

Program EVALPLOT  
(Version 2021-1)

by

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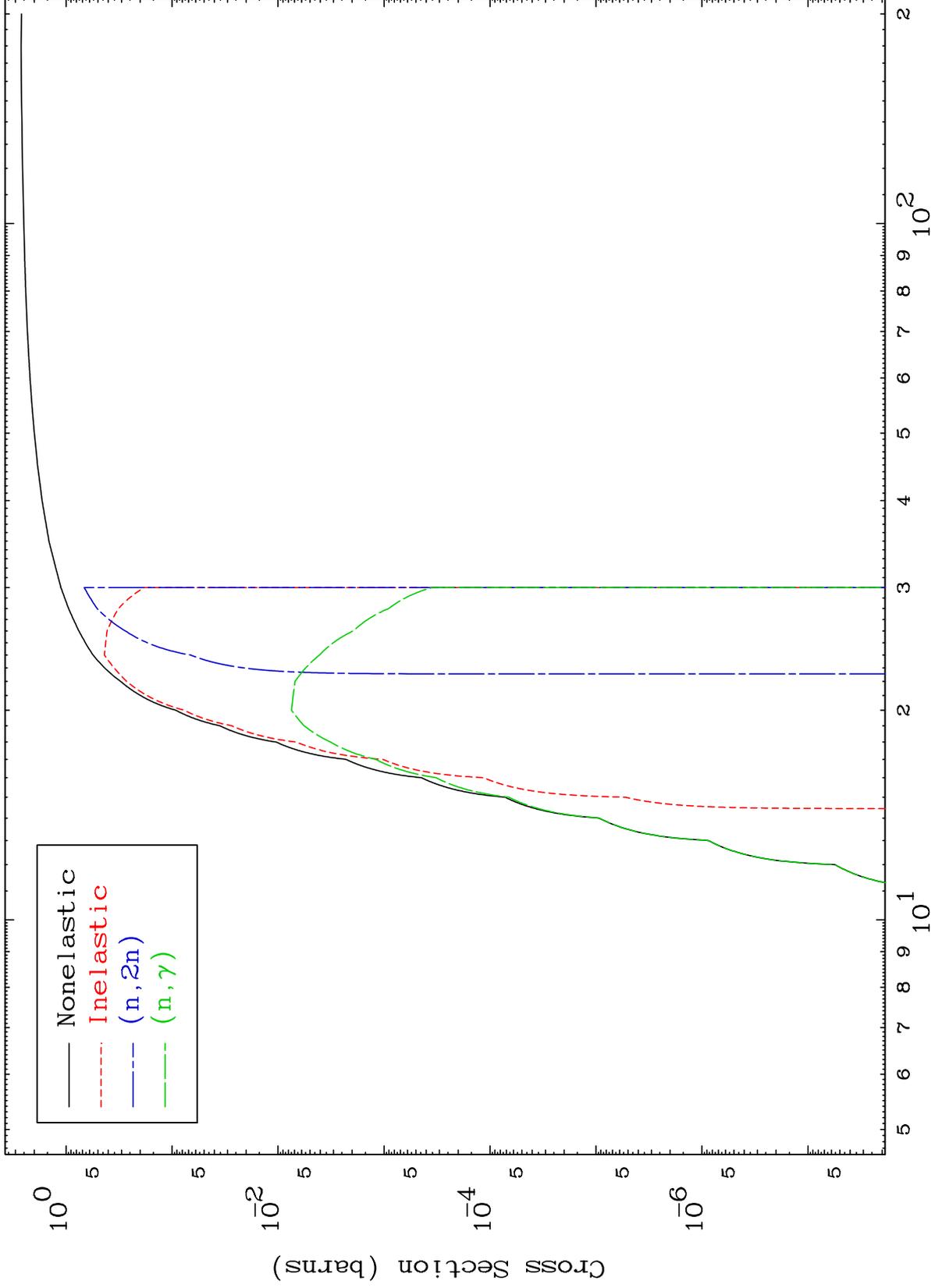
Press Mouse Button to Start

MAT 7701

0 Kelvin Cross Sections

$\alpha$  Major

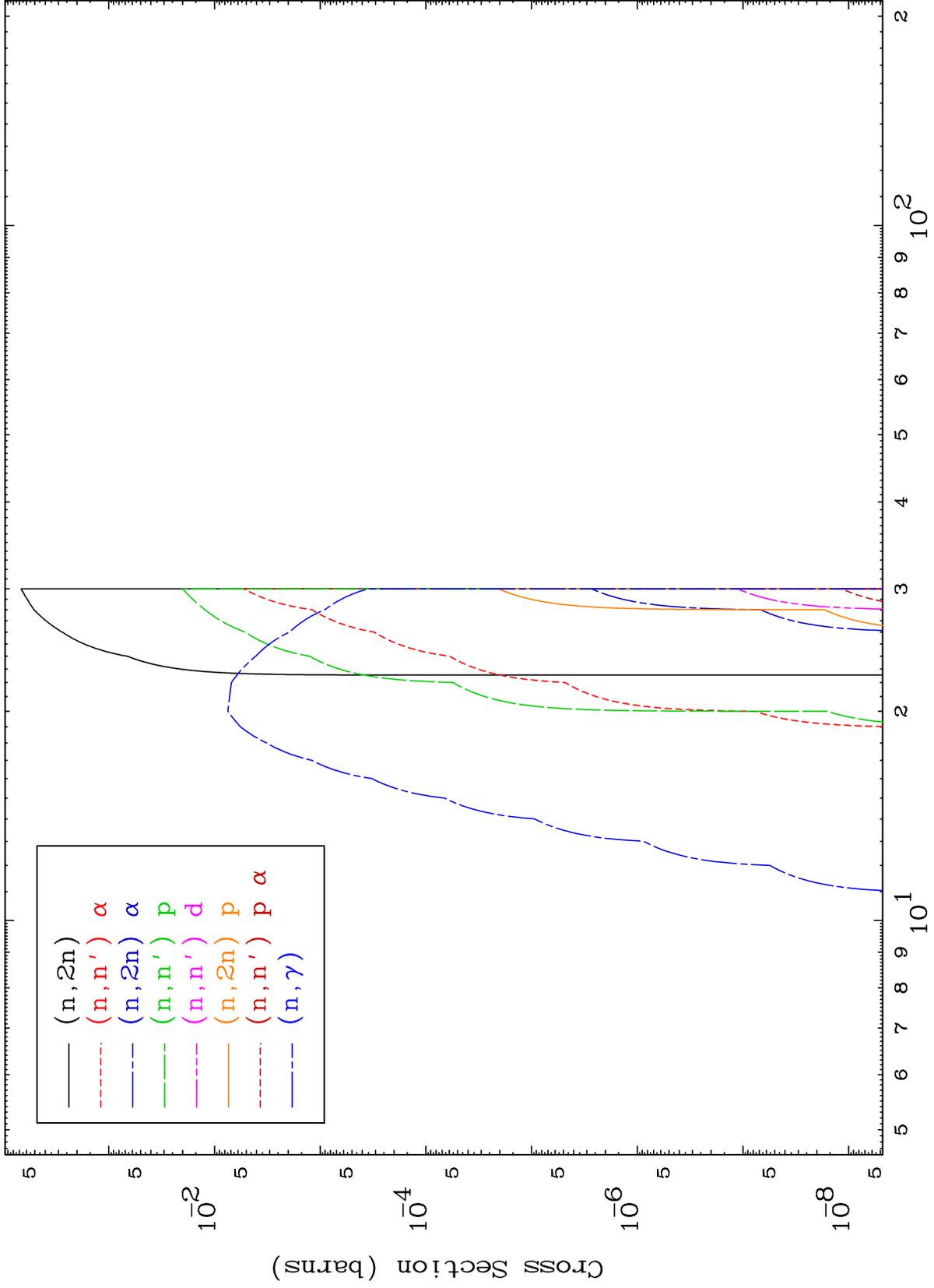
77-Ir-183



1

Incident Energy (MeV)

