

Program EVALPLOT  
(Version 2021-1)

by

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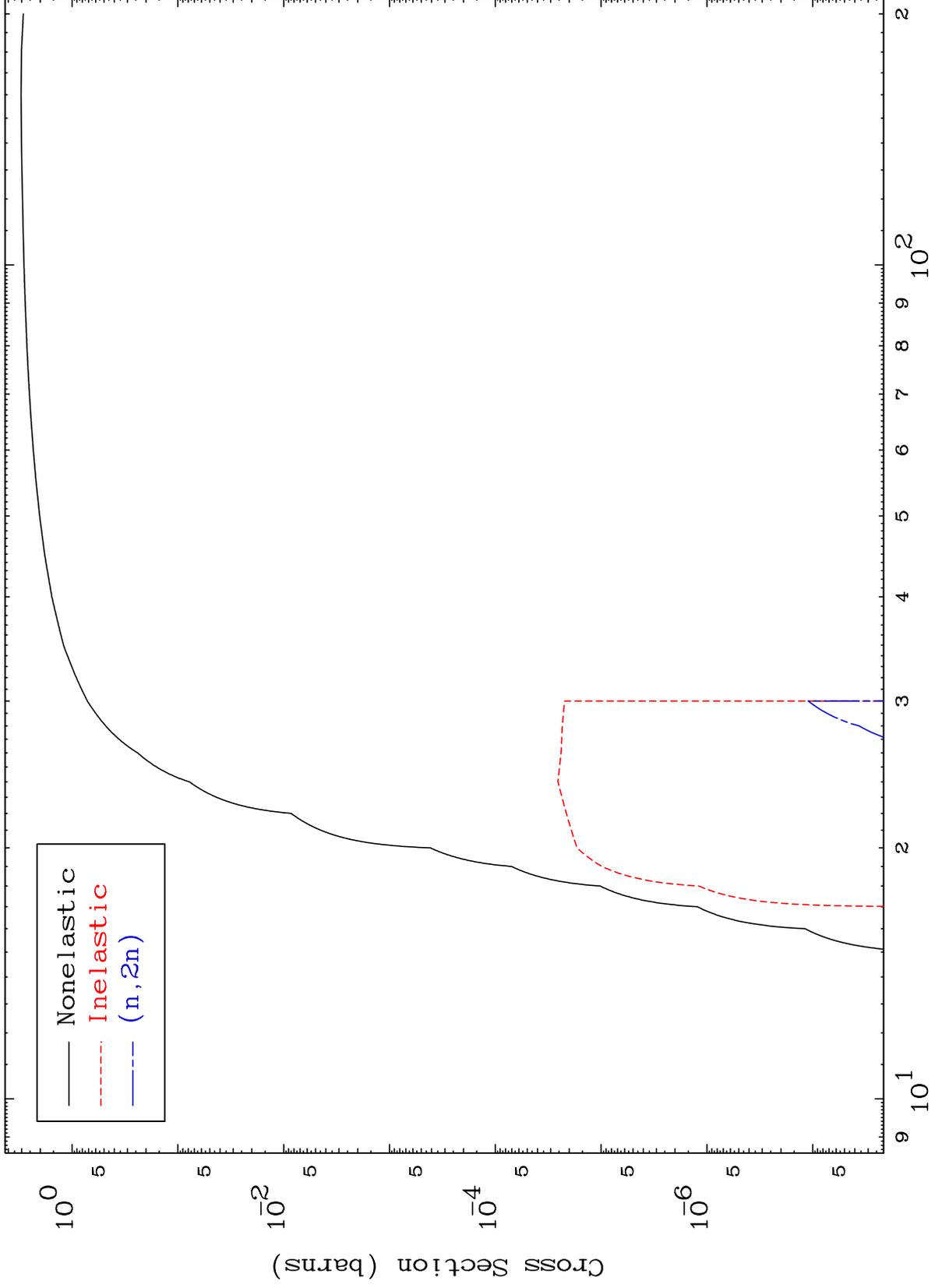
Press Mouse Button to Start

