

Program EVALPLOT  
(Version 2021-1)

by

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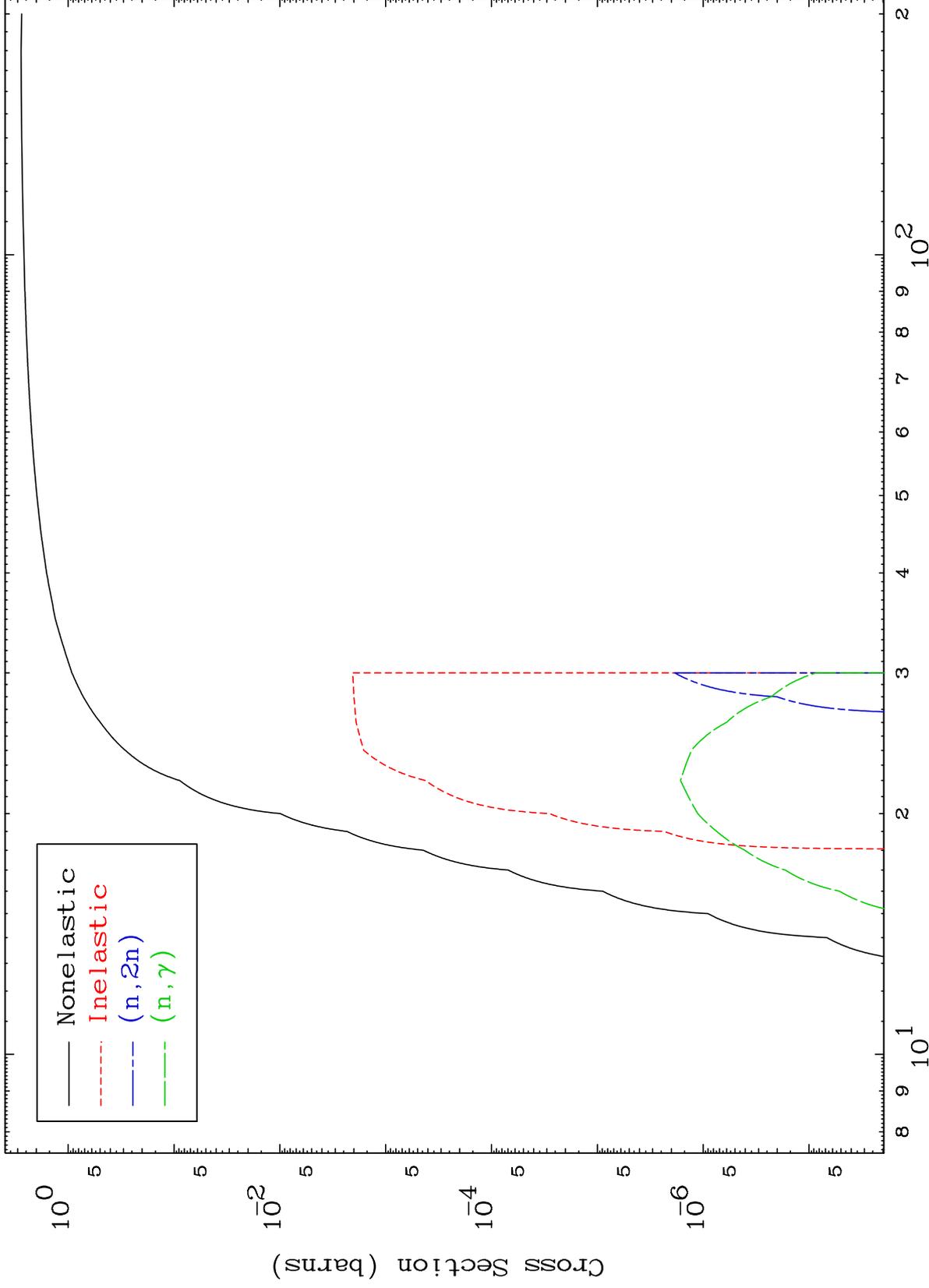
Press Mouse Button to Start

MAT 8704

0 Kelvin Cross Sections

$\alpha$  Major

87-Fr-205



87-Fr-205

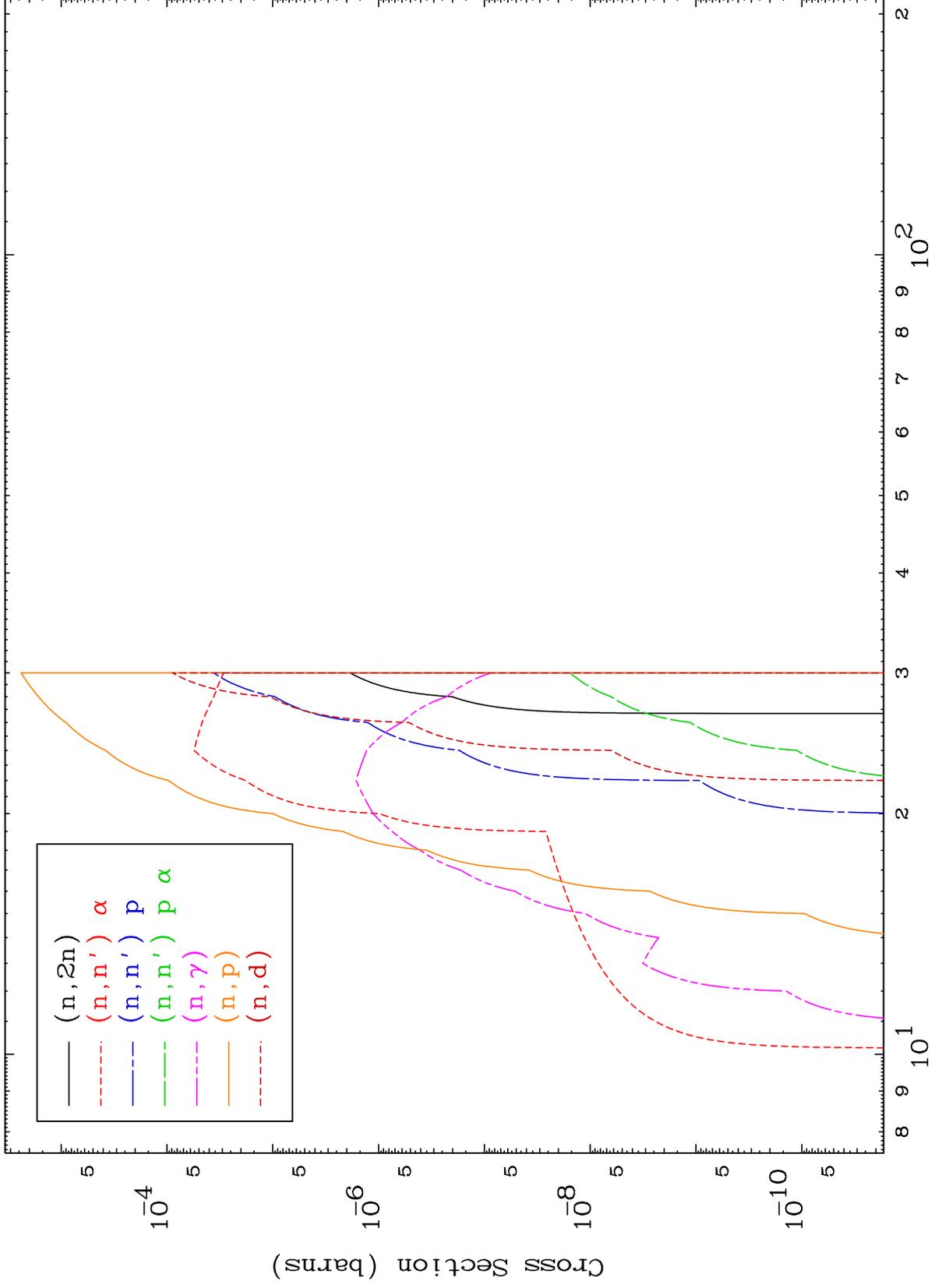
Incident Energy (MeV)