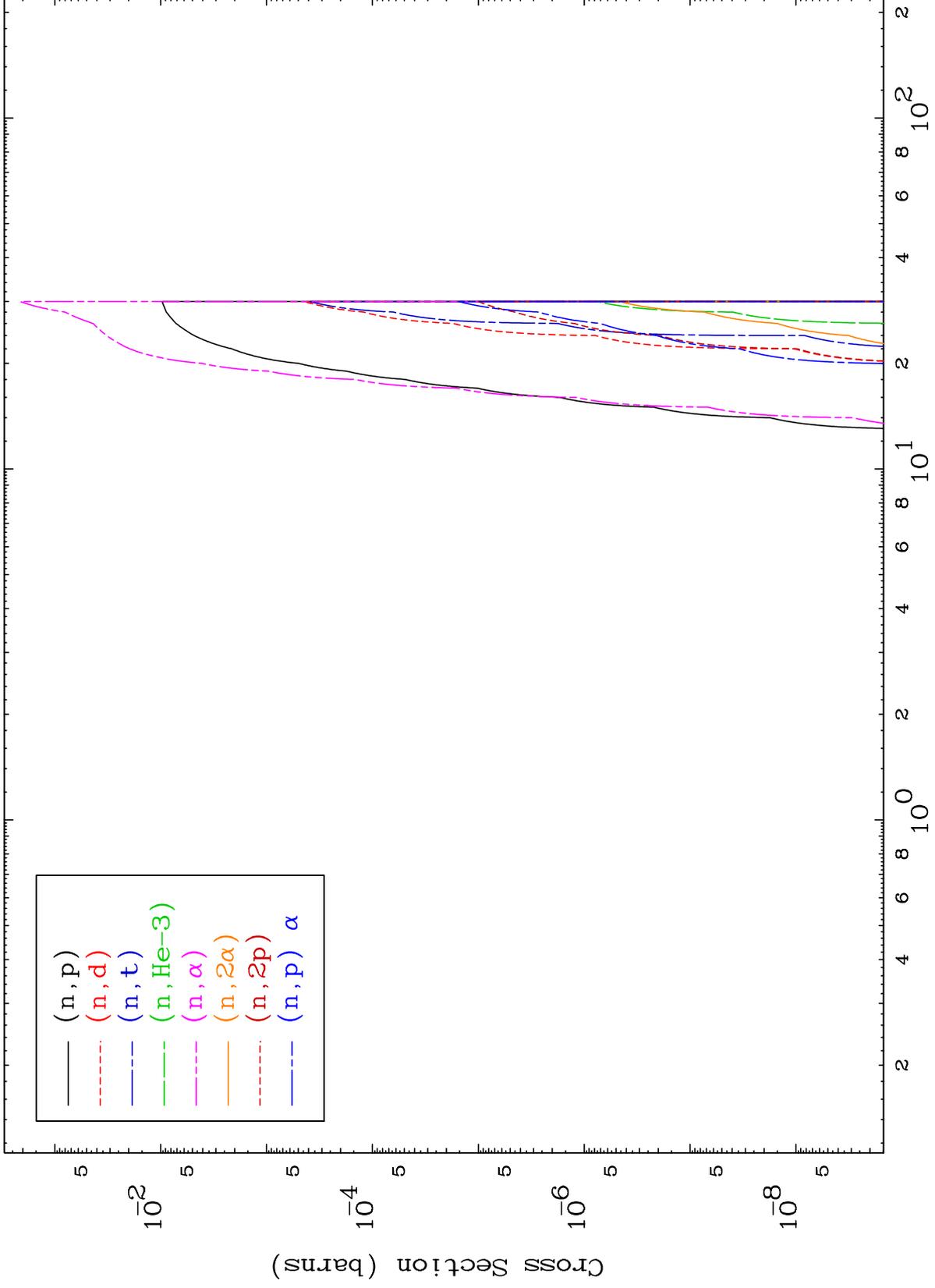
77-Ir-183



MAT 7701

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

