

Program EVALPLOT  
(Version 2021-1)

by

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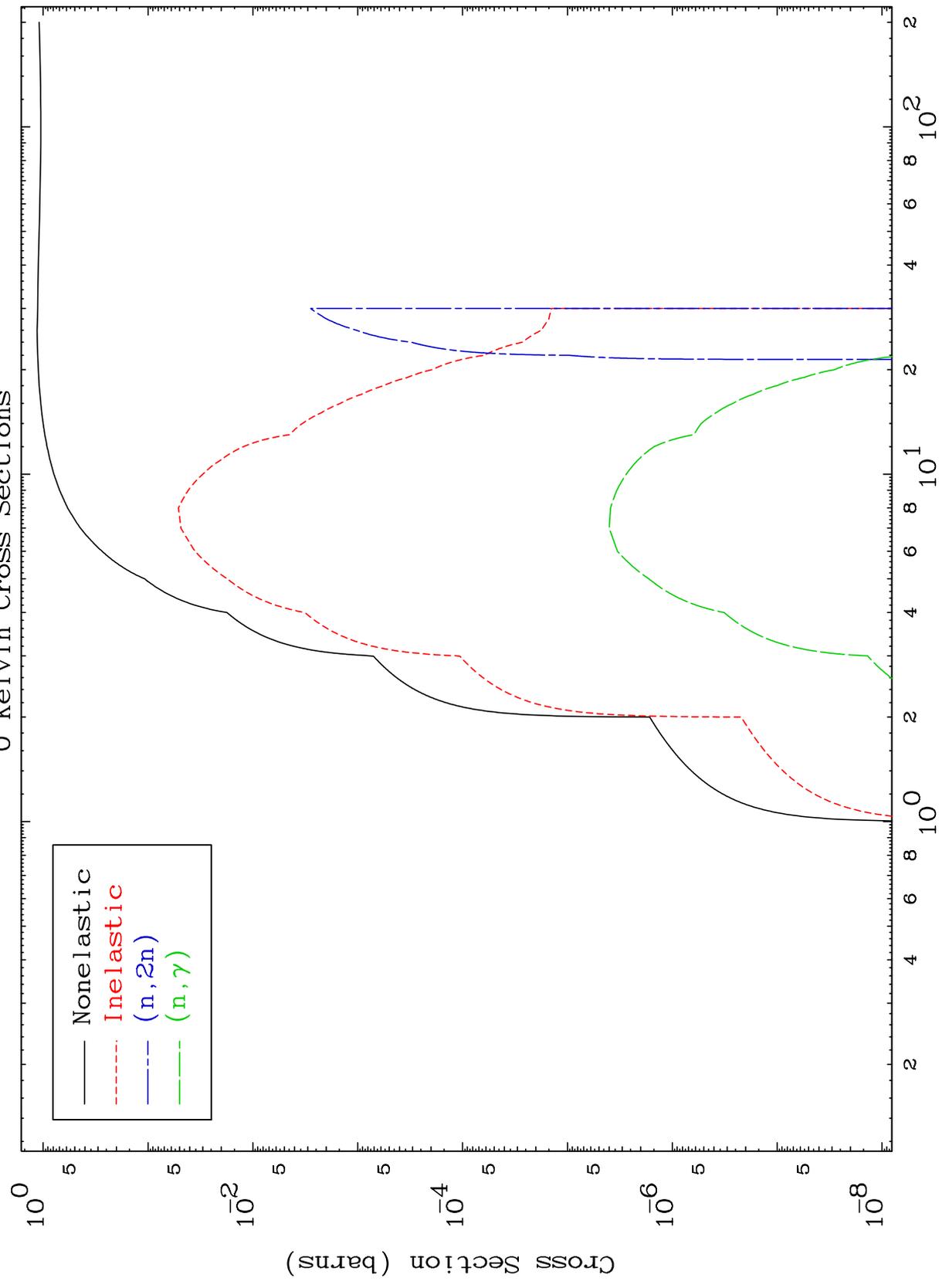
Press Mouse Button to Start

