

Program EVALPLOT  
(Version 2021-1)

by

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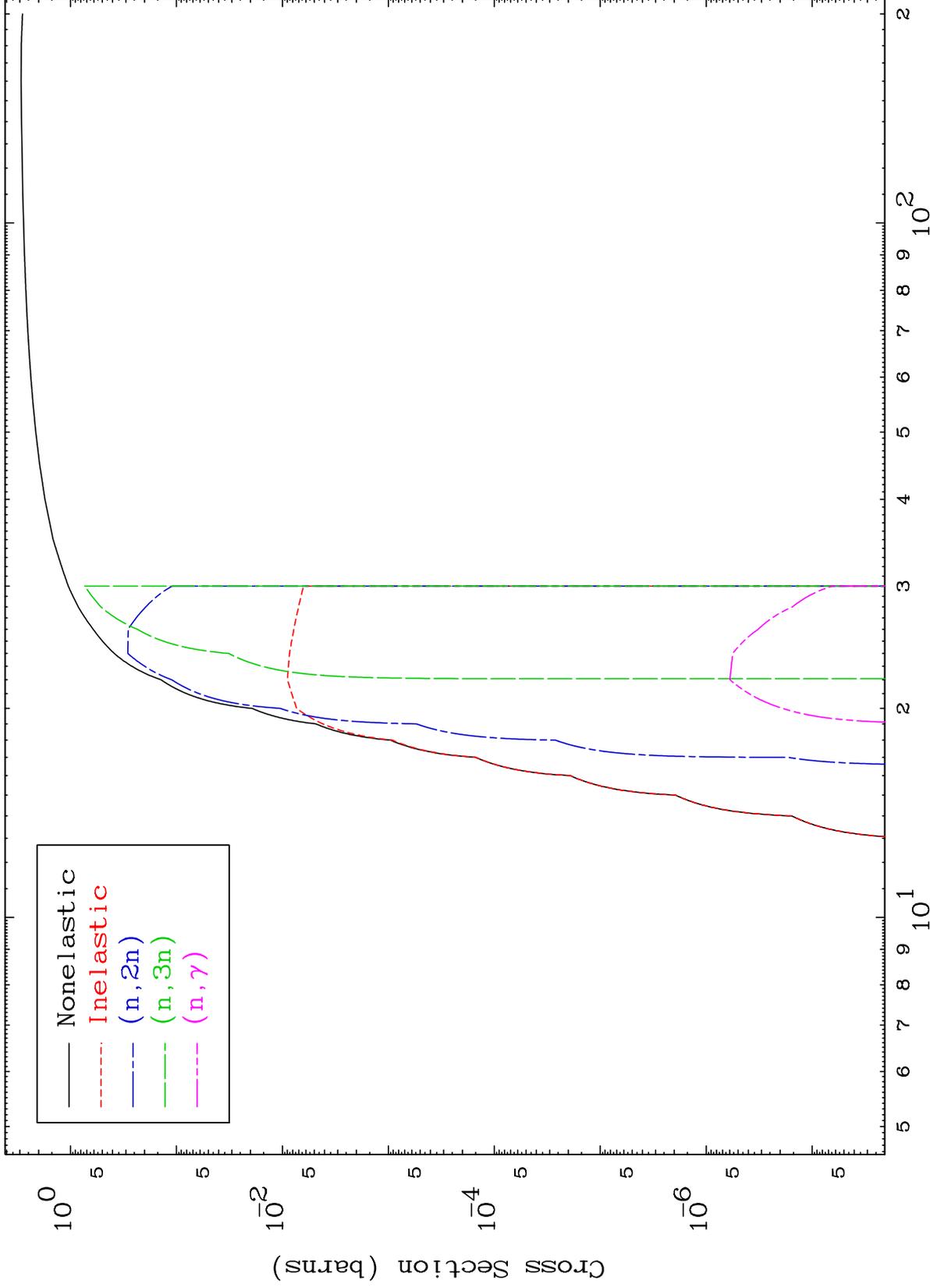
Press Mouse Button to Start