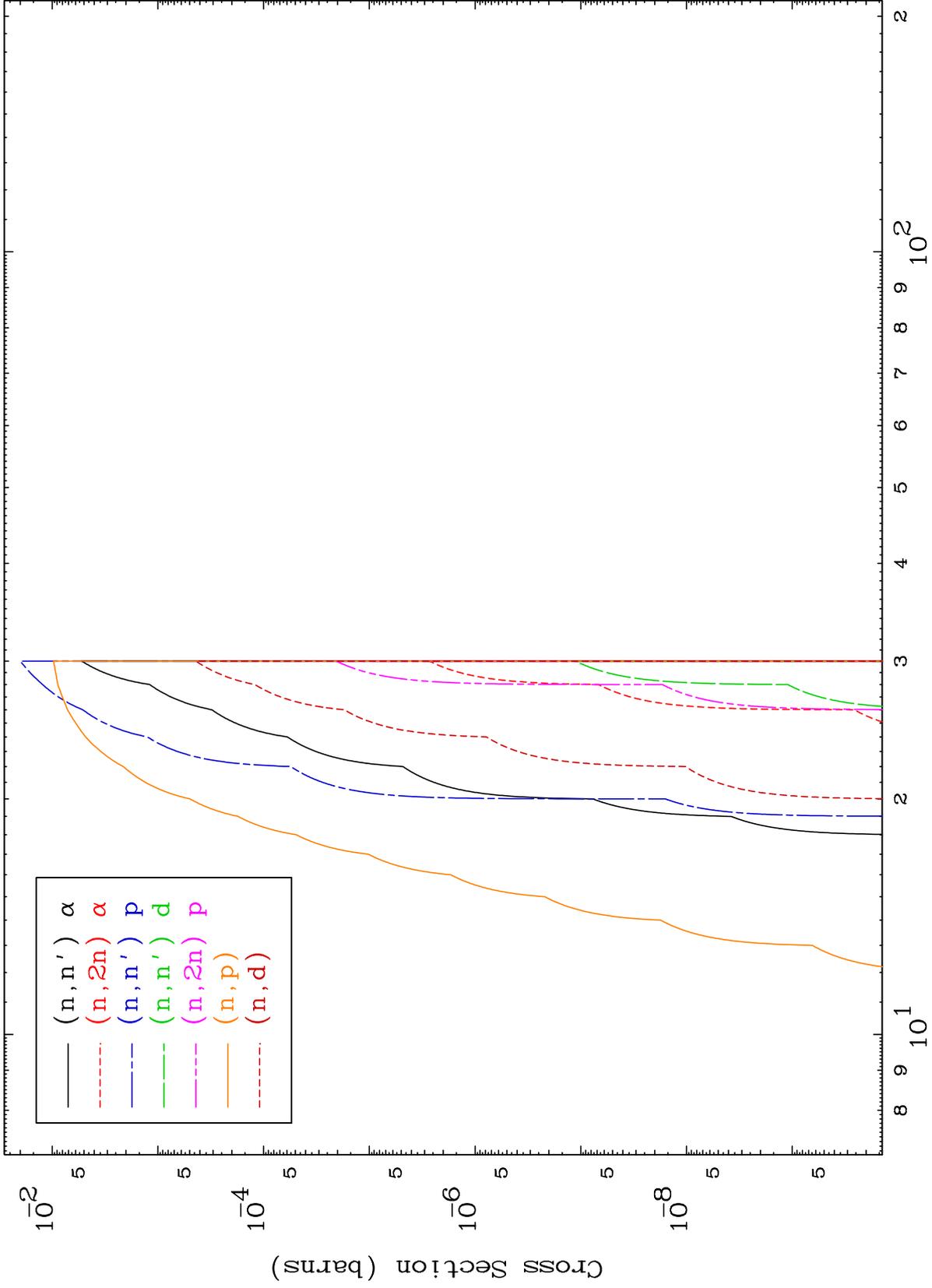
77-Ir-183

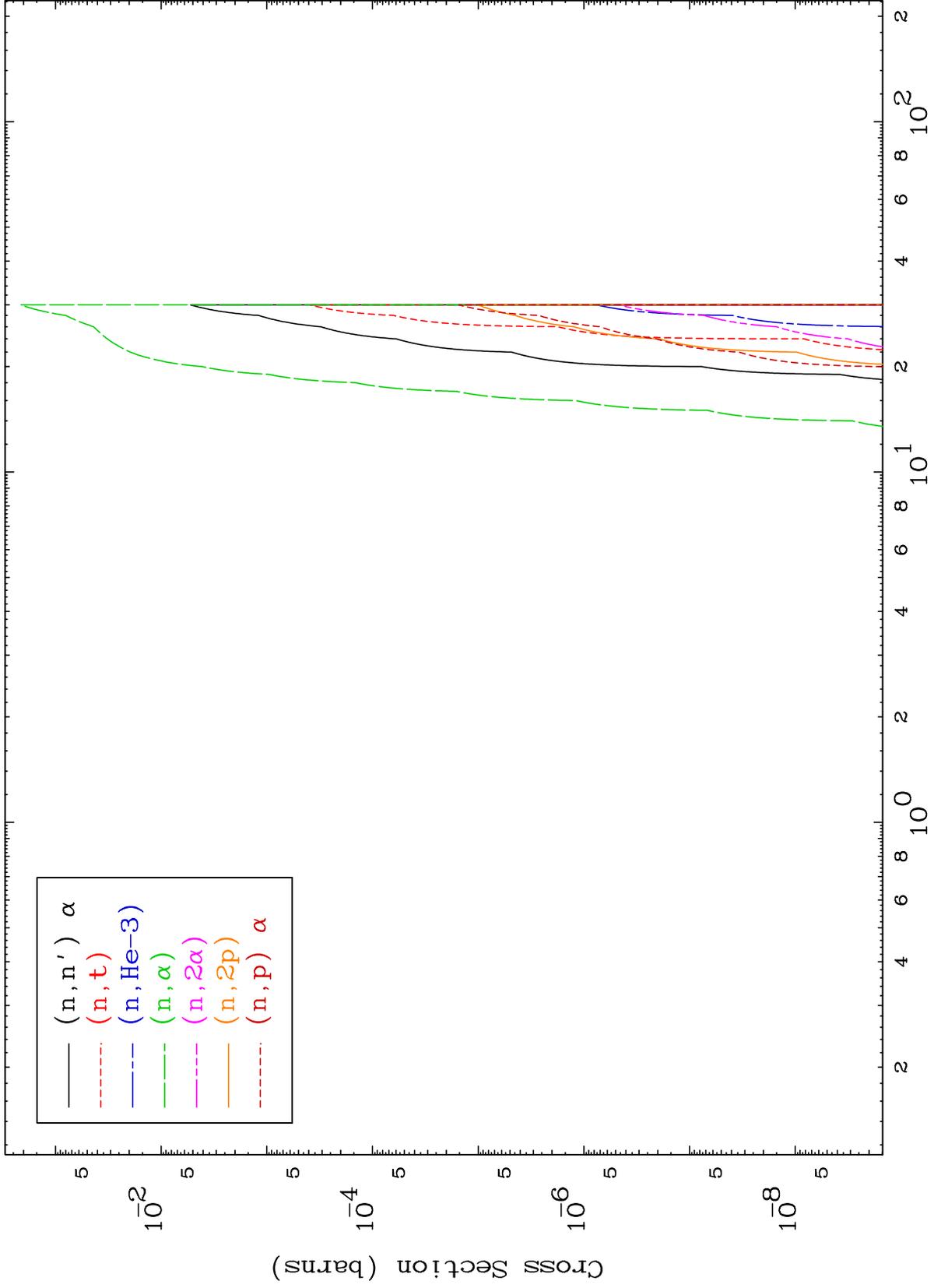


3

Incident Energy (MeV)