MAT 1323

0 Kelvin

$\alpha$  Major

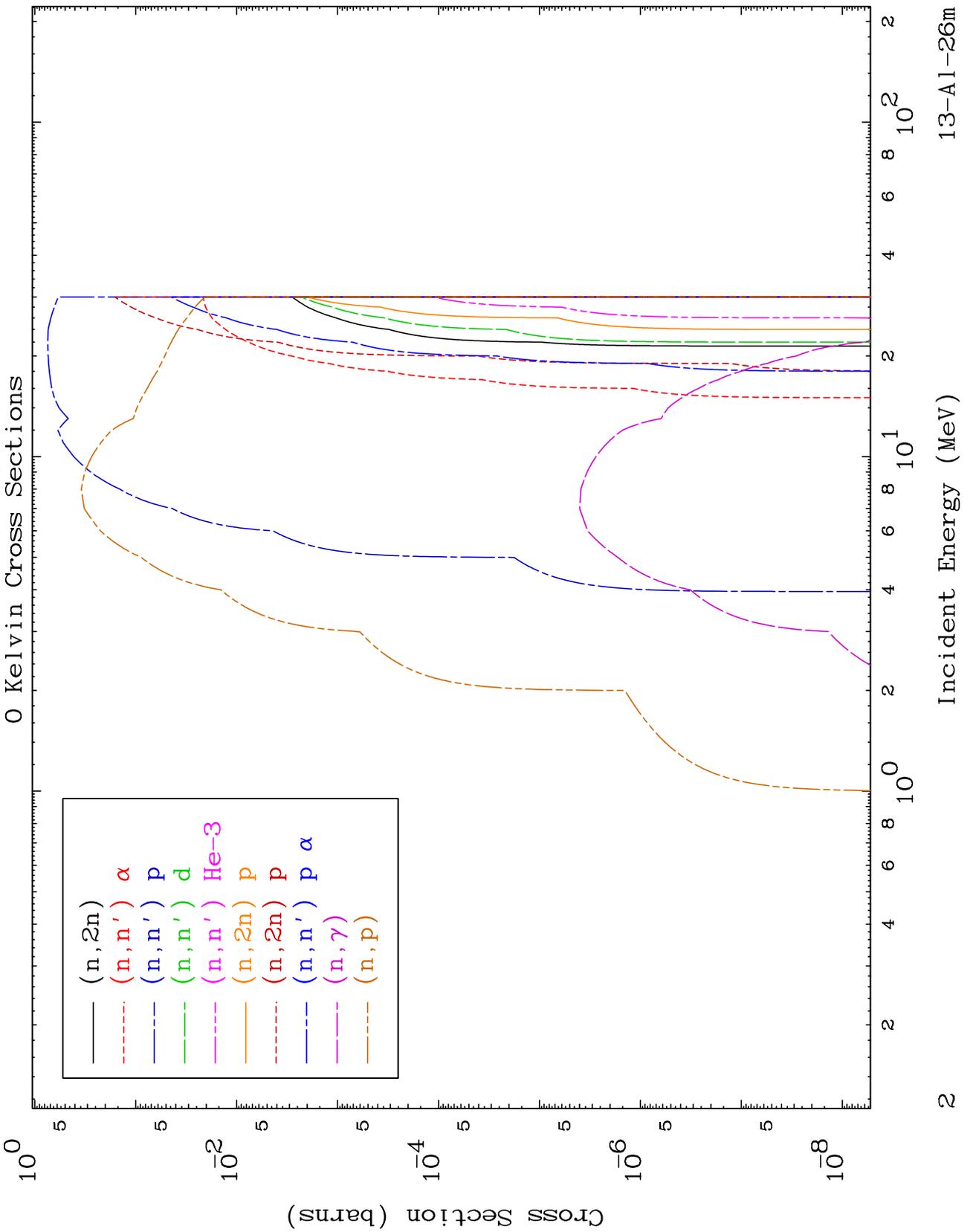
13-Al-26m



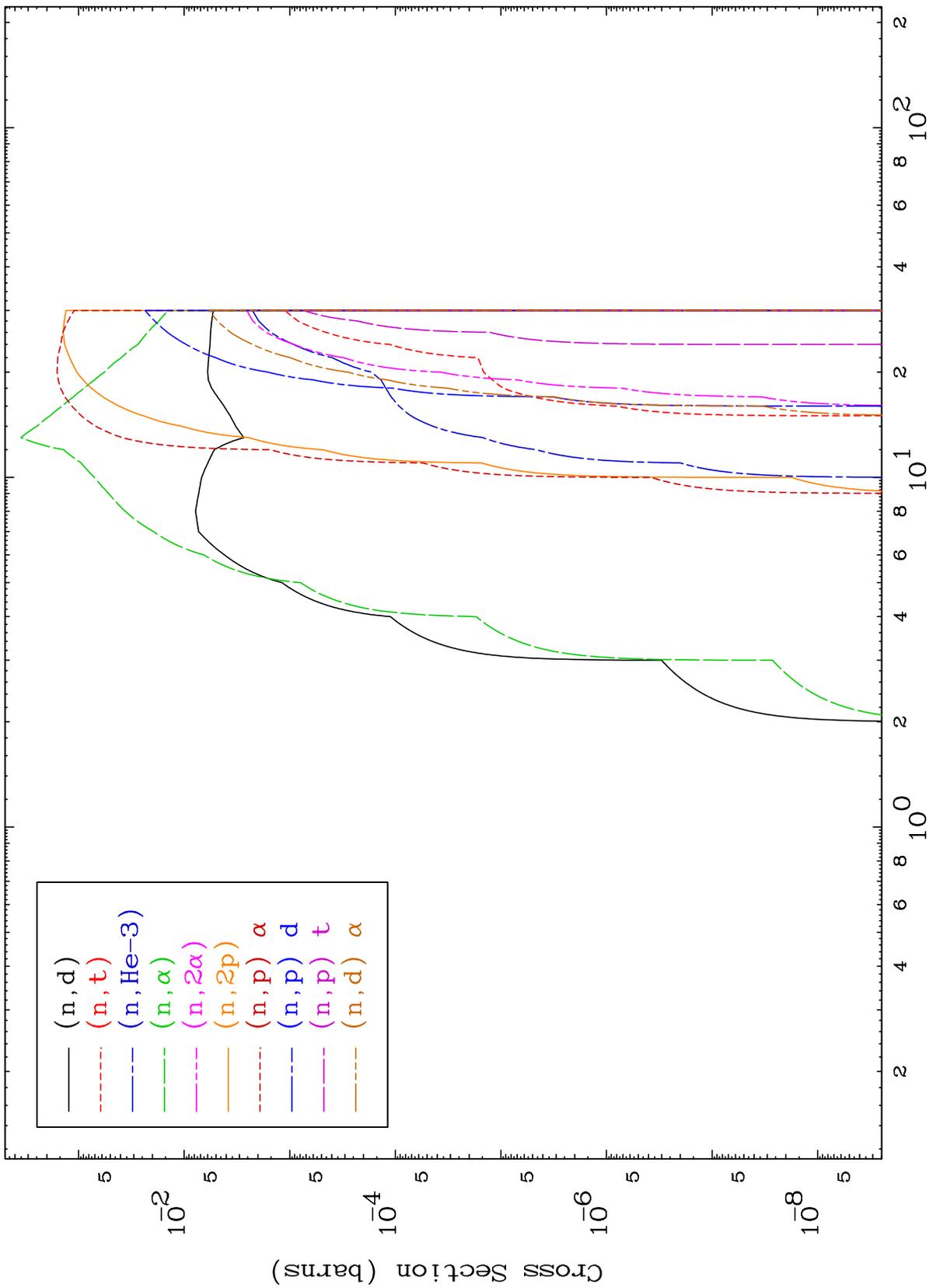
MAT 1323

$\alpha$  Neutron Absorption