MAT 9933

0 Kelvin Cross Sections

$\alpha$  Major

100-Fm-247m



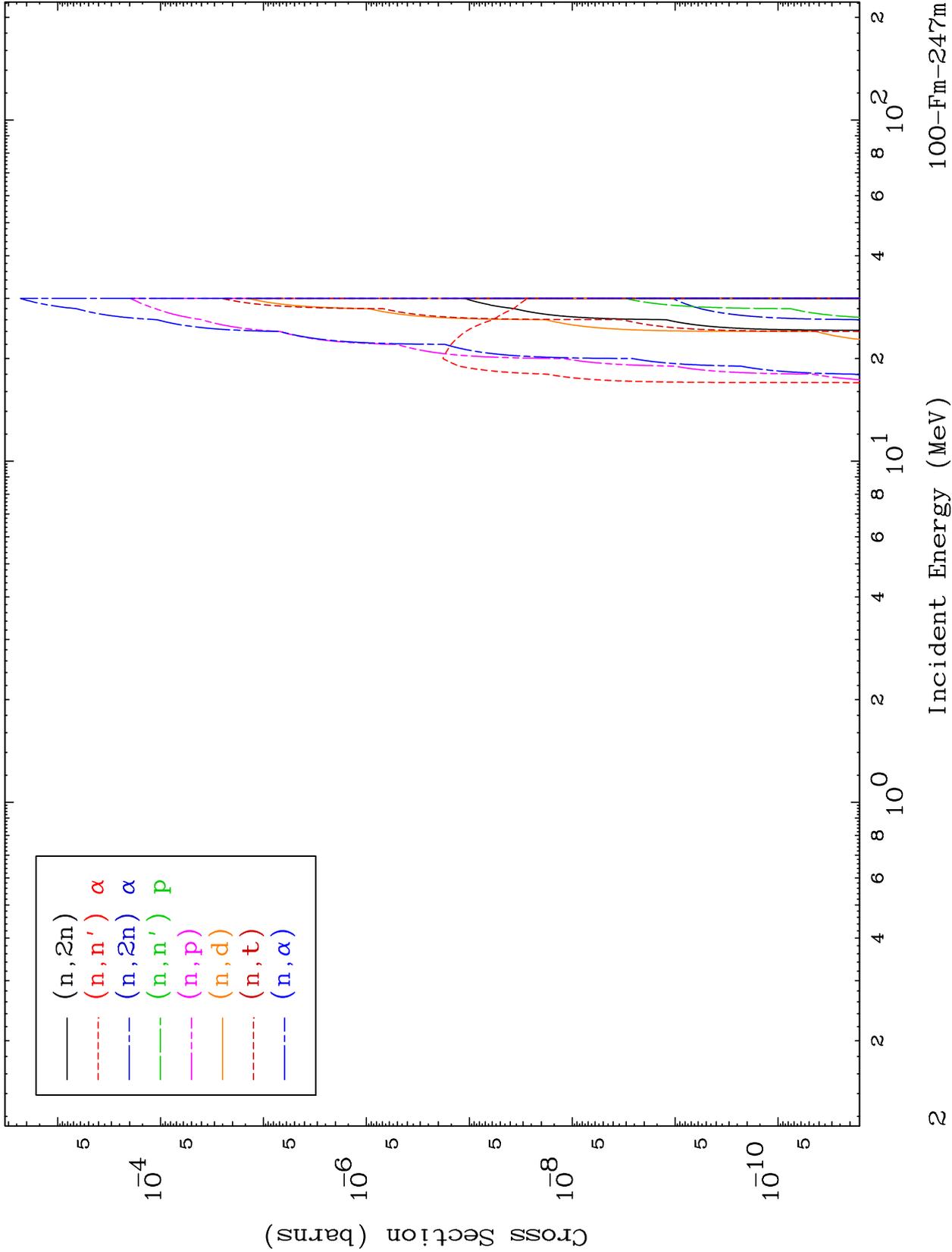
Incident Energy (MeV)

