

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

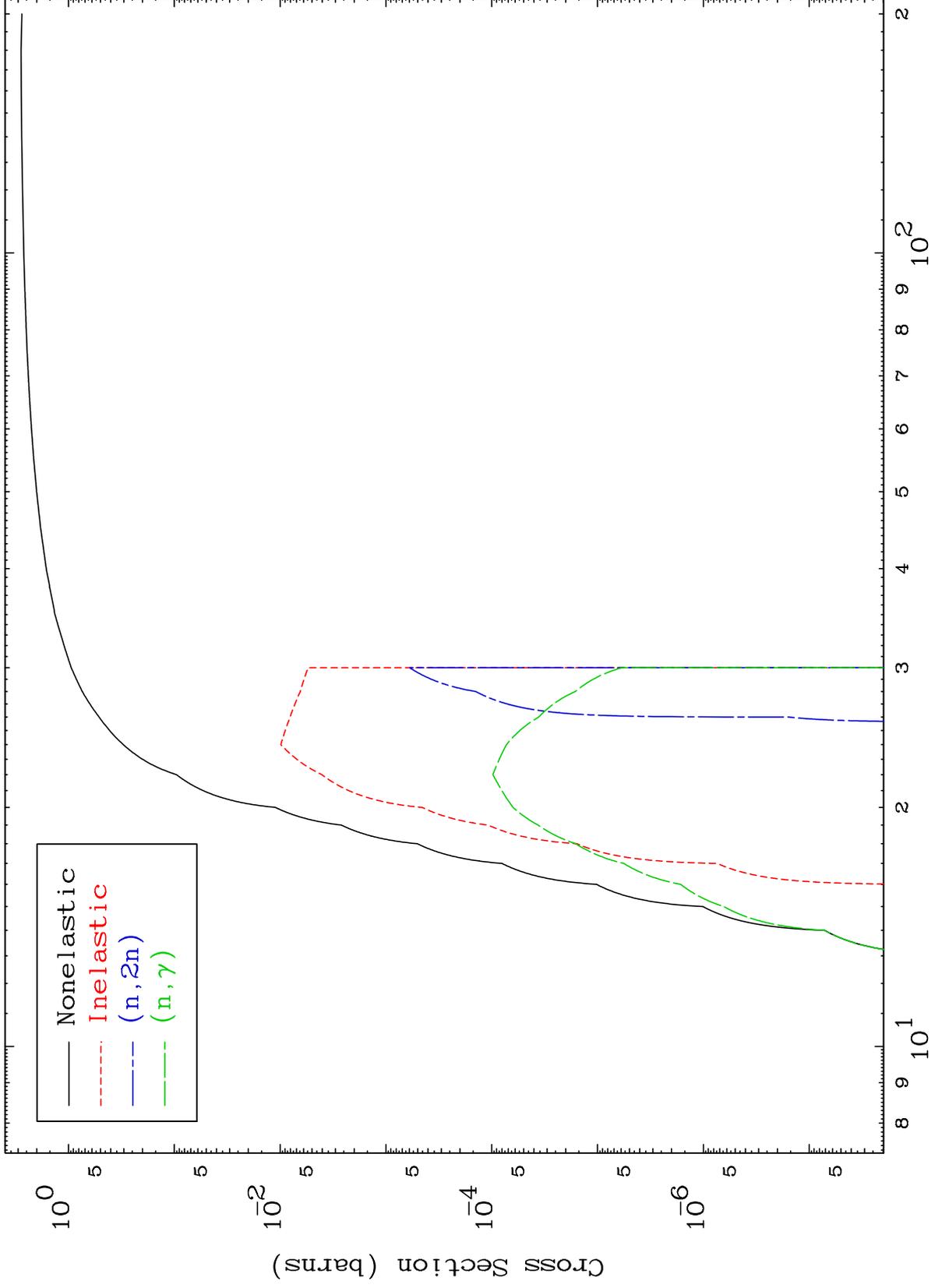
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Press Mouse Button to Start

MAT 8713

0 Kelvin Cross Sections

87-Fr-208



1

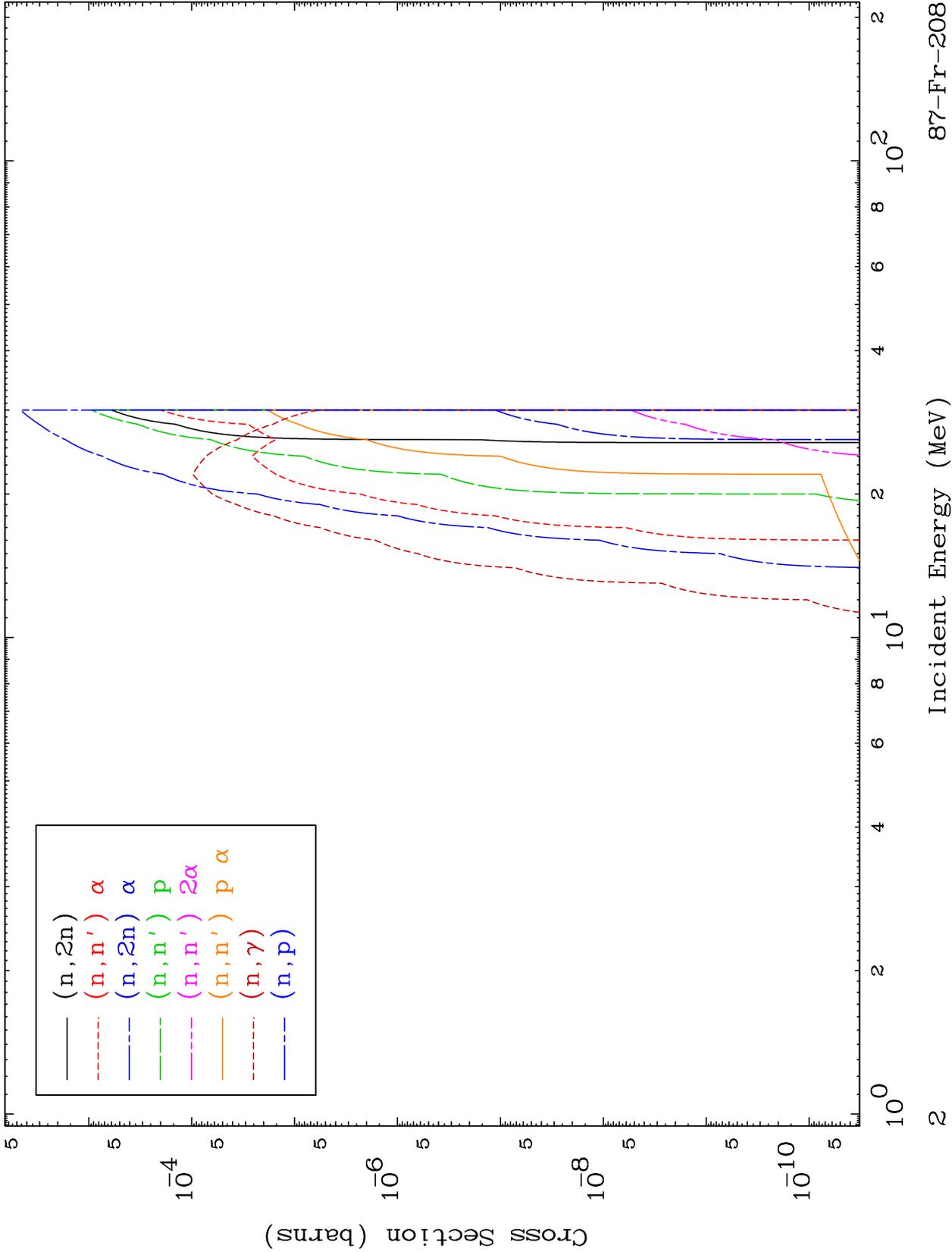
Incident Energy (MeV)