13-Al-26m



$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

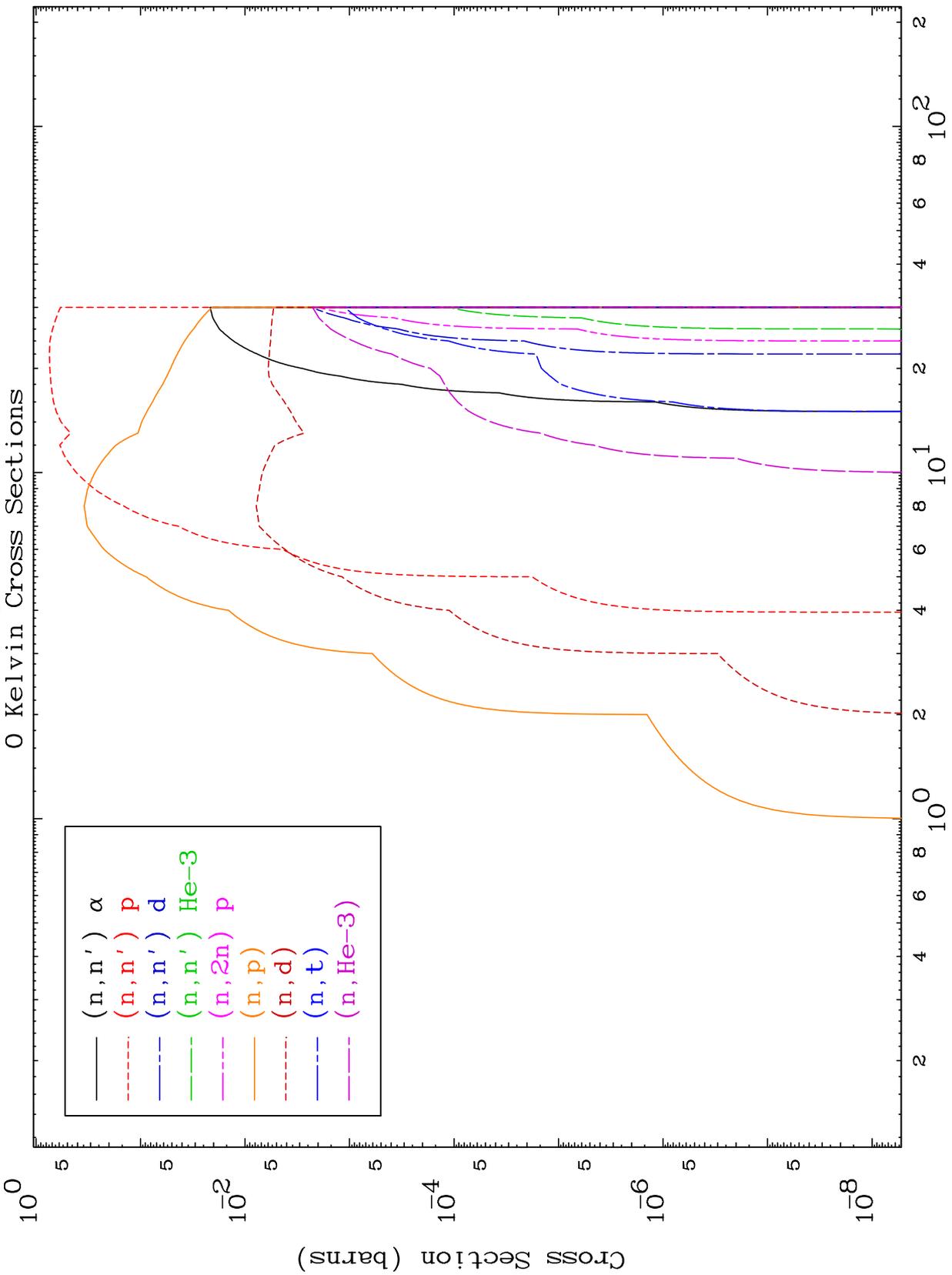


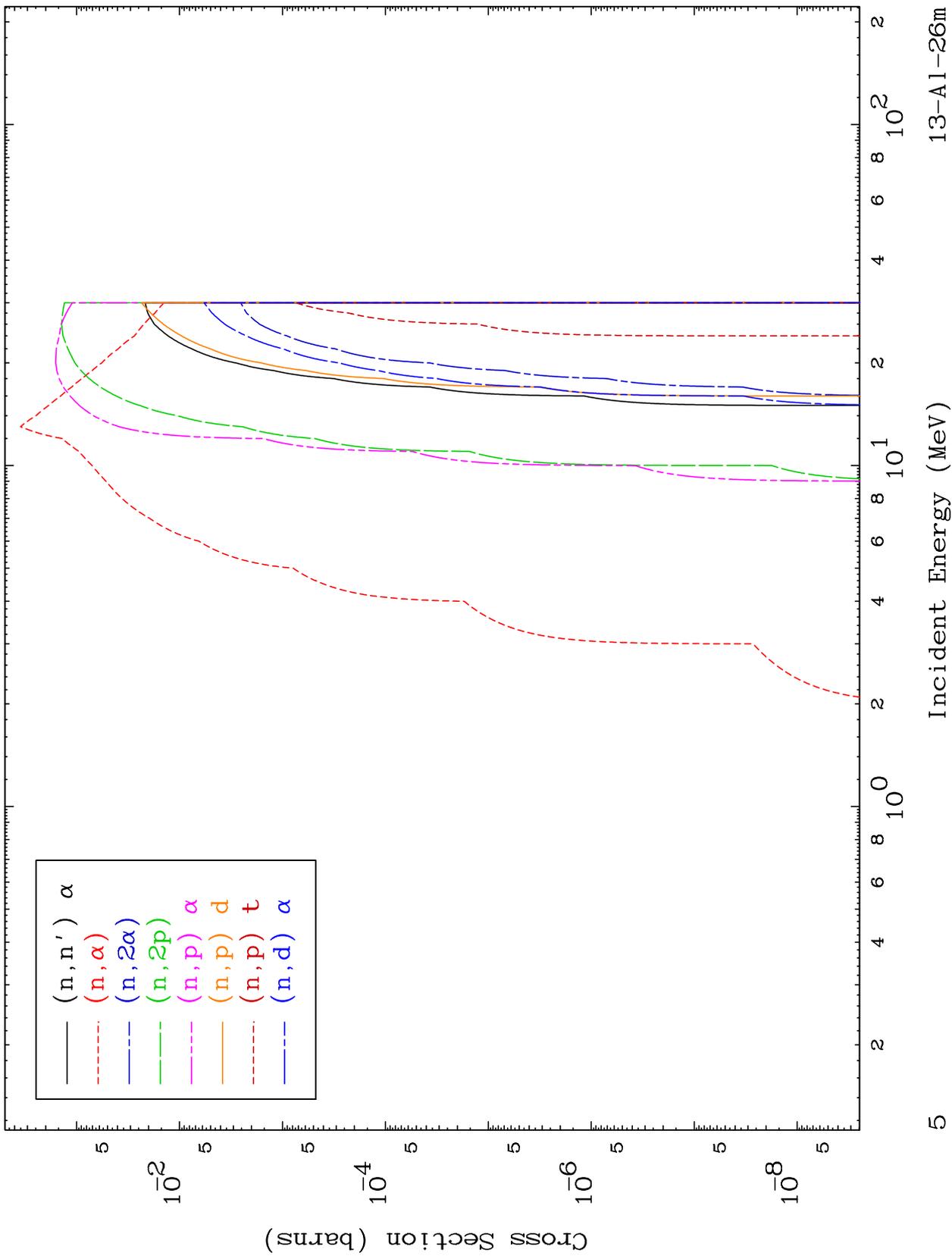
Incident Energy (MeV)