1

MAT 8704

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

87-Fr-205



2

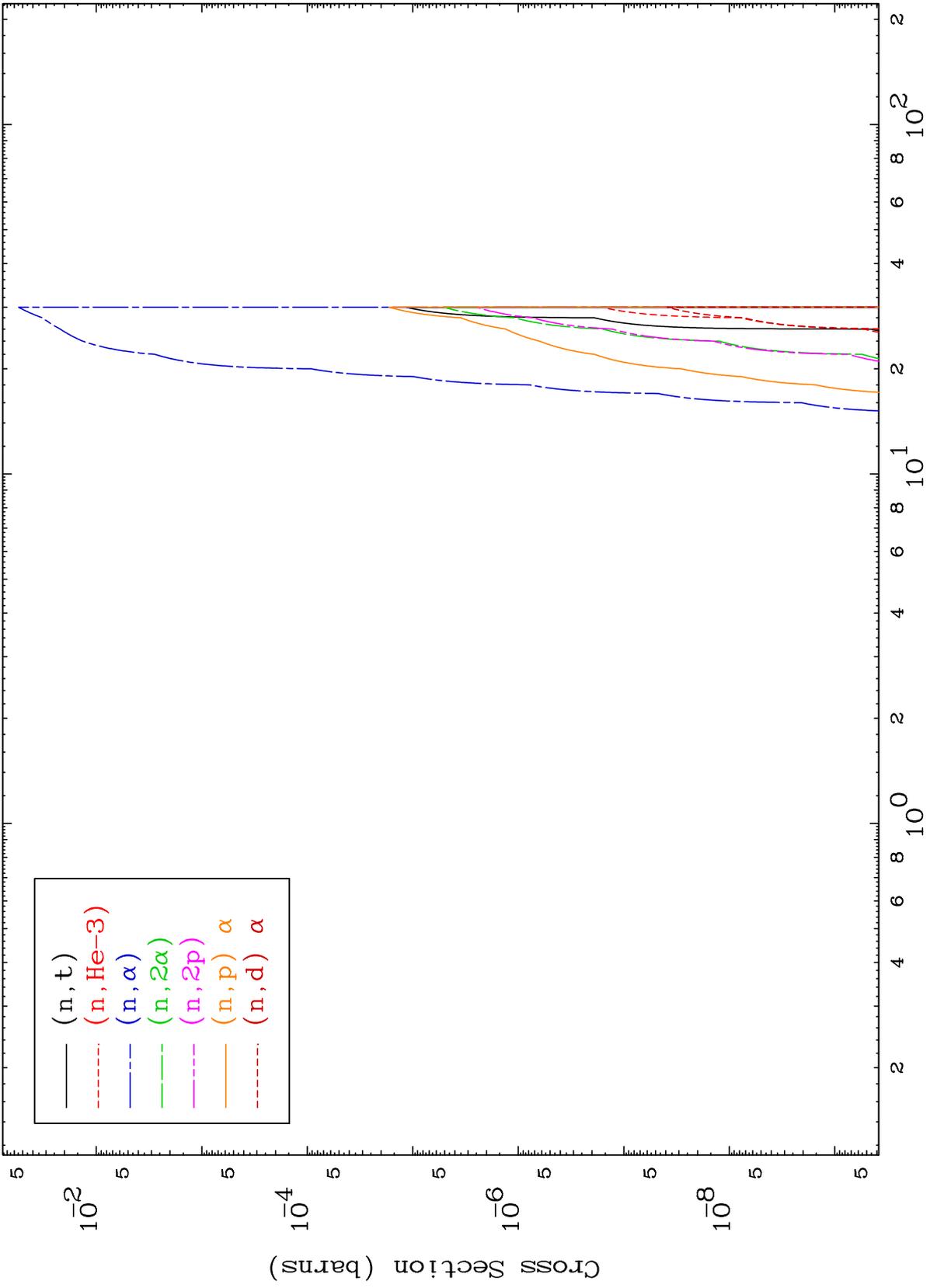
Incident Energy (MeV)

