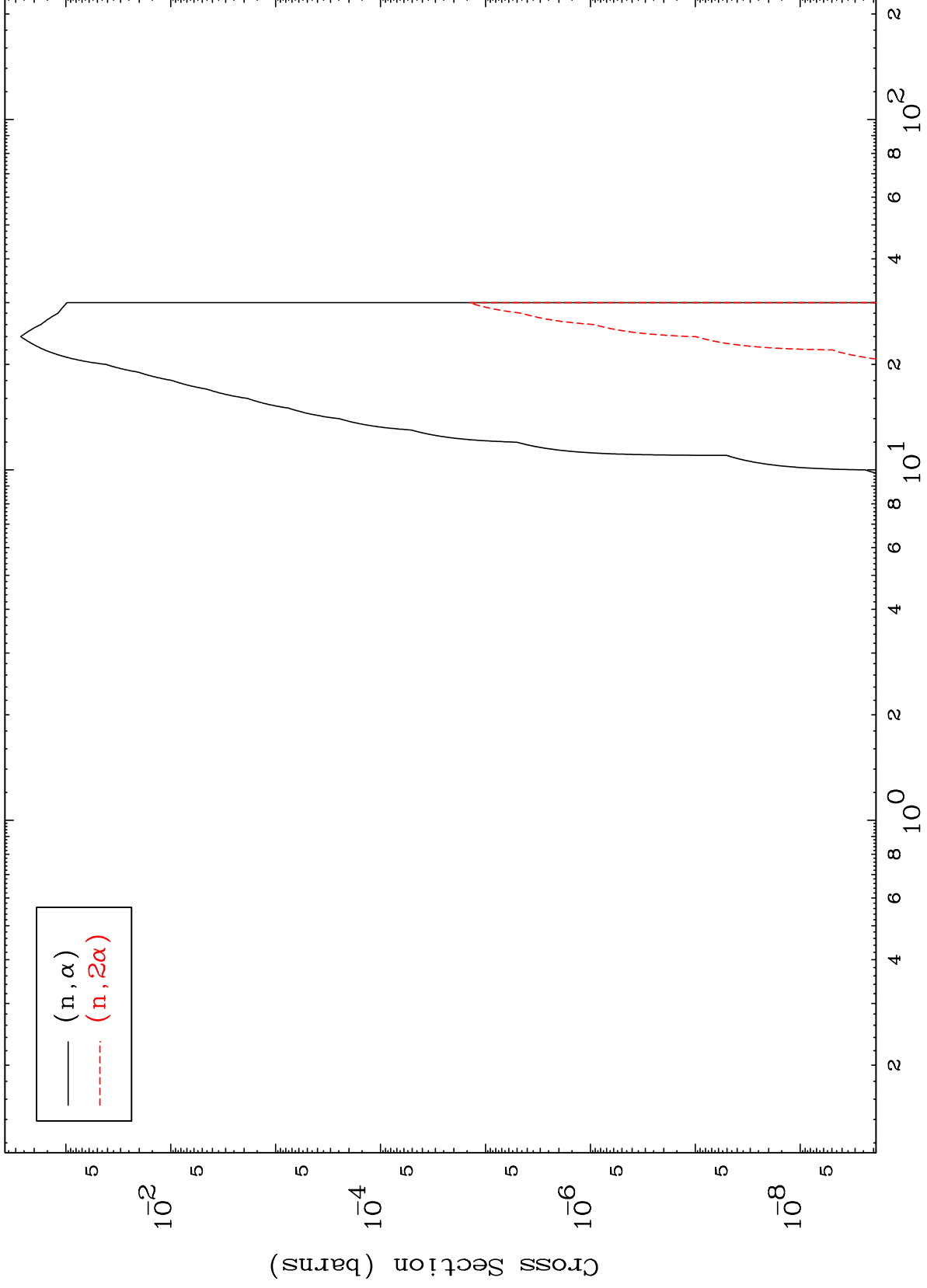
47-Ag-106m

MAT 4723

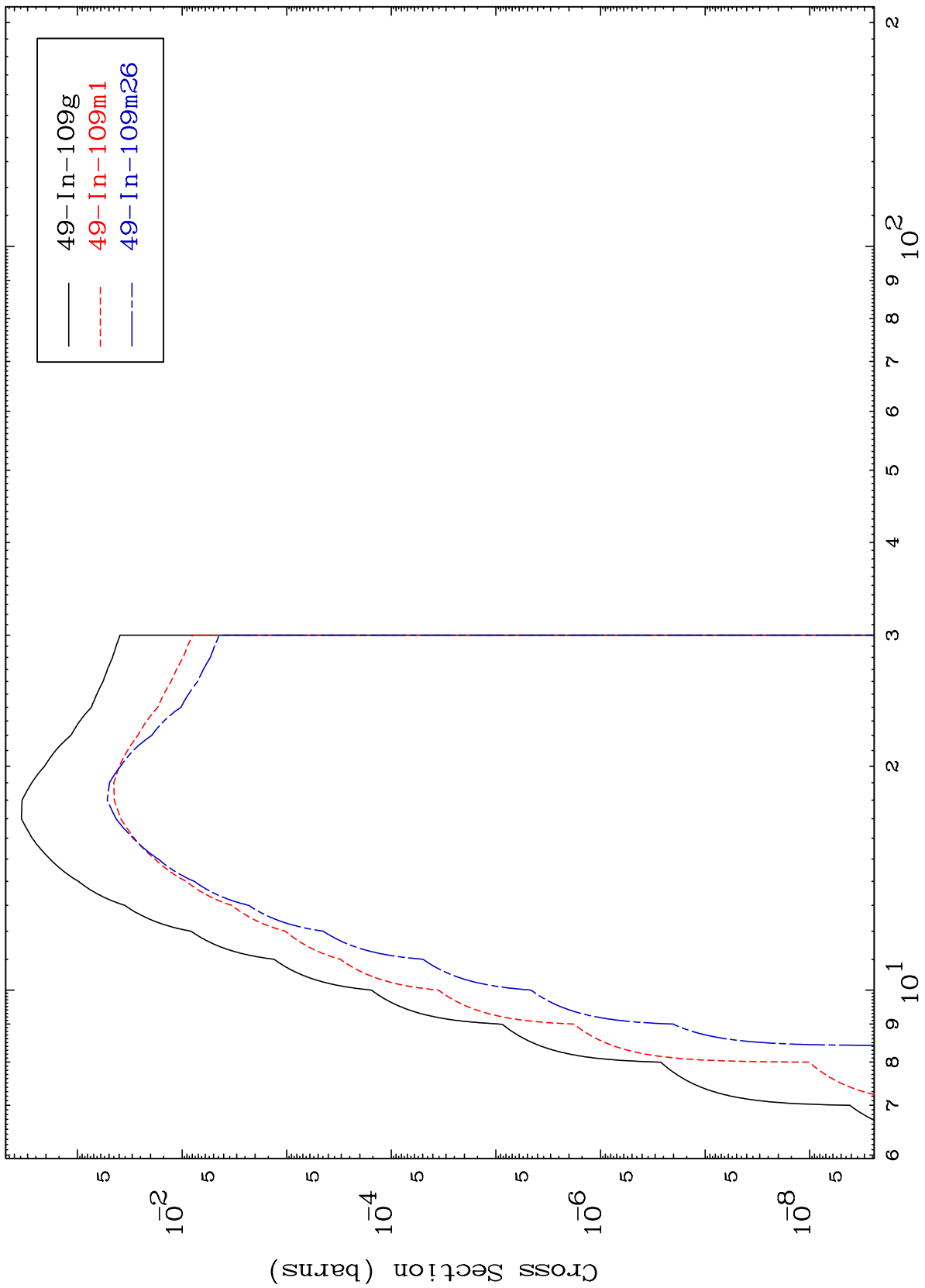
( $\alpha, \alpha$ ) Levels

47-Ag-106m

0 Kelvin Cross Sections



Inelastic  
Radionuclide Production Cross Section

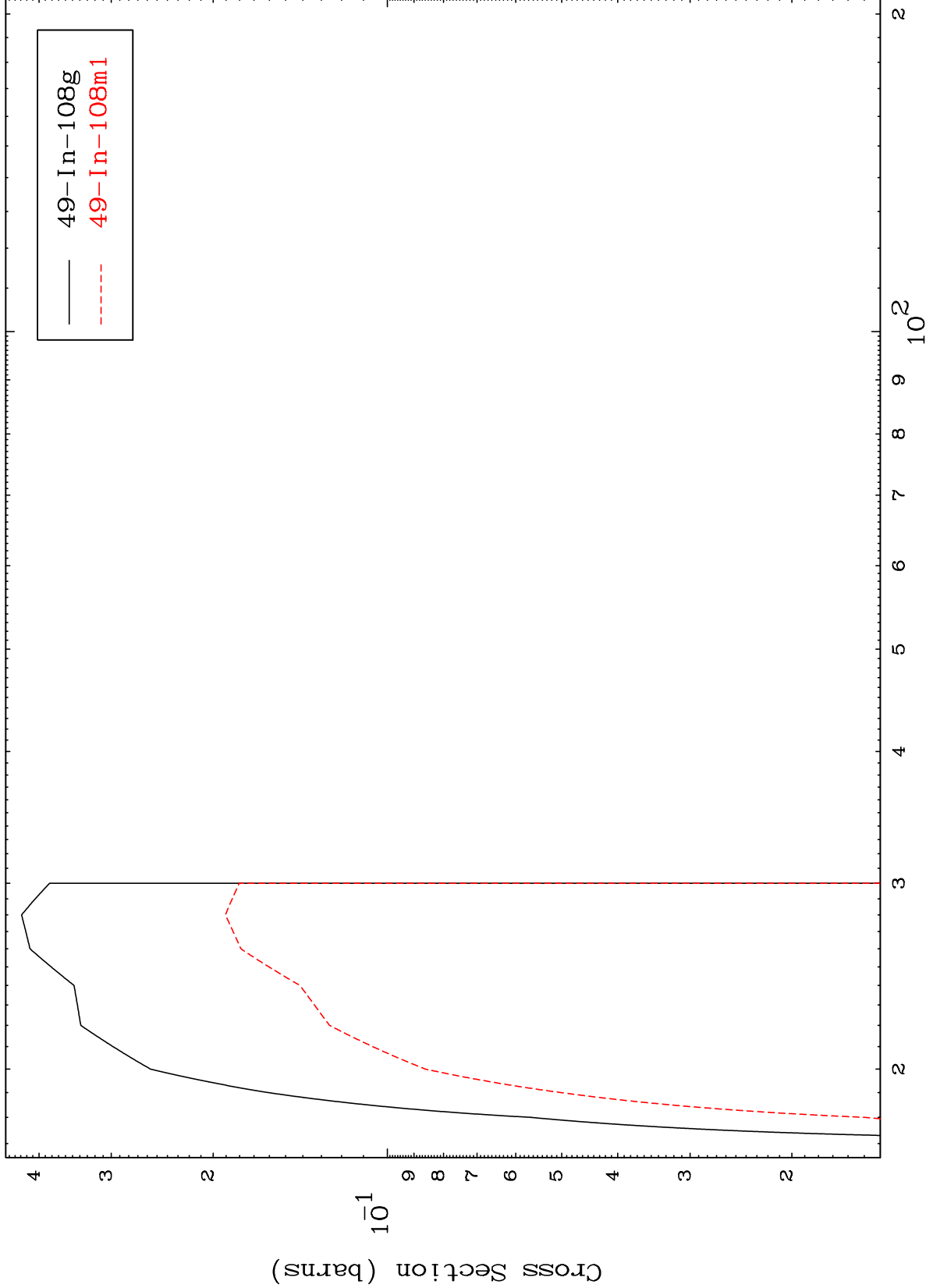