MAT 1323

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

13-Al-26m



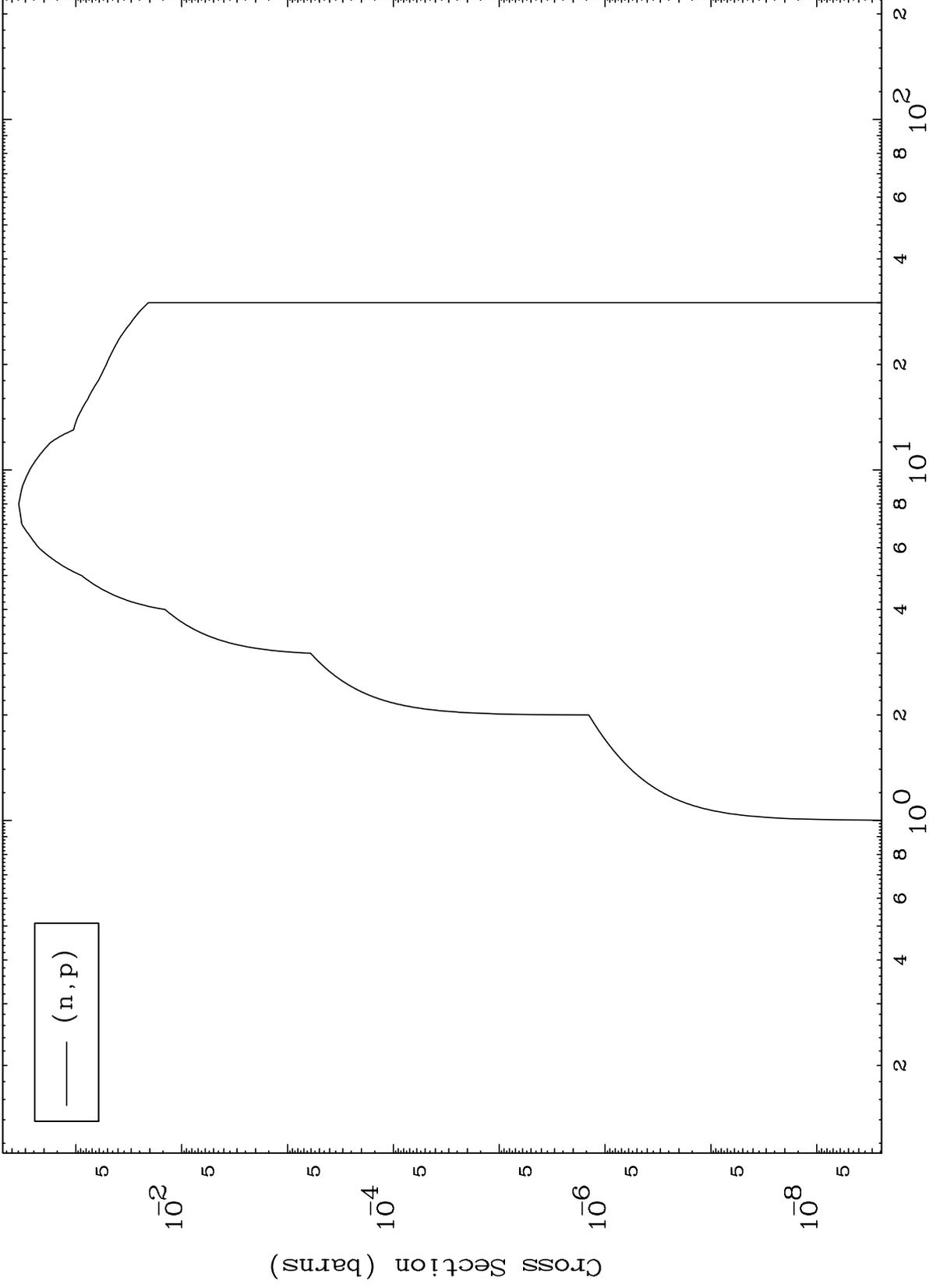


MAT 1323

( $\alpha, p$ ) Levels

13-Al-26m

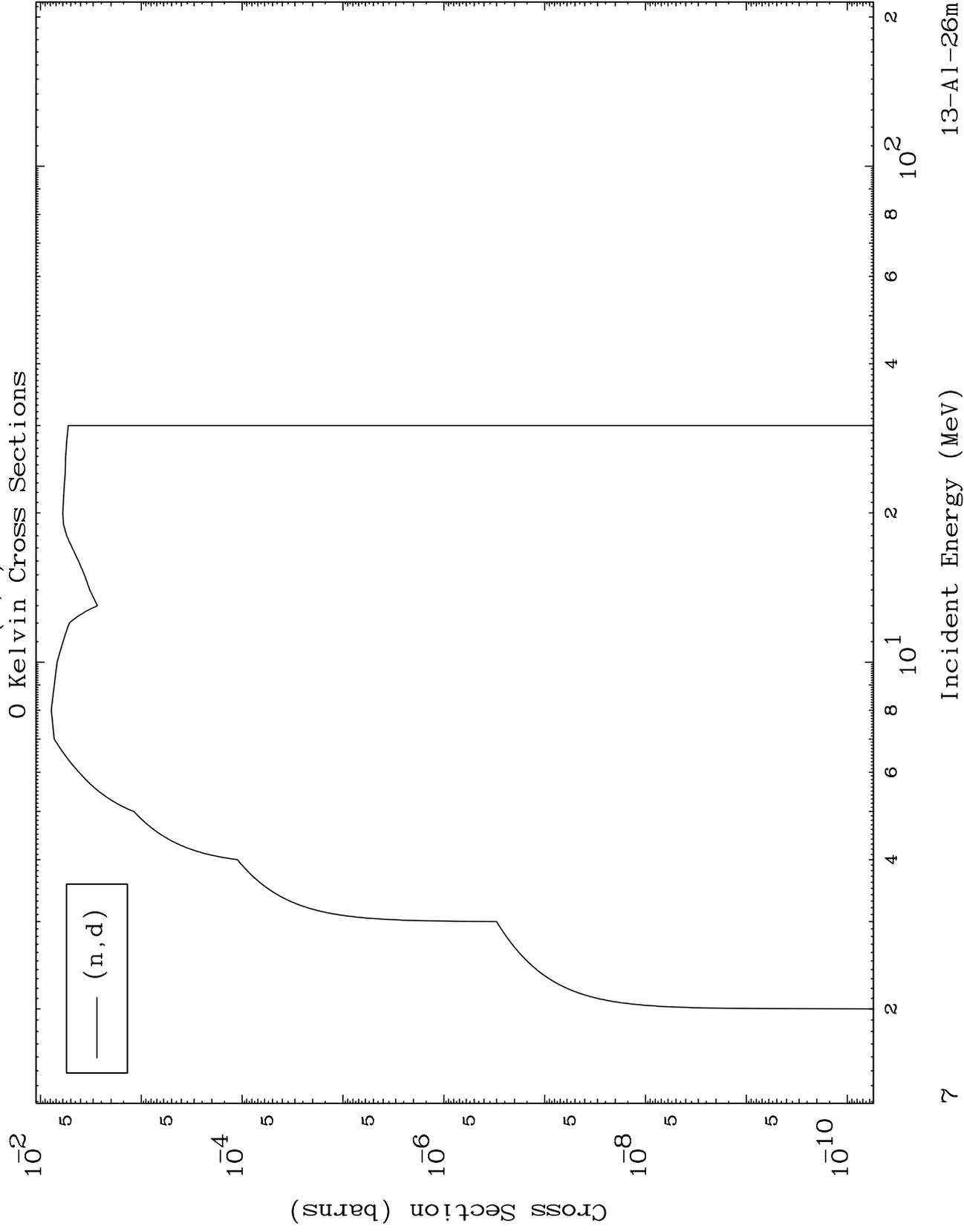
0 Kelvin Cross Sections