87-Fr-208

MAT 8713

α Neutron Absorption
0 Kelvin Cross Sections

87-Fr-208



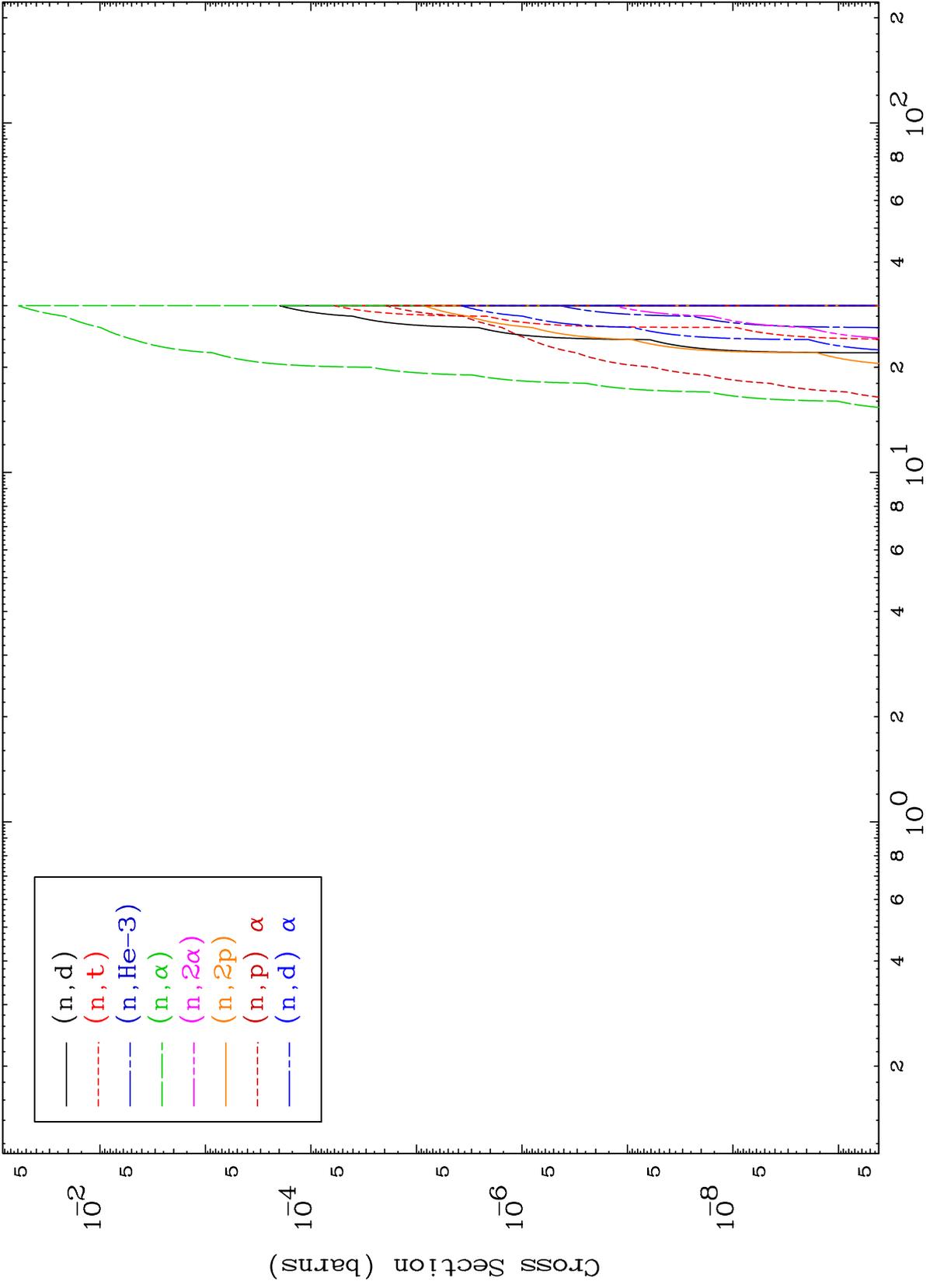
Incident Energy (MeV)

