

Program EVALPLOT
(Version 2021-1)

by

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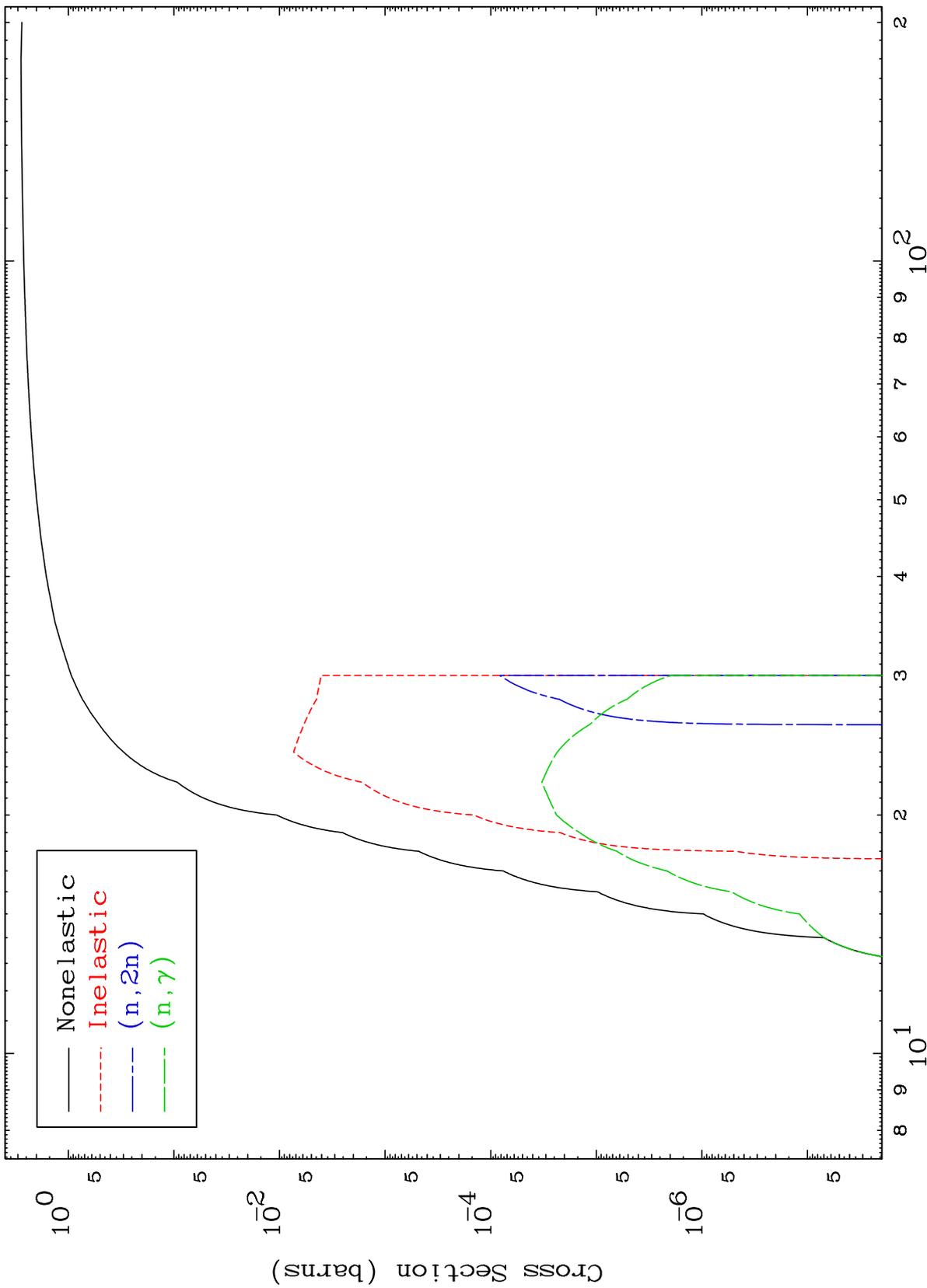
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Press Mouse Button to Start

MAT 8710

0 Kelvin Cross Sections

87-Fr-207



87-Fr-207

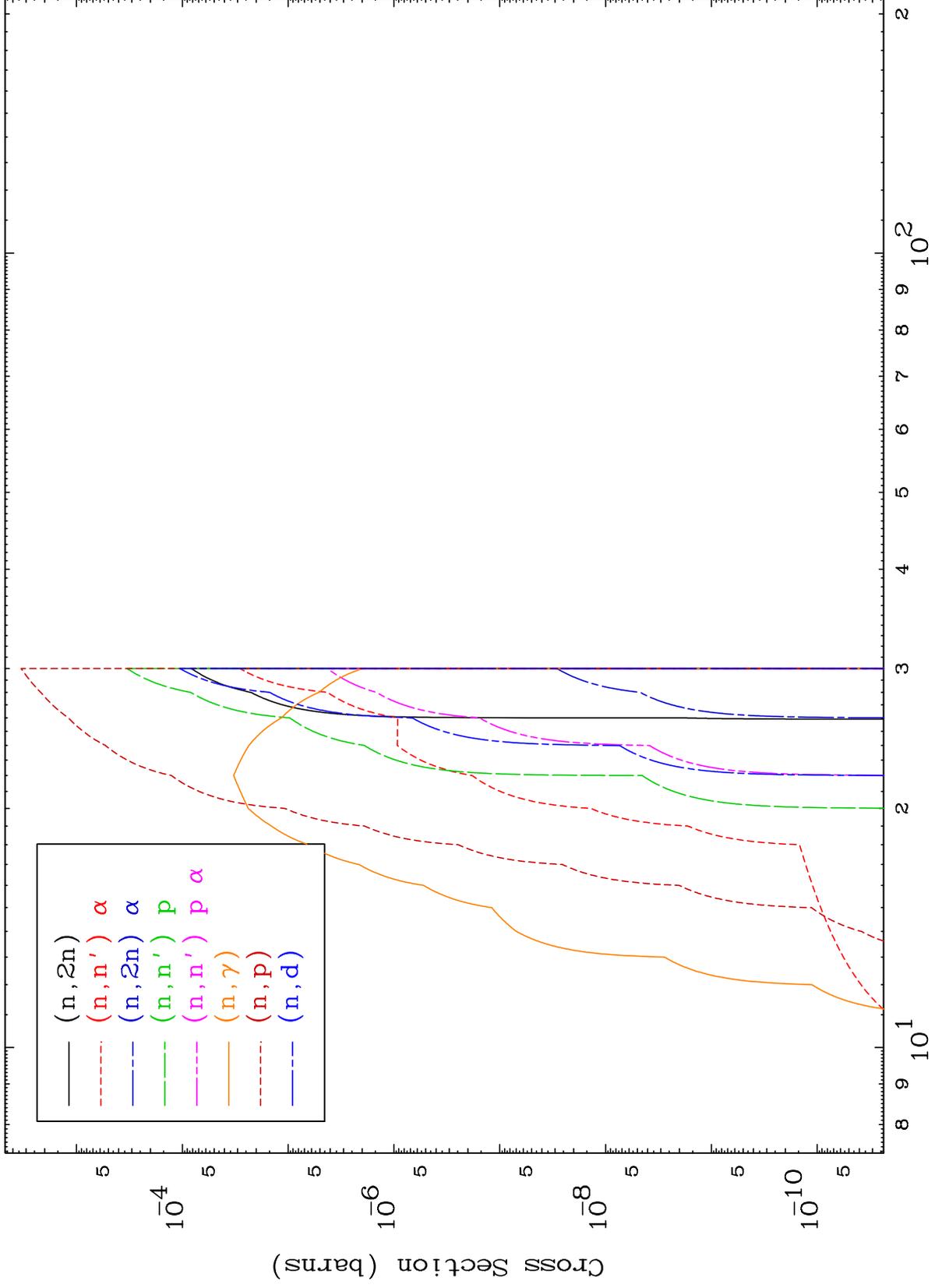
Incident Energy (MeV)