87-Fr-205

MAT 8704

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

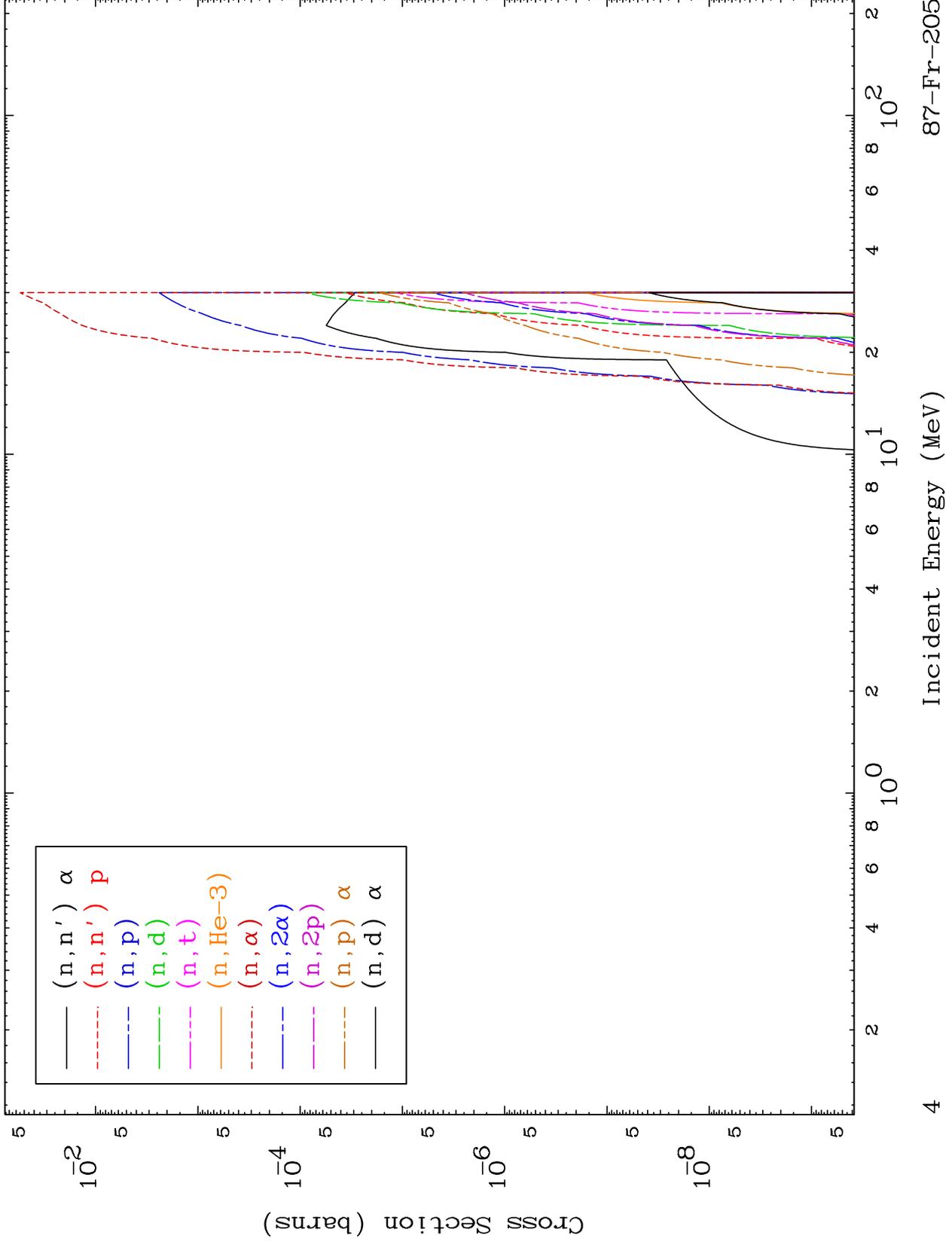
87-Fr-205



MAT 8704

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

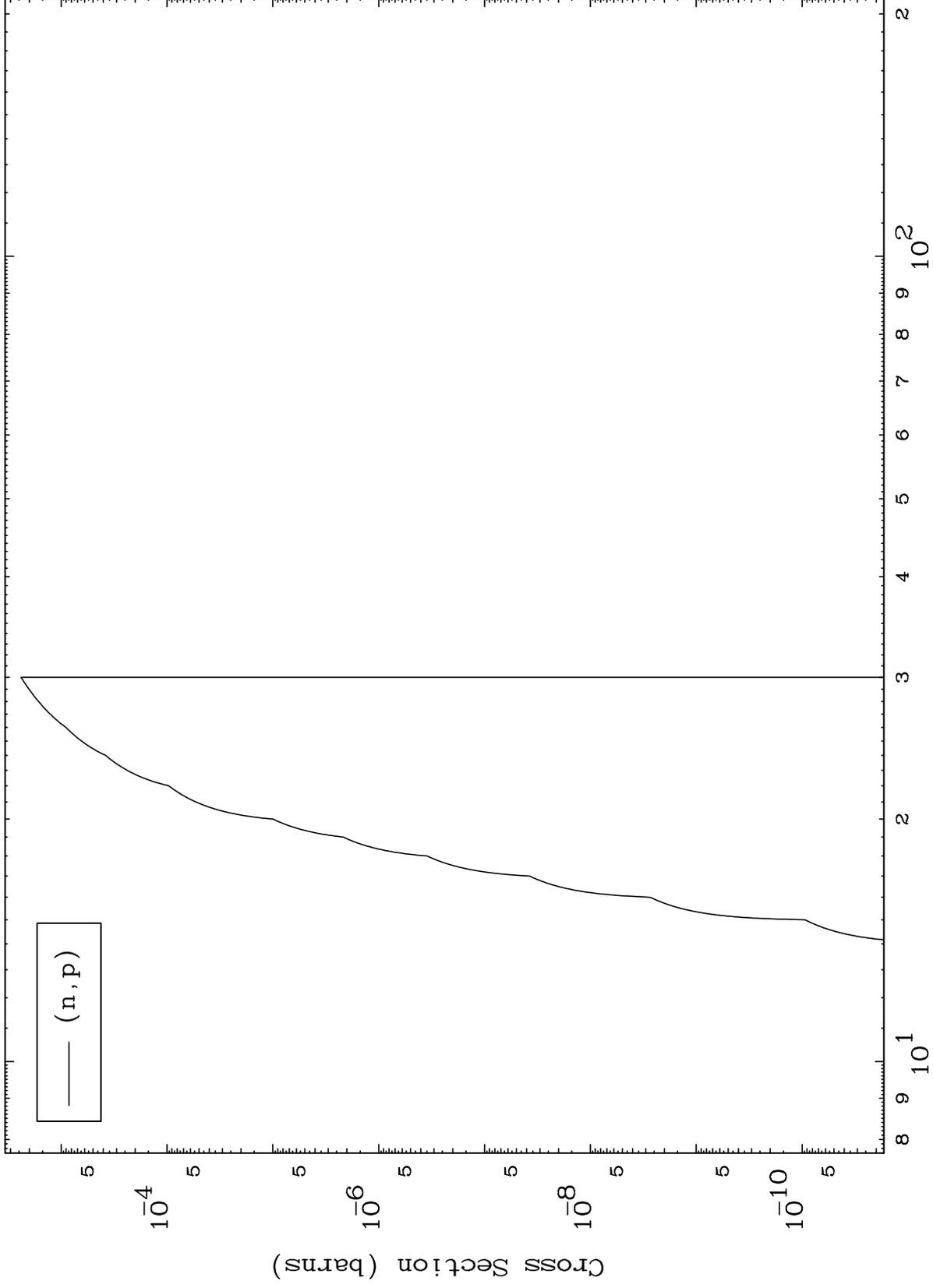
87-Fr-205



MAT 8704

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

87-Fr-205



Incident Energy (MeV)