87-Fr-208

MAT 8713

α Neutron Absorption
0 Kelvin Cross Sections

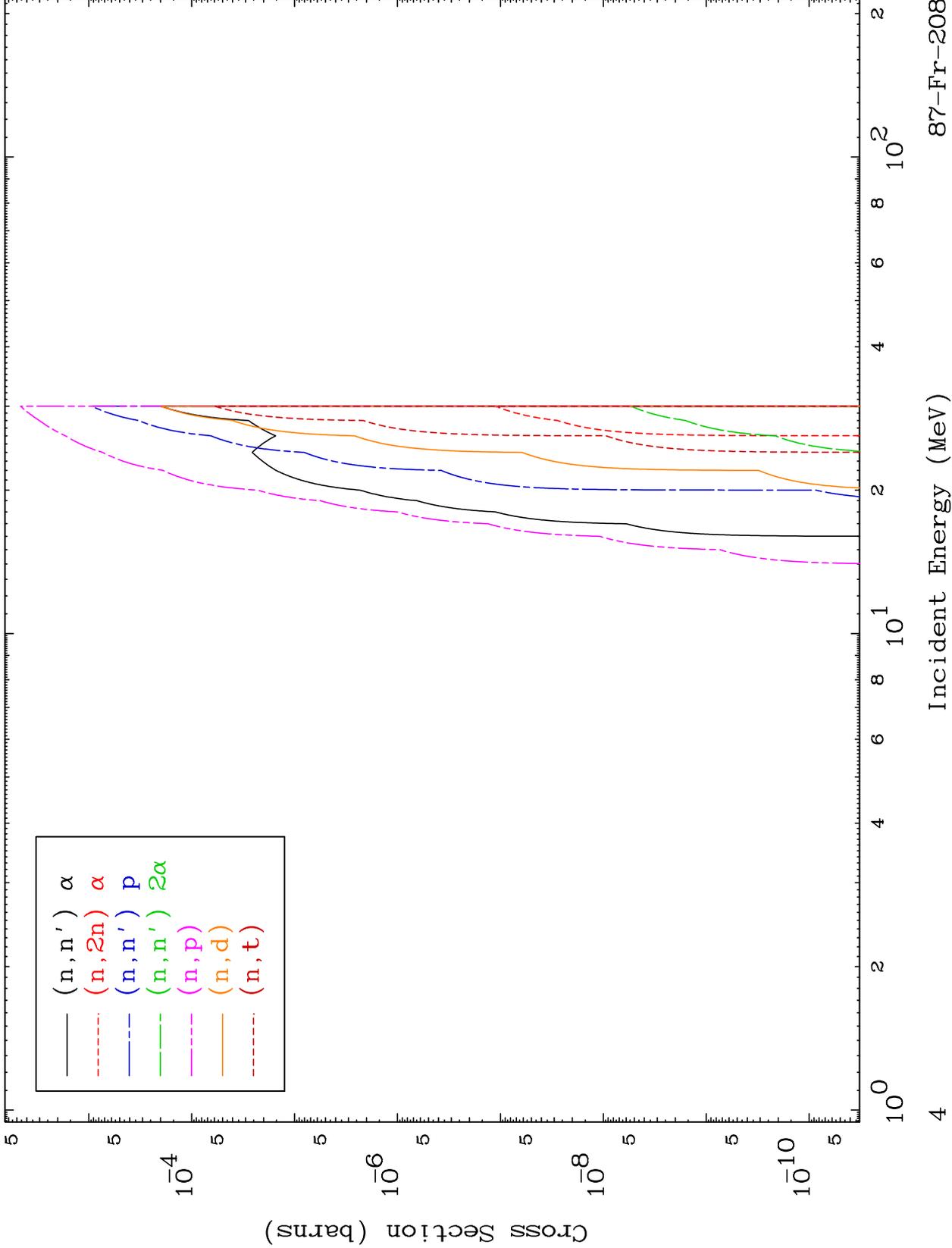
87-Fr-208



MAT 8713

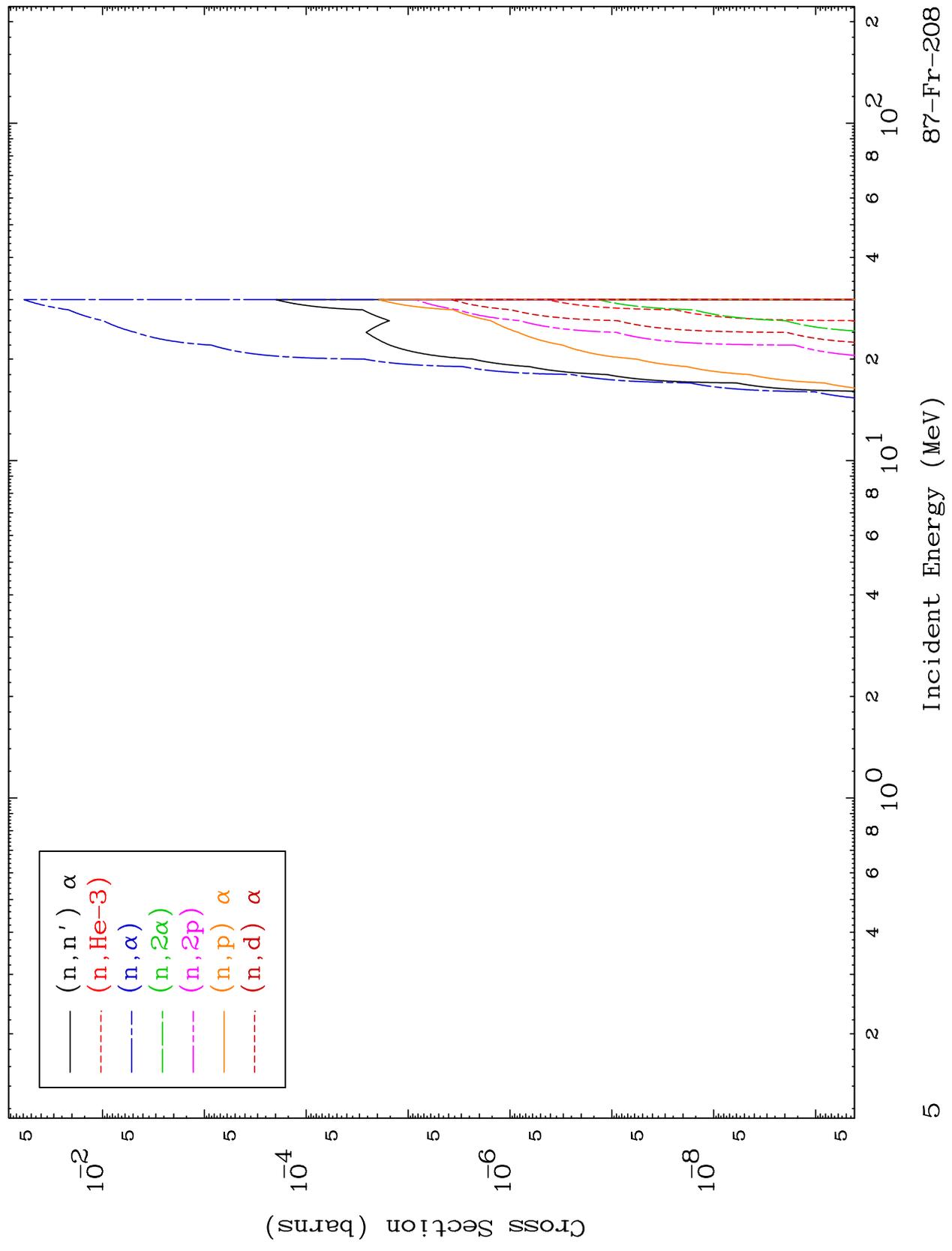
α Charged Particle
0 Kelvin Cross Sections