1

MAT 8710

α Neutron Absorption
0 Kelvin Cross Sections

87-Fr-207



Incident Energy (MeV)

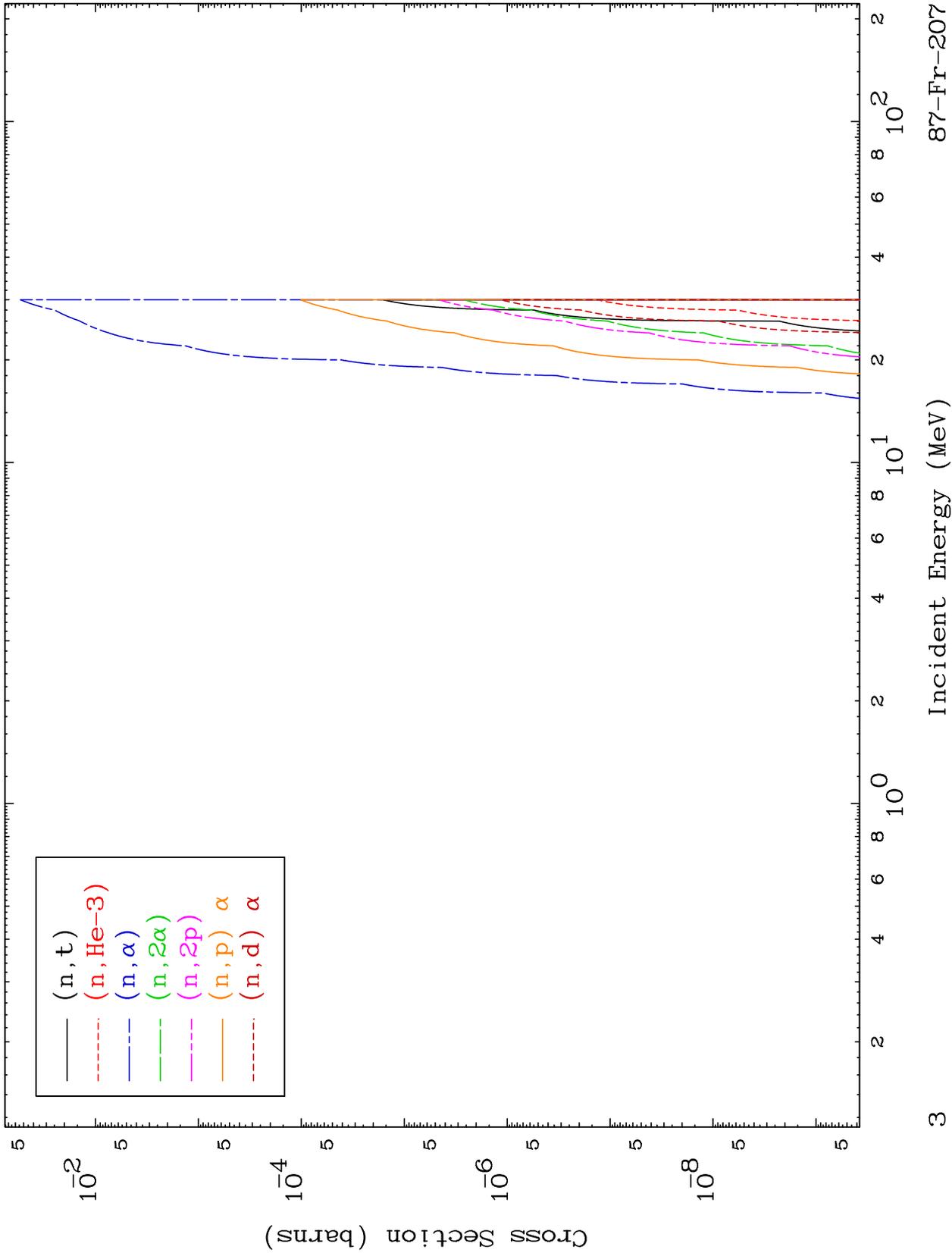
87-Fr-207

2

MAT 8710

α Neutron Absorption
0 Kelvin Cross Sections