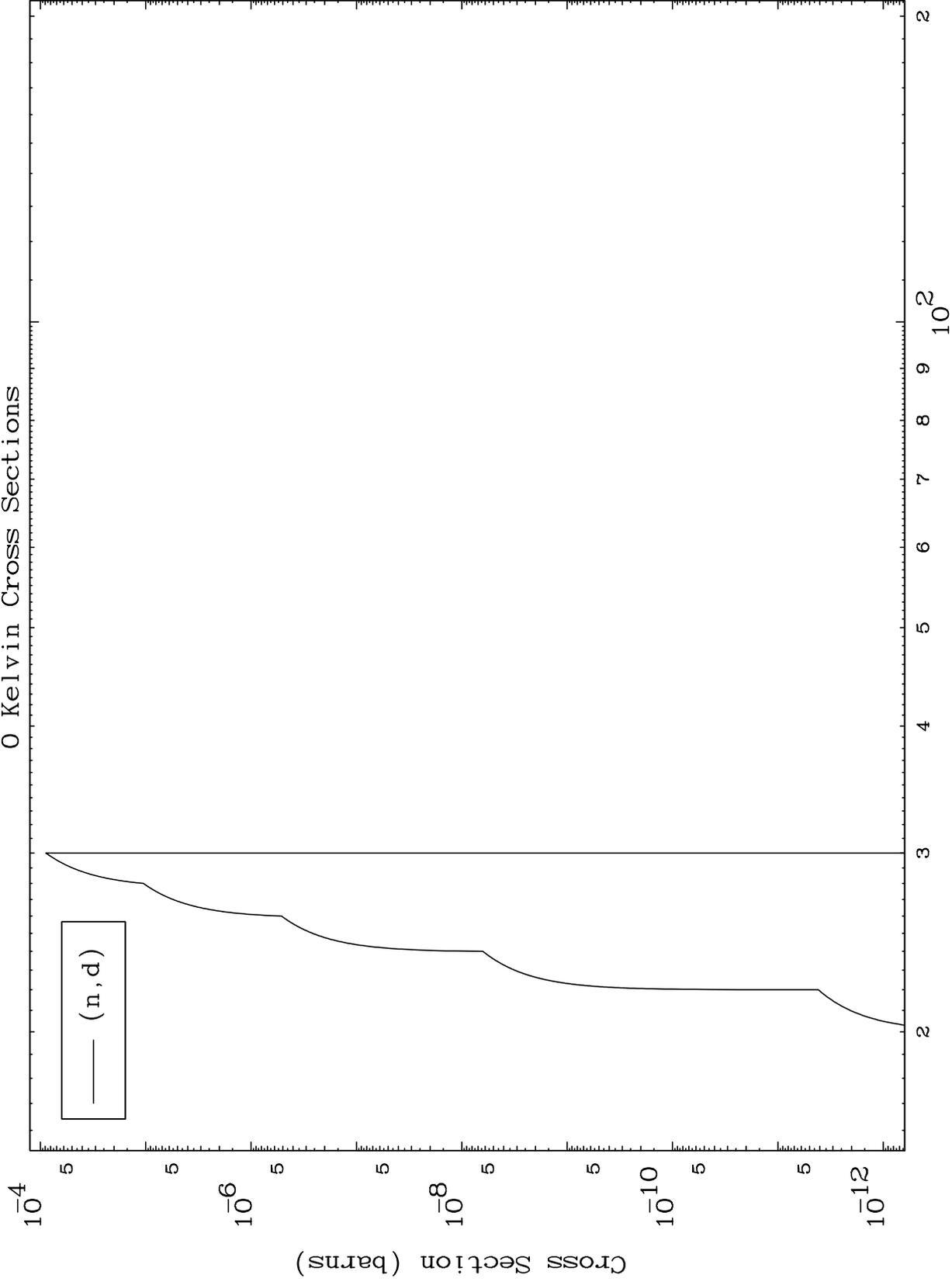
87-Fr-205

5

MAT 8704

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

87-Fr-205



6

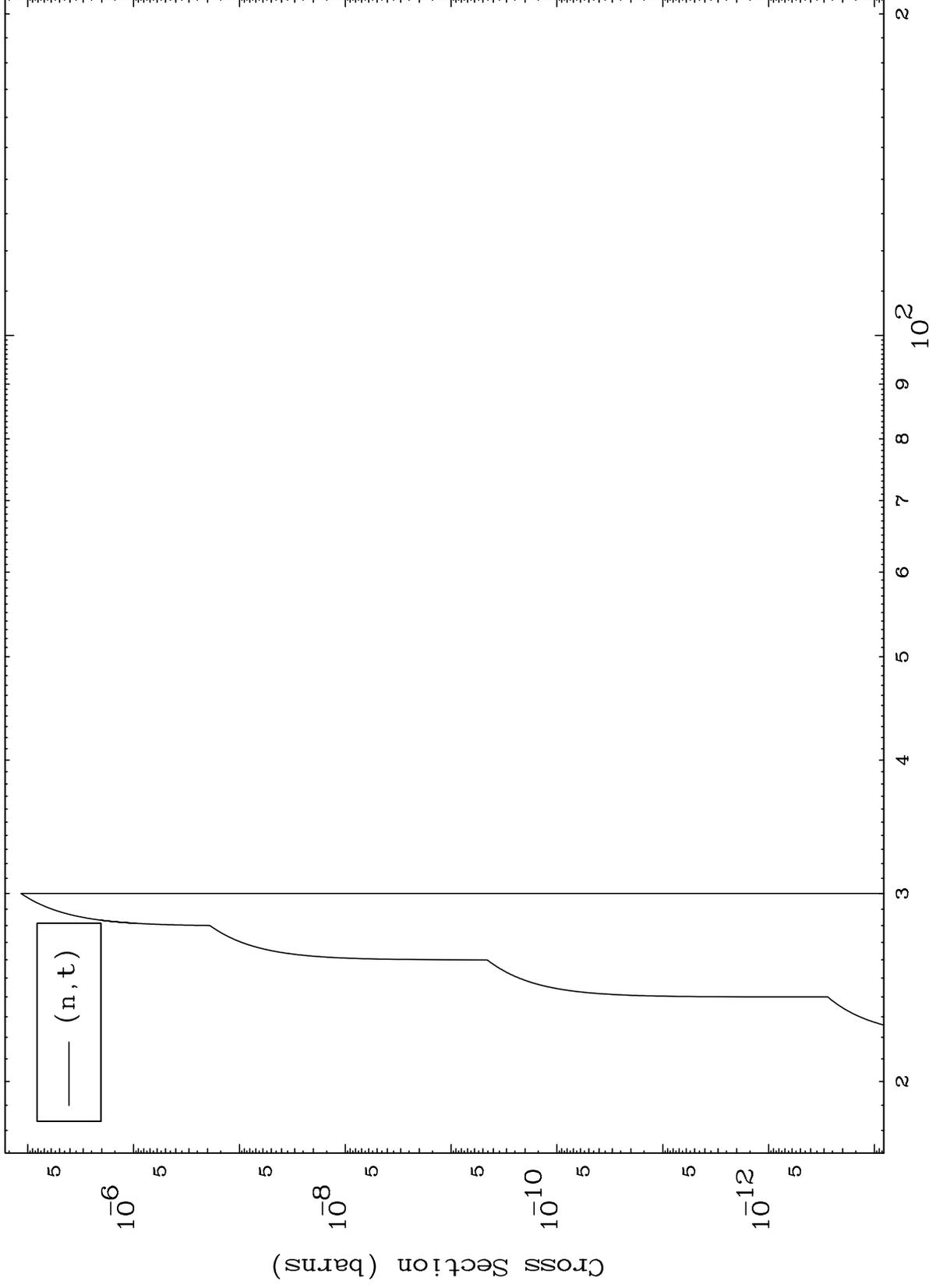
Incident Energy (MeV)