87-Fr-208



Incident Energy (MeV)

87-Fr-208

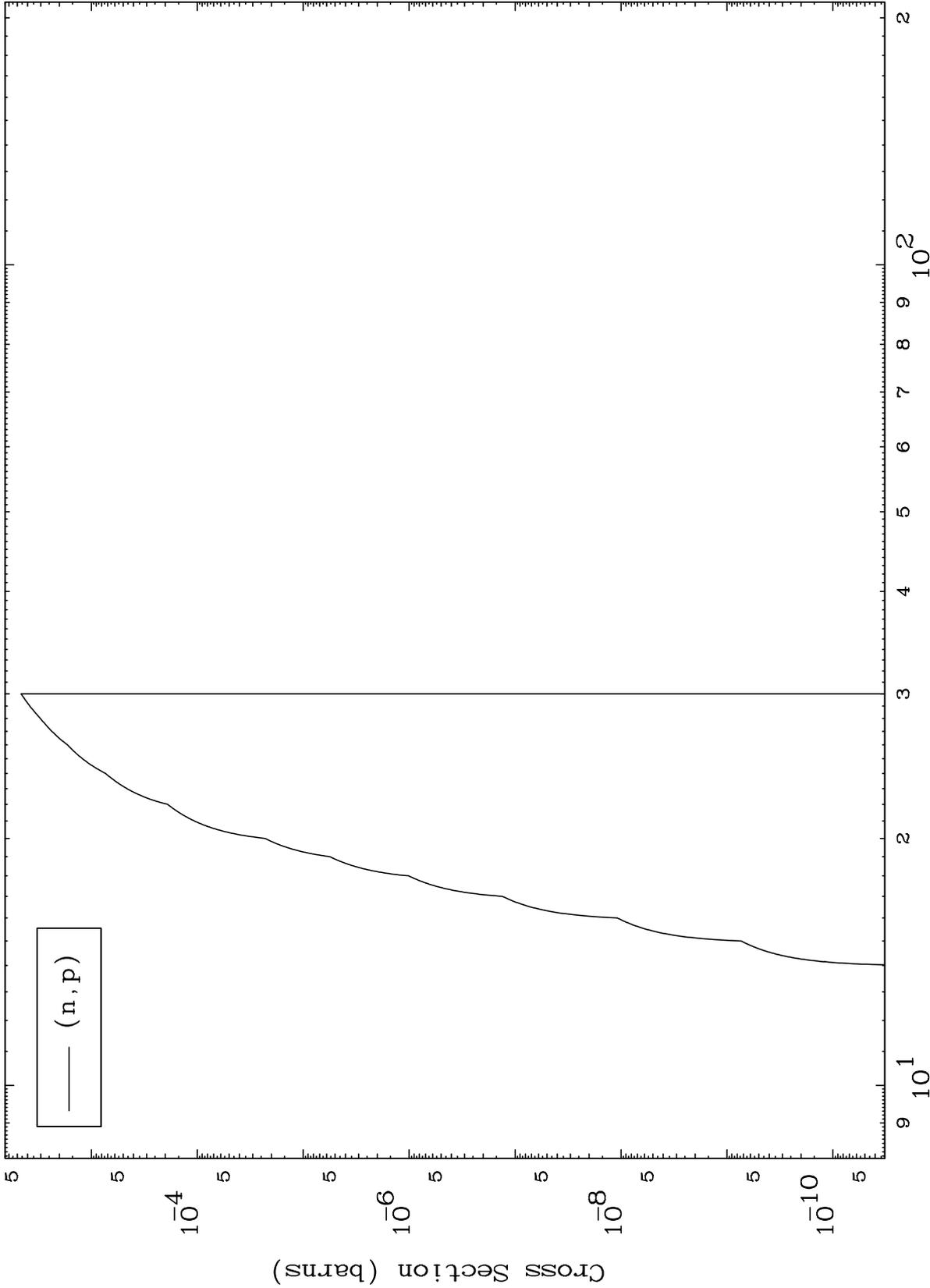


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(α, p) Levels

87-Fr-208

0 Kelvin Cross Sections



Incident Energy (MeV)