MAT 8761

0 Kelvin Cross Sections

$\alpha$  Major

87-Fr-224



1

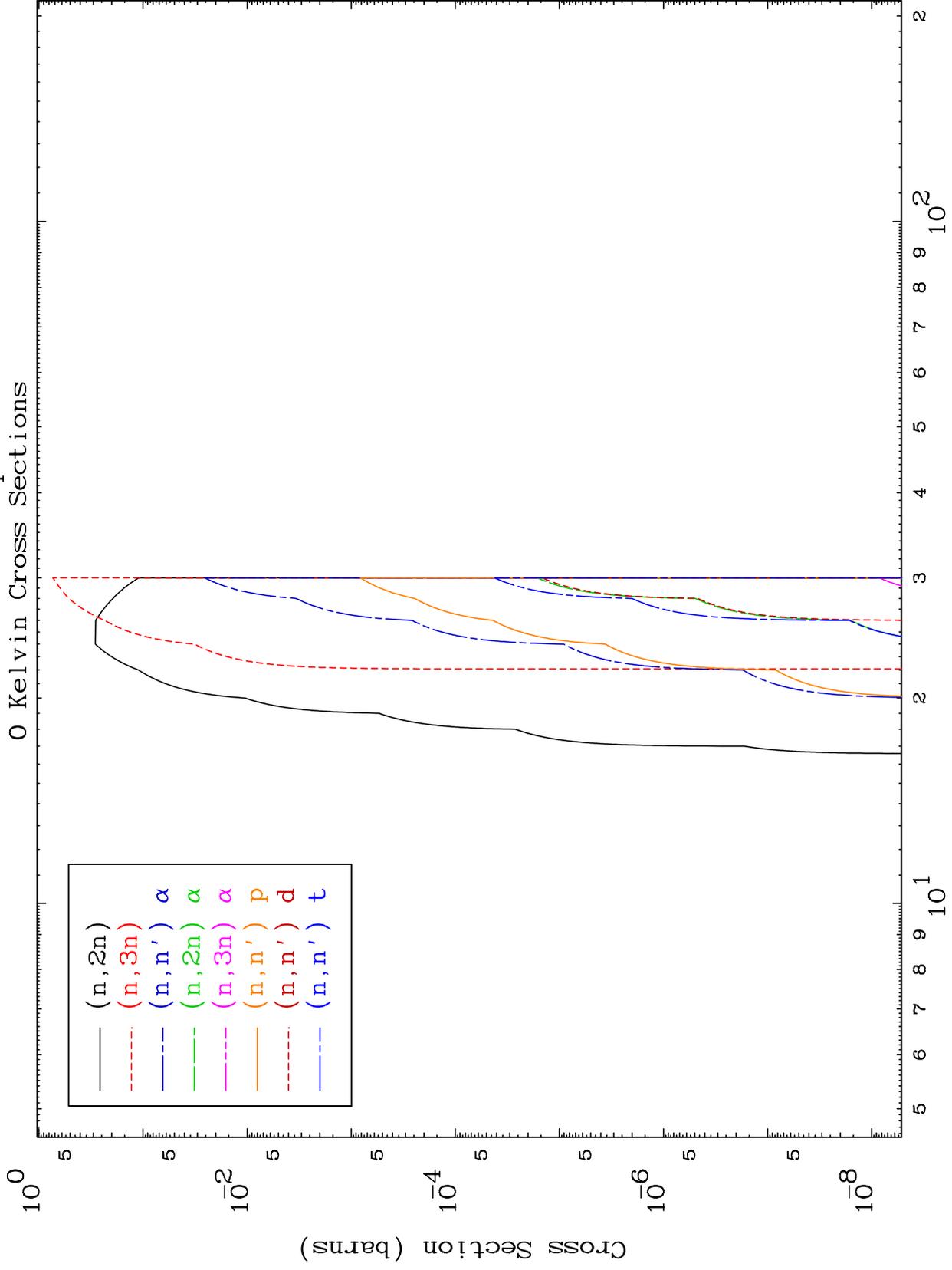
Incident Energy (MeV)

87-Fr-224

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$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

87-Fr-224



2

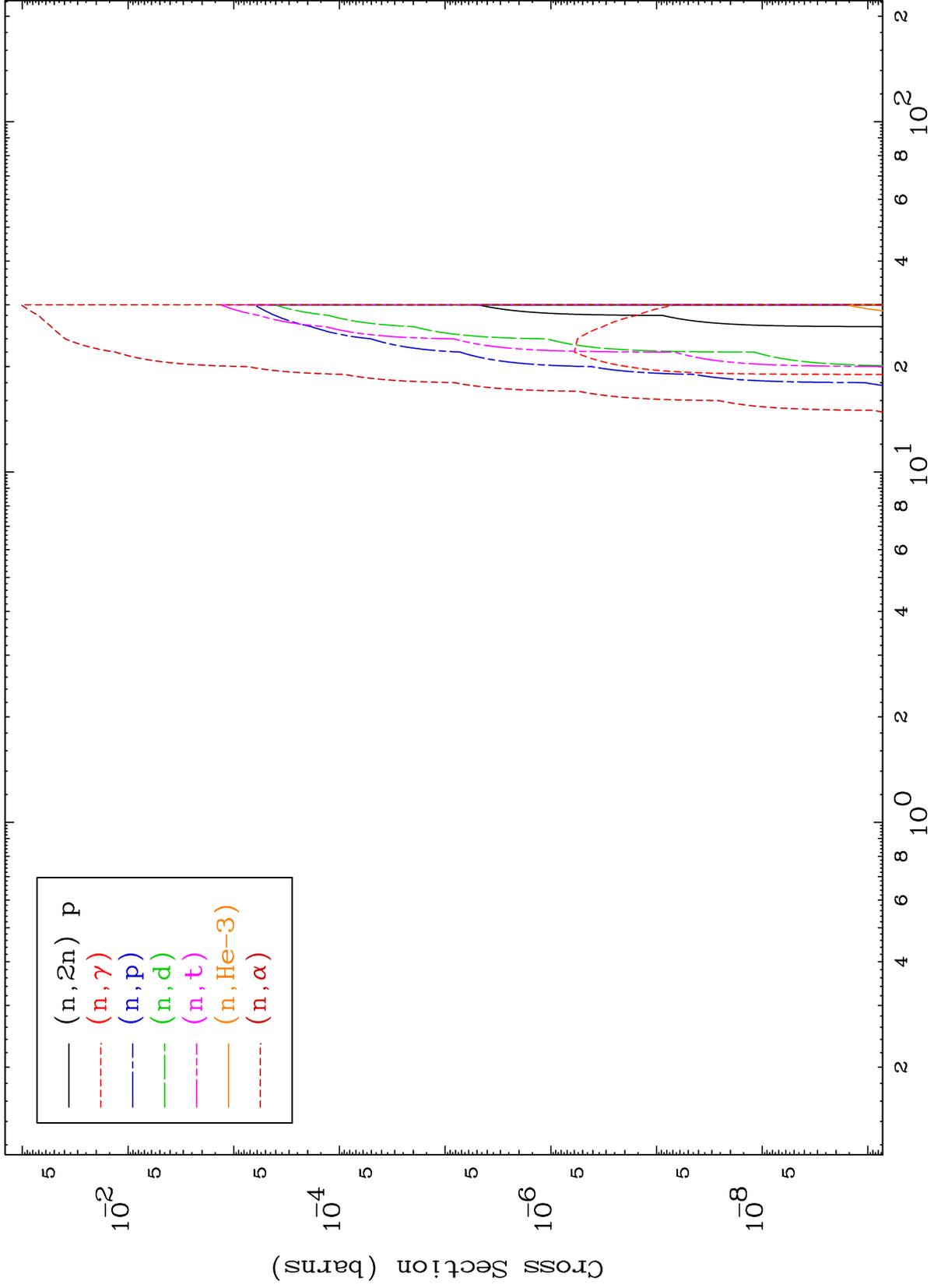
Incident Energy (MeV)