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(n,2n)

47-Ag-106m

Radionuclide Production Cross Section



49-In-108g  
49-In-108m1

13

Incident Energy (MeV)

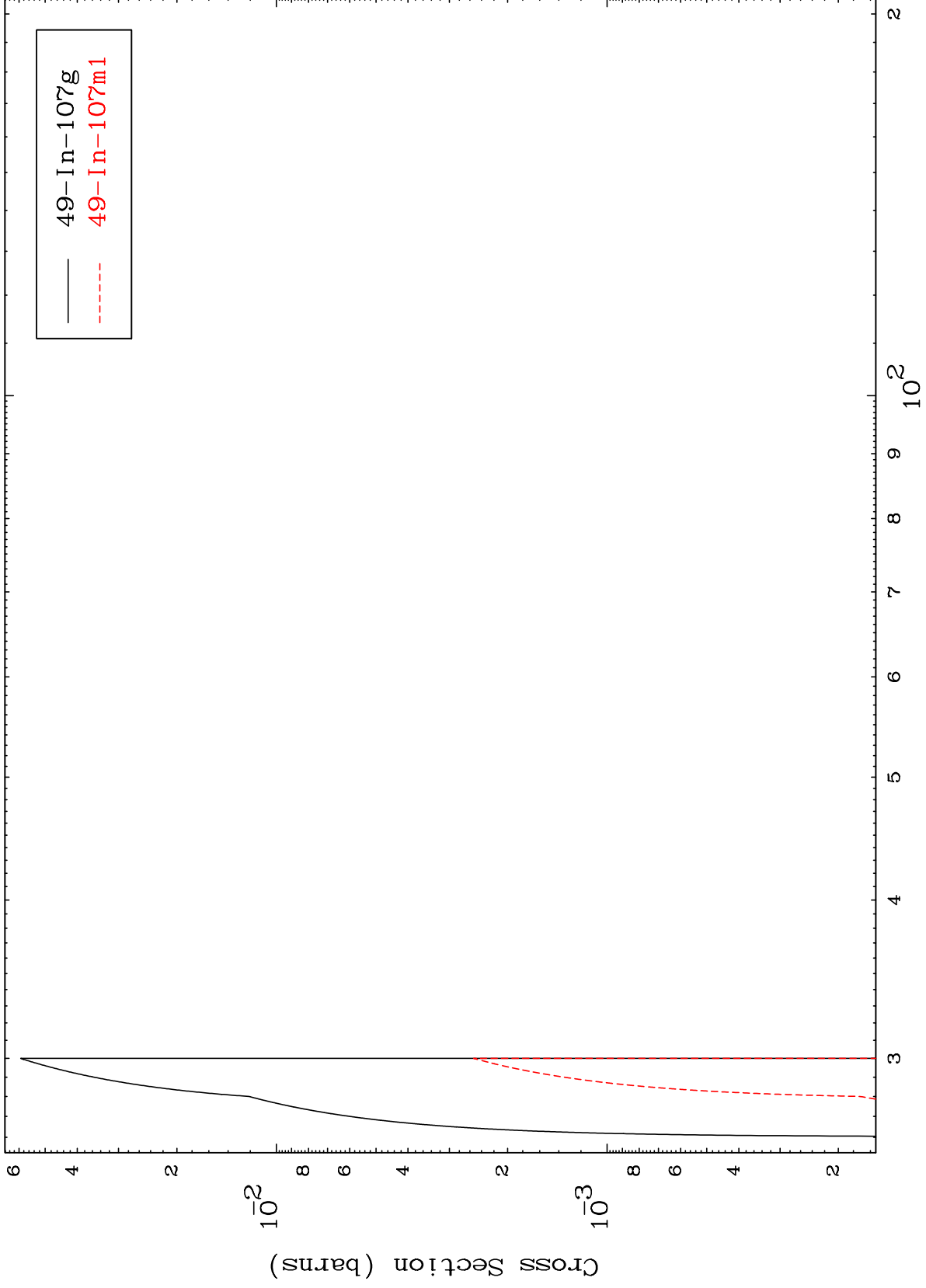
47-Ag-106m

MAT 4723

(n,3n)