100-Fm-247m

MAT 9933

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

100-Fm-247m

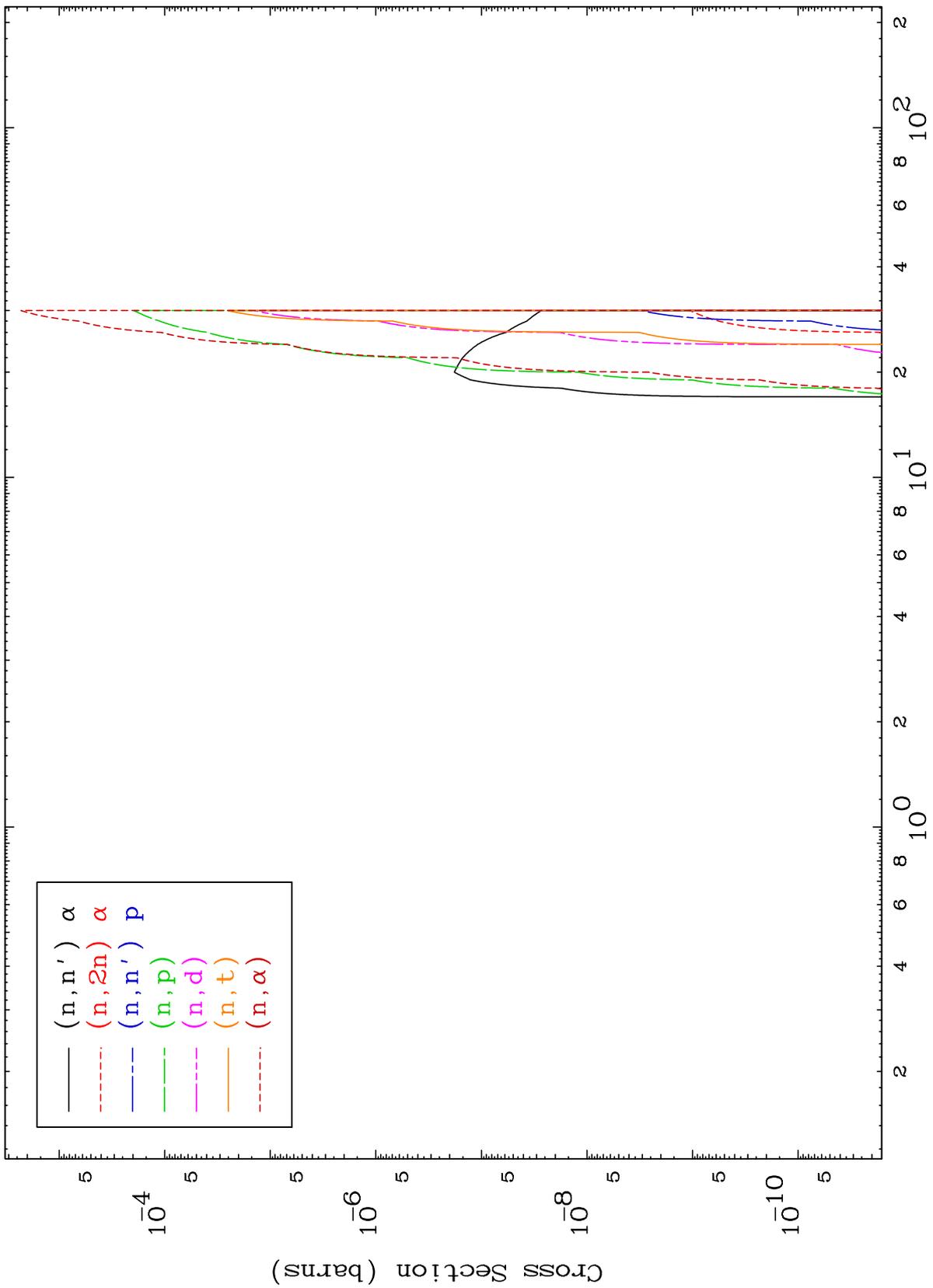


MAT 9933

$\alpha$  Charged Particle