87-Fr-224

MAT 8761

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

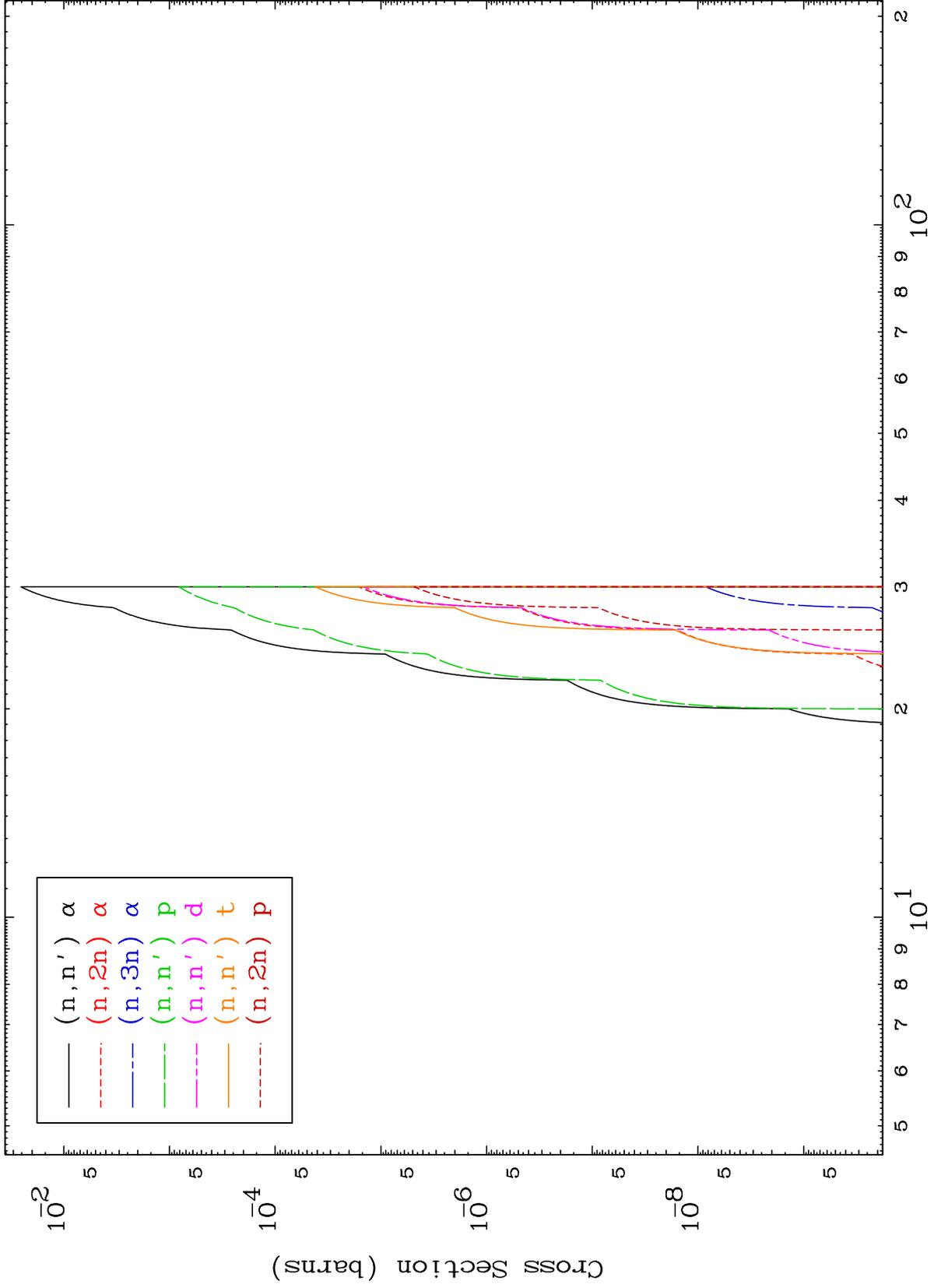
87-Fr-224



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$\alpha$  Charged Particle  
0 Kelvin Cross Sections