77-Ir-183

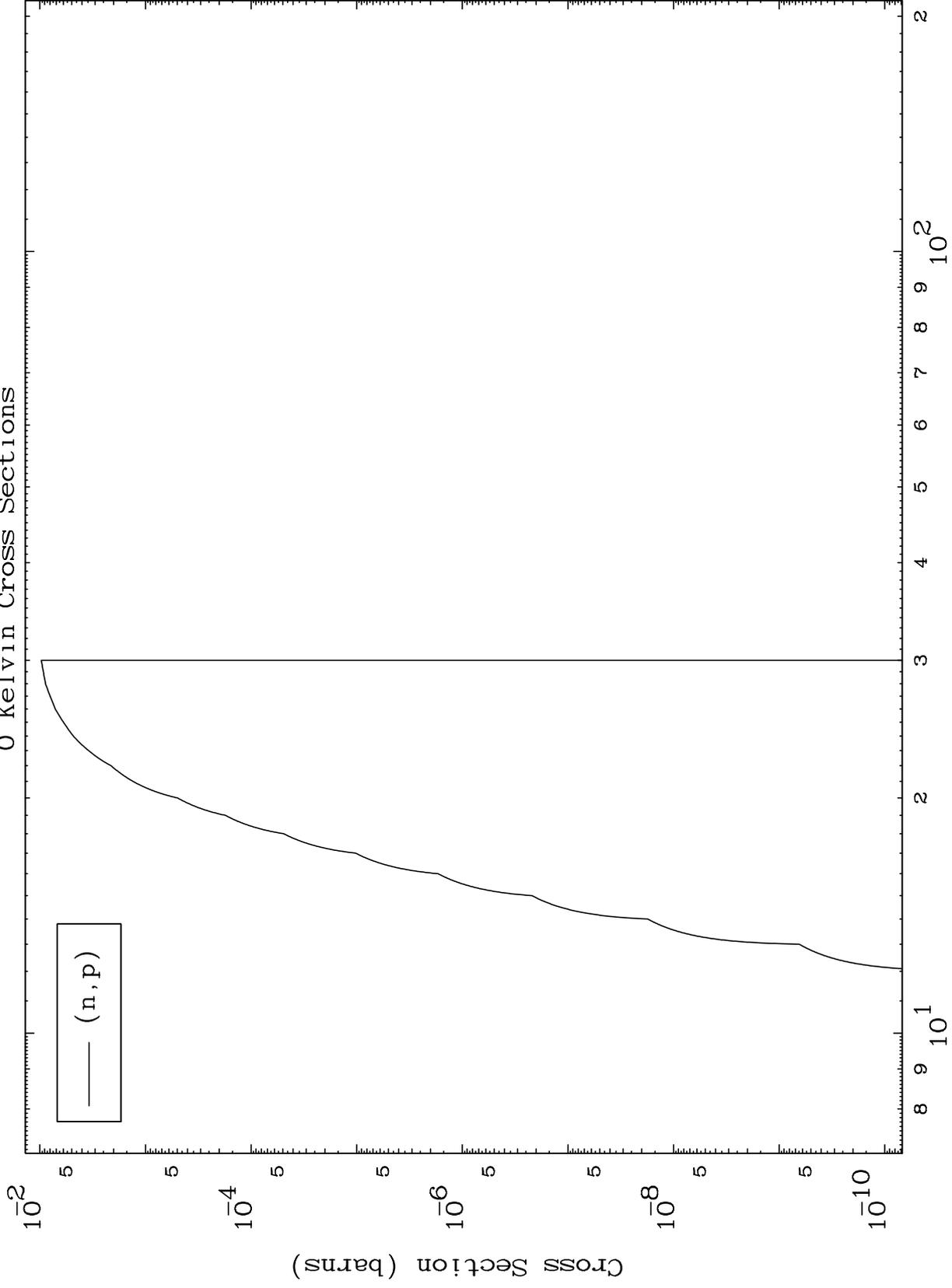




MAT 7701

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