100-Fm-247m

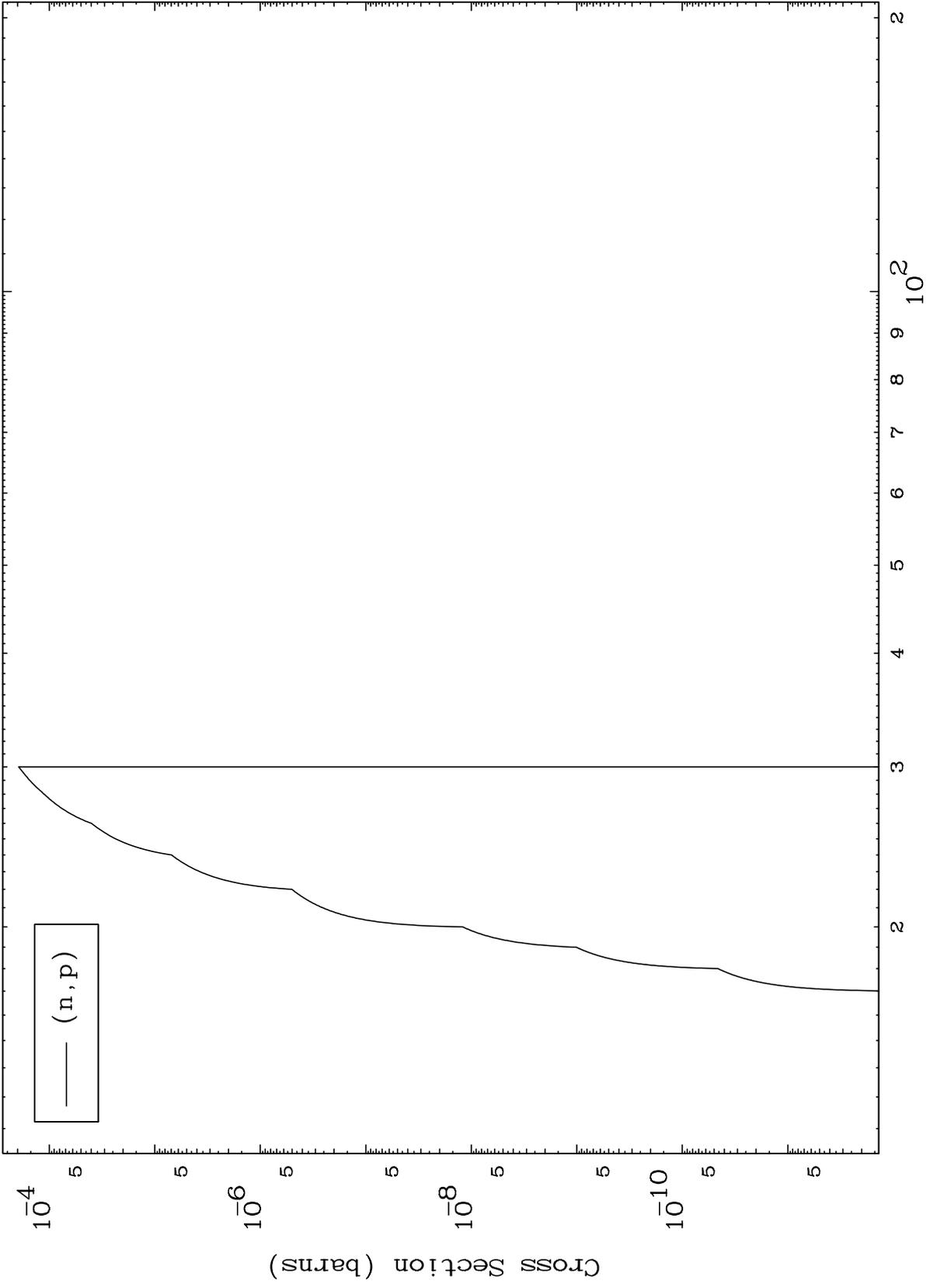
0 Kelvin Cross Sections



MAT 9933

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

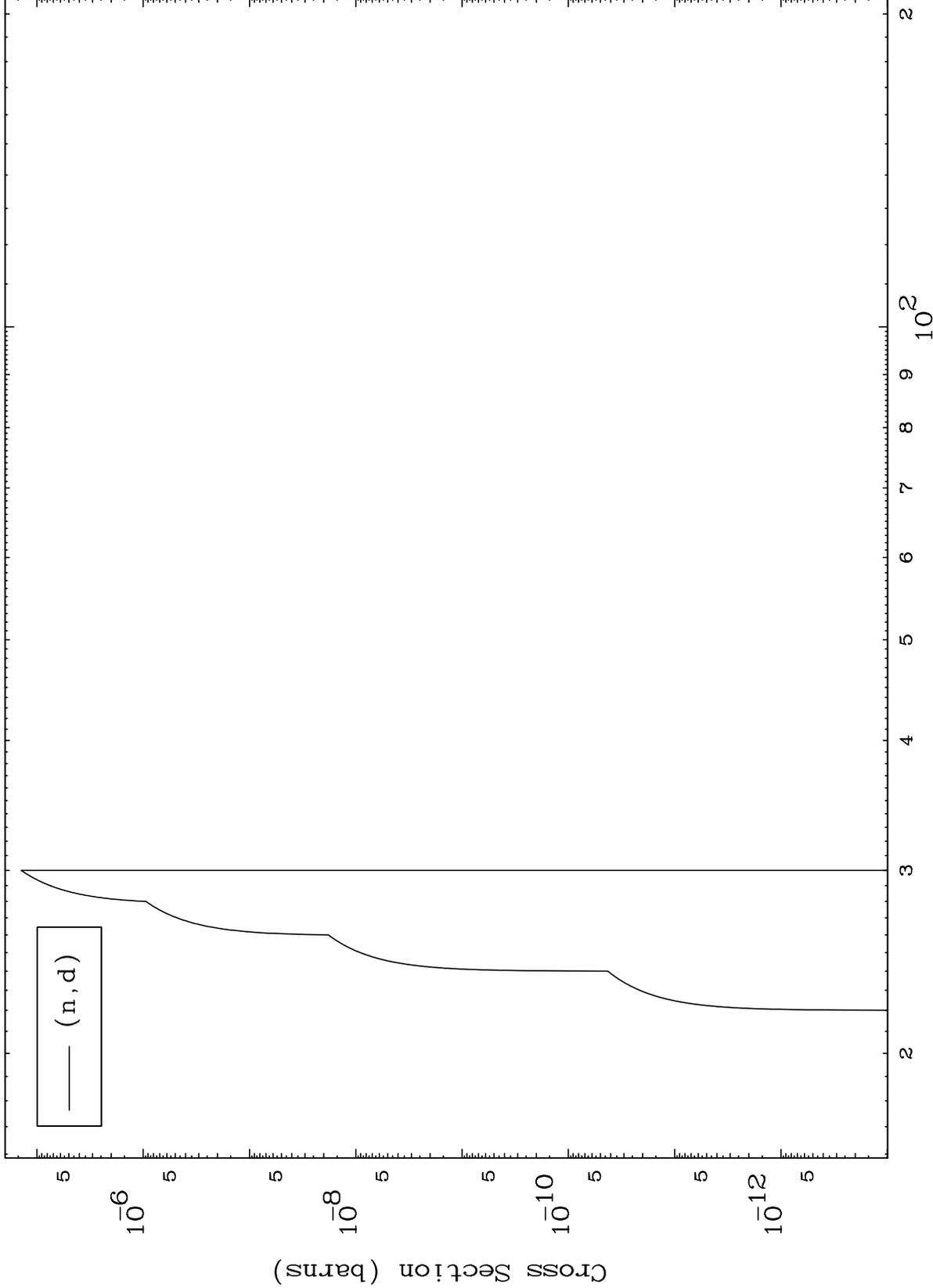