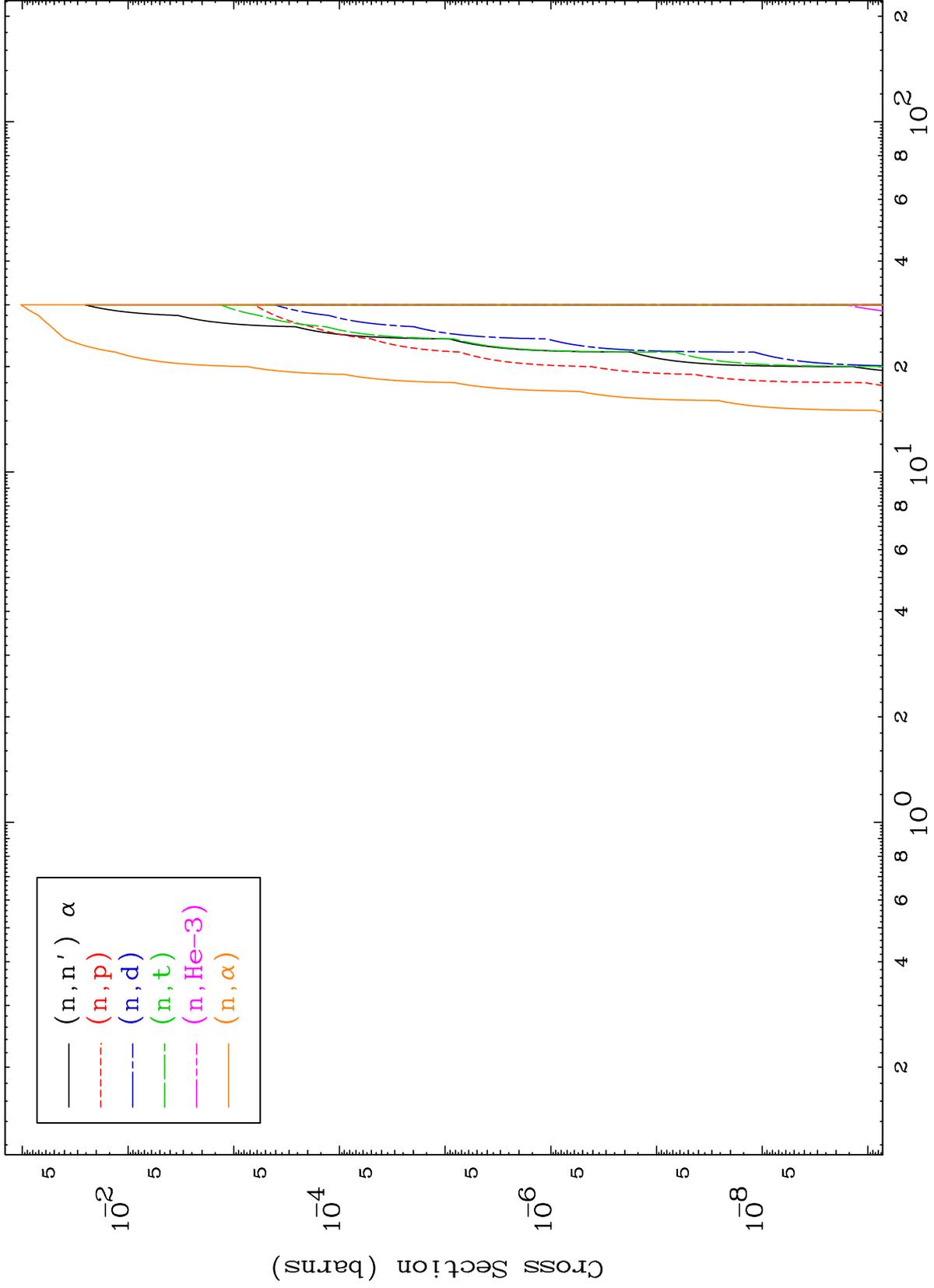
87-Fr-224



4

Incident Energy (MeV)