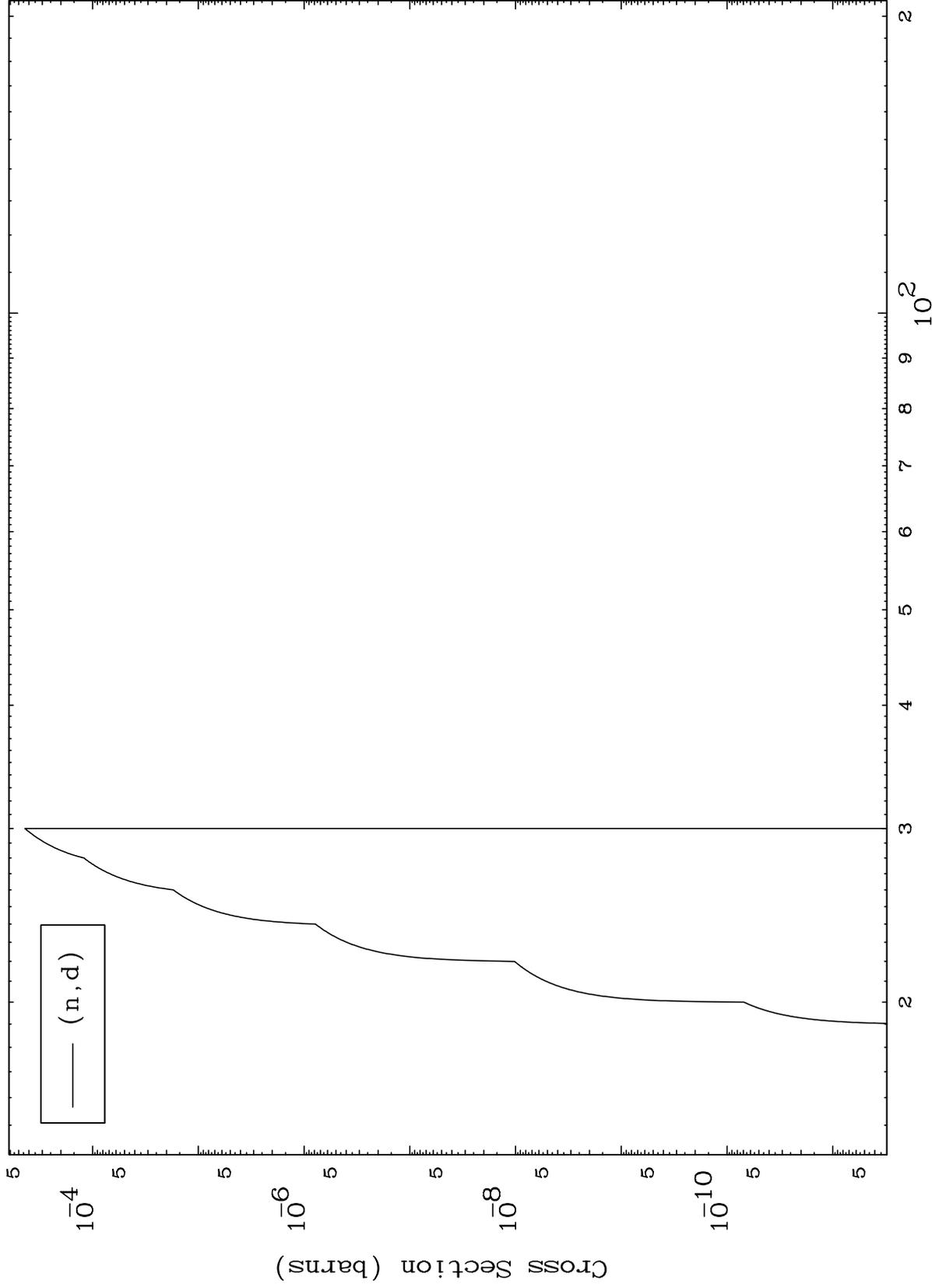
77-Ir-183

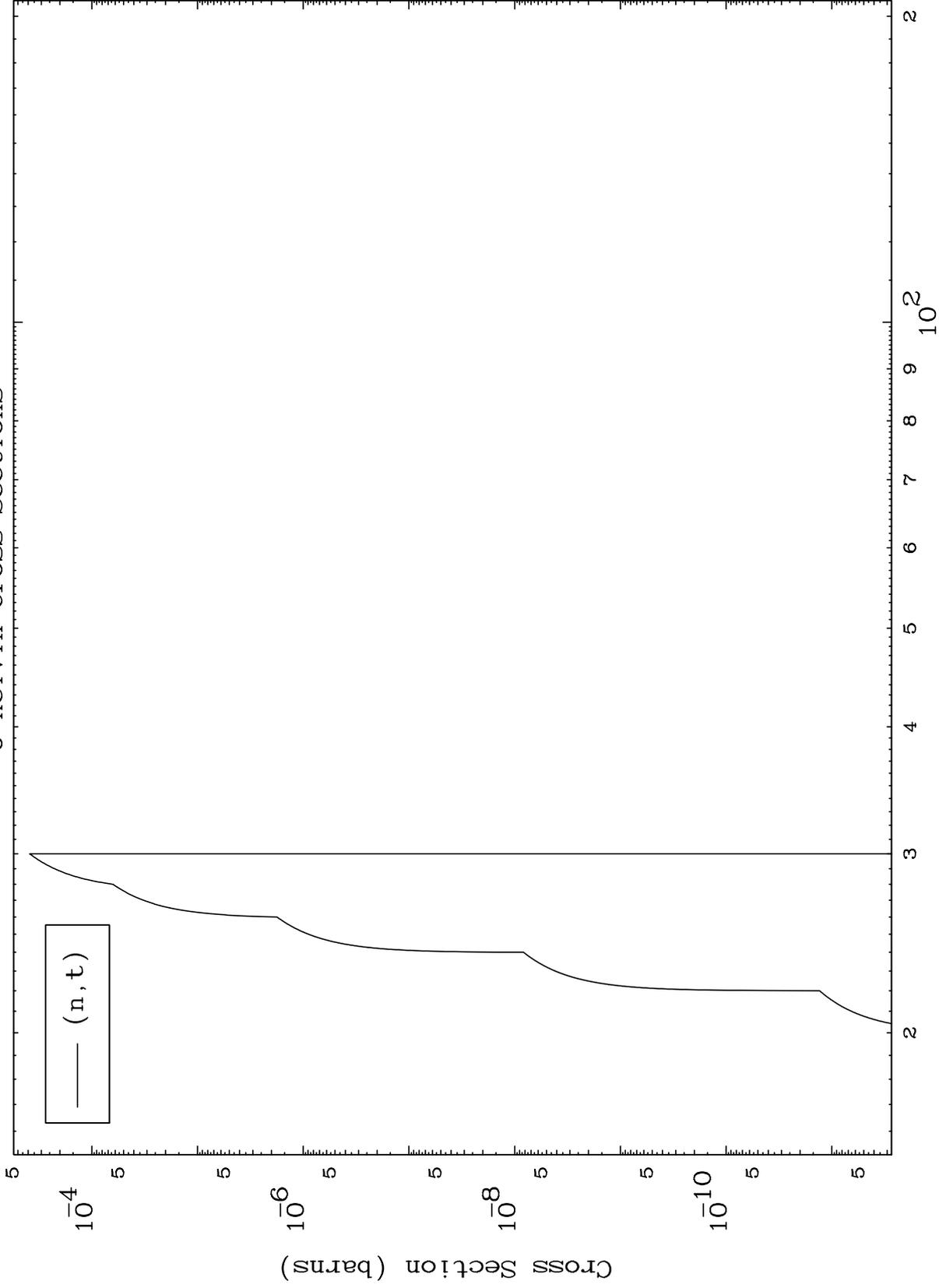


6

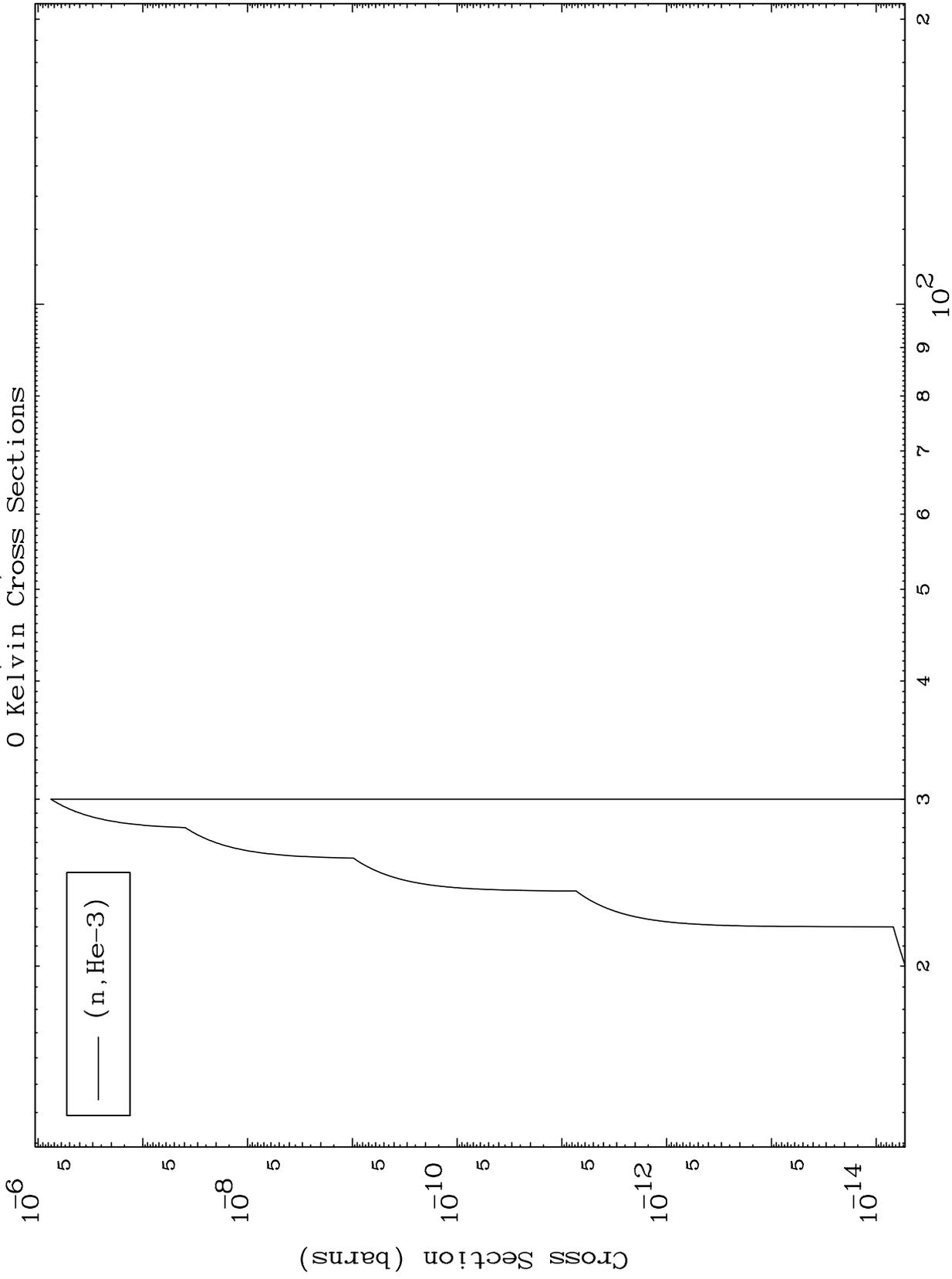
Incident Energy (MeV)