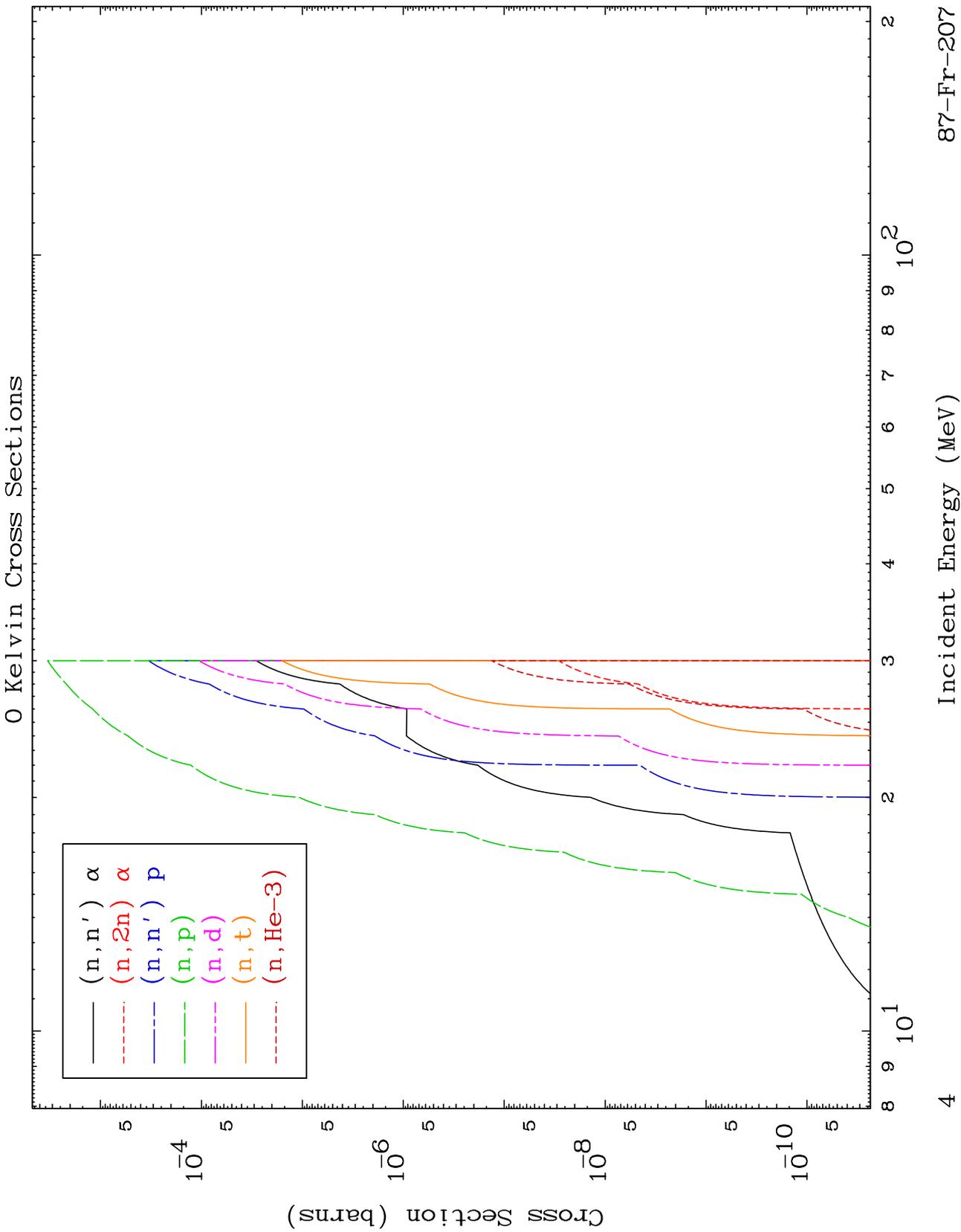
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α Charged Particle

87-Fr-207



4

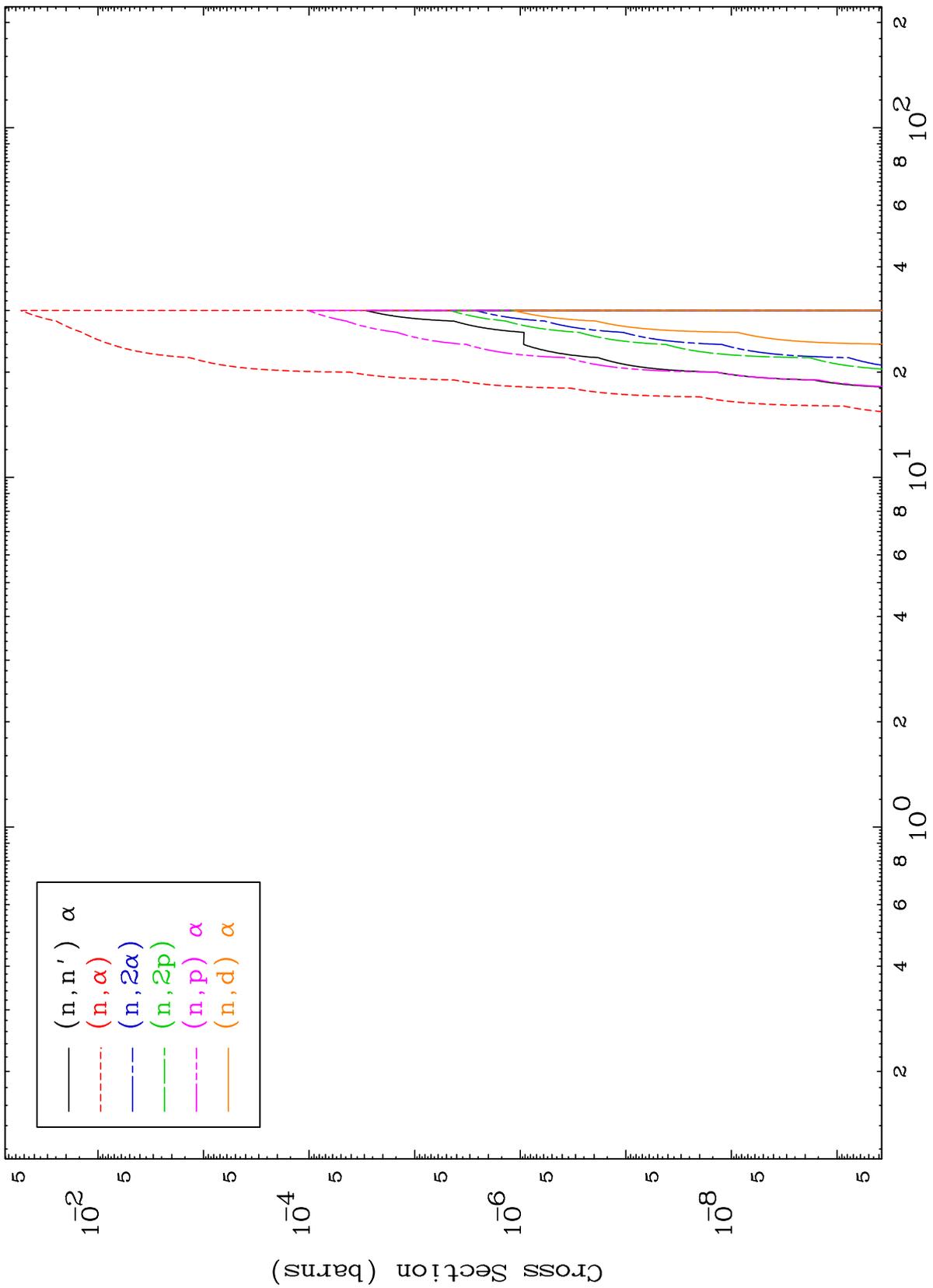
Incident Energy (MeV)