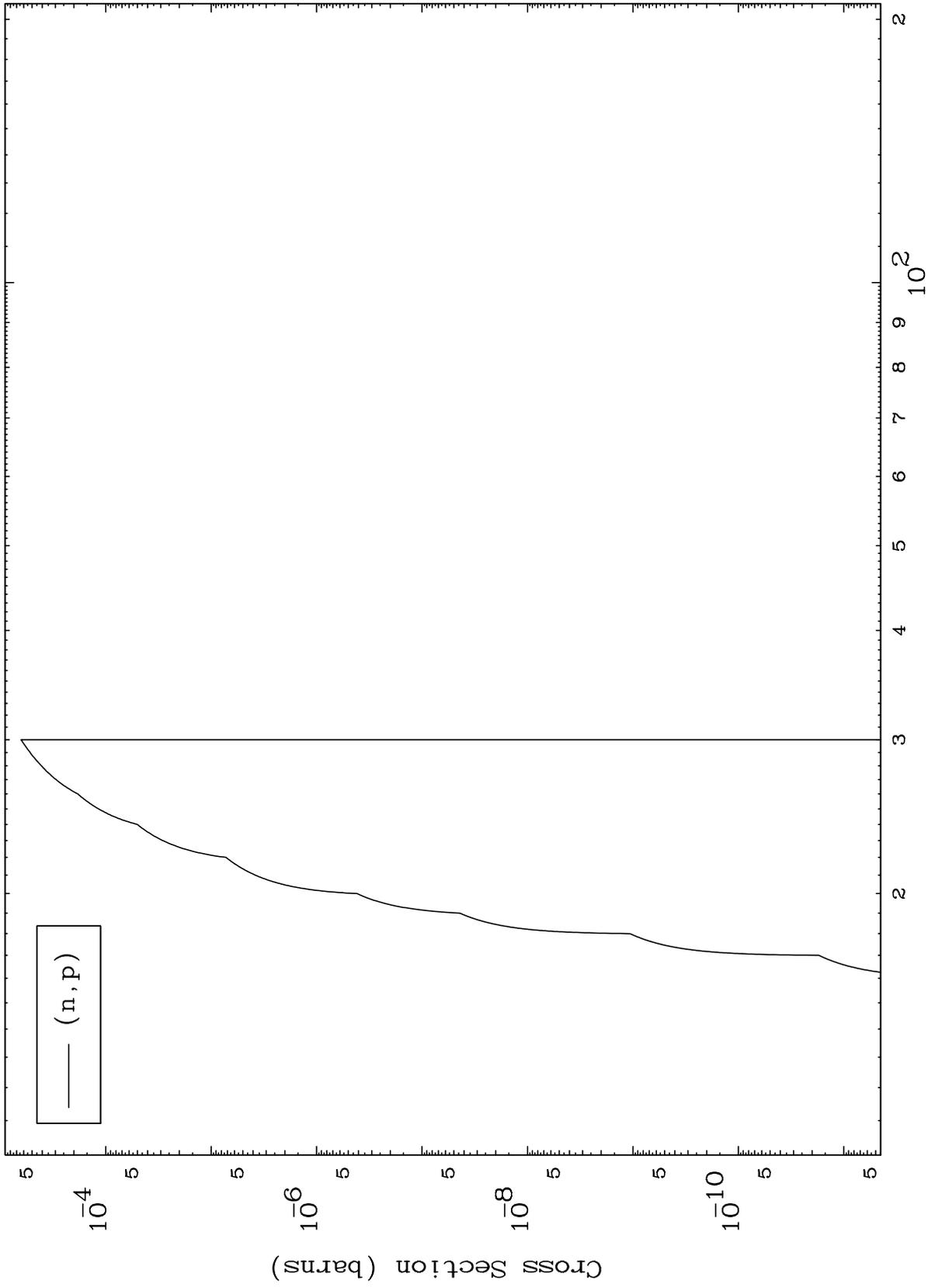
87-Fr-224



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( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