77-Ir-183





( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

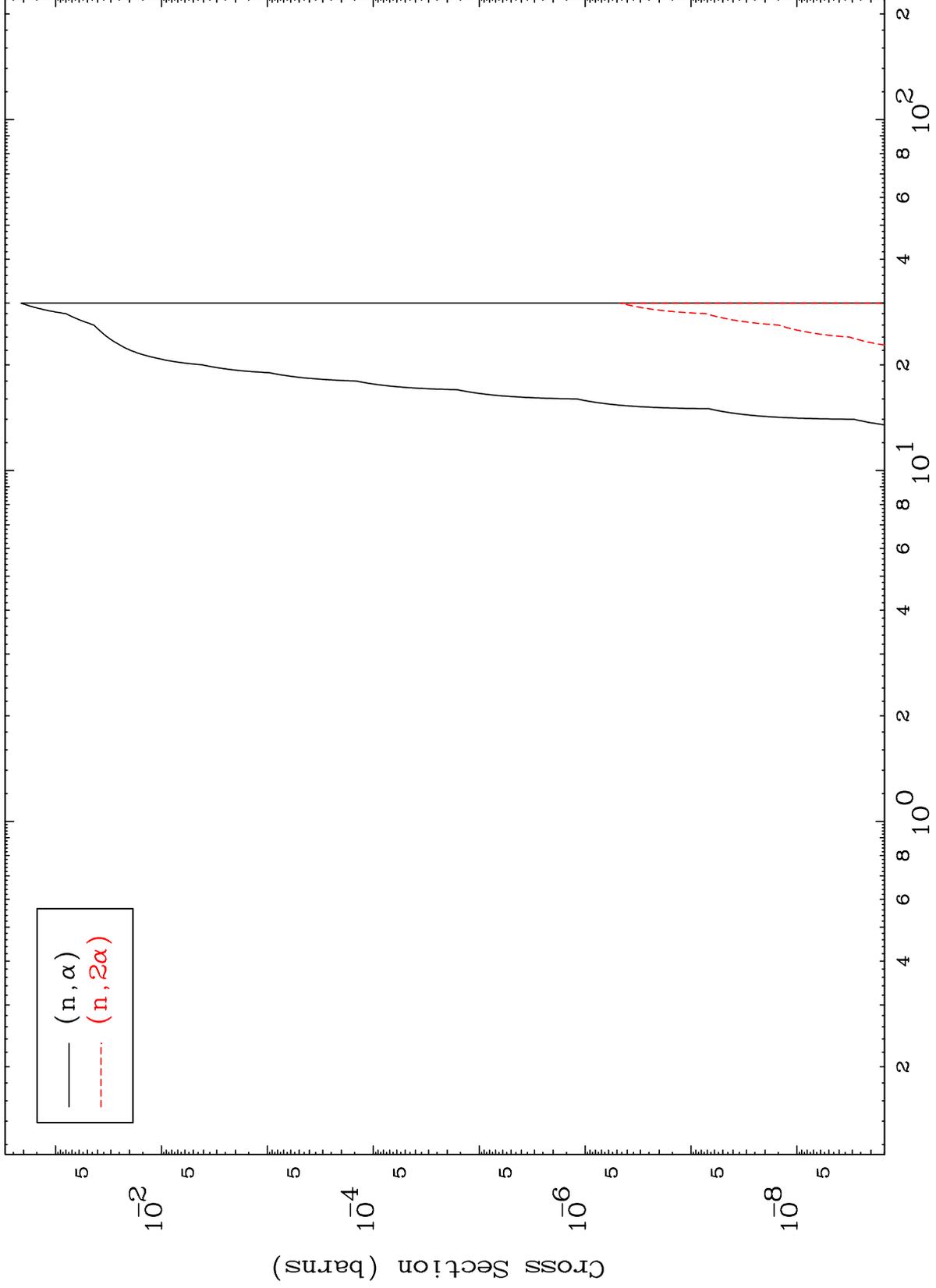


MAT 7701

( $\alpha, \alpha$ ) Levels

77-Ir-183

0 Kelvin Cross Sections



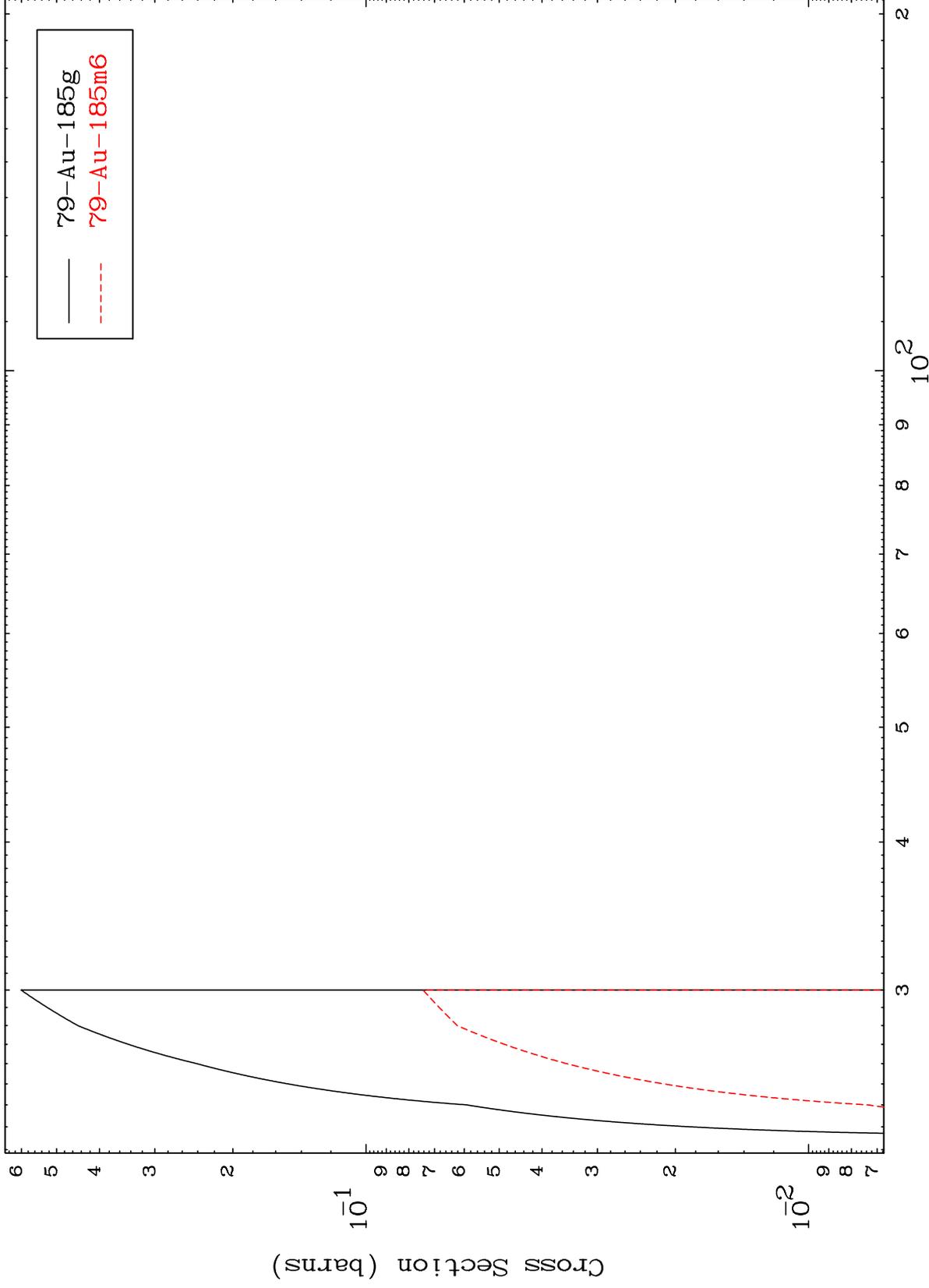
— ( $n, \alpha$ )  
- - - ( $n, 2\alpha$ )

10

Incident Energy (MeV)