MAT 1323

( $\alpha, d$ ) Levels

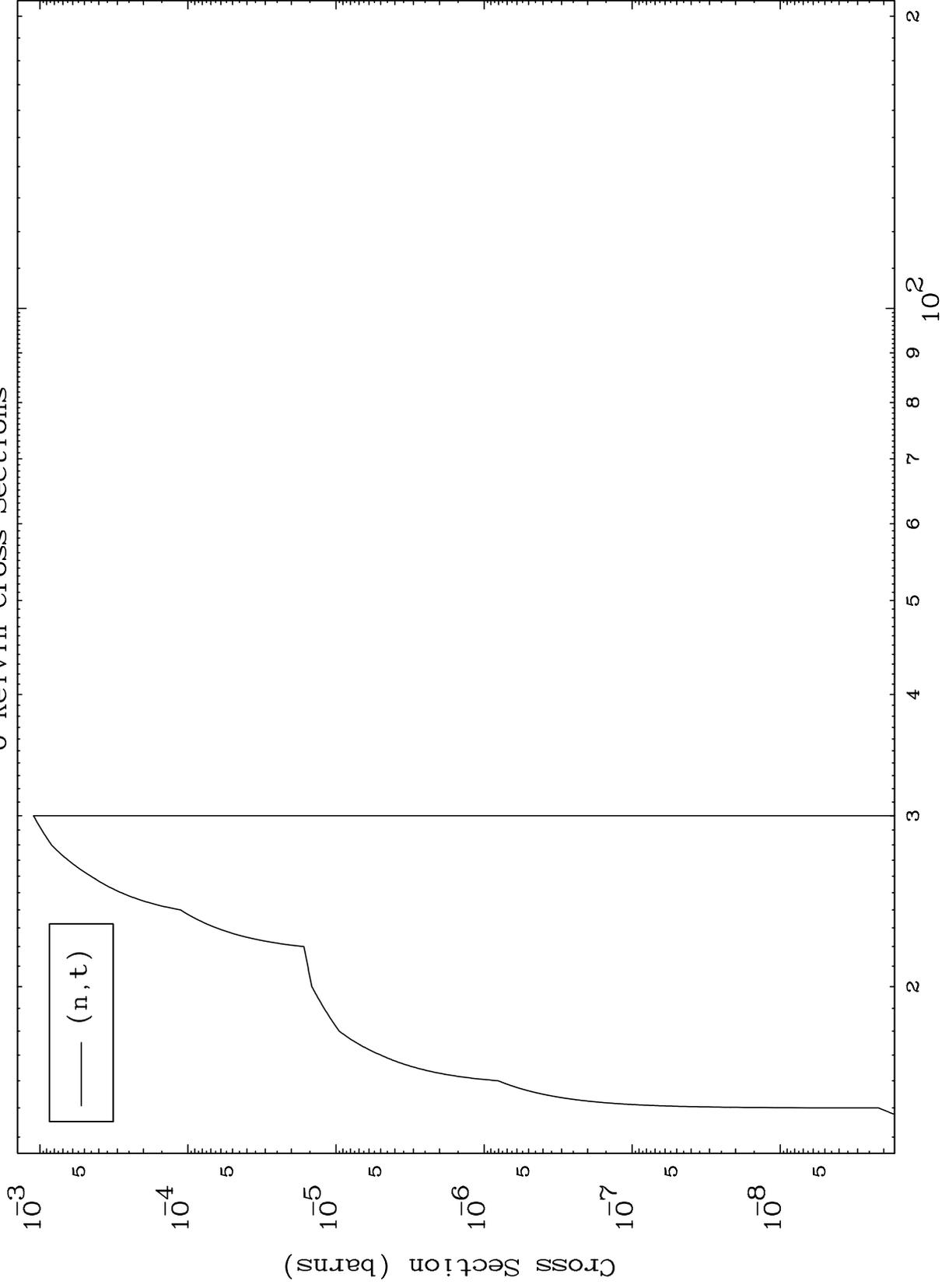
$^{13}\text{Al}-26\text{m}$



MAT 1323

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

13-Al-26m



8

Incident Energy (MeV)