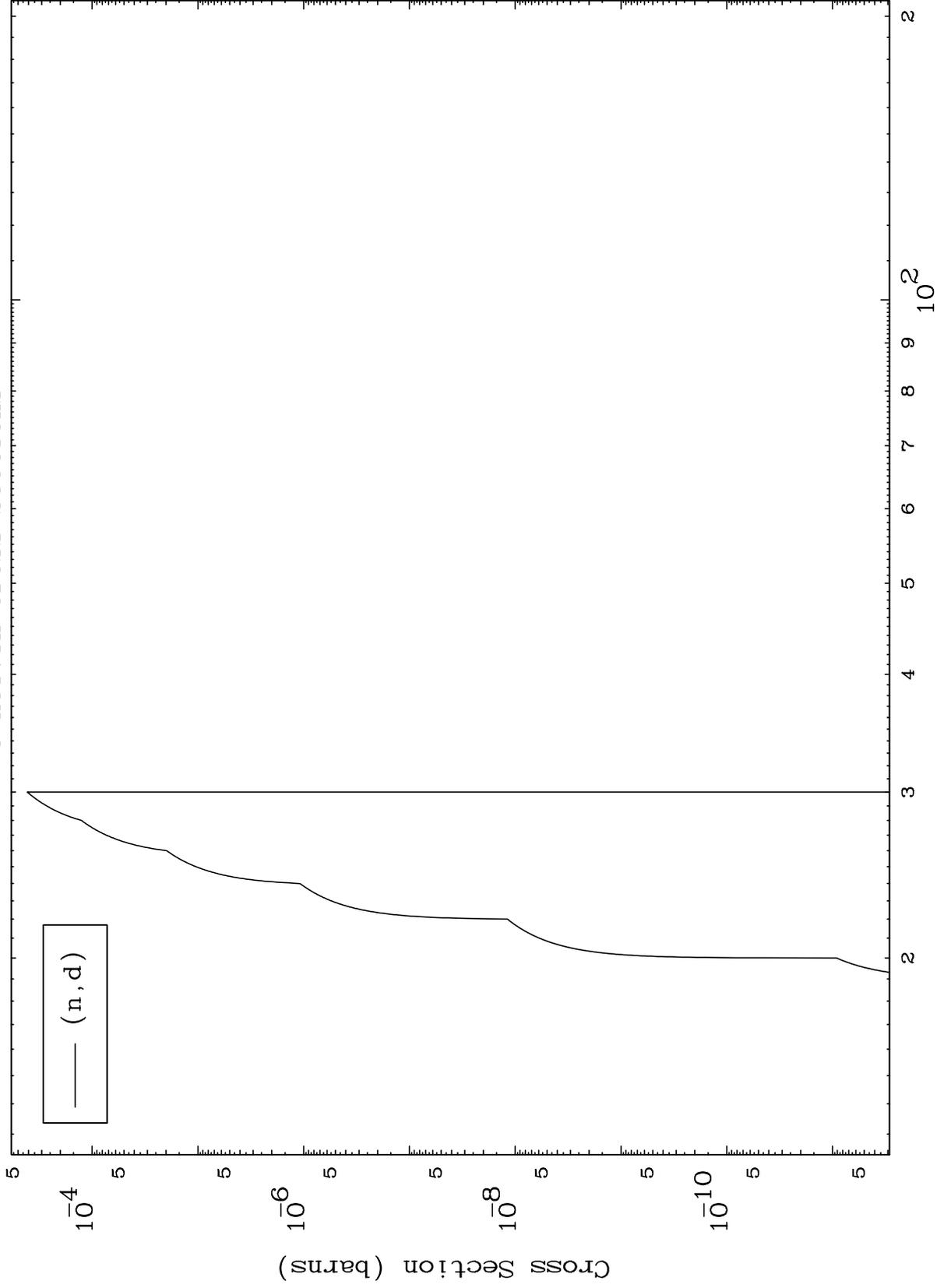
87-Fr-224



MAT 8761

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

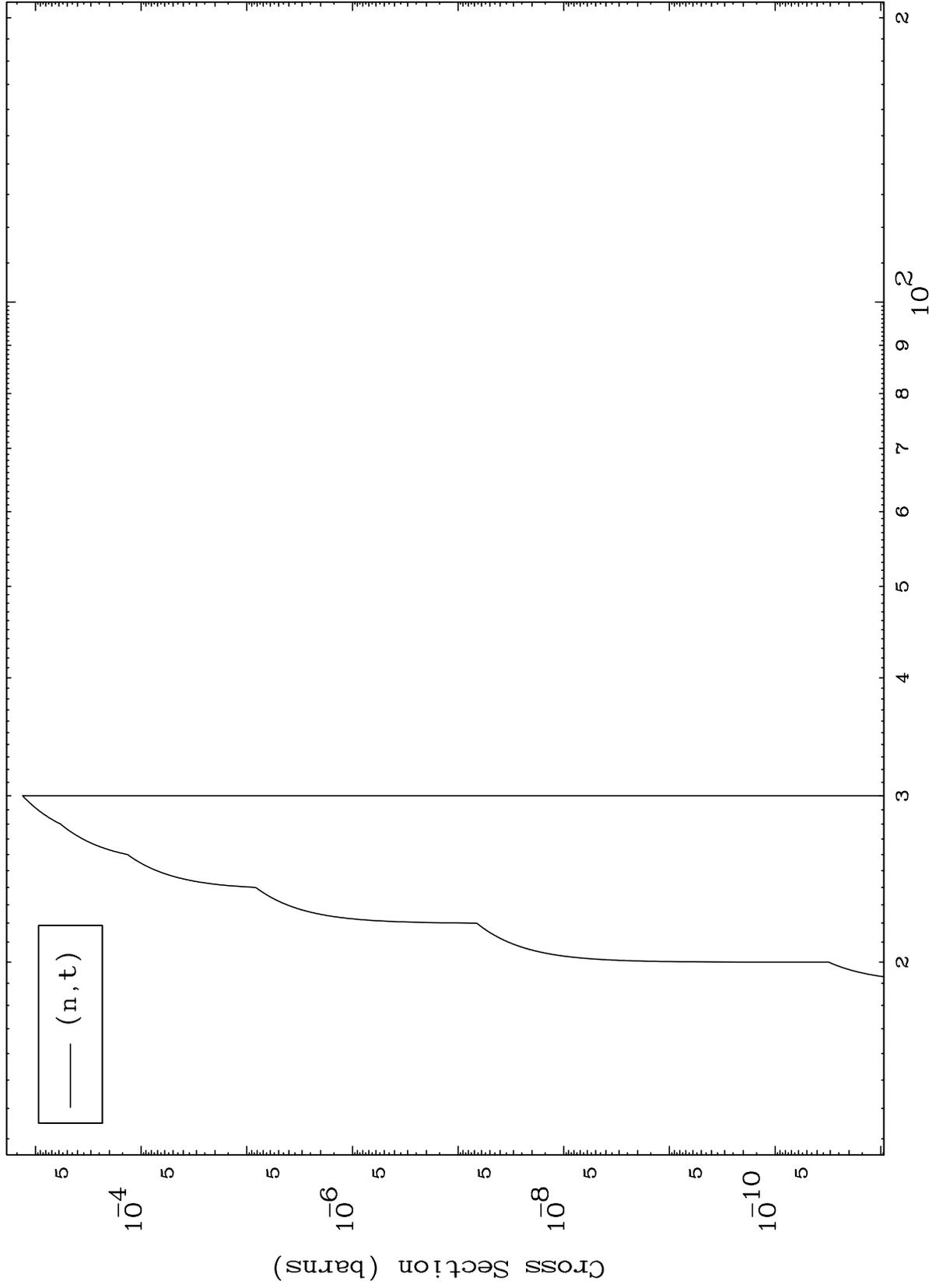
87-Fr-224