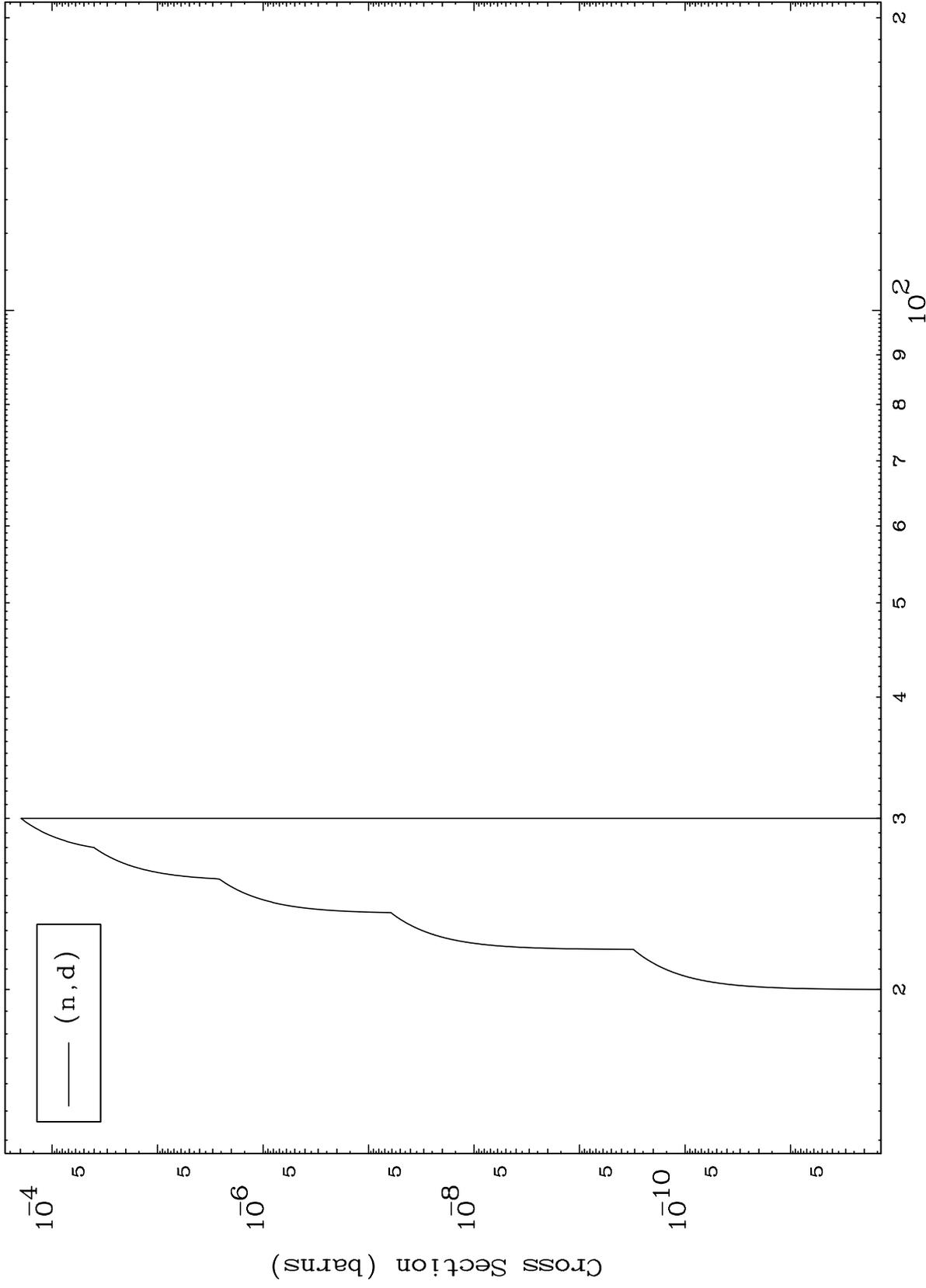
87-Fr-208

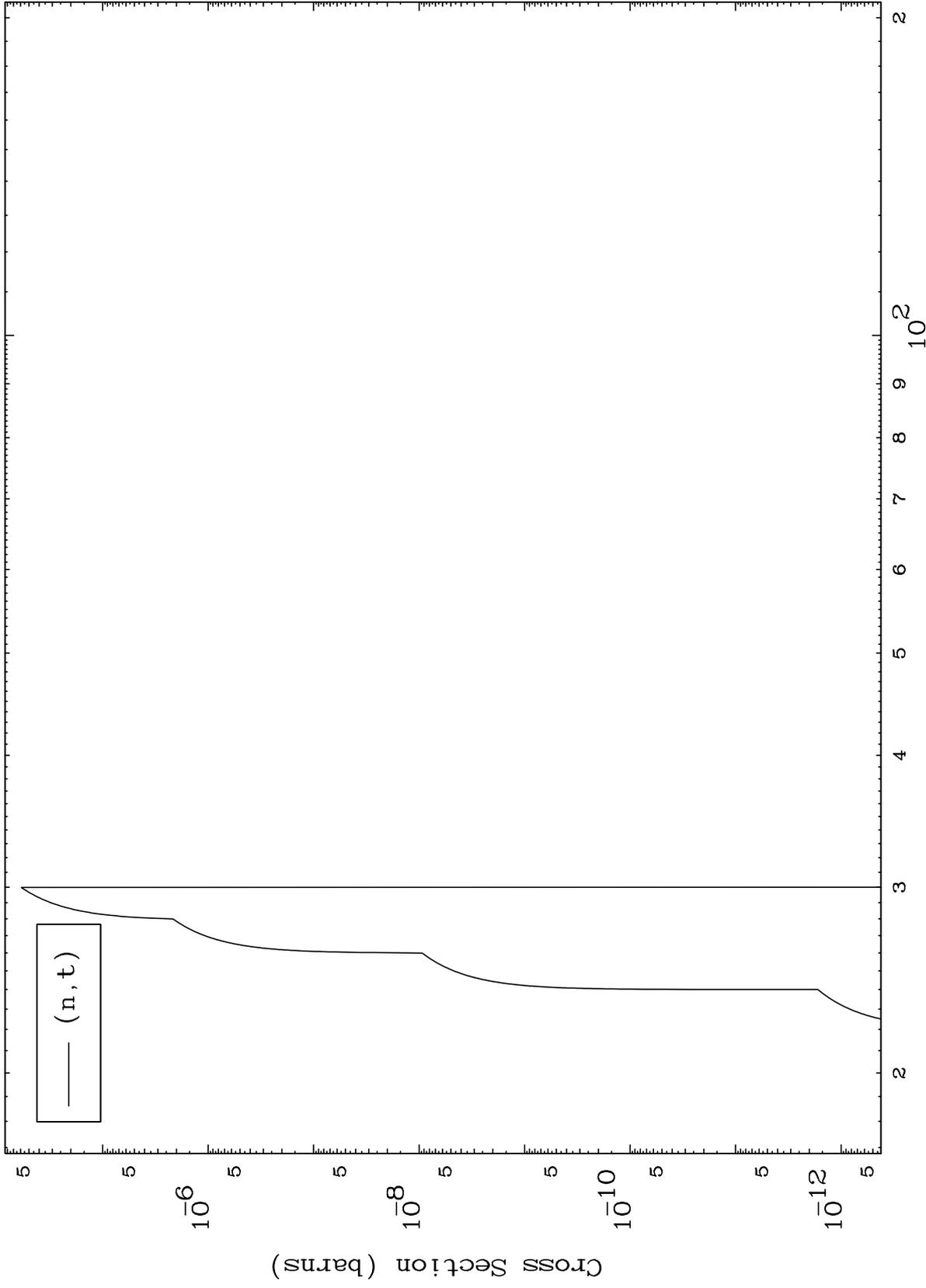
6

MAT 8713

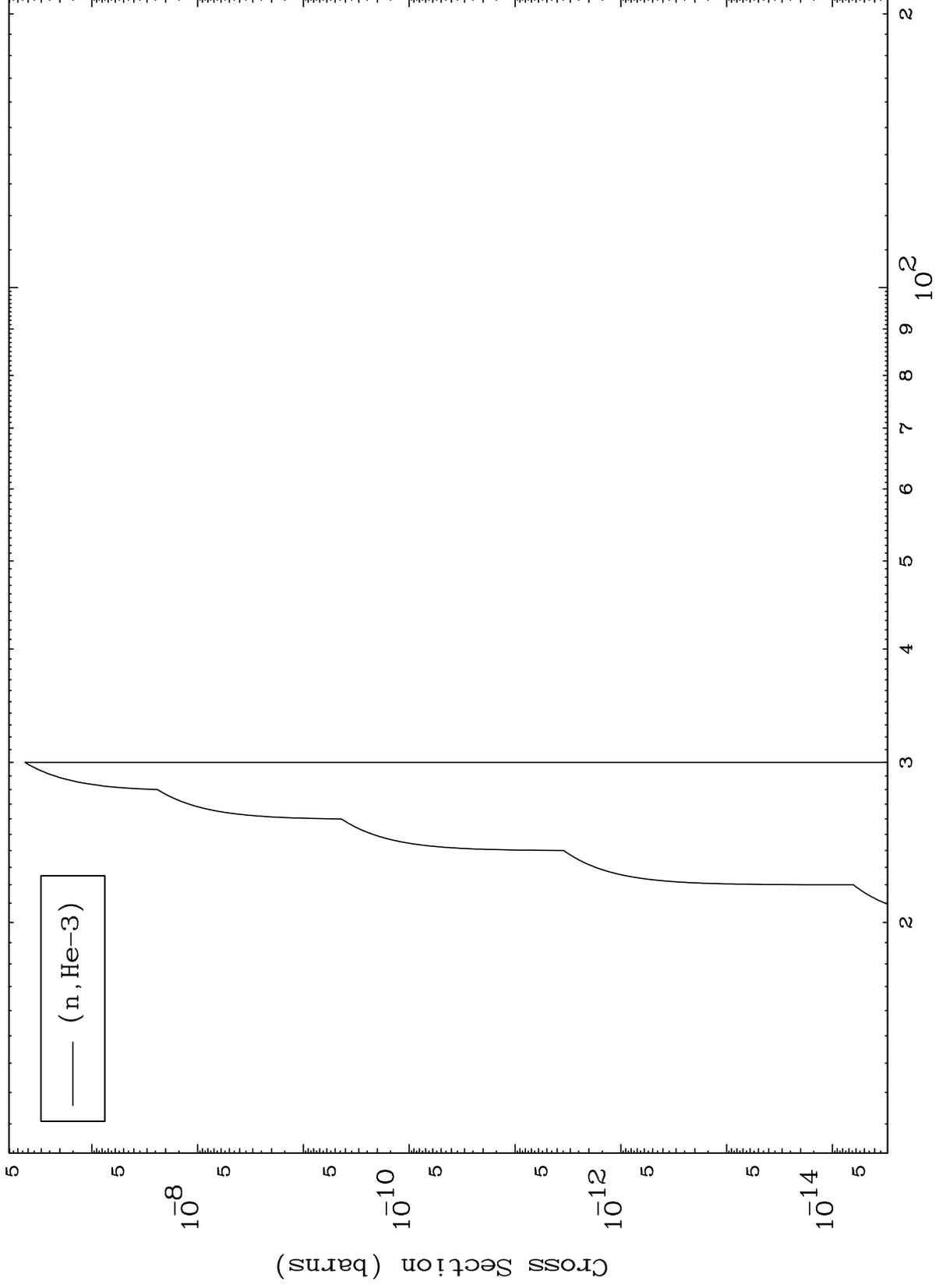