87-Fr-207

MAT 8710

α Charged Particle
0 Kelvin Cross Sections

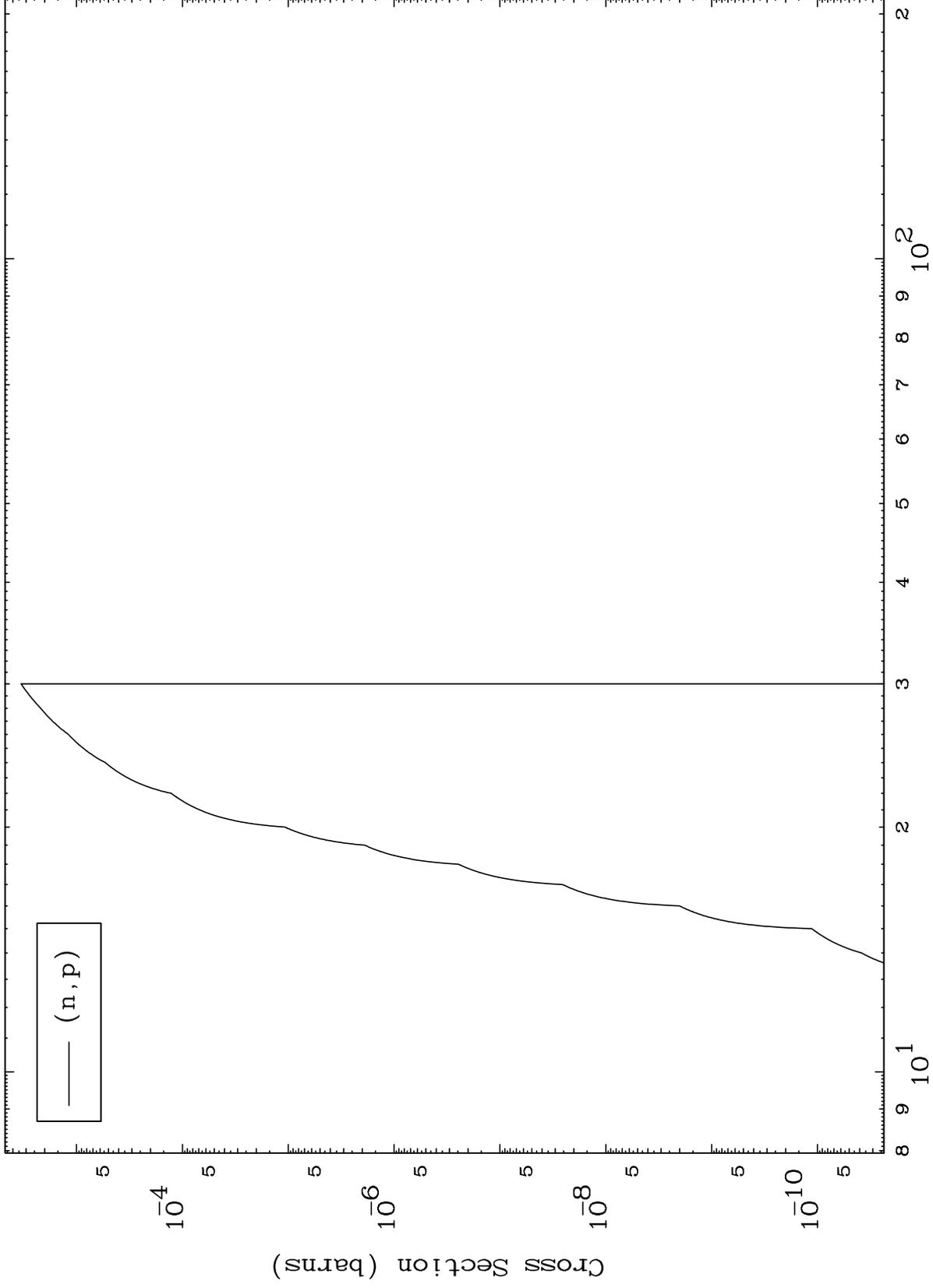
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(α, p) Levels
0 Kelvin Cross Sections

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Incident Energy (MeV)

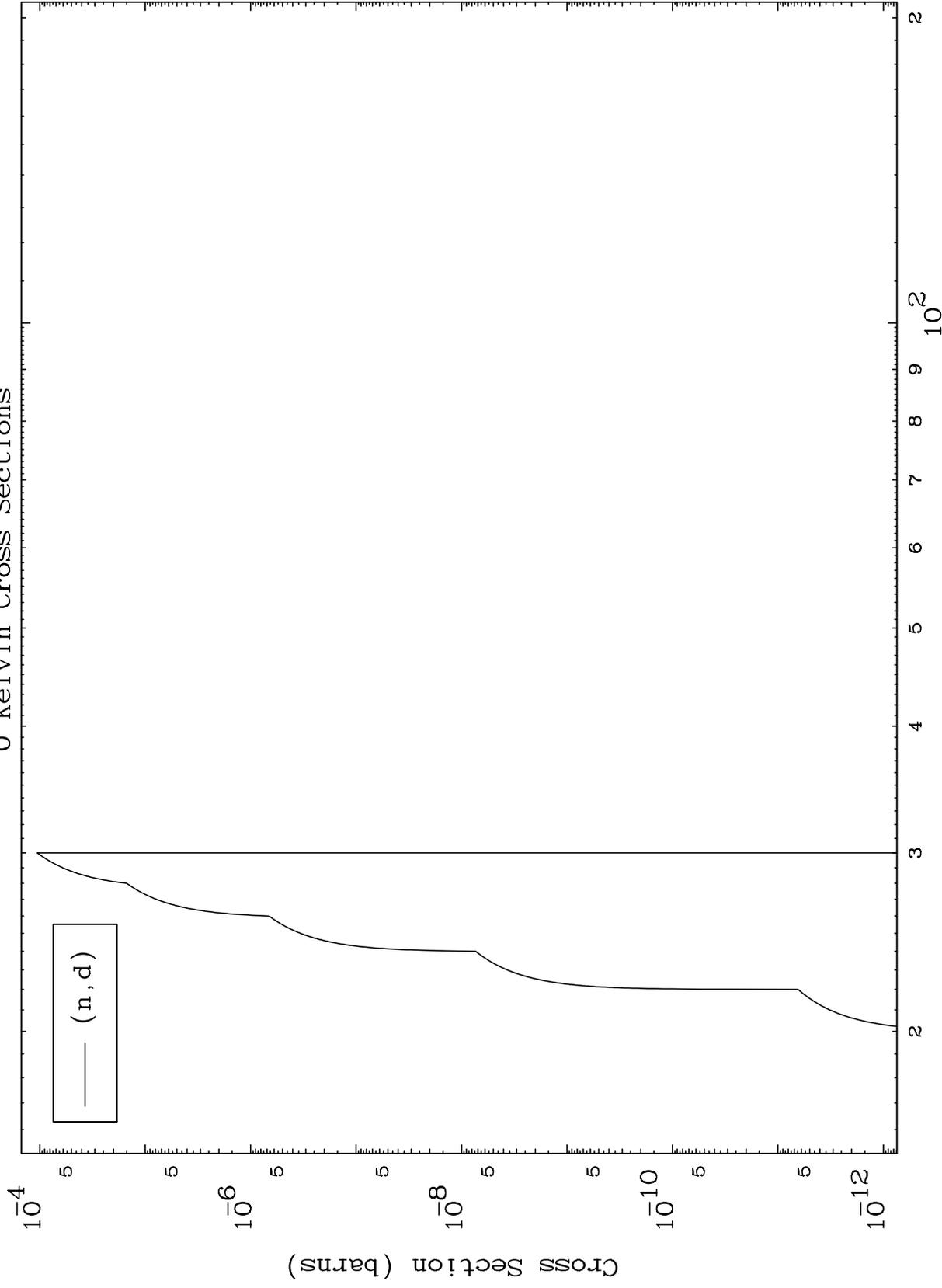
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(α, d) Levels
0 Kelvin Cross Sections