47-Ag-106m

Radionuclide Production Cross Section



14

Incident Energy (MeV)

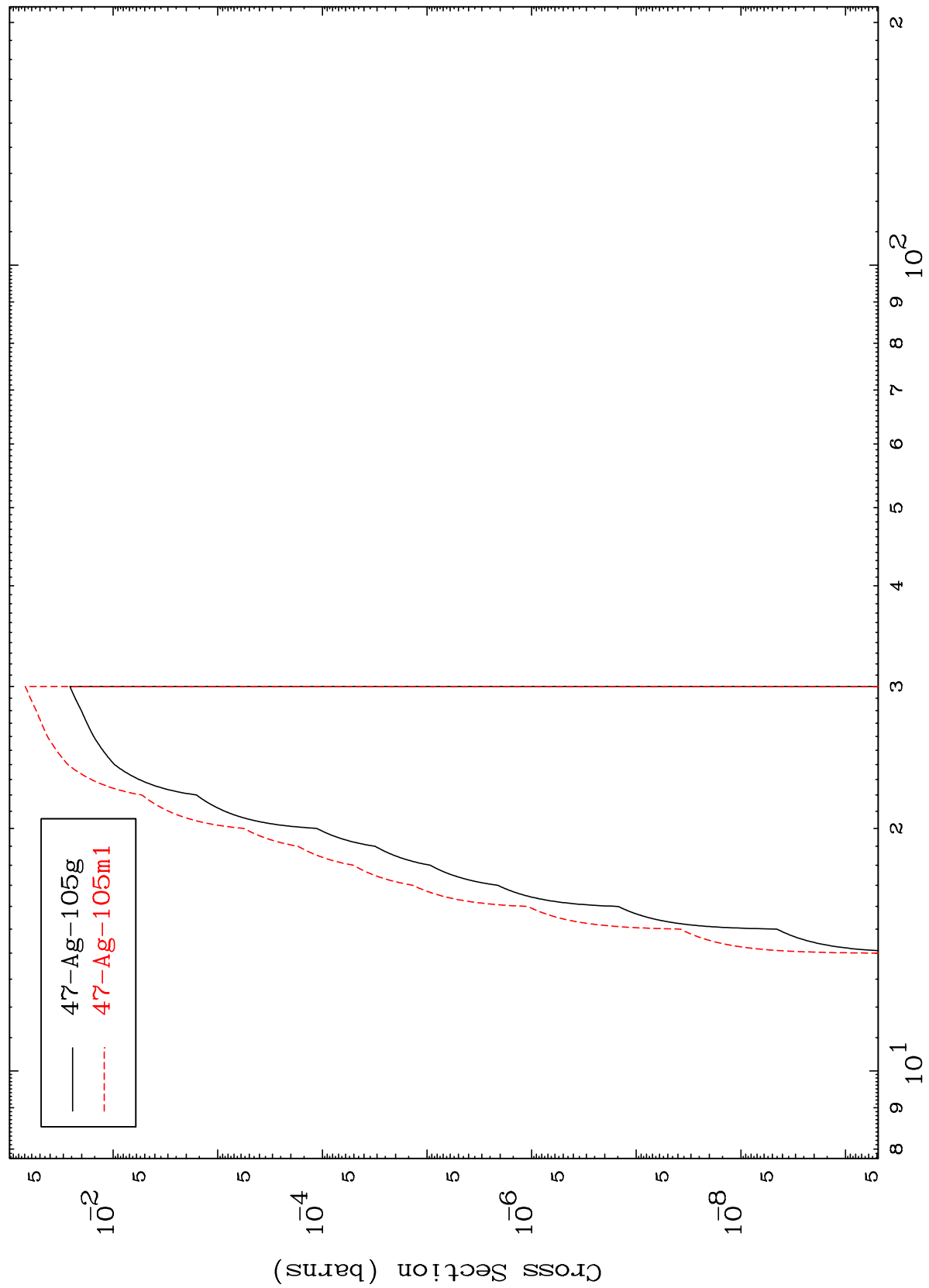
47-Ag-106m

MAT 4723

47-Ag-106m

(n,n')  $\alpha$

Radionuclide Production Cross Section



15

Incident Energy (MeV)