77-Ir-183

(n,2n)  
Radionuclide Production Cross Section

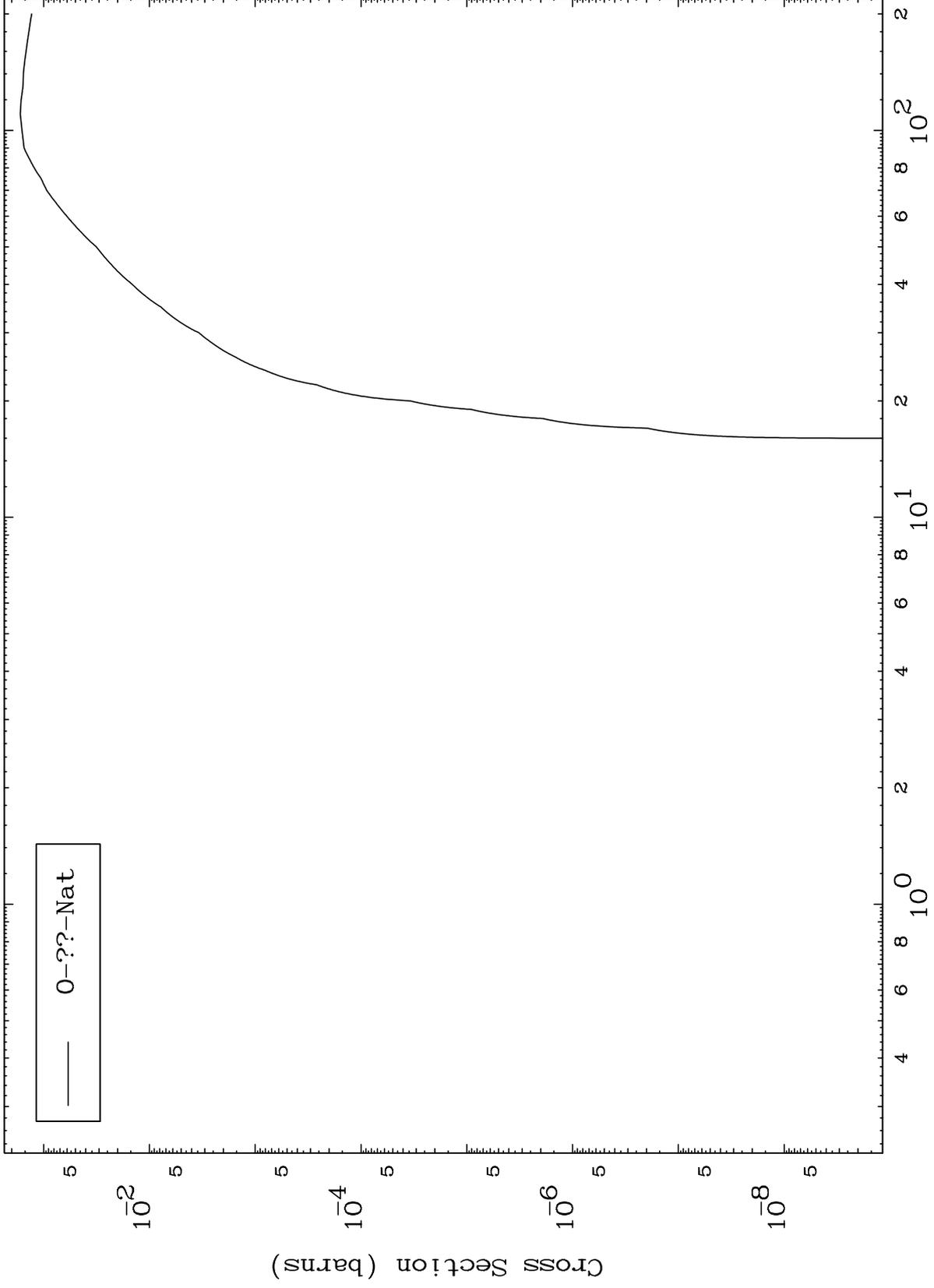


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Fission

<sup>77</sup>Ir-183

Radionuclide Production Cross Section

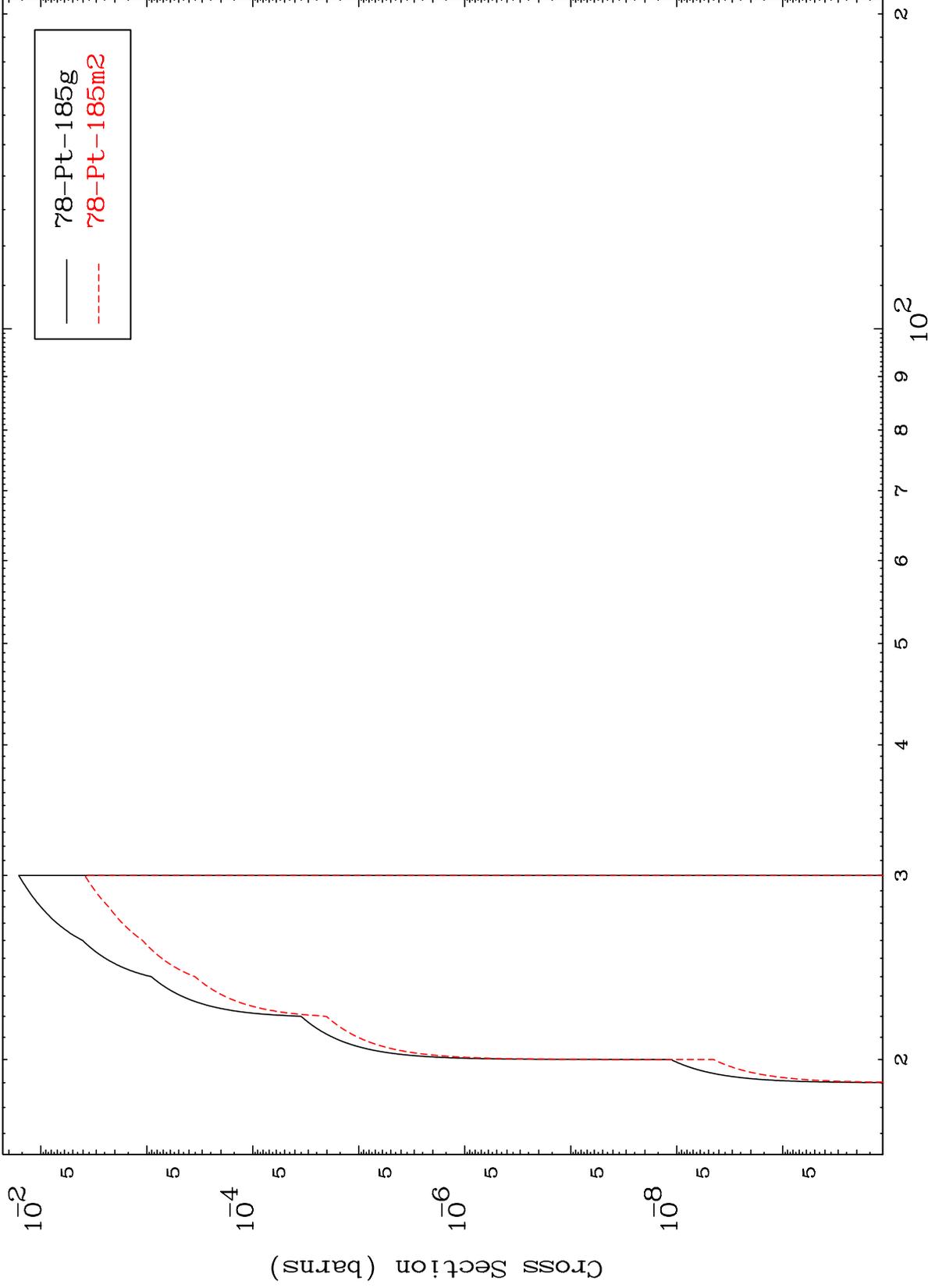


12

Incident Energy (MeV)