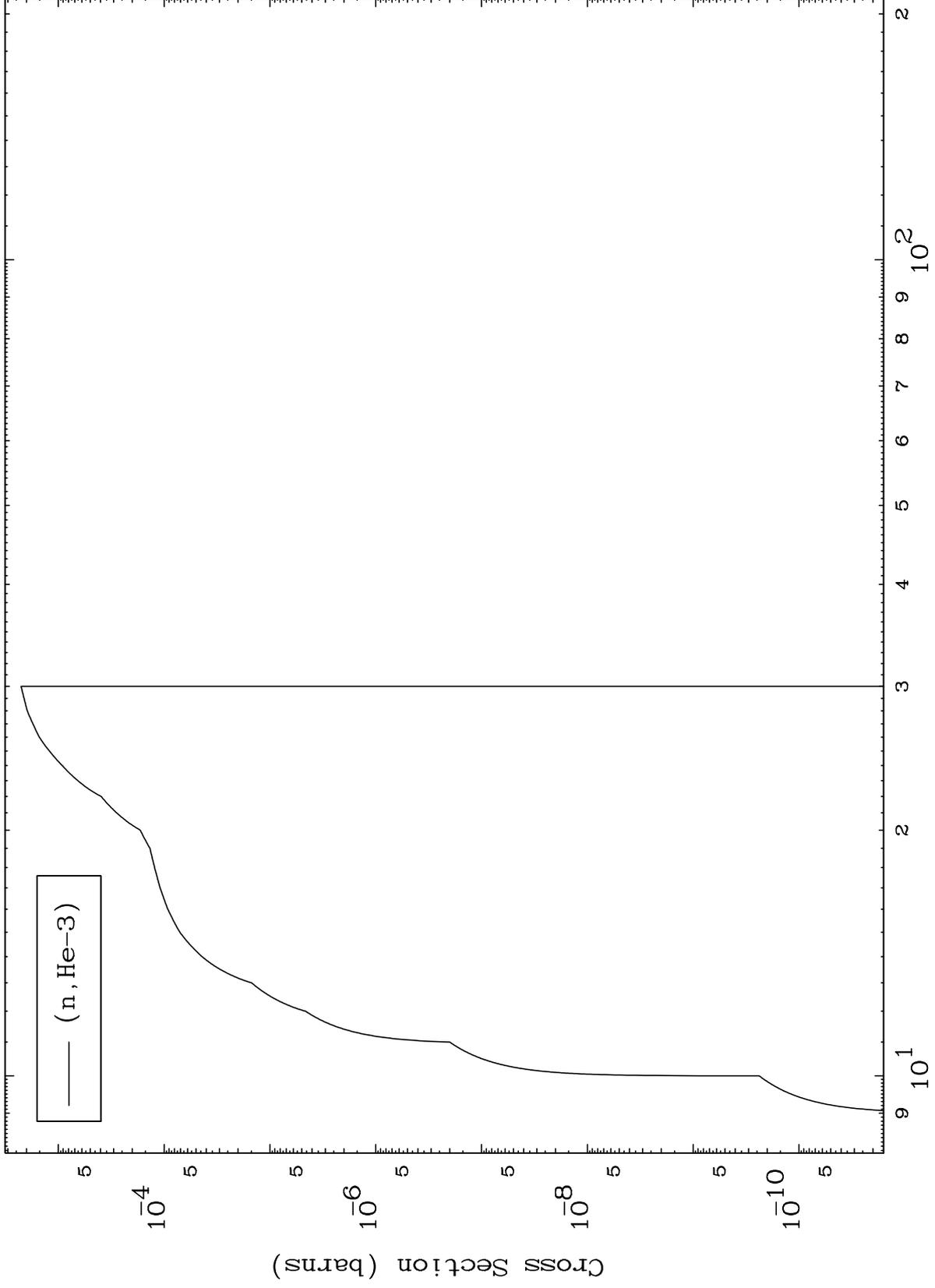
13-Al-26m

MAT 1323

( $\alpha, \text{He3}$ ) Levels

13-Al-26m

0 Kelvin Cross Sections



9

Incident Energy (MeV)