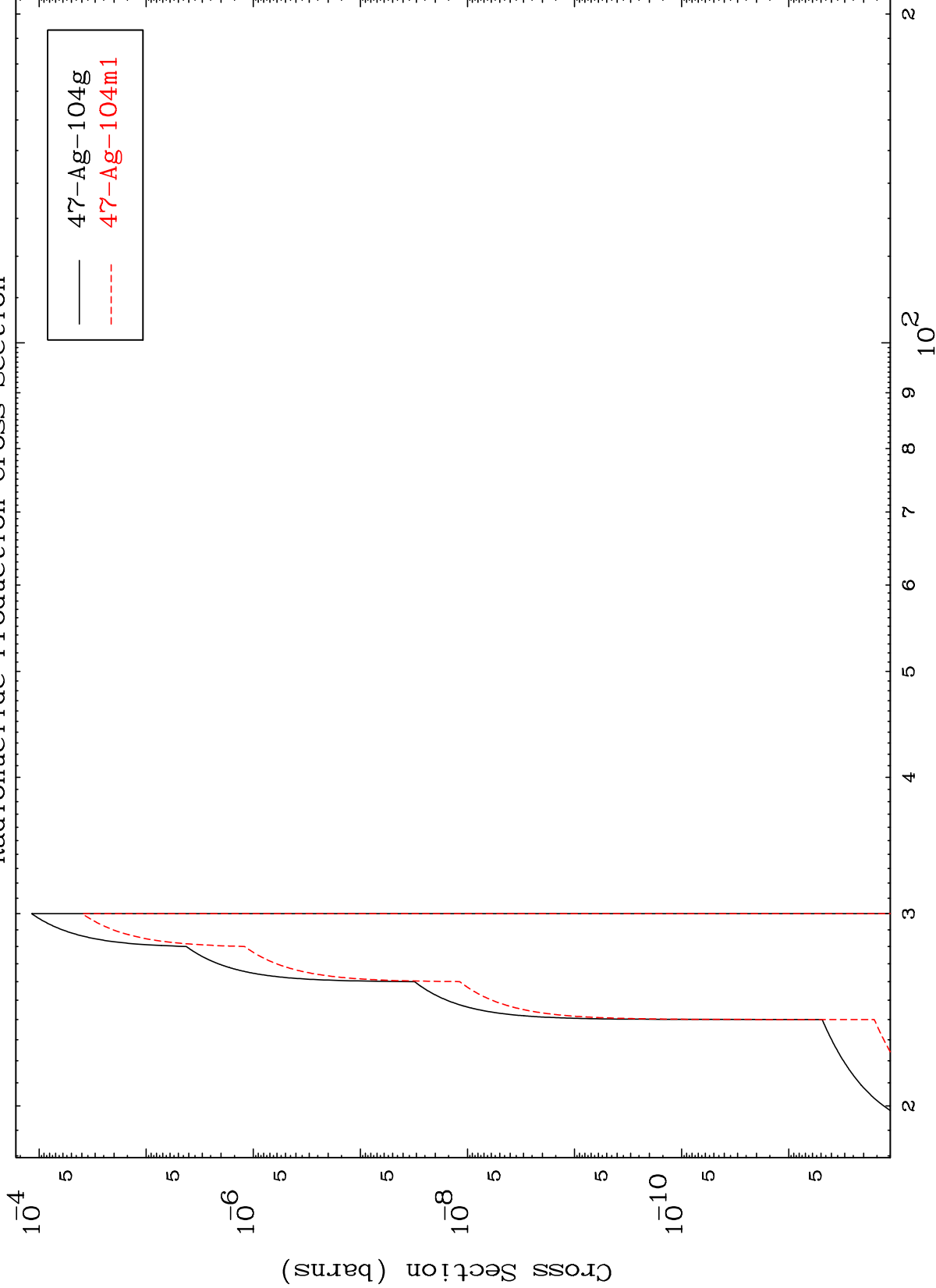
47-Ag-106m

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(n,2n)  $\alpha$

47-Ag-106m

Radionuclide Production Cross Section



16

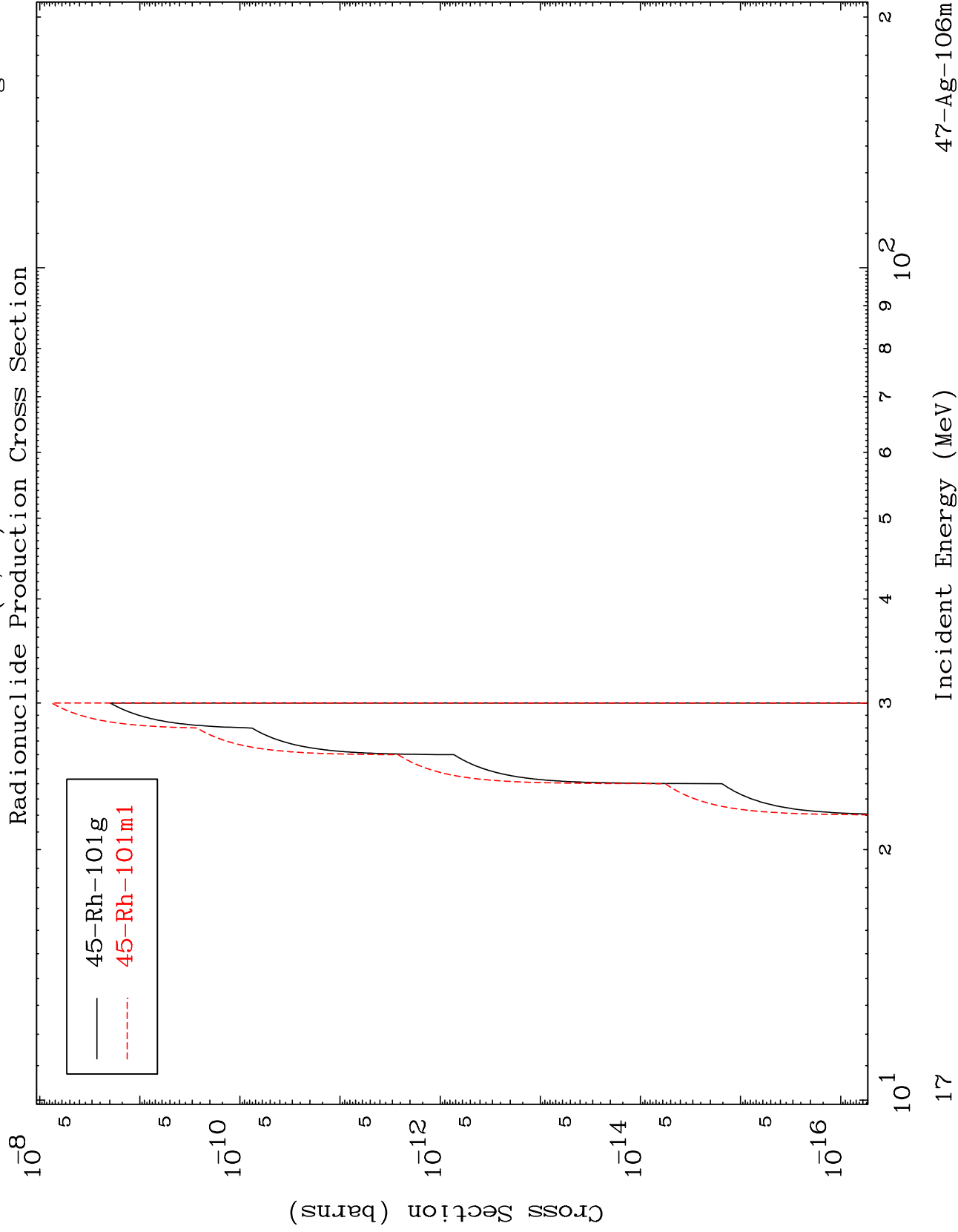
Incident Energy (MeV)