100-Fm-247m



MAT 9933

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

100-Fm-247m



5

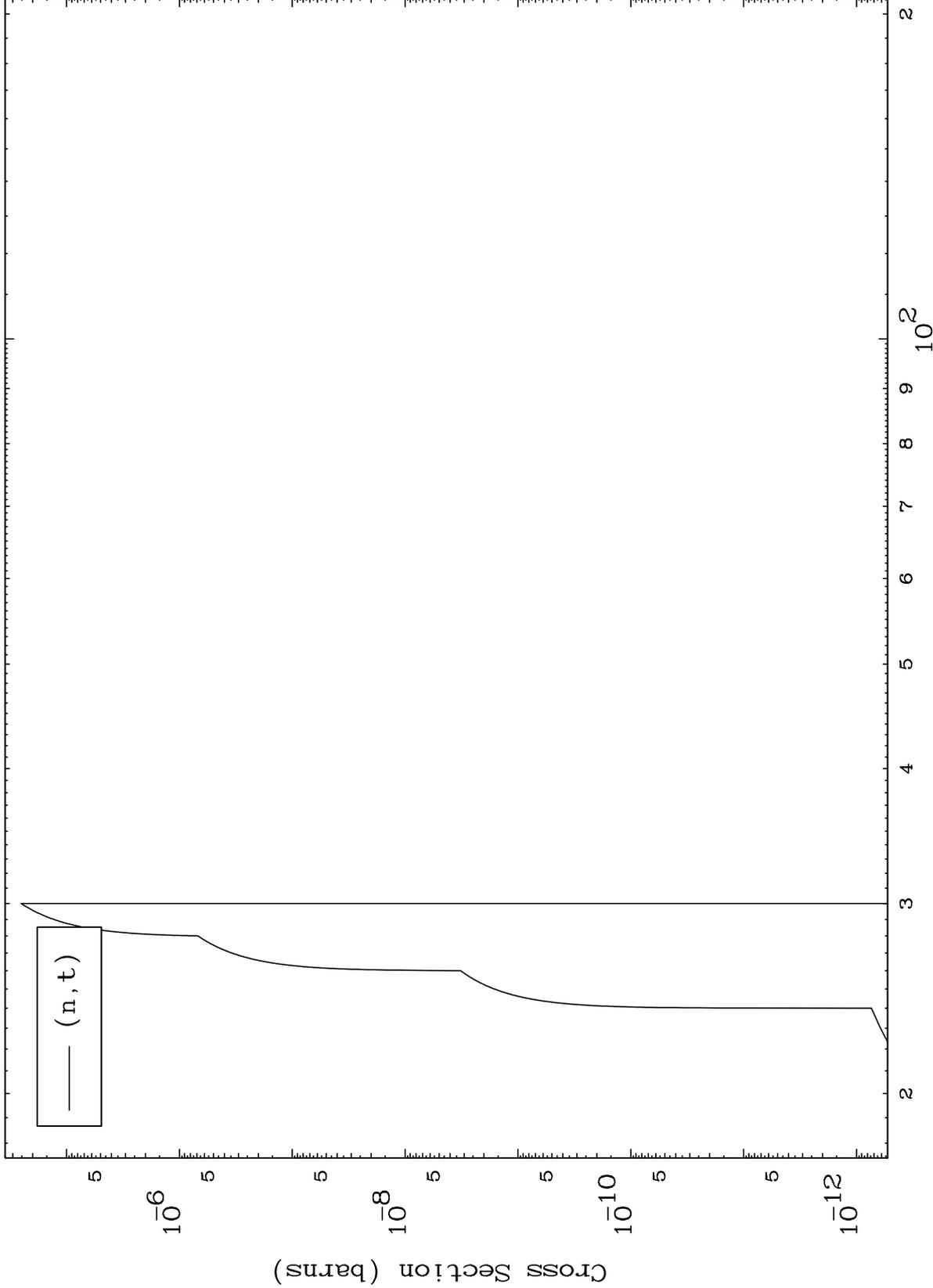
Incident Energy (MeV)