87-Fr-205

MAT 8704

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

87-Fr-205

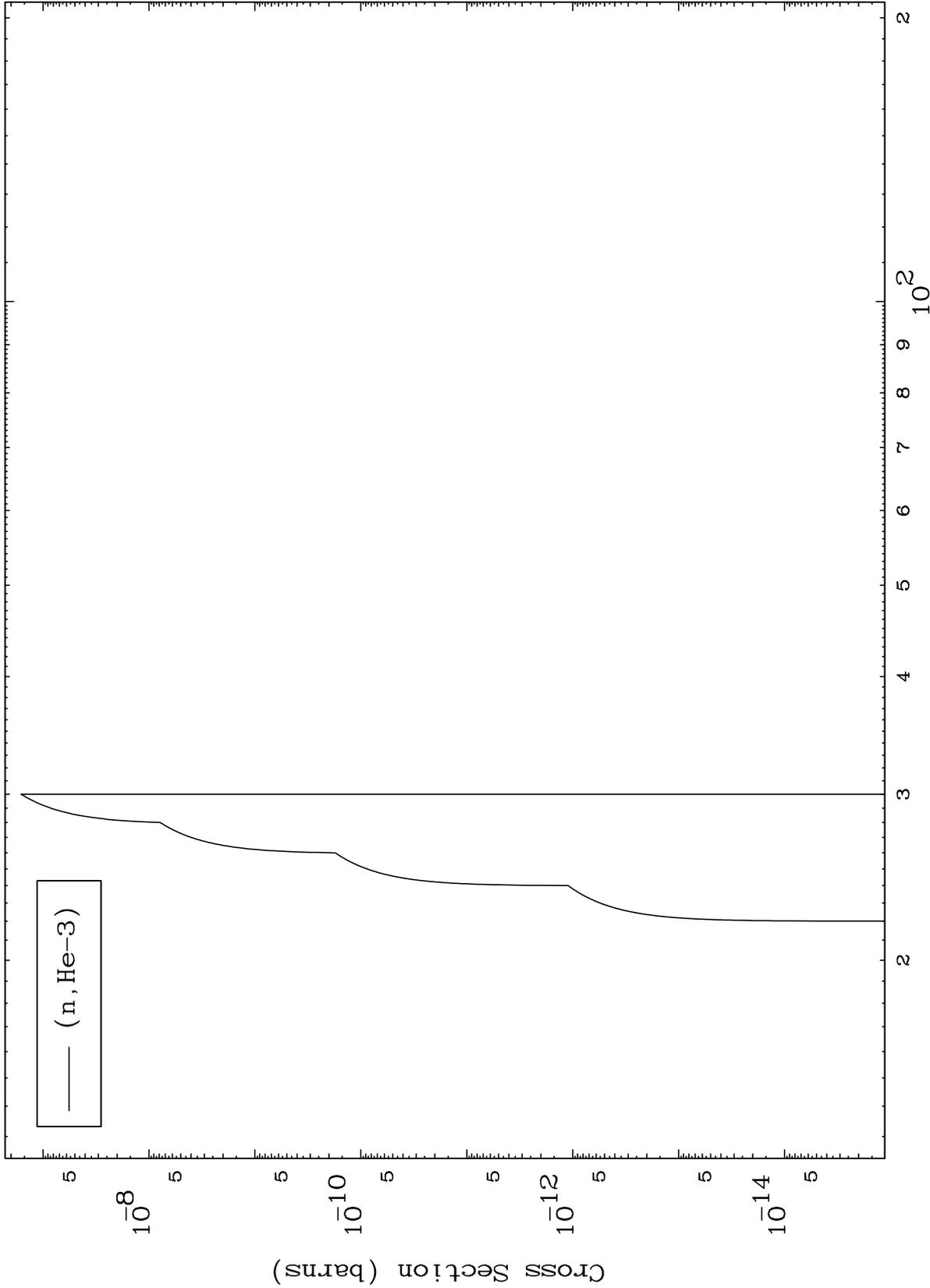


7

Incident Energy (MeV)

87-Fr-205

( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections

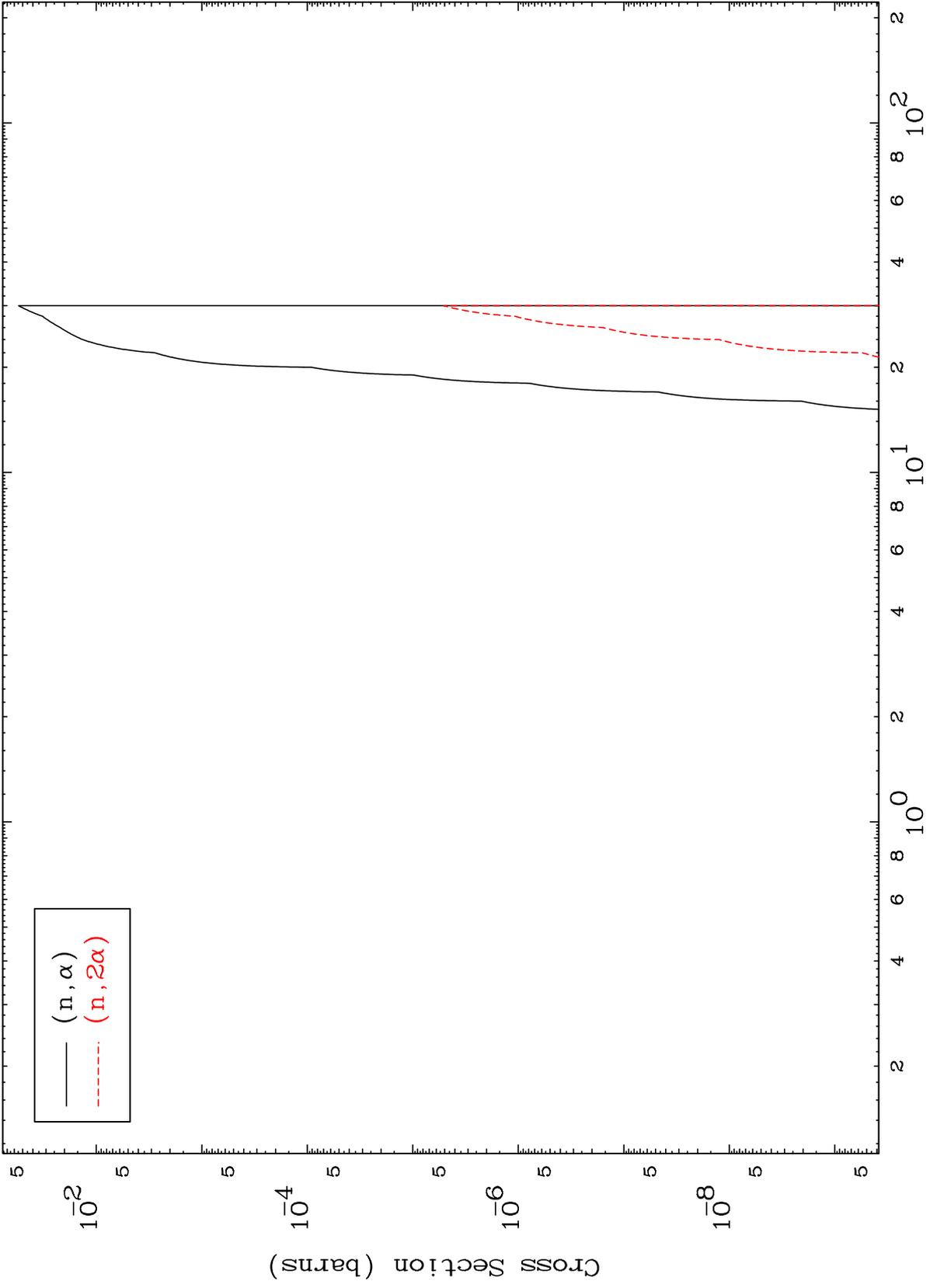


(n, He-3)