47-Ag-106m

MAT 4723

(n,n') 2α

47-Ag-106m

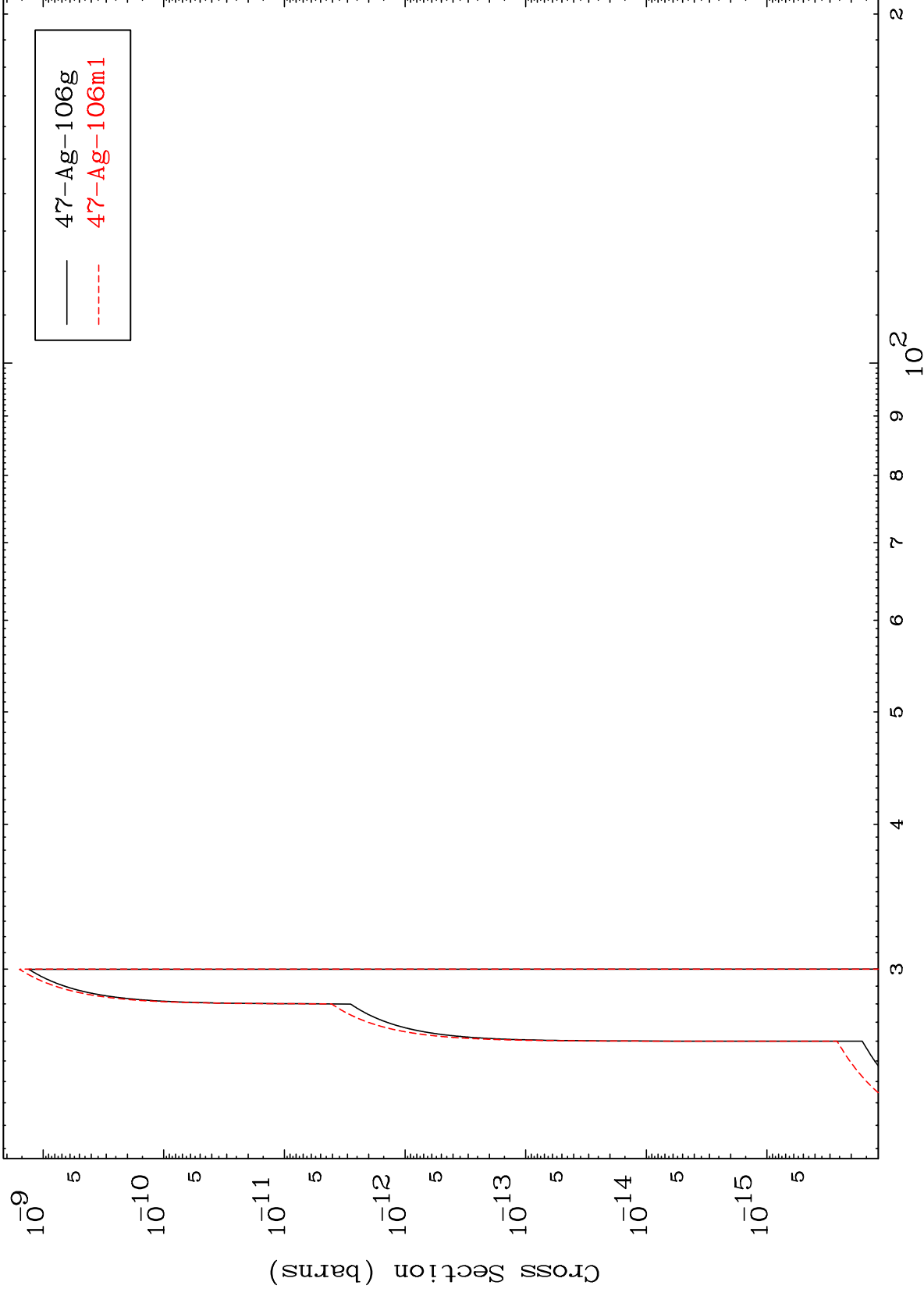


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(n,n') He-3

47-Ag-106m

Radionuclide Production Cross Section



47-Ag-106g  
47-Ag-106m1

18

Incident Energy (MeV)