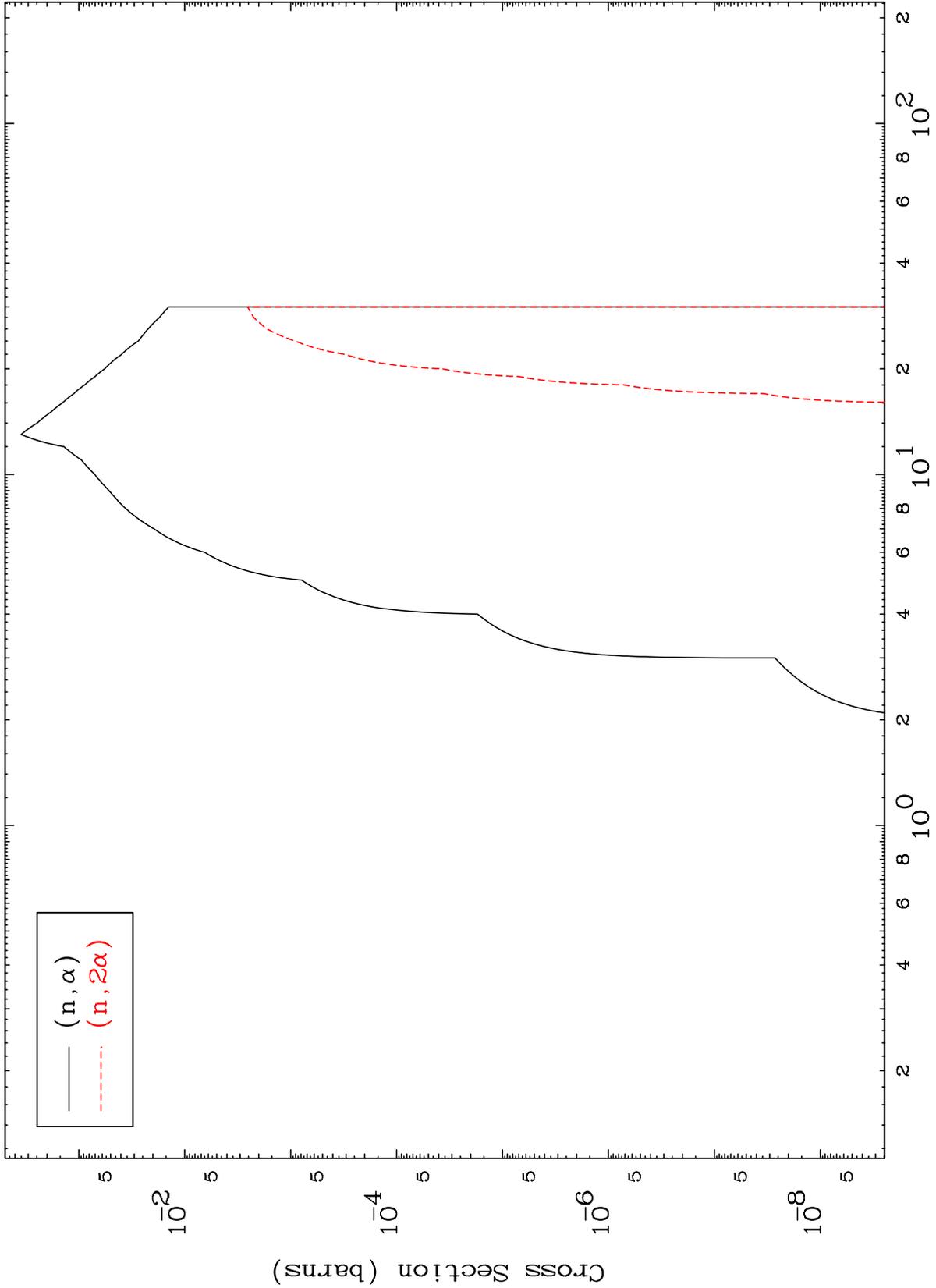
13-Al-26m

MAT 1323

( $\alpha, \alpha$ ) Levels

13-Al-26m

0 Kelvin Cross Sections



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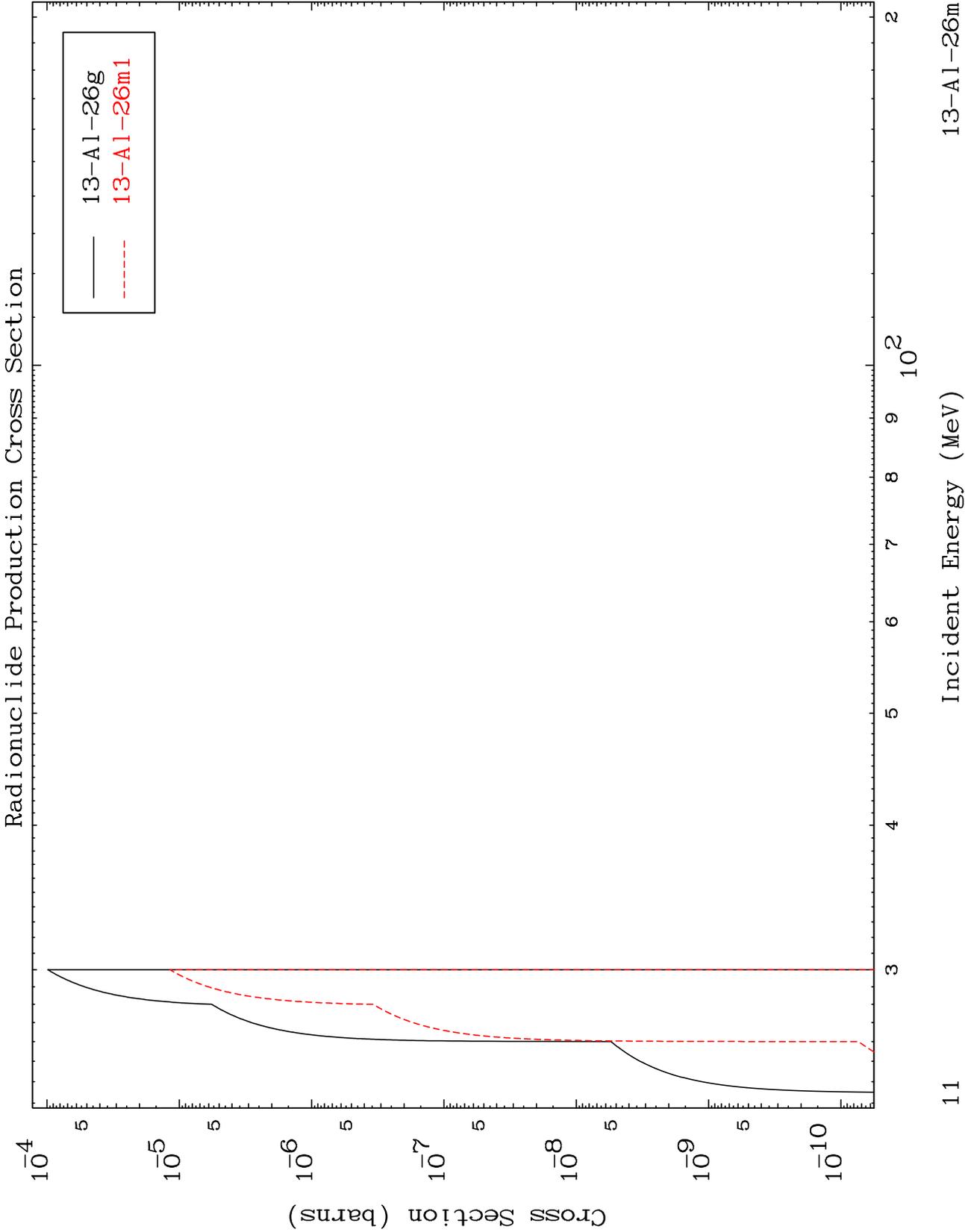
Incident Energy (MeV)