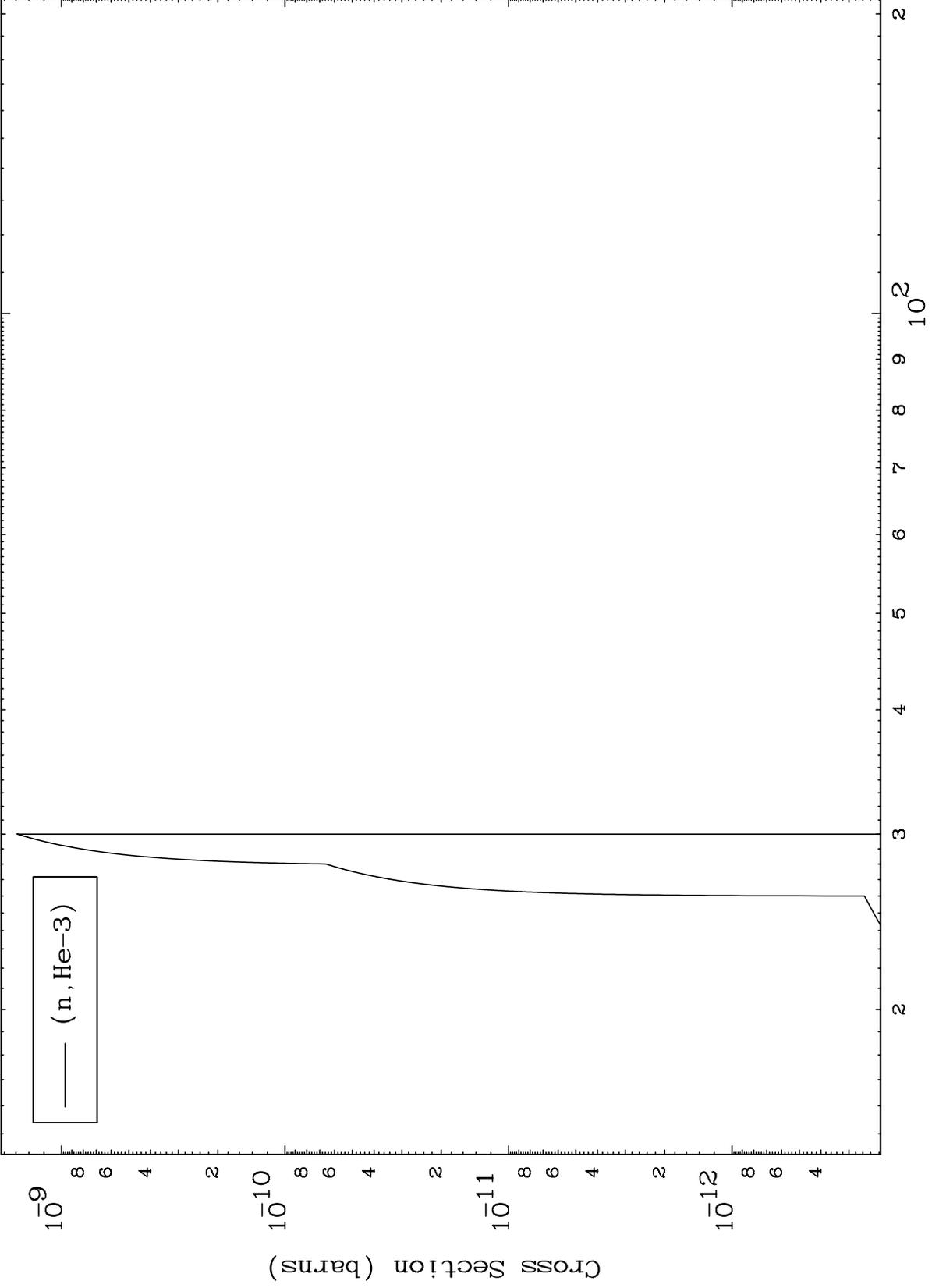


7

Incident Energy (MeV)

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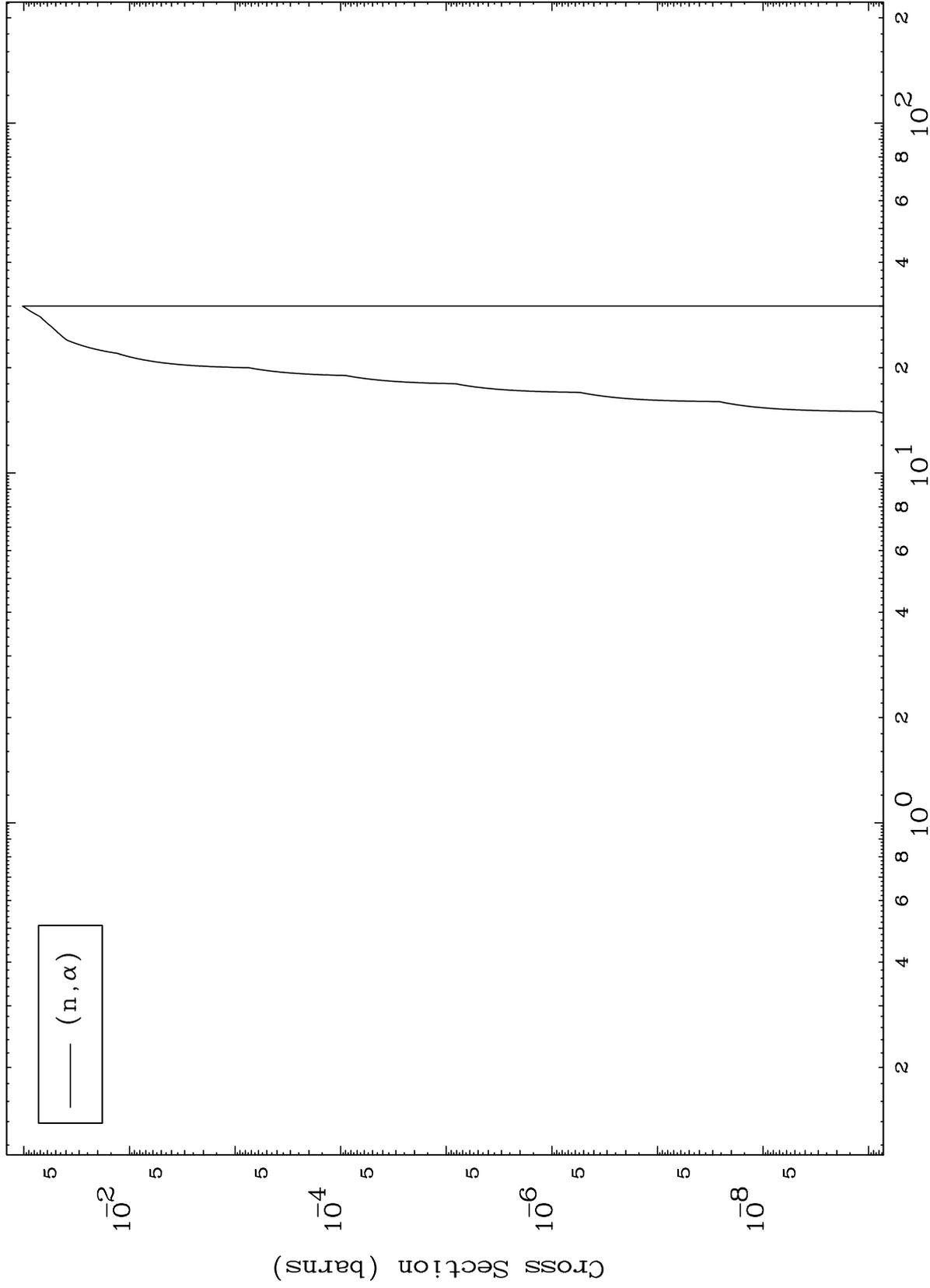