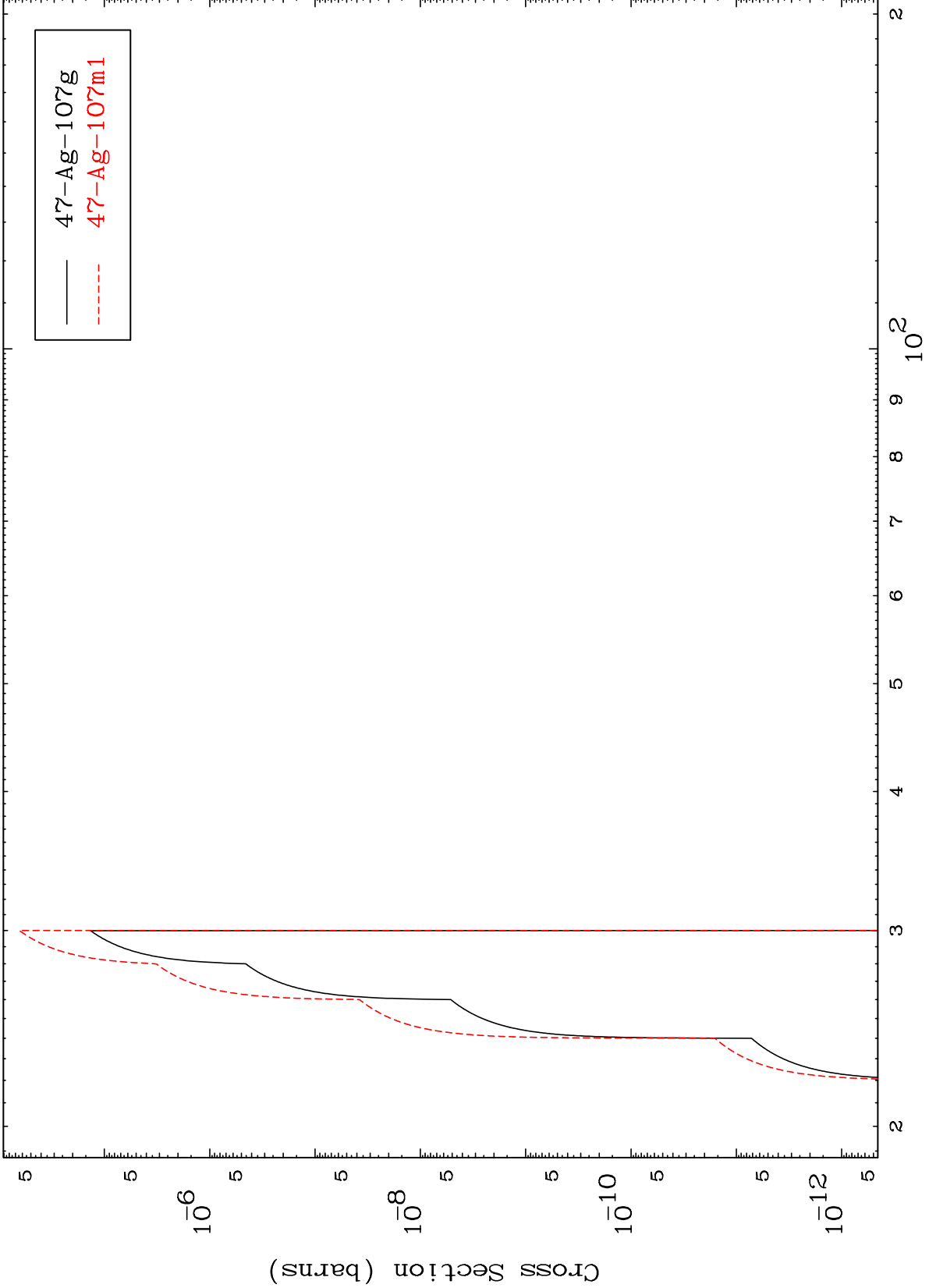
47-Ag-106m

MAT 4723

(n,2n) p

47-Ag-106m

Radionuclide Production Cross Section



19

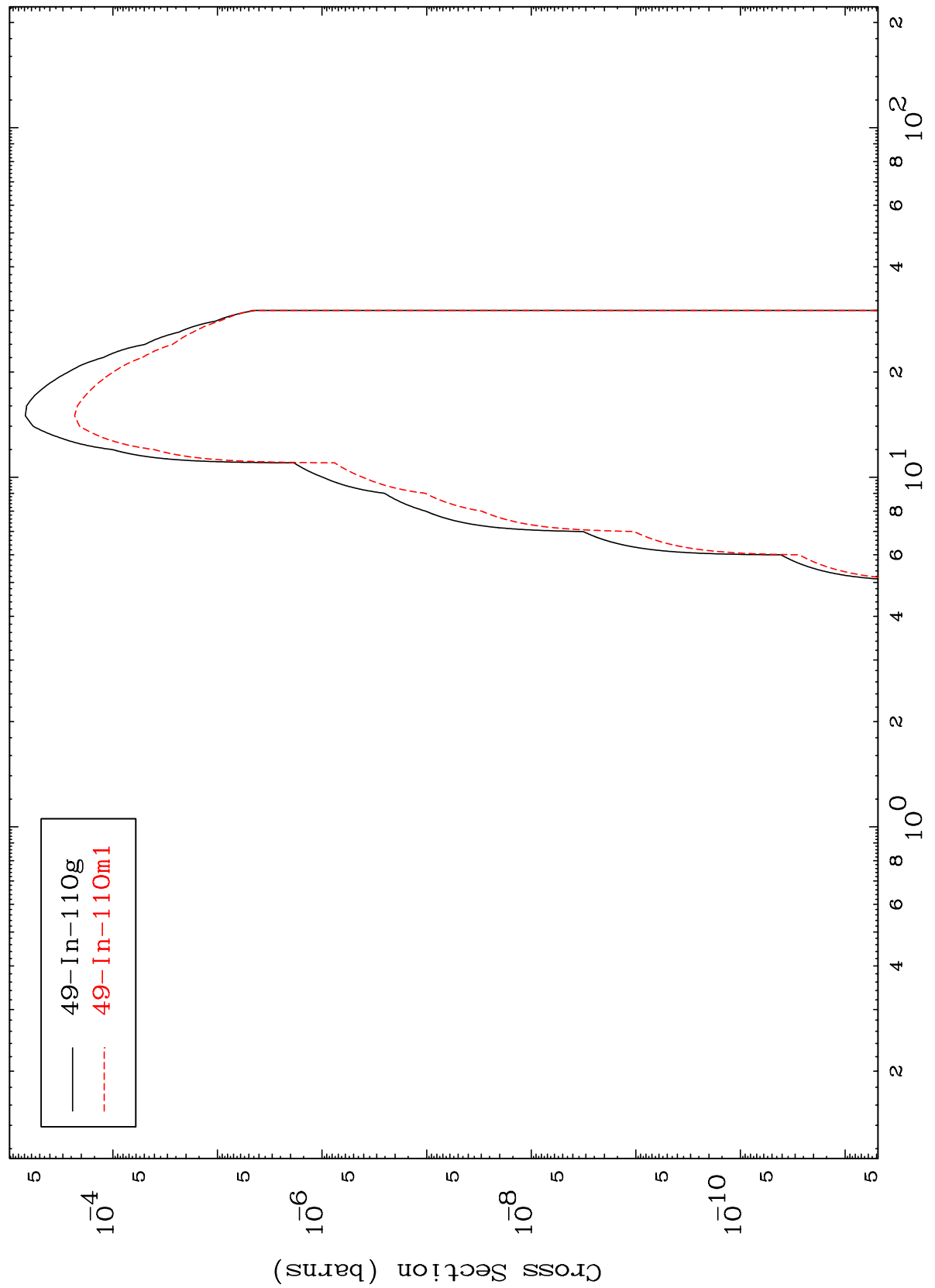
Incident Energy (MeV)

47-Ag-106m

MAT 4723

47-Ag-106m

Radionuclide Production Cross Section  
(n,  $\gamma$ )