(α, d) Levels
0 Kelvin Cross Sections

87-Fr-208





0 Kelvin Cross Sections



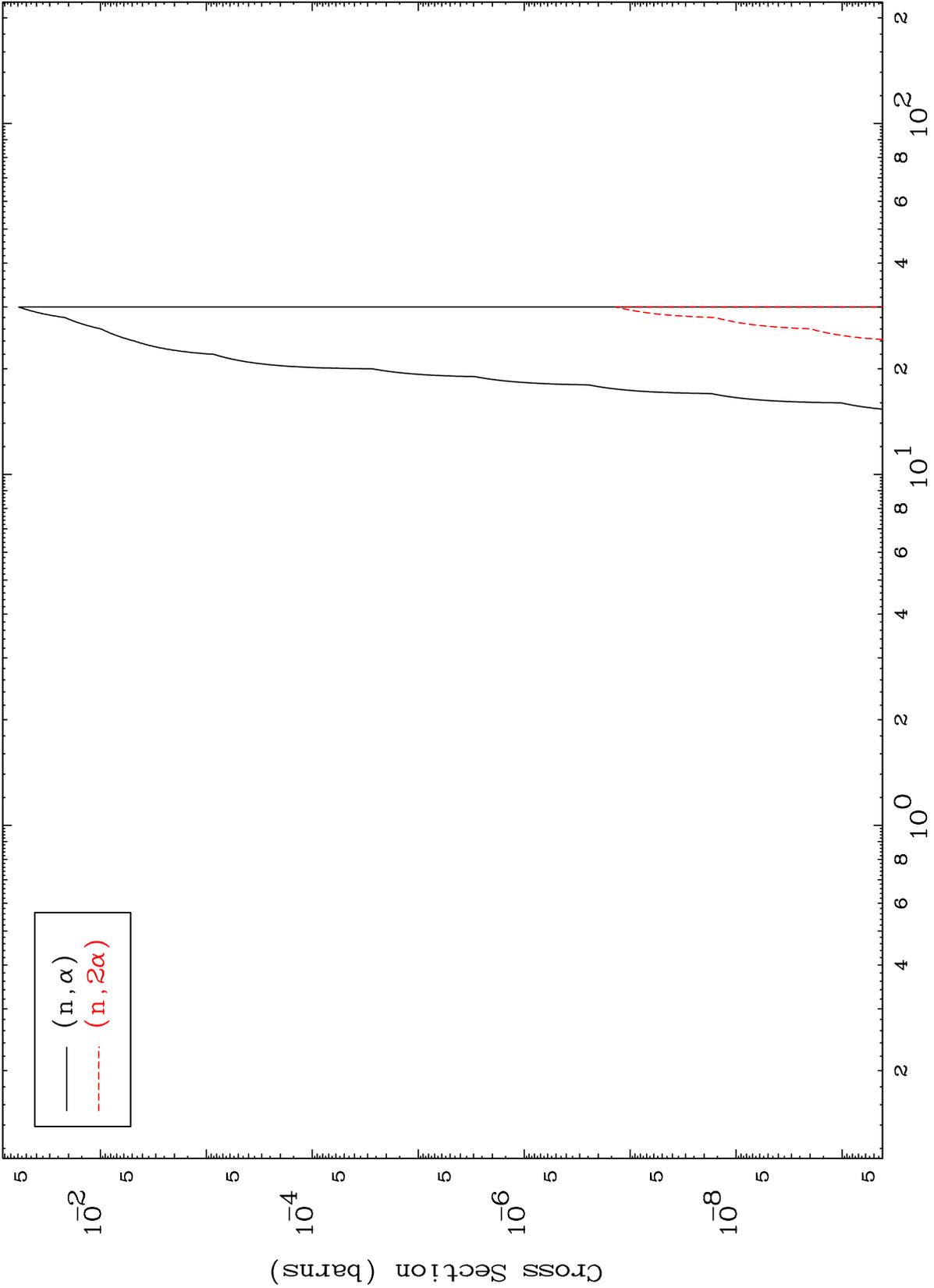
(n, He-3)