MAT 8704

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

87-Fr-205

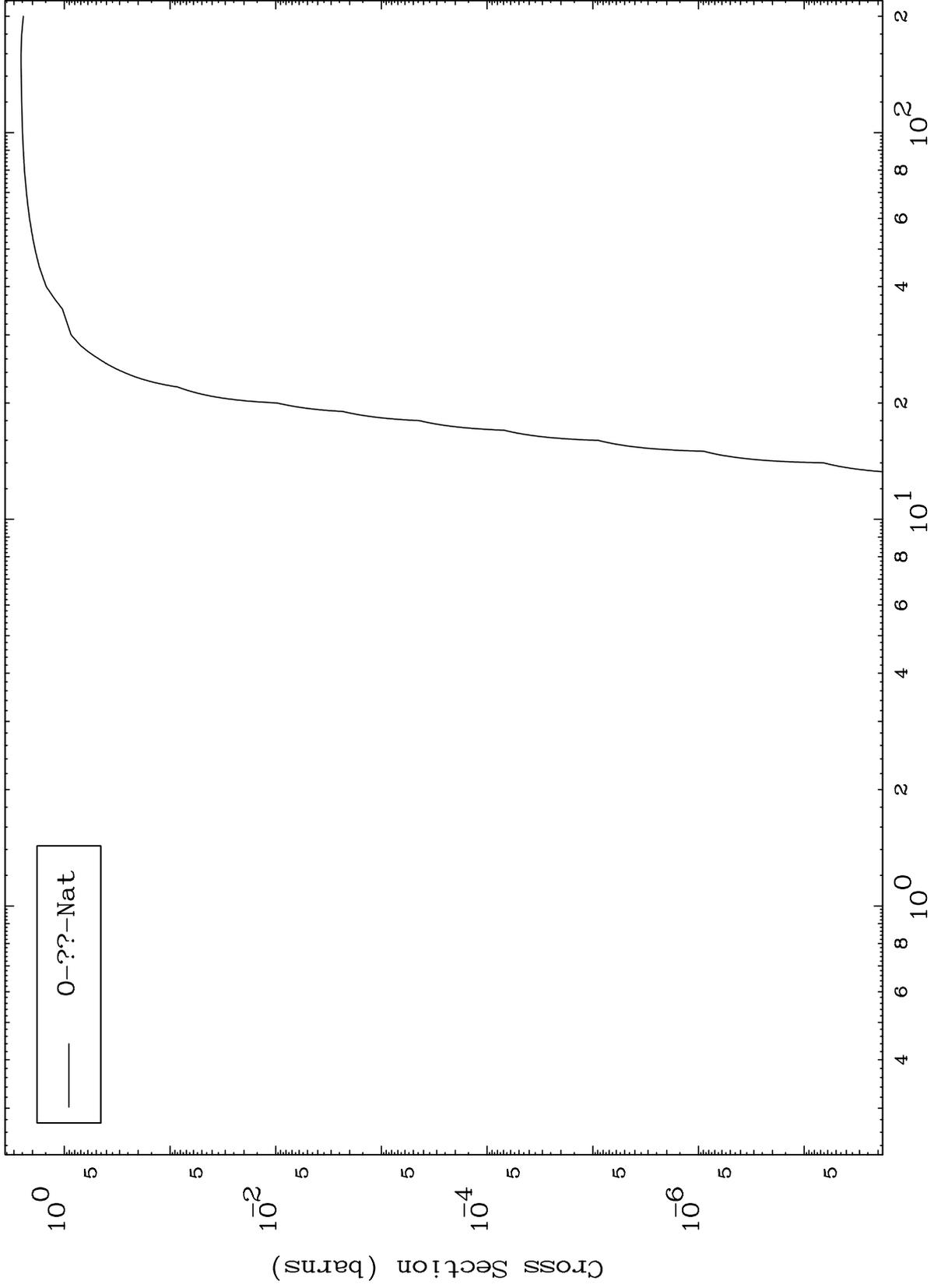


MAT 8704

Fission

87-Fr-205

Radionuclide Production Cross Section



10

Incident Energy (MeV)