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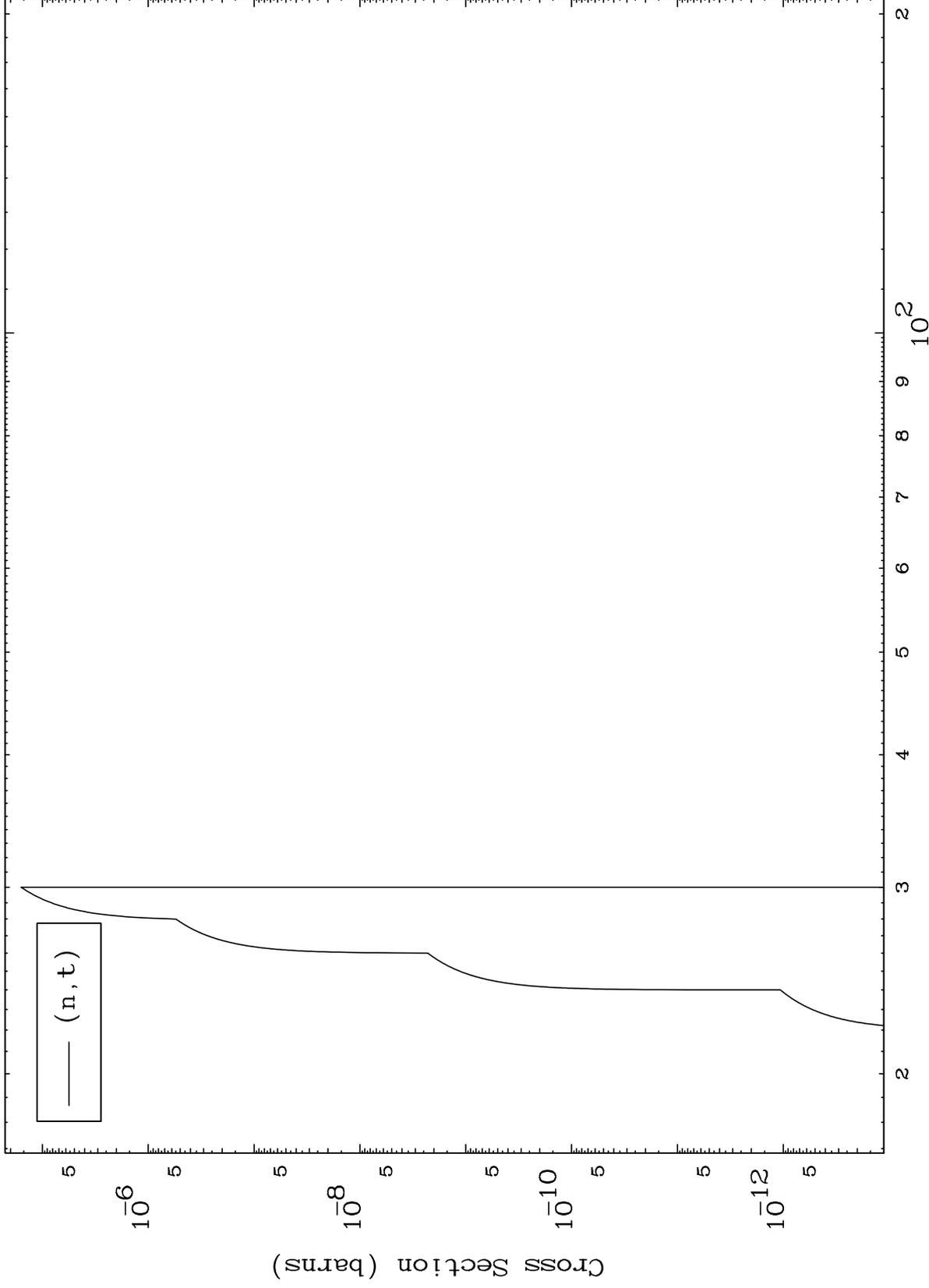
Incident Energy (MeV)

87-Fr-207

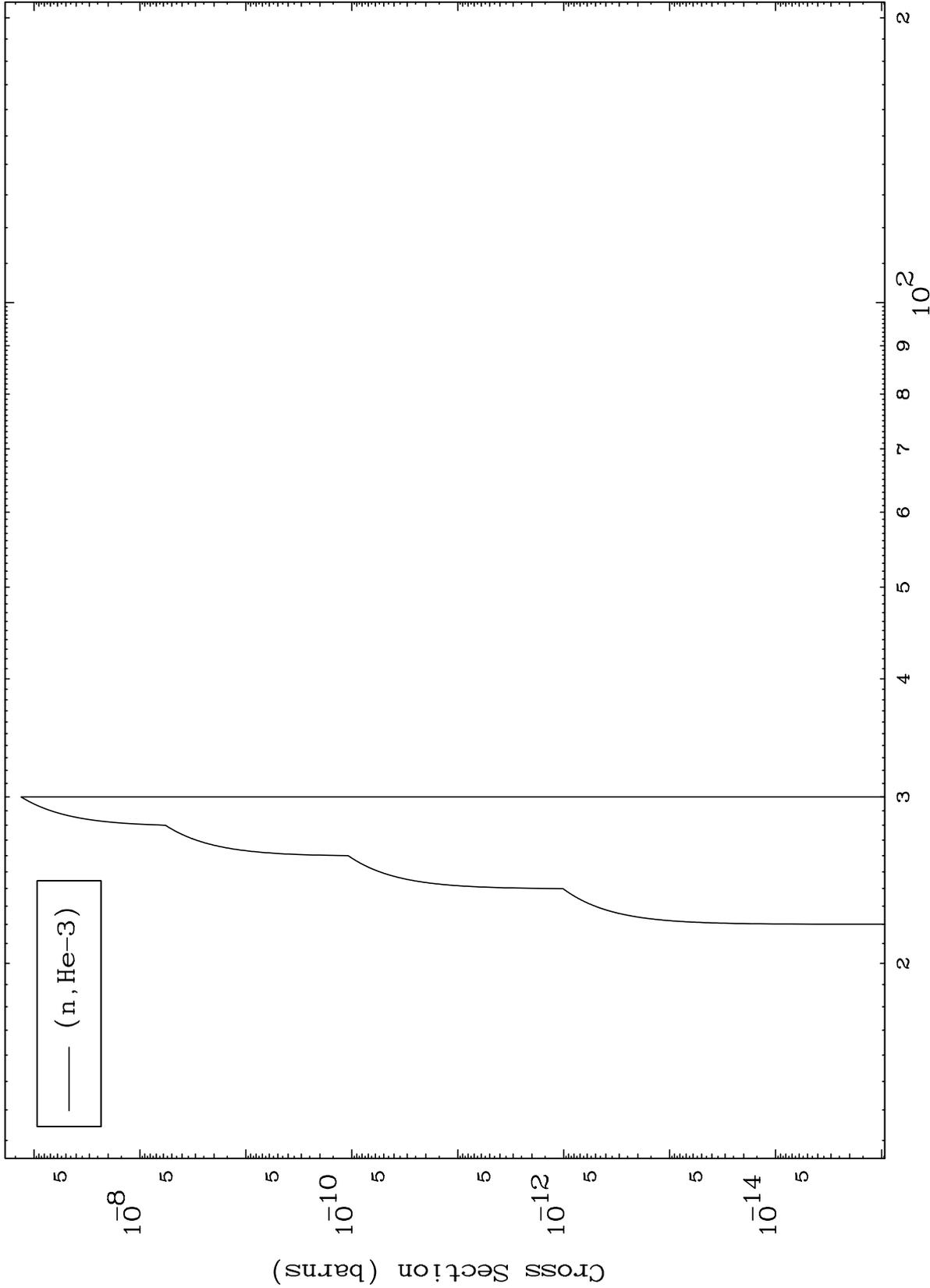
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(α, t) Levels
0 Kelvin Cross Sections