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(α, α) Levels

87-Fr-208

0 Kelvin Cross Sections



10

Incident Energy (MeV)

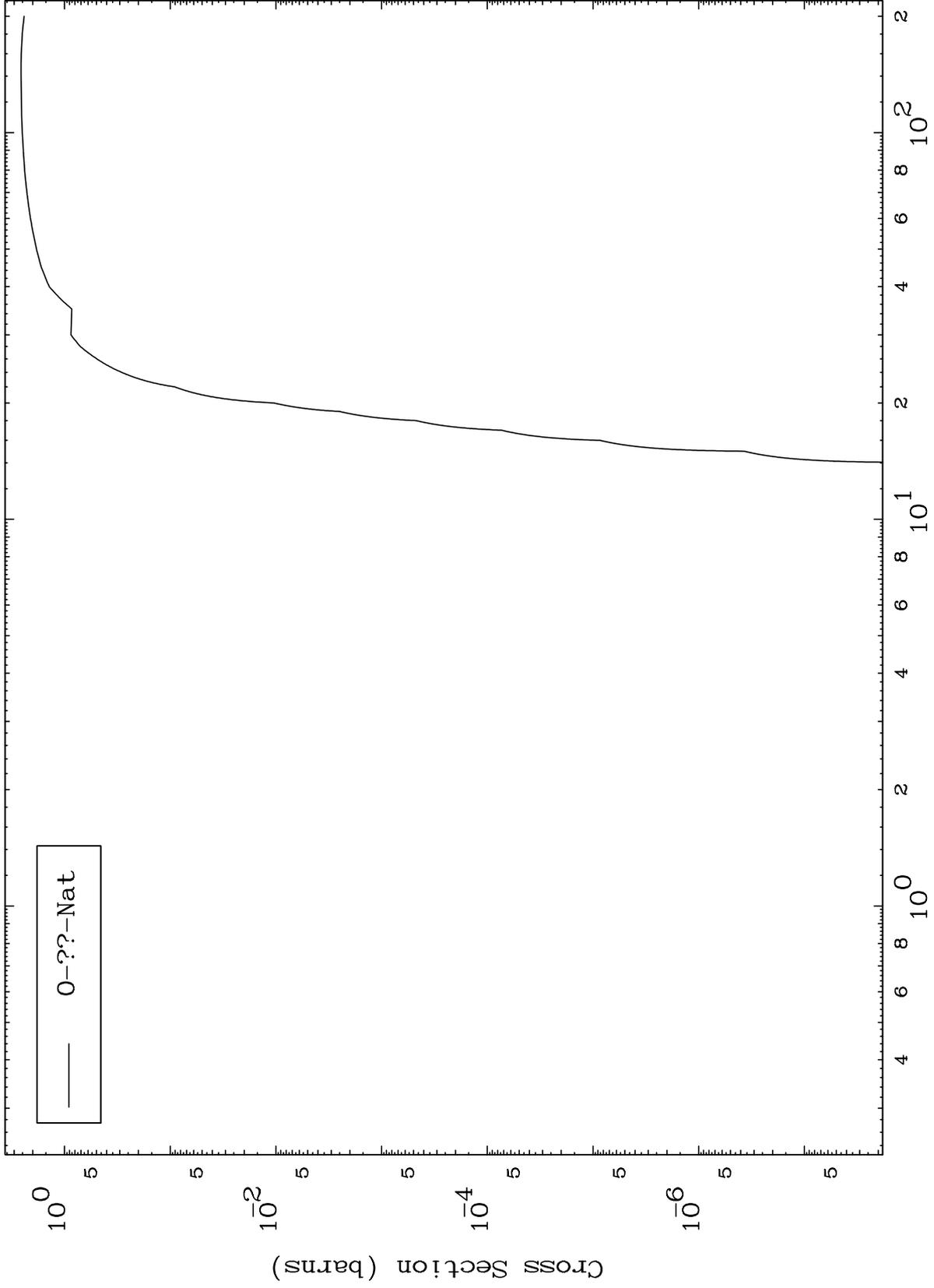
87-Fr-208

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Fission

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Radionuclide Production Cross Section