100-Fm-247m

MAT 9933

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

100-Fm-247m



6

Incident Energy (MeV)

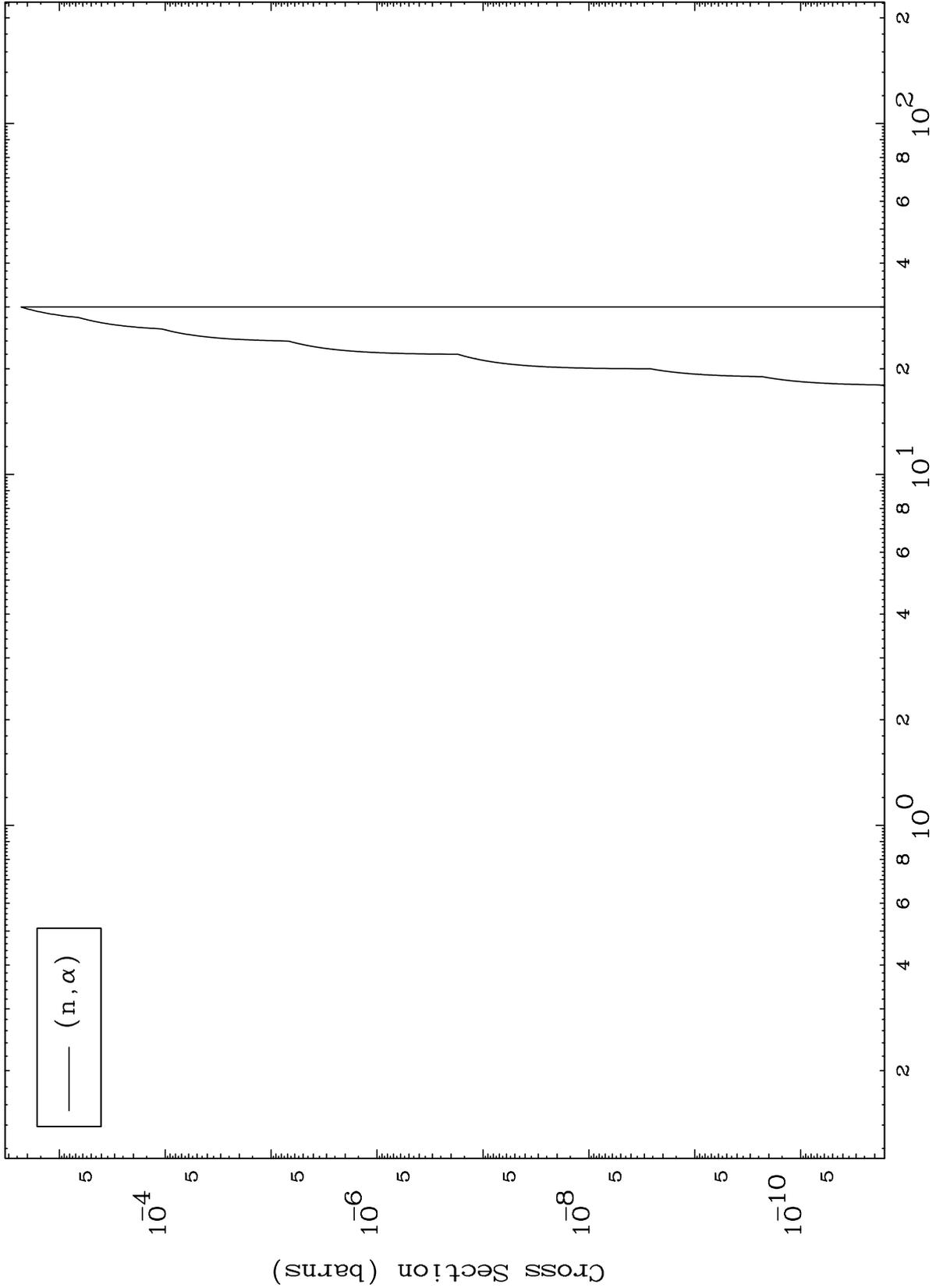
100-Fm-247m

MAT 9933

( $\alpha, \alpha$ ) Levels

100-Fm-247m

0 Kelvin Cross Sections