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($\alpha, \text{He3}$) Levels
0 Kelvin Cross Sections

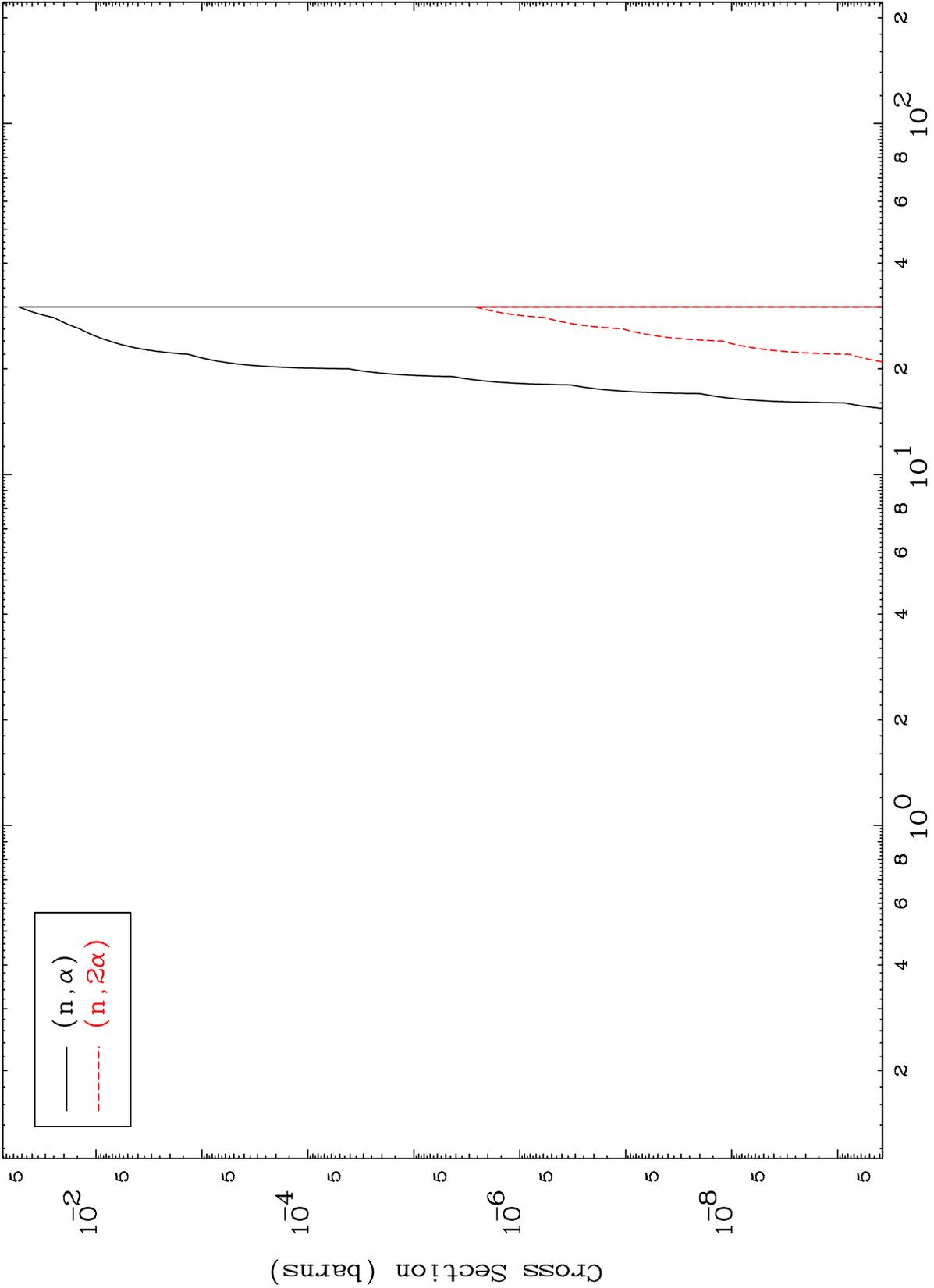


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(α, α) Levels

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0 Kelvin Cross Sections



— (n, α)
- - - ($n, 2\alpha$)

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Incident Energy (MeV)

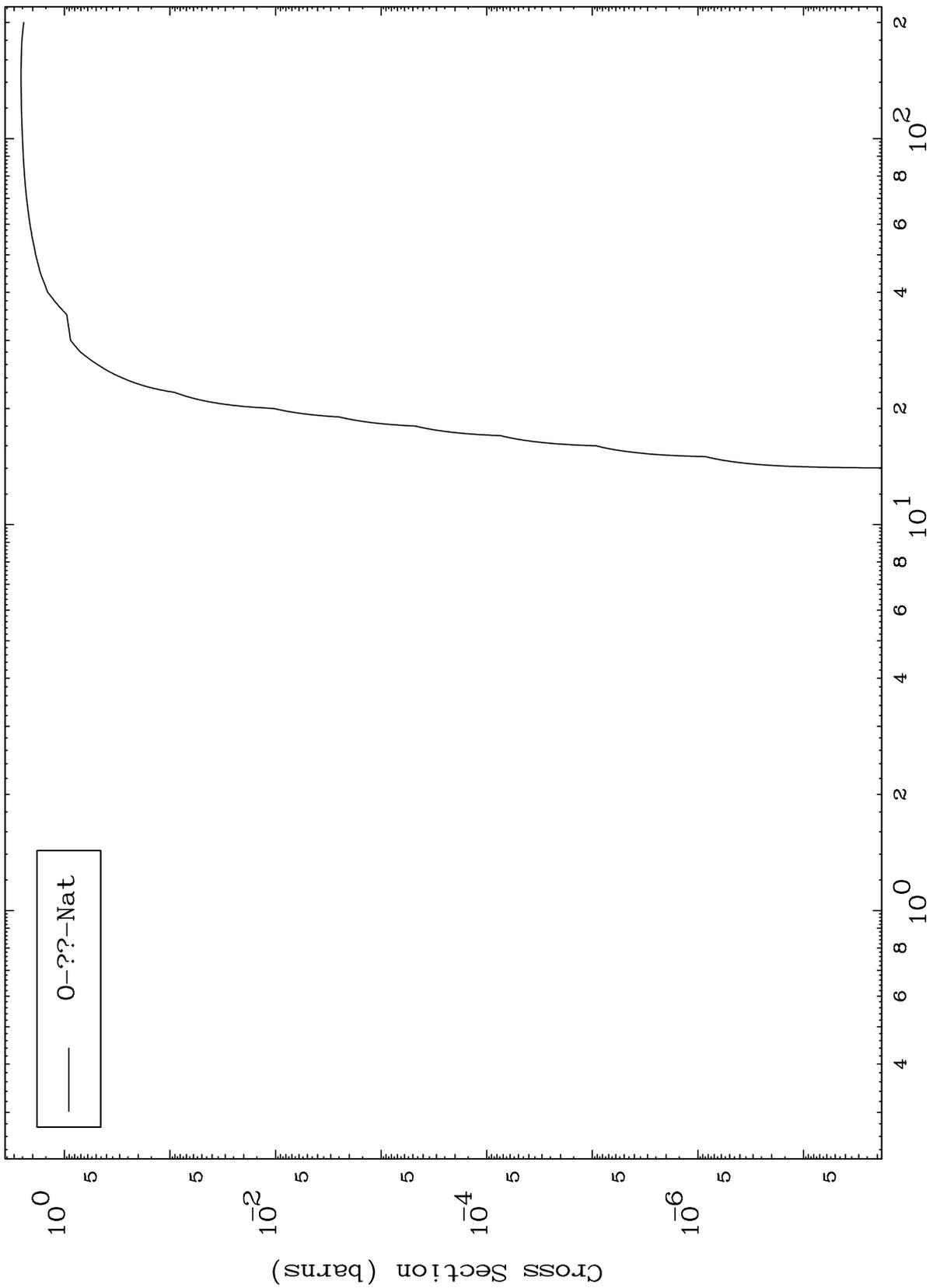
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Fission