13-Al-26m

MAT 1323

(n,n') He-3

13-Al-26m



11

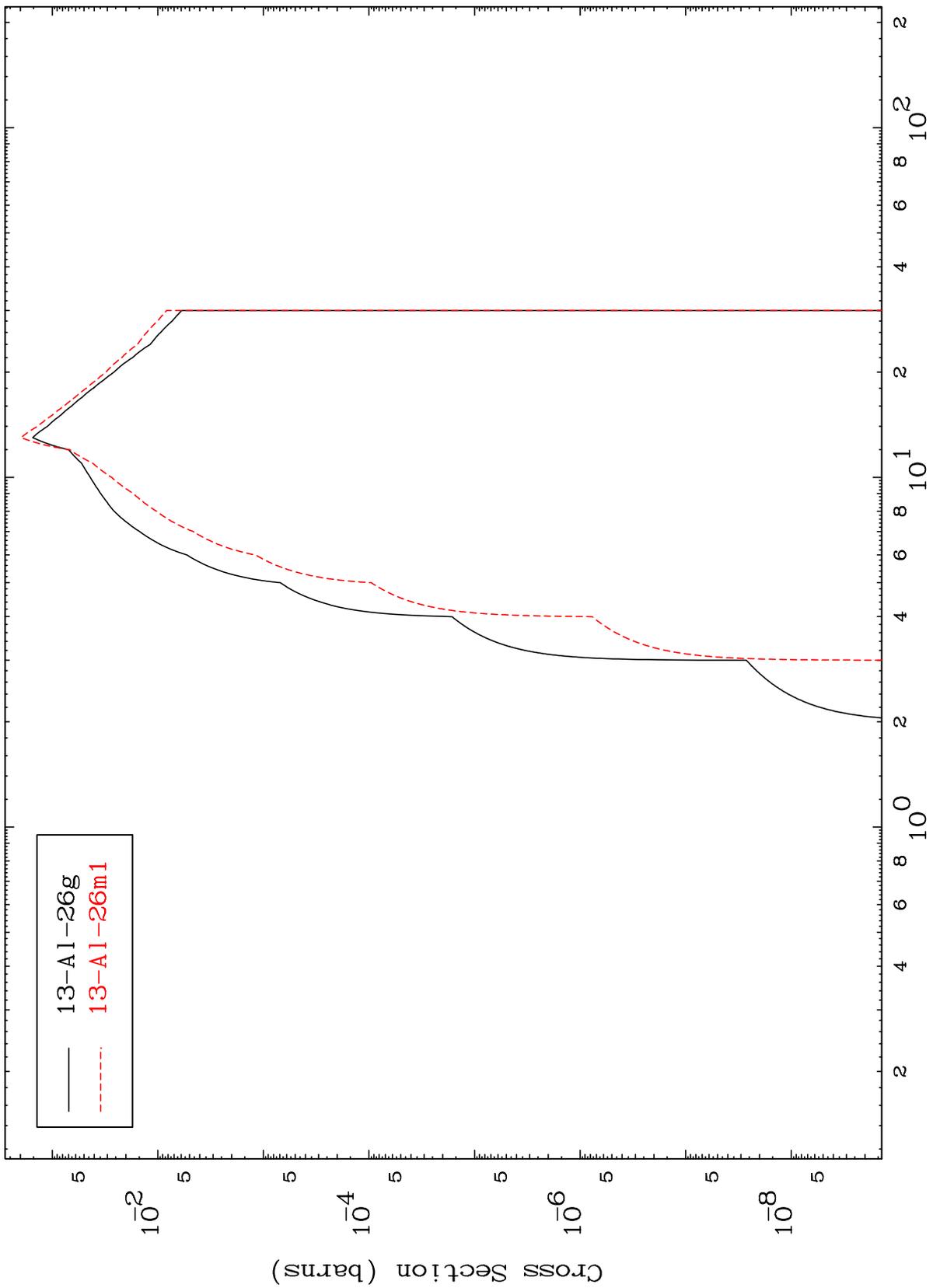
Incident Energy (MeV)

13-Al-26m

MAT 1323

13-Al-26m

(n,  $\alpha$ )  
Radionuclide Production Cross Section



— 13-Al-26g  
- - - 13-Al-26m1

12

13-Al-26m

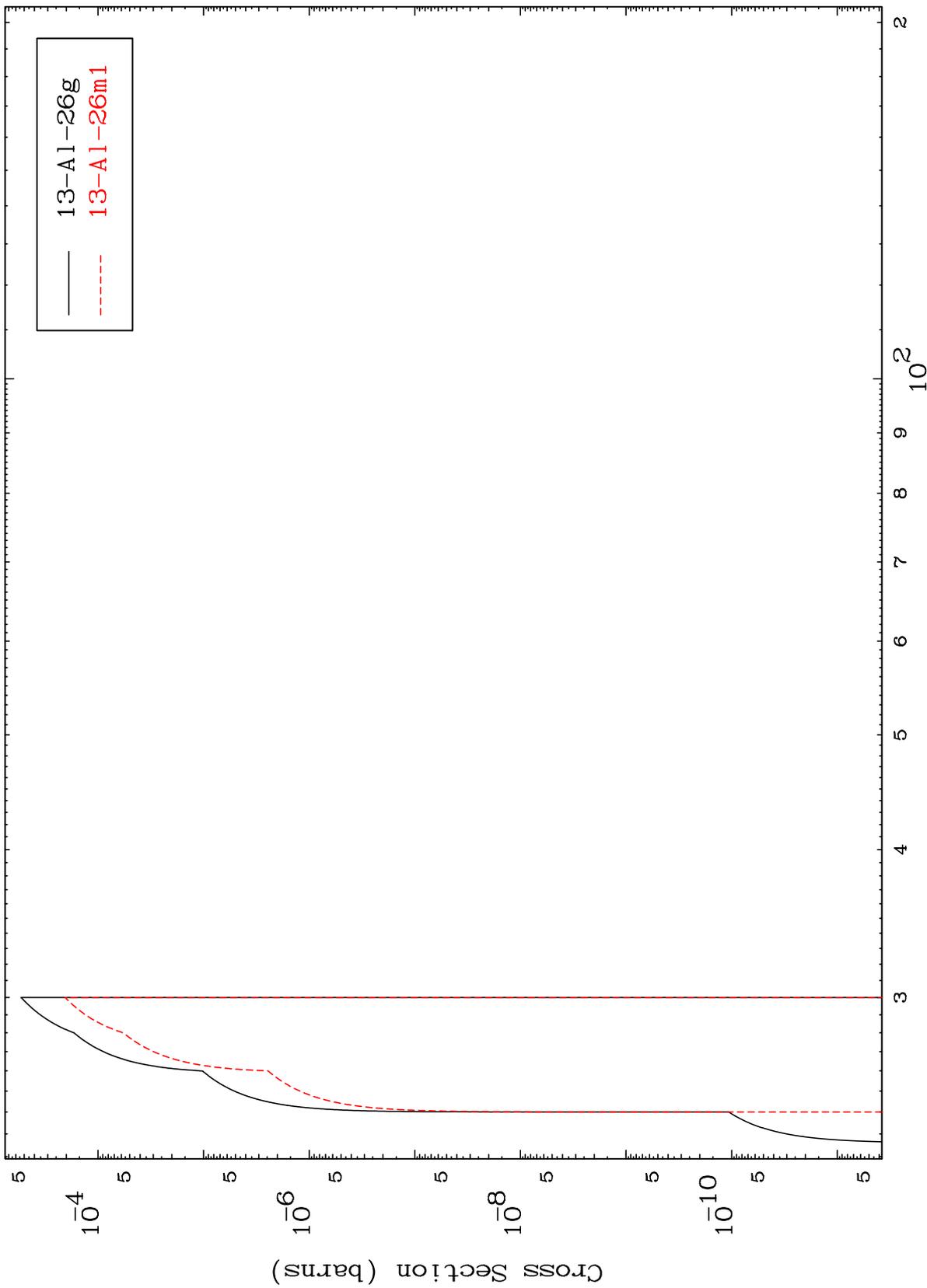
Incident Energy (MeV)

MAT 1323

(n,p) t

13-Al-26m

Radionuclide Production Cross Section



13

Incident Energy (MeV)

13-Al-26m