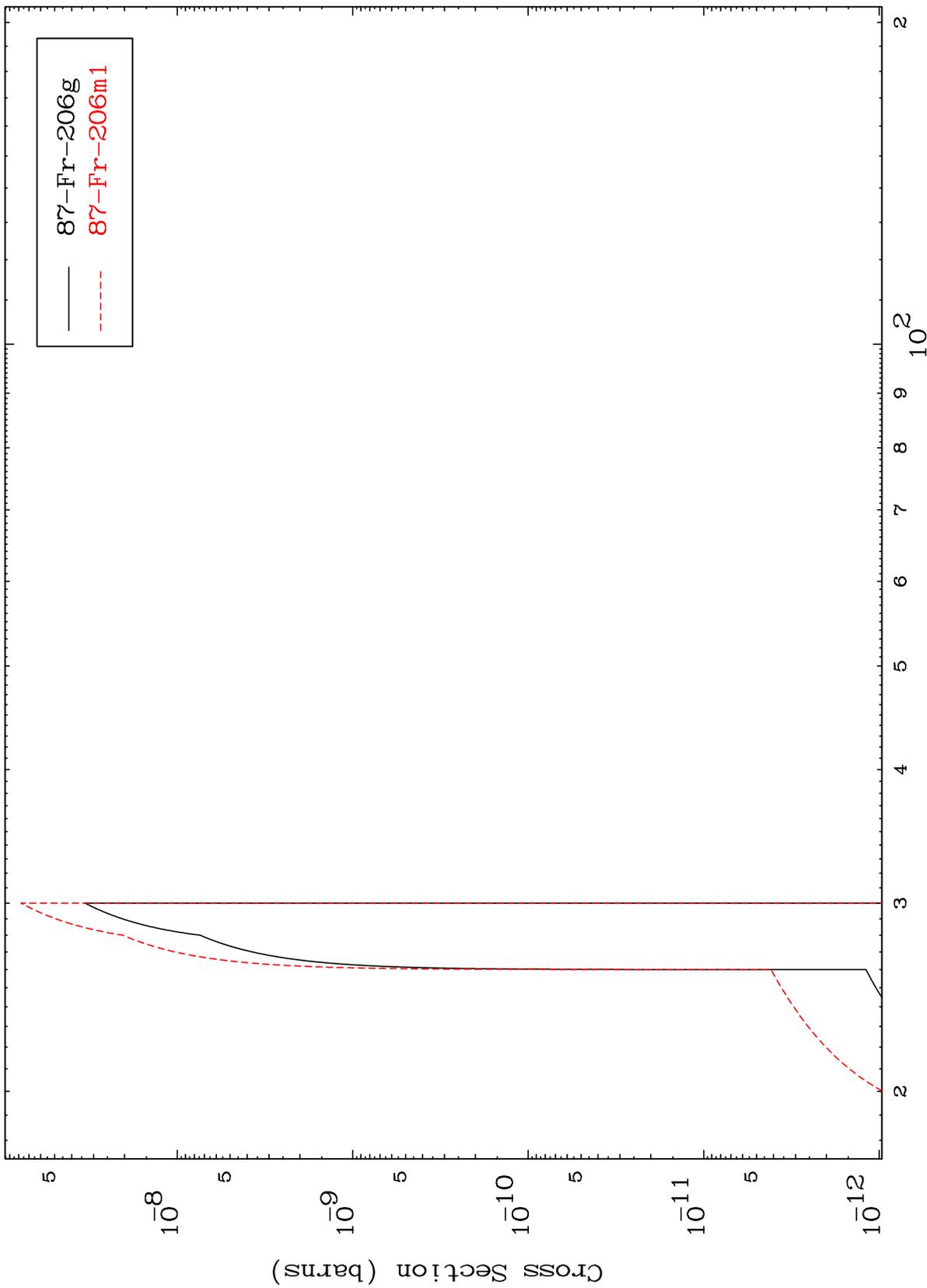


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(n,2n) α

87-Fr-208

Radionuclide Production Cross Section



12

Incident Energy (MeV)

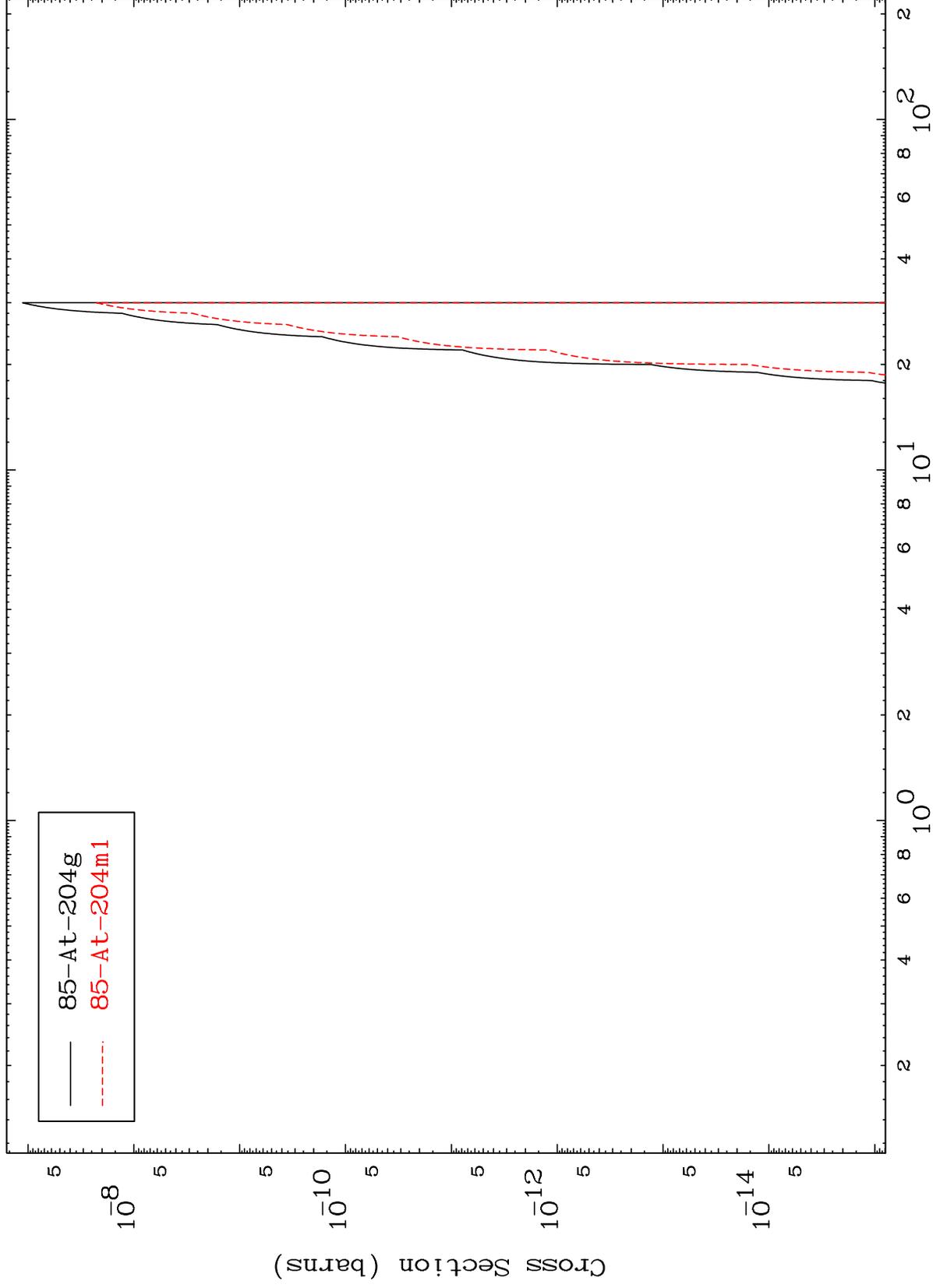
87-Fr-208

MAT 8713

(n,2α)

87-Fr-208

Radionuclide Production Cross Section



13

Incident Energy (MeV)

87-Fr-208