<sup>77</sup>Ir-183

Radionuclide Production Cross Section

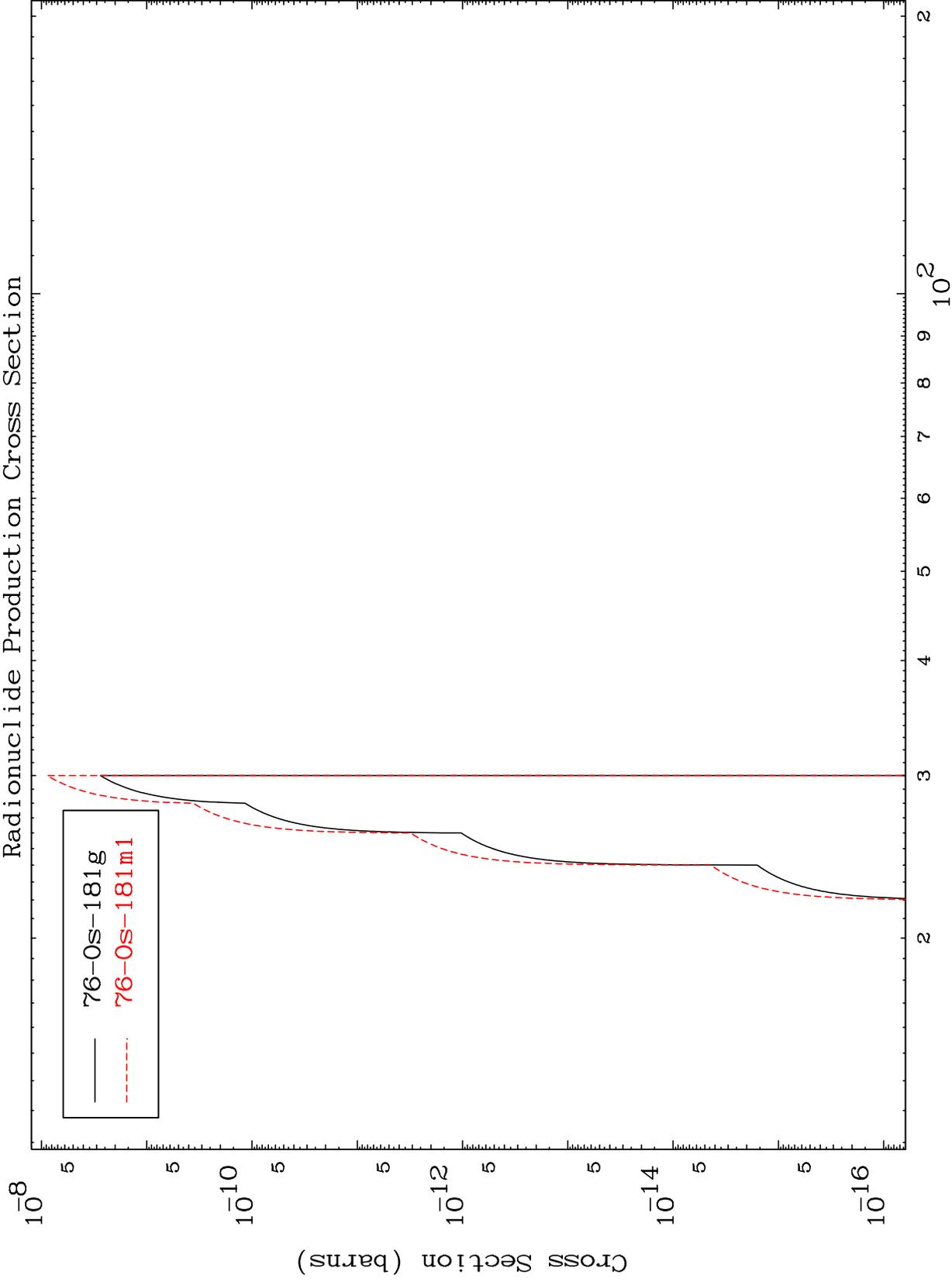


MAT 7701

<sup>77</sup>Ir-183

(n,n') p α

Radionuclide Production Cross Section



<sup>77</sup>Ir-183

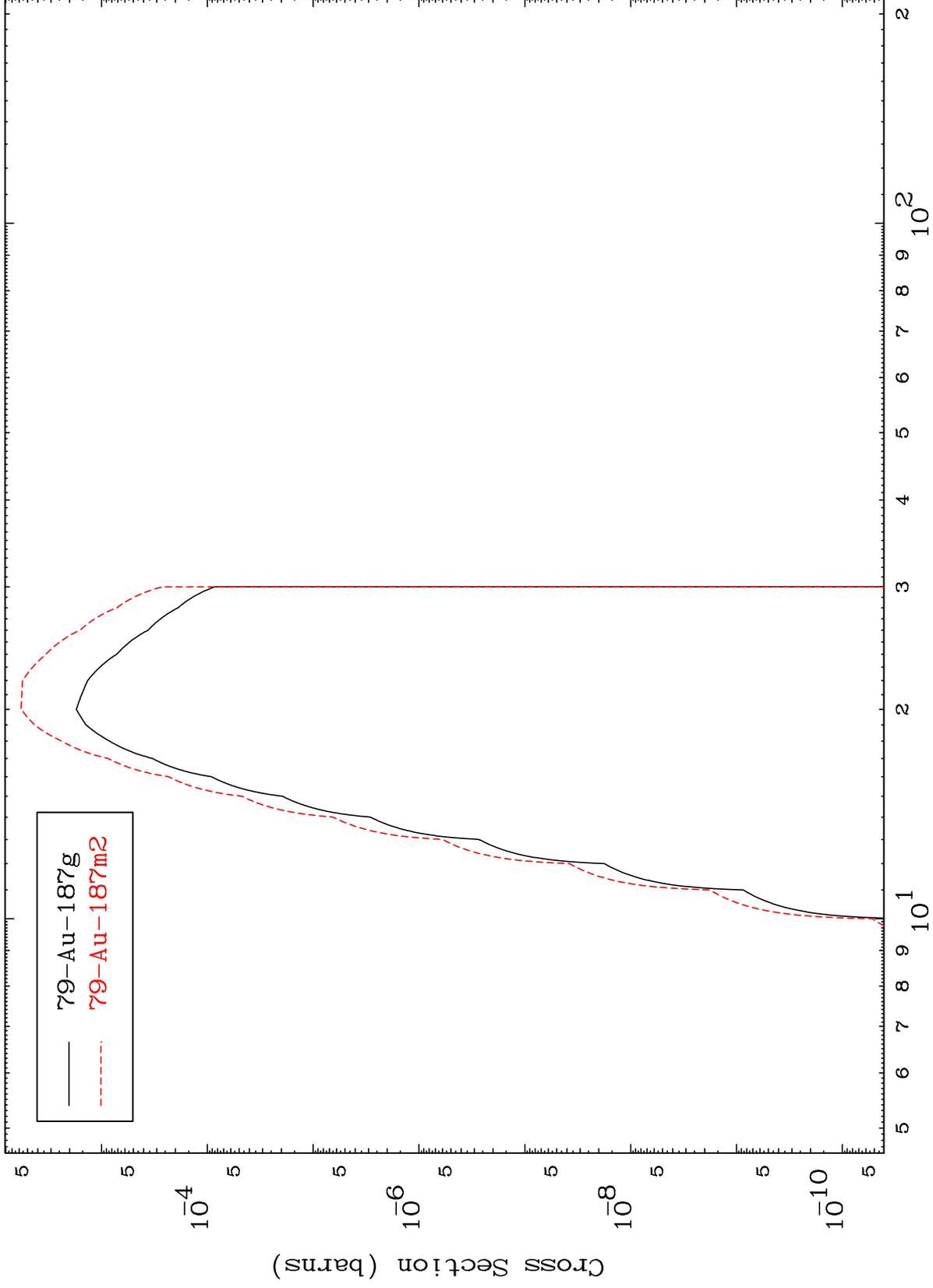
Incident Energy (MeV)

14

MAT 7701

77-Ir-183

(n,γ)  
Radionuclide Production Cross Section



15

Incident Energy (MeV)

77-Ir-183

Radionuclide Production Cross Section