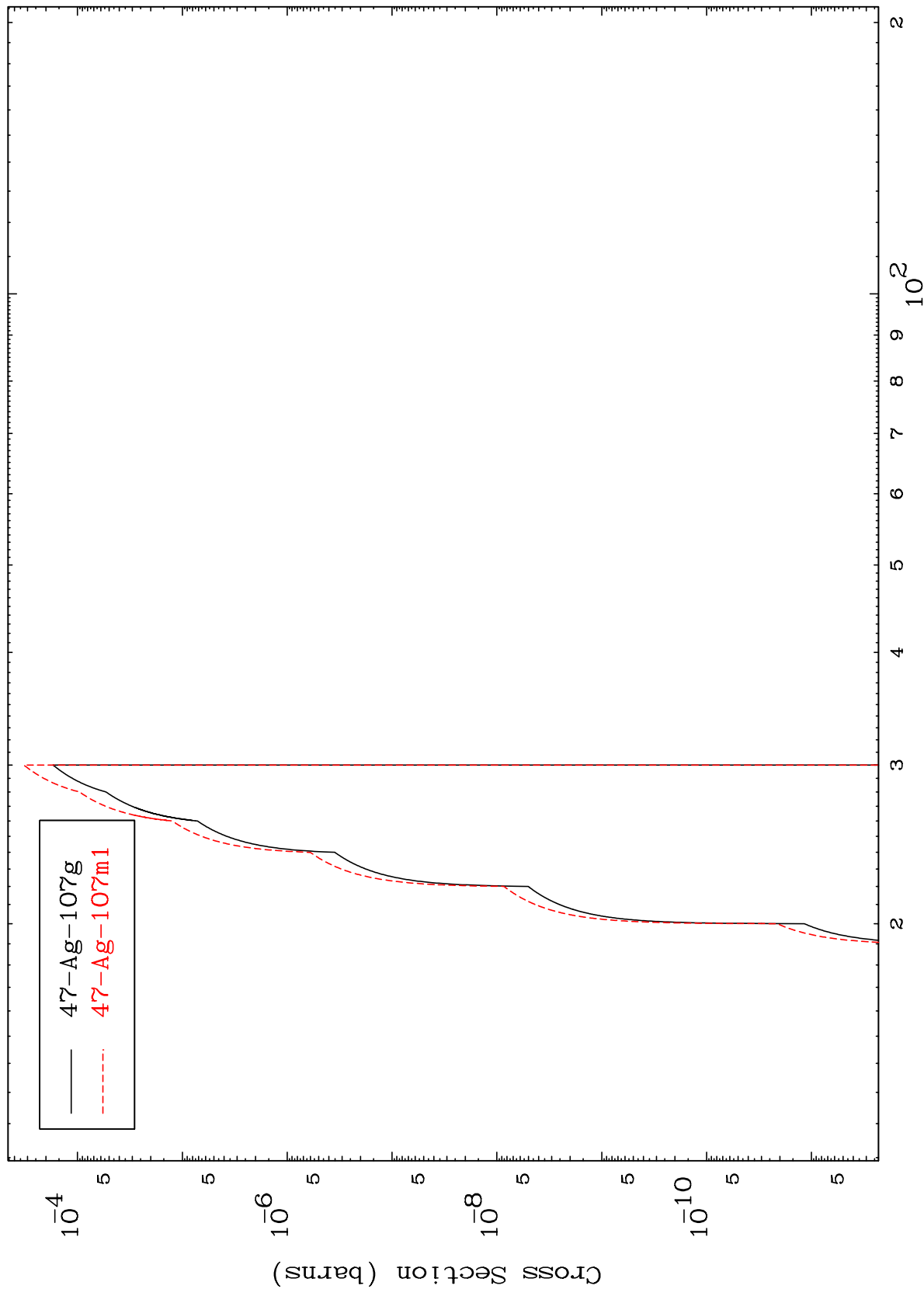


— 49-In-110g  
- - - 49-In-110m1

MAT 4723

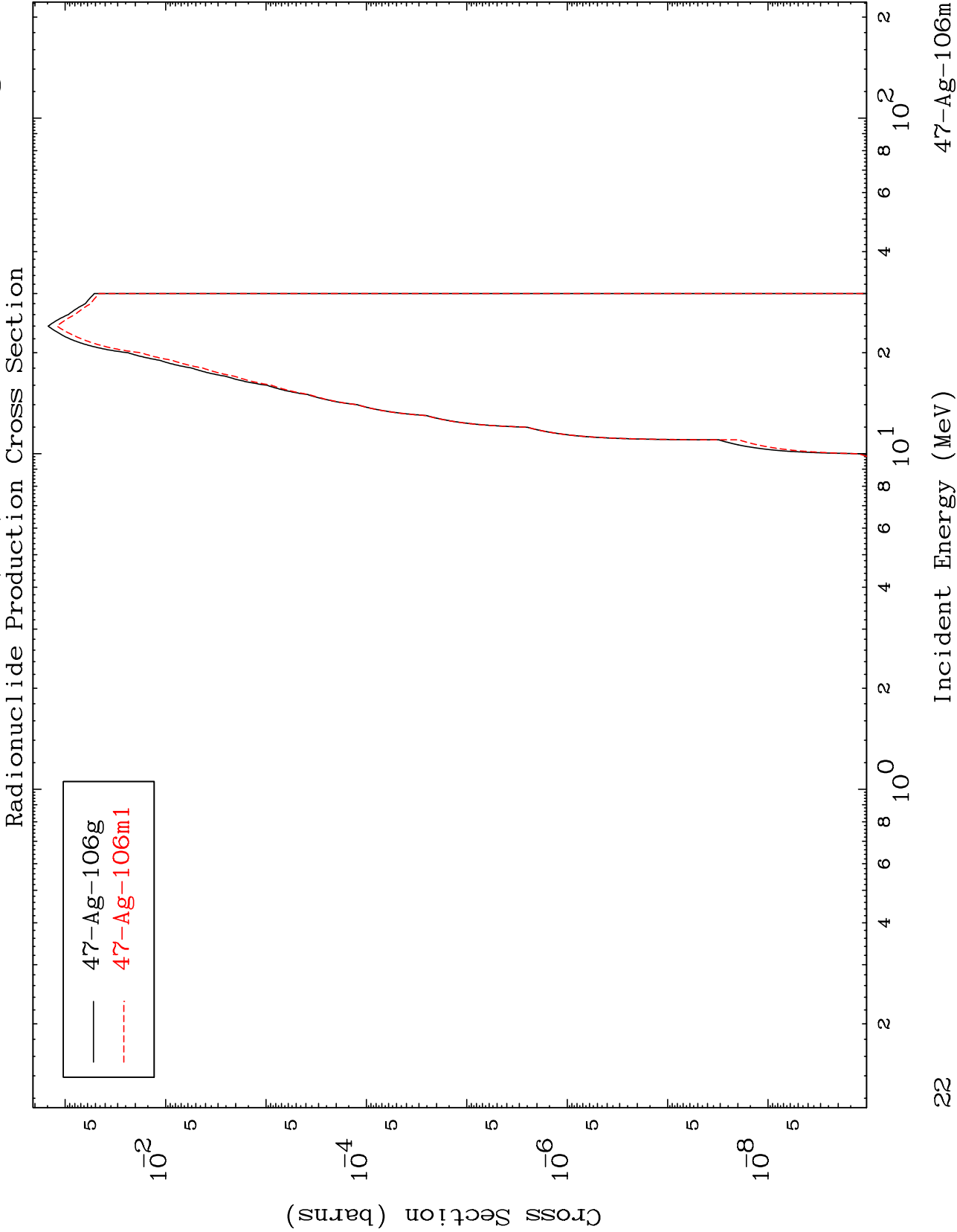
47-Ag-106m

(n,He-3)  
Radionuclide Production Cross Section



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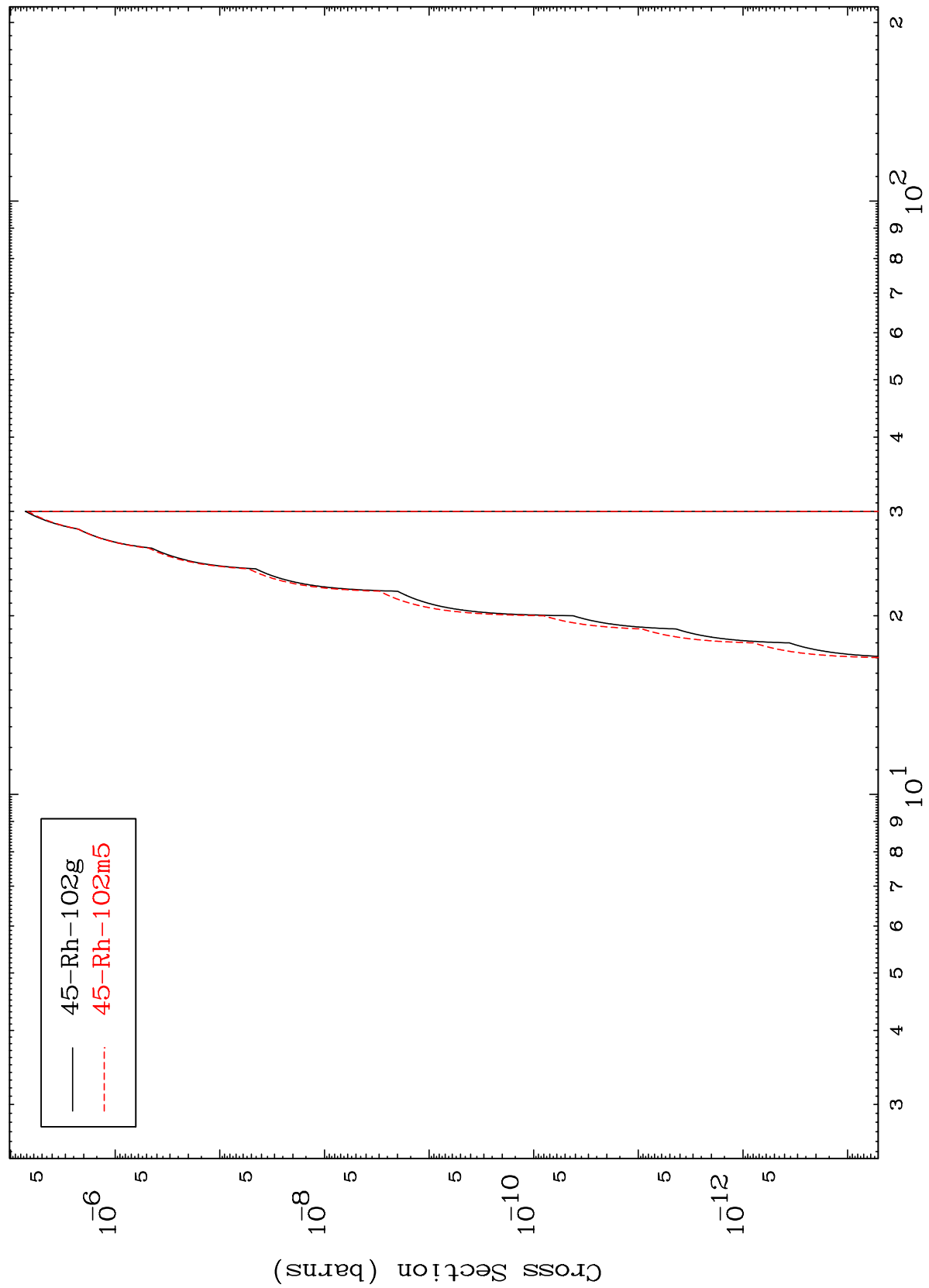
47-Ag-106m