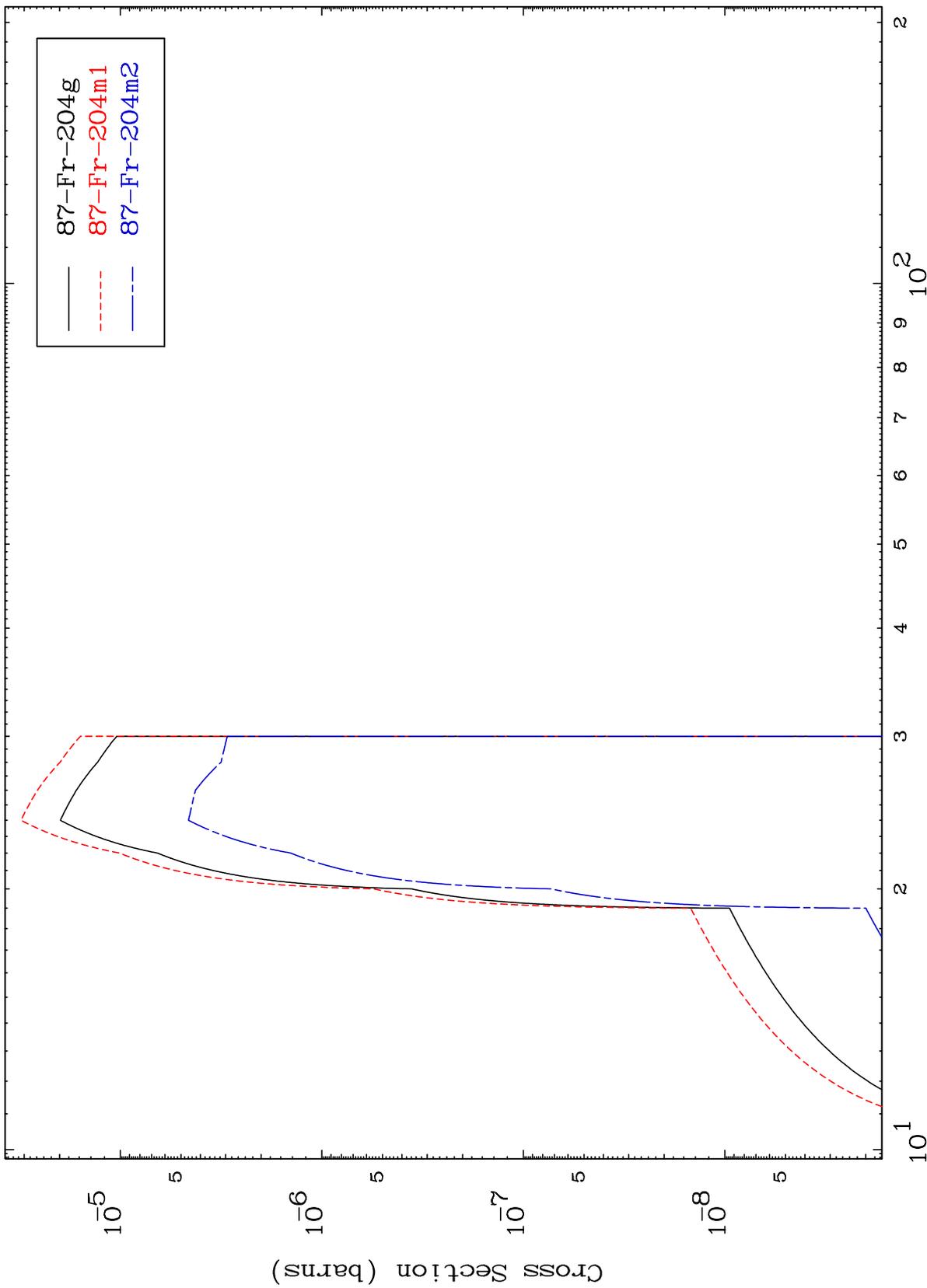
87-Fr-205

MAT 8704

$(n, n') \alpha$

87-Fr-205

Radionuclide Production Cross Section



Incident Energy (MeV)