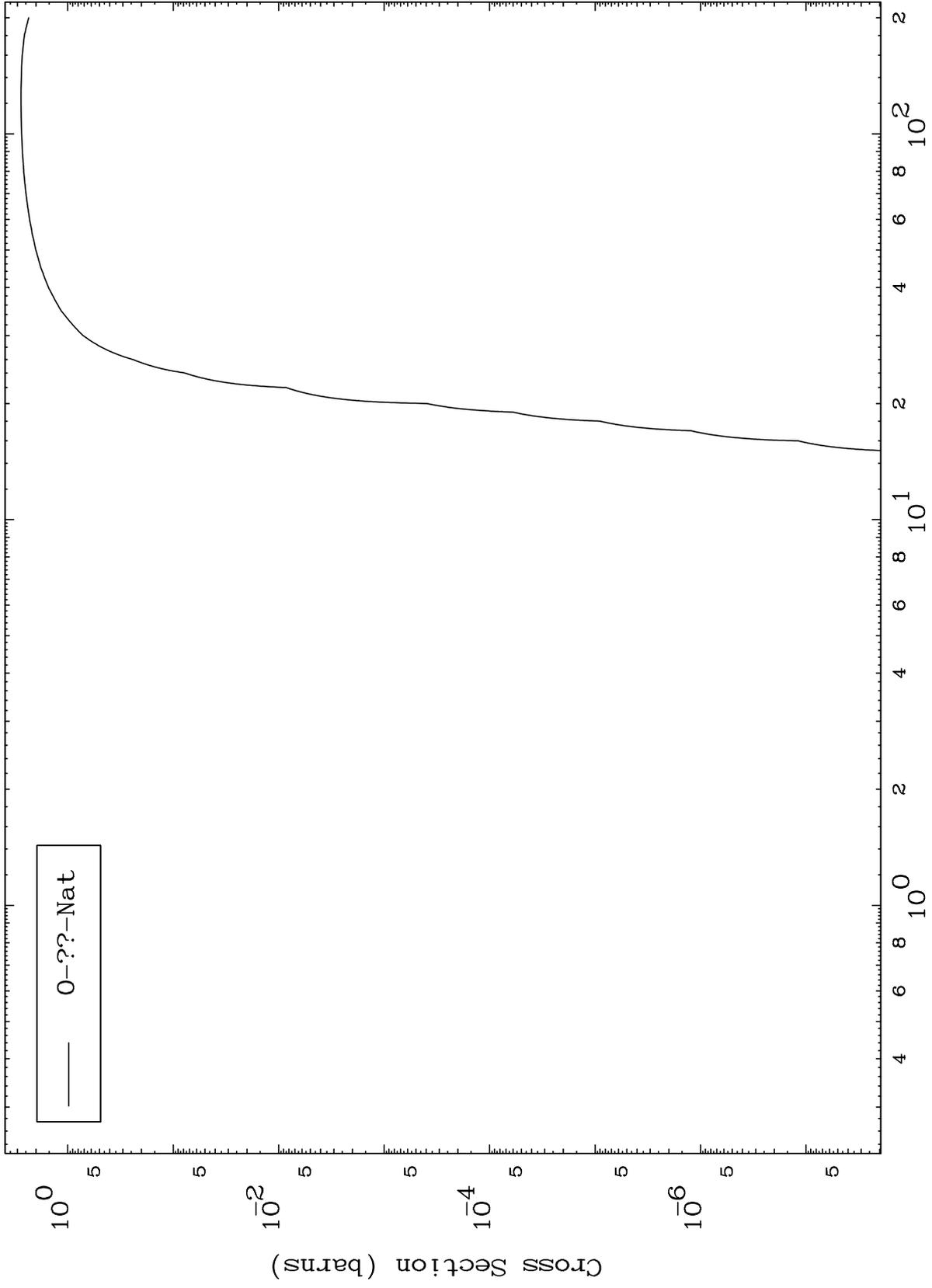


MAT 9933

100-Fm-247m

Fission

Radionuclide Production Cross Section

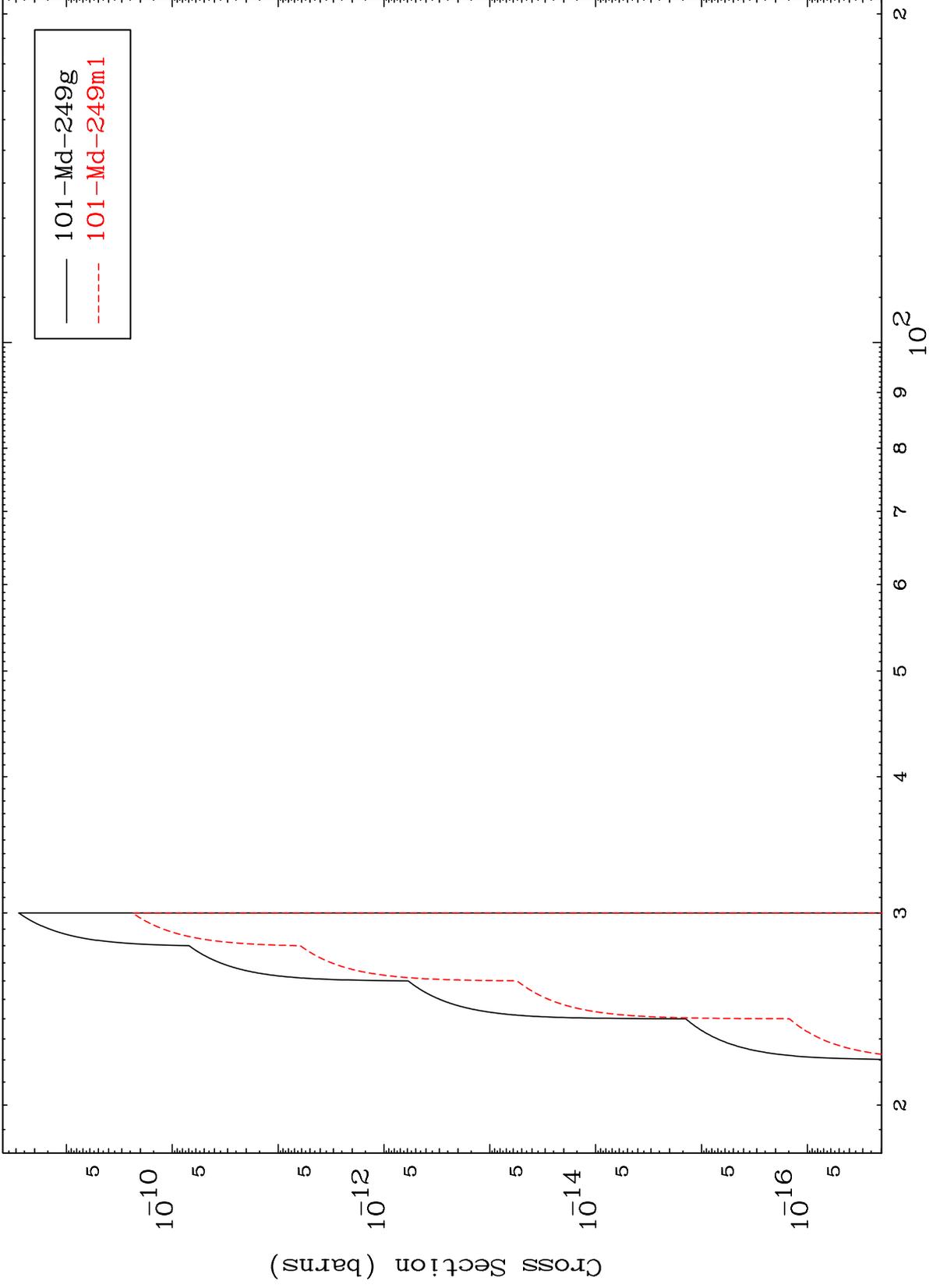


MAT 9933

(n,n') p

100-Fm-247m

Radionuclide Production Cross Section



9

Incident Energy (MeV)