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( $\alpha, \alpha$ ) Levels

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0 Kelvin Cross Sections



10

Incident Energy (MeV)

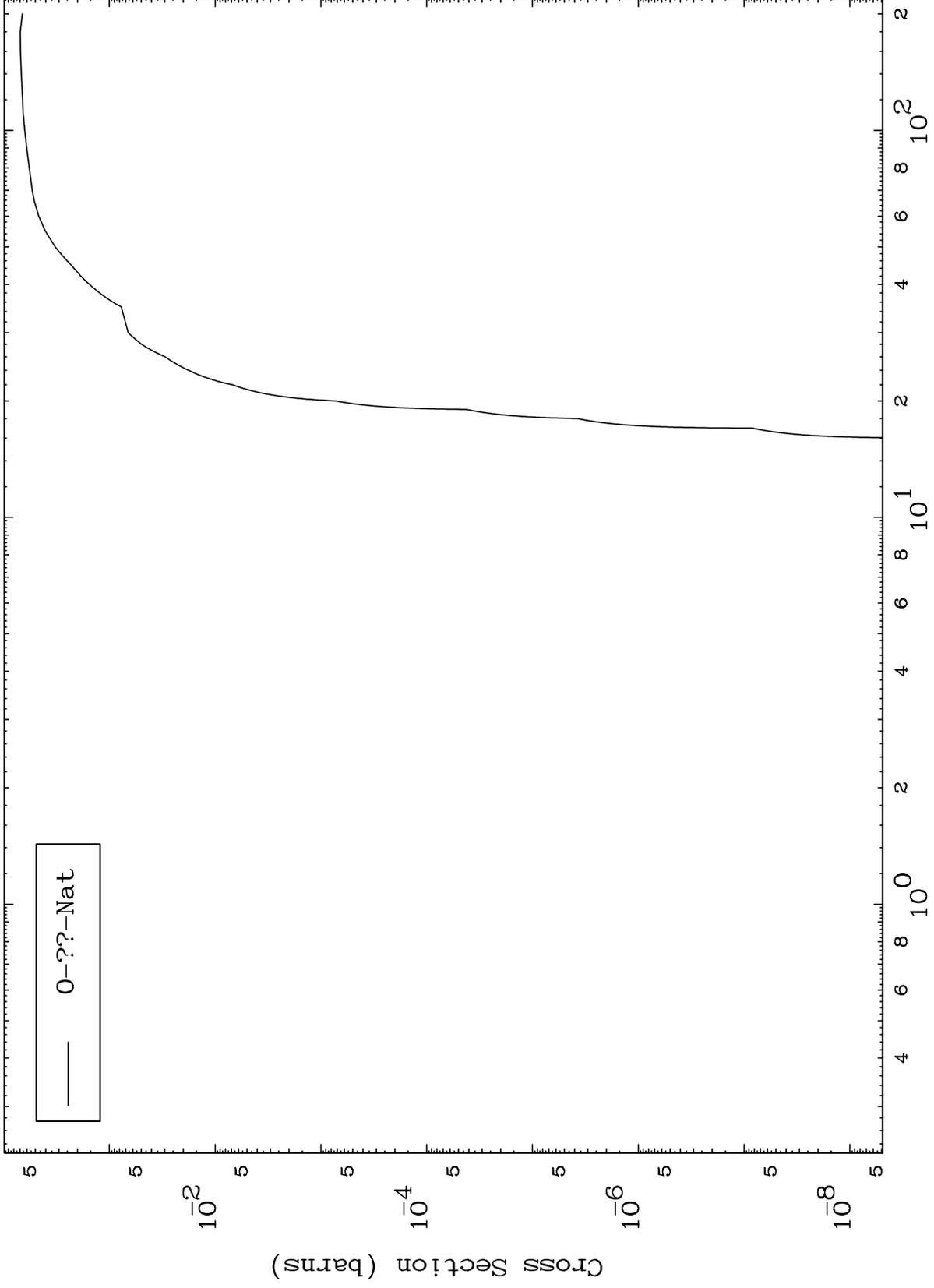
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Fission

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Radionuclide Production Cross Section



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$^{87}\text{Fr-224}$