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Radionuclide Production Cross Section

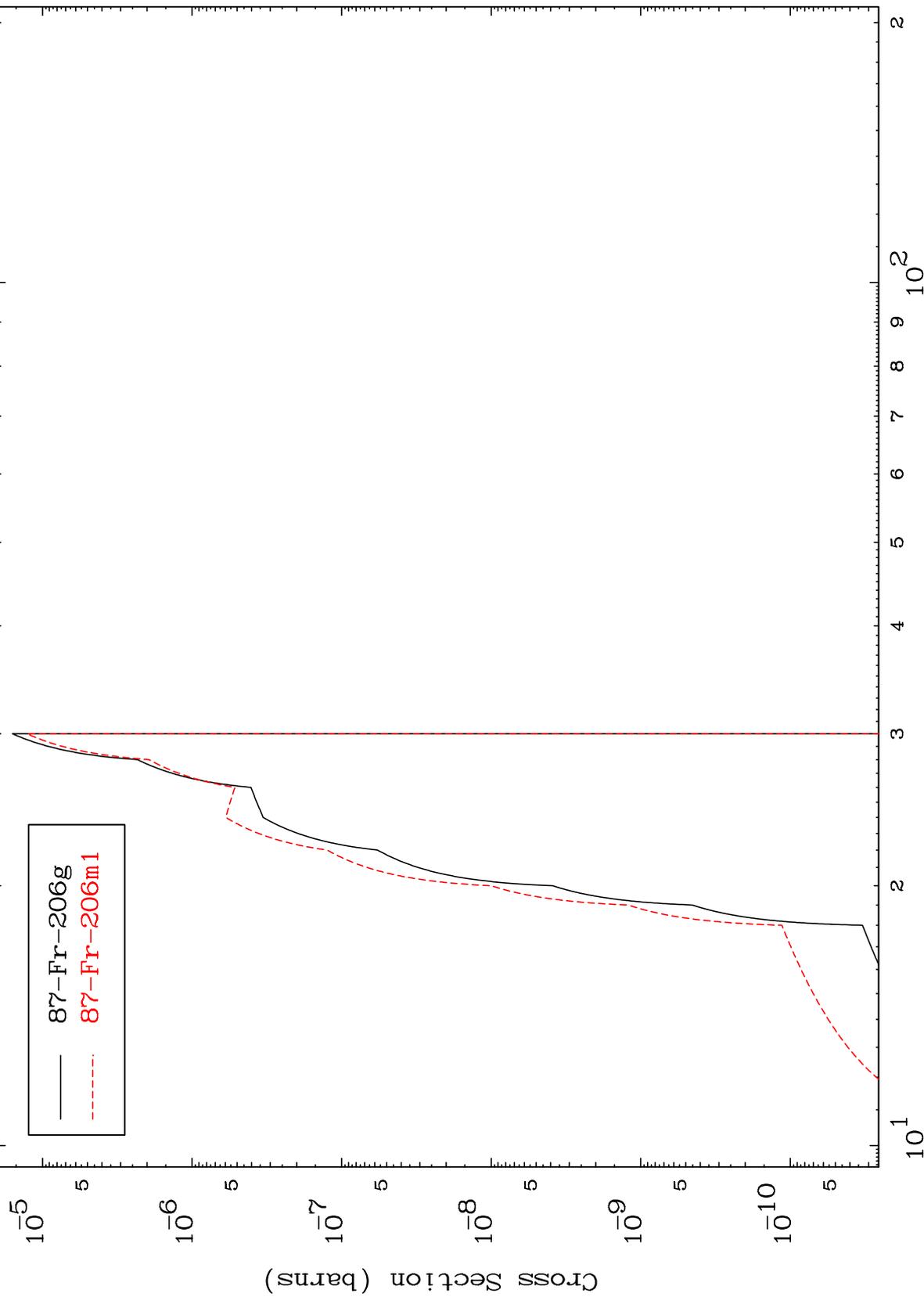


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$(n, n') \alpha$

87-Fr-207

Radionuclide Production Cross Section



Incident Energy (MeV)