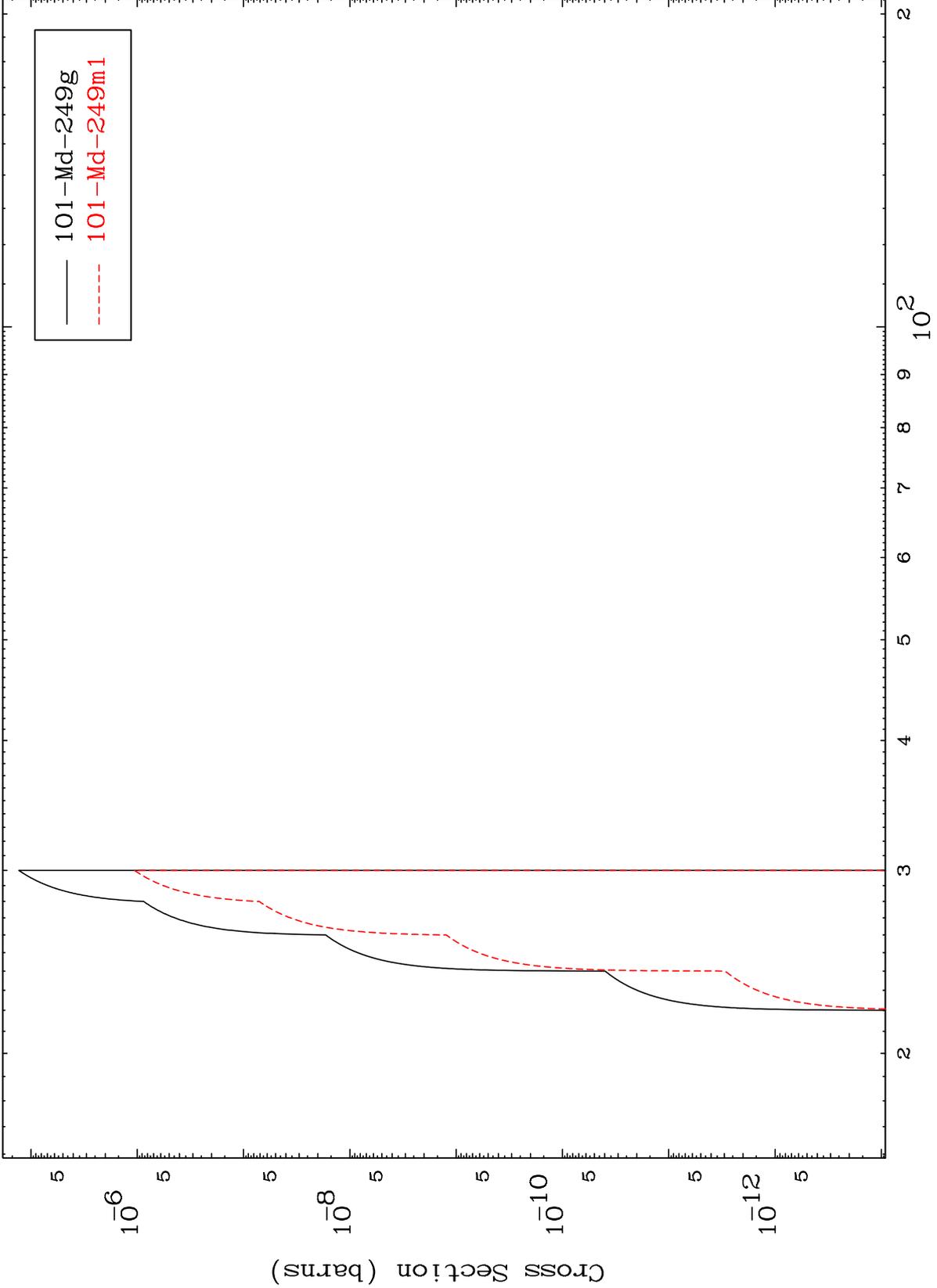
100-Fm-247m

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(n,d)

100-Fm-247m

Radionuclide Production Cross Section



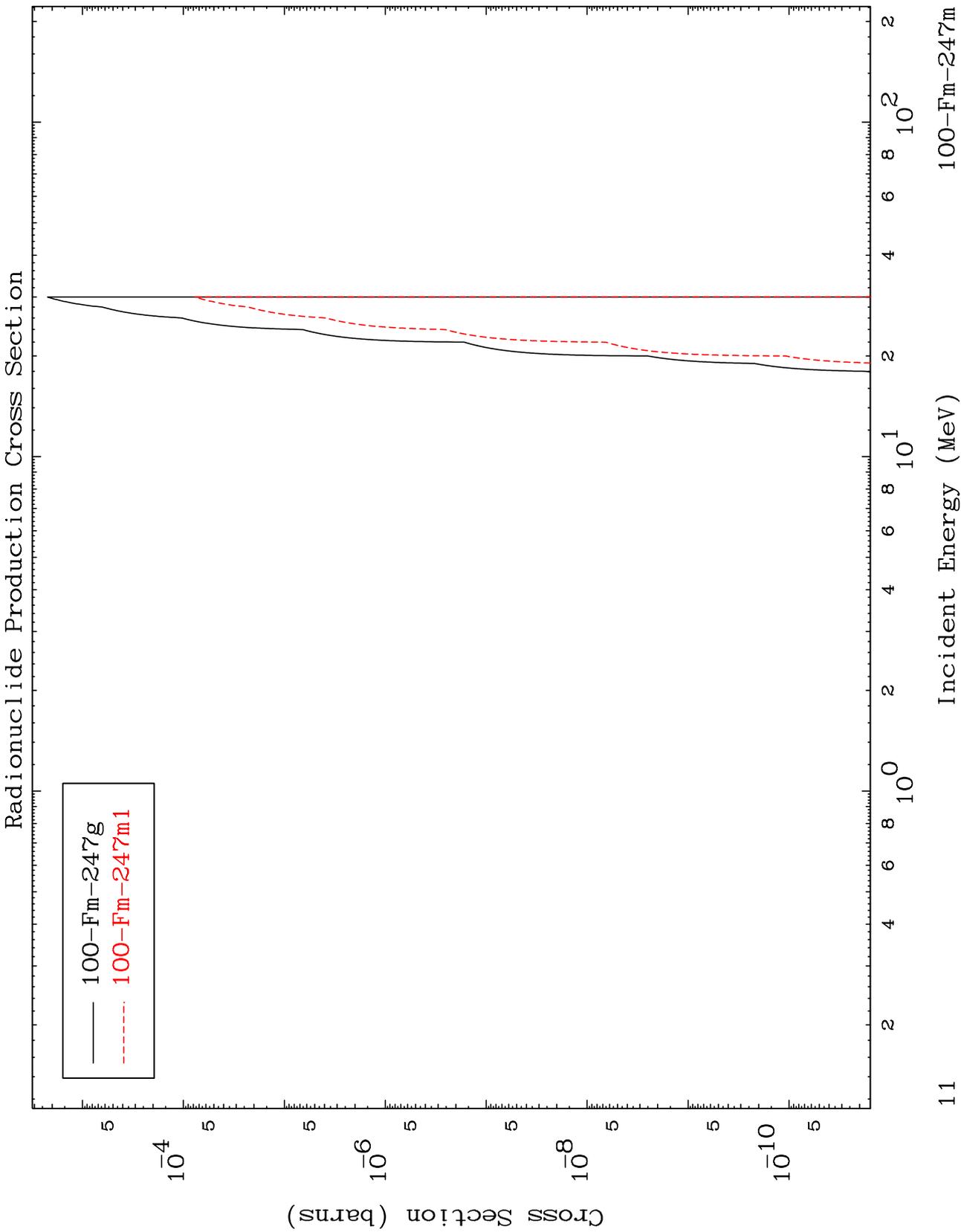
10

Incident Energy (MeV)

100-Fm-247m

MAT 9933

100-Fm-247m



100-Fm-247m