MAT 4723

47-Ag-106m

(n,2α)  
Radionuclide Production Cross Section



47-Ag-106m

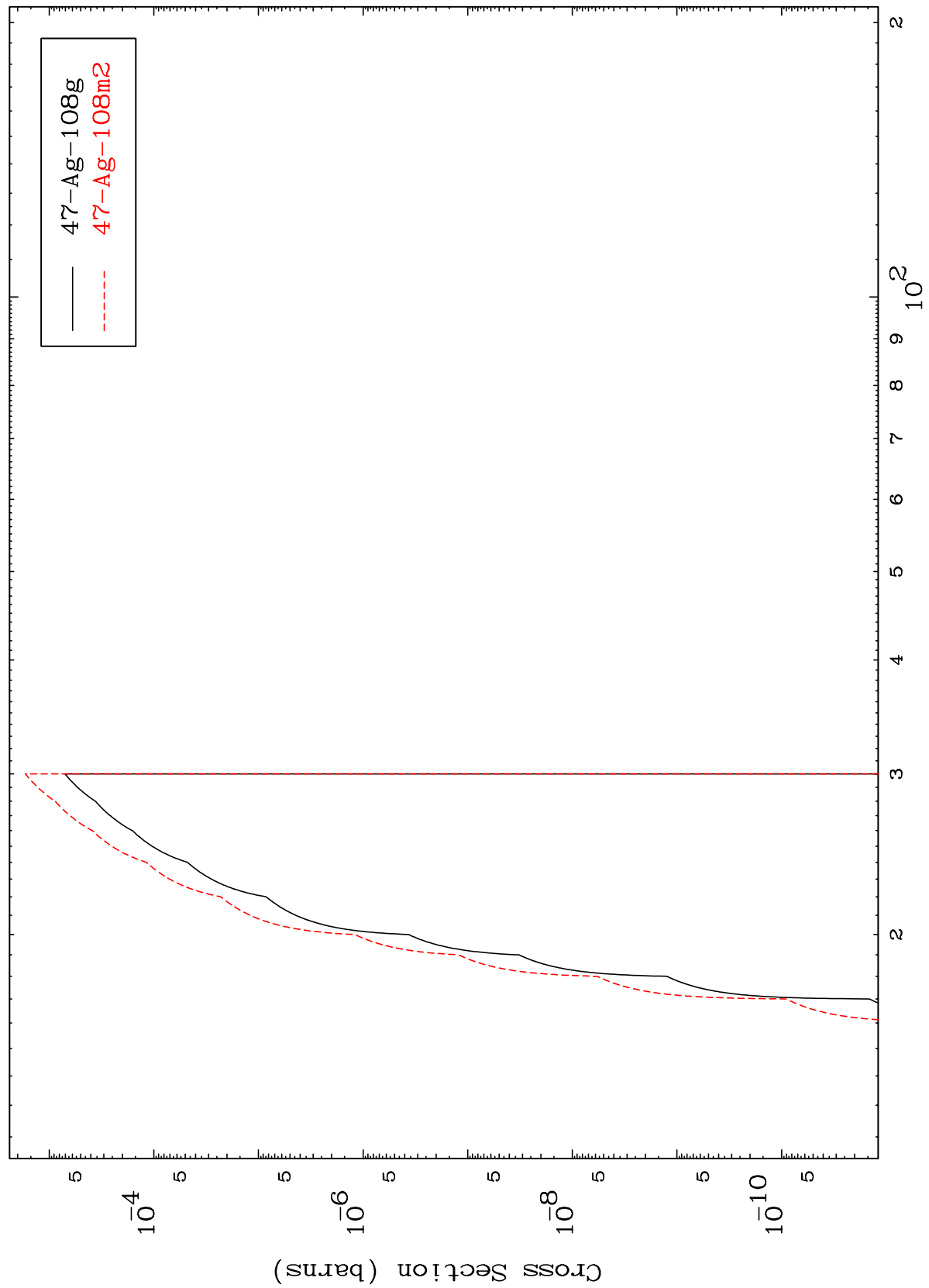
Incident Energy (MeV)

23

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47-Ag-106m

(n,2p)  
Radionuclide Production Cross Section



24

Incident Energy (MeV)

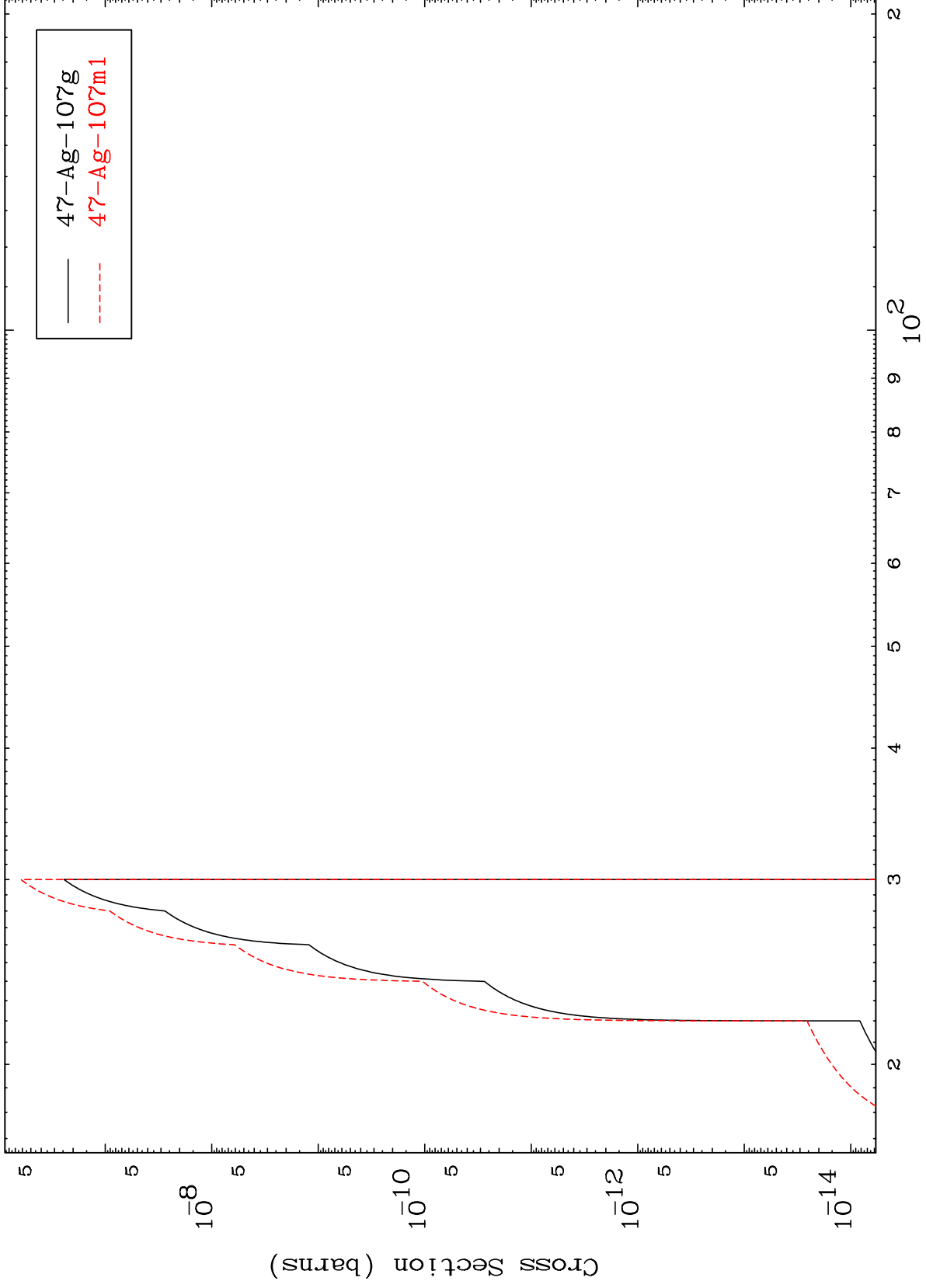
47-Ag-106m

MAT 4723

(n,p) d

47-Ag-106m

Radionuclide Production Cross Section



25

Incident Energy (MeV)

47-Ag-106m