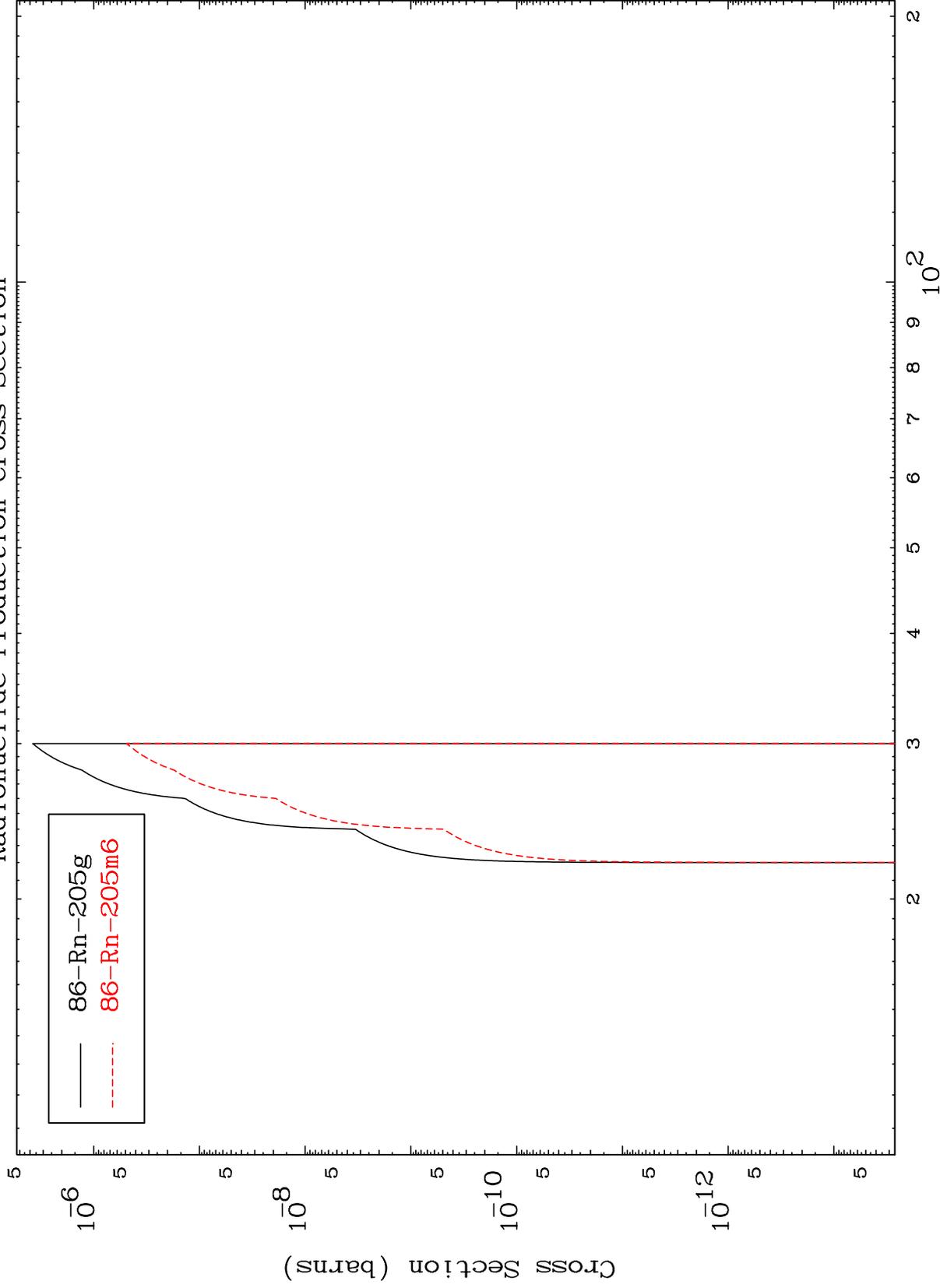
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(n,n') p α

Radionuclide Production Cross Section



Incident Energy (MeV)

87-Fr-207

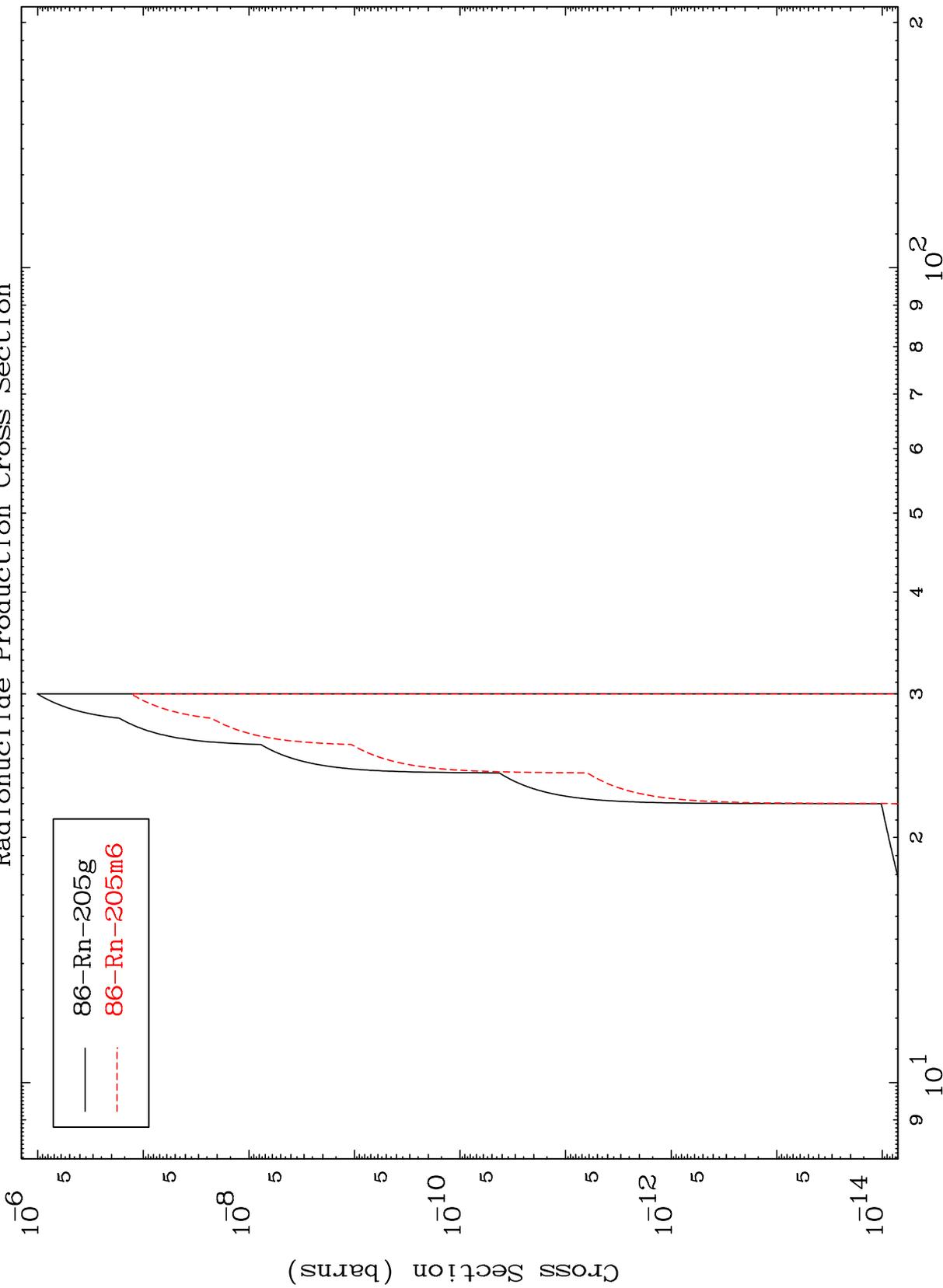
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(n,d) α

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Radionuclide Production Cross Section



86-Rn-205g
86-Rn-205m6

Incident Energy (MeV)

87-Fr-207

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