87-Fr-205

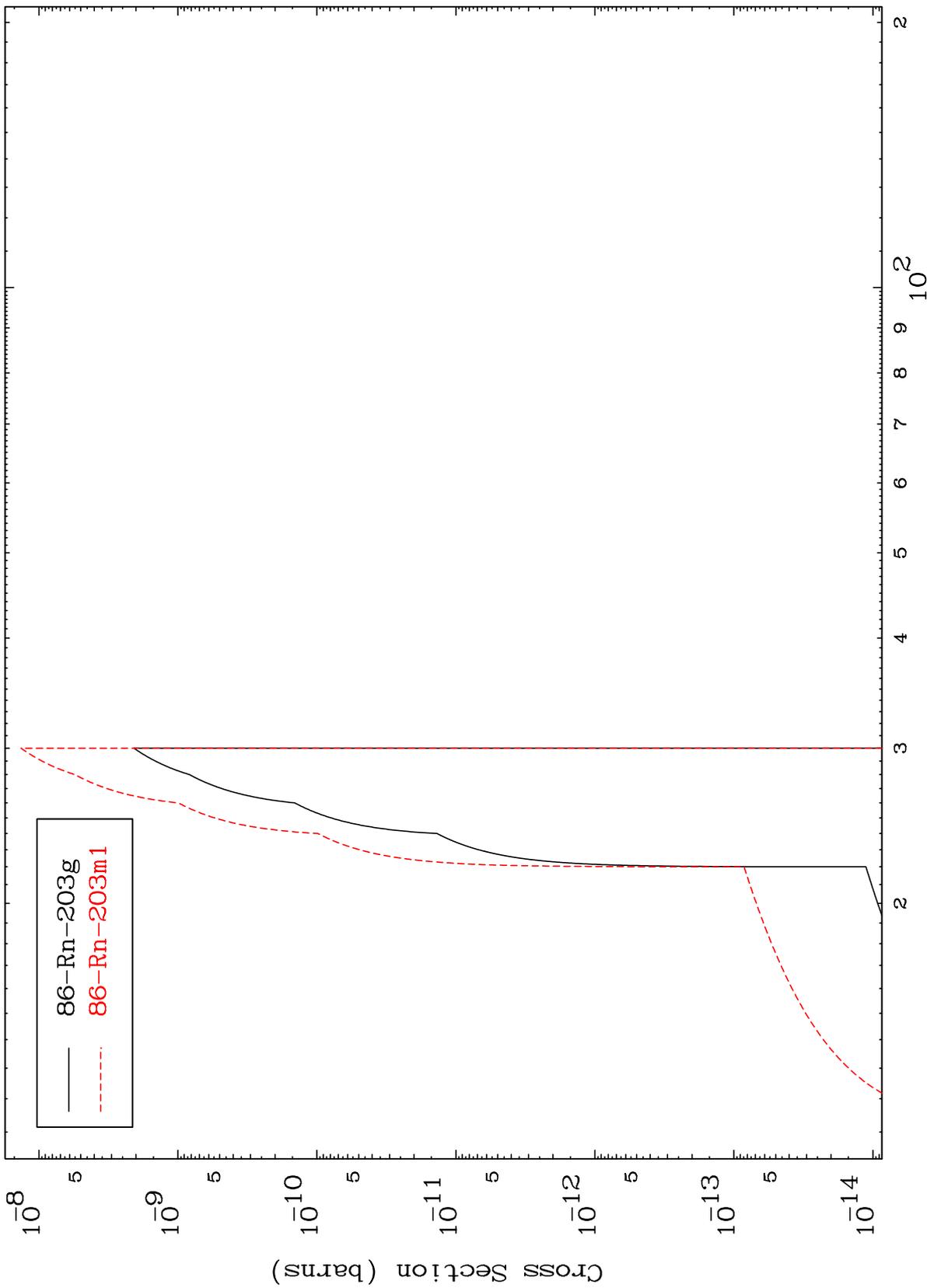
11

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(n,n') p  $\alpha$

87-Fr-205

Radionuclide Production Cross Section



12

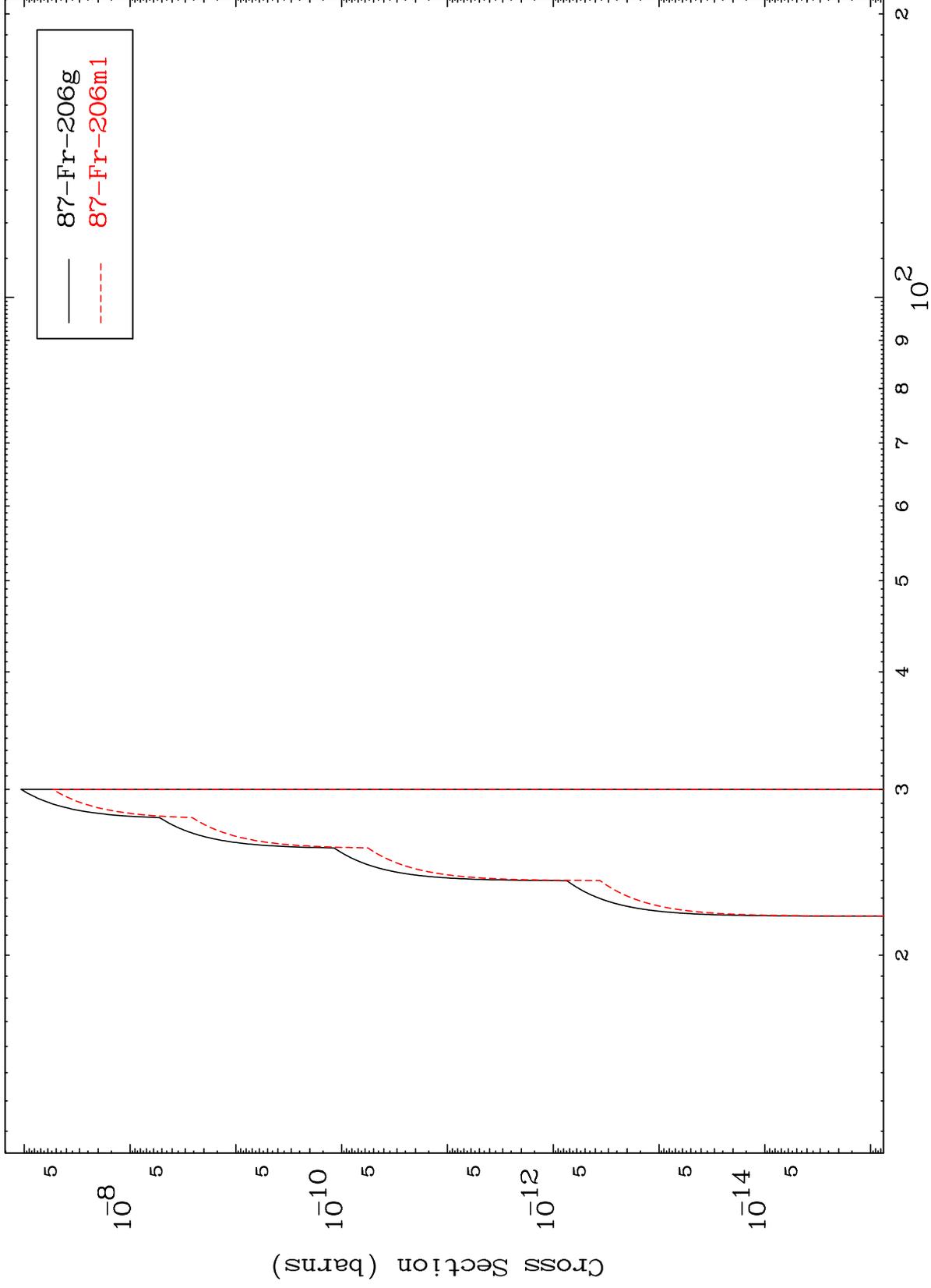
Incident Energy (MeV)

87-Fr-205

MAT 8704

87-Fr-205

(n,He-3)  
Radionuclide Production Cross Section



13

87-Fr-205

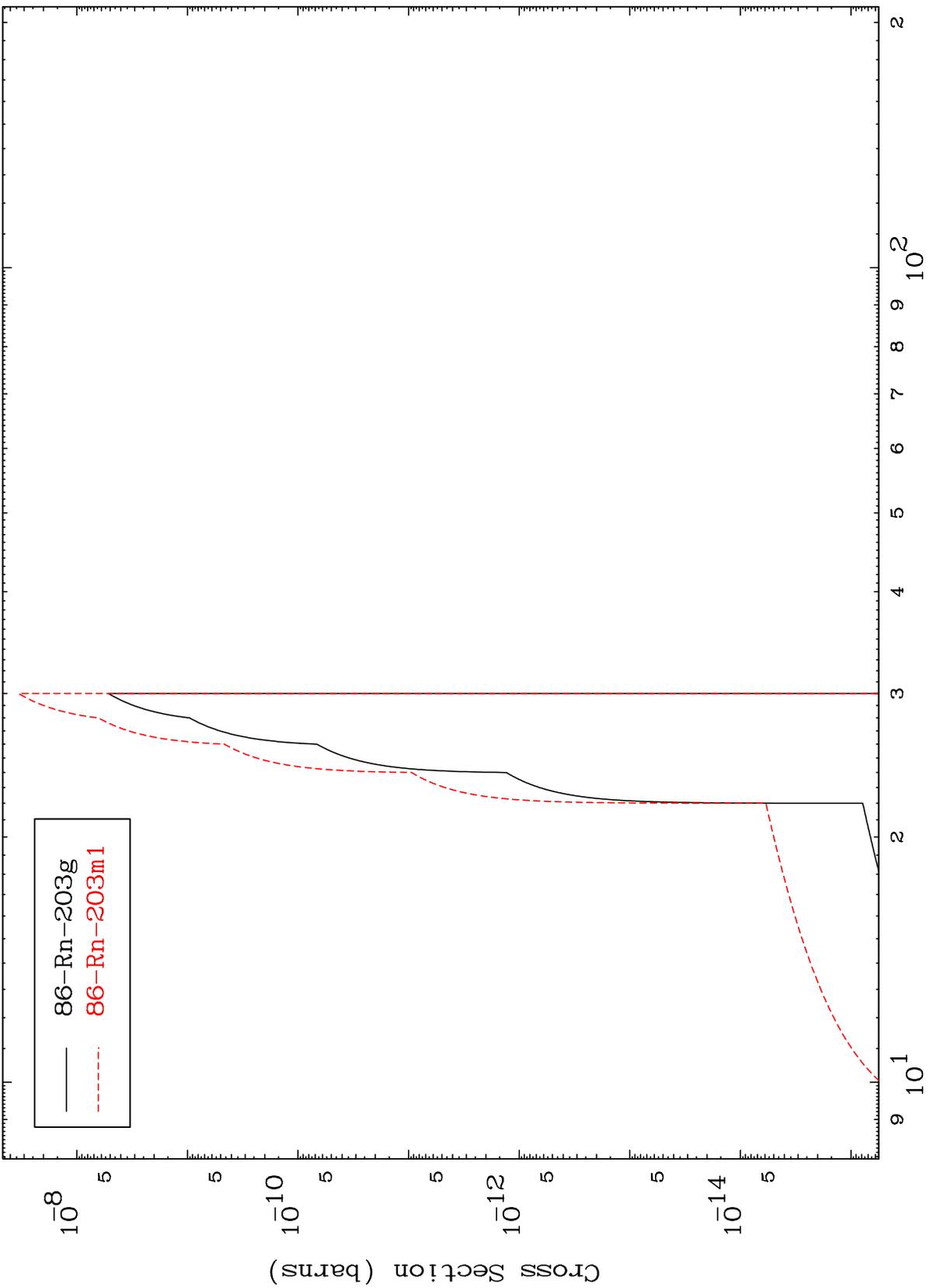
Incident Energy (MeV)

MAT 8704

(n,d)  $\alpha$

87-Fr-205

Radionuclide Production Cross Section



86-Rn-203g  
86-Rn-203m1

14

Incident Energy (MeV)

87-Fr-205