

Program EVALPLOT
(Version 2021-1)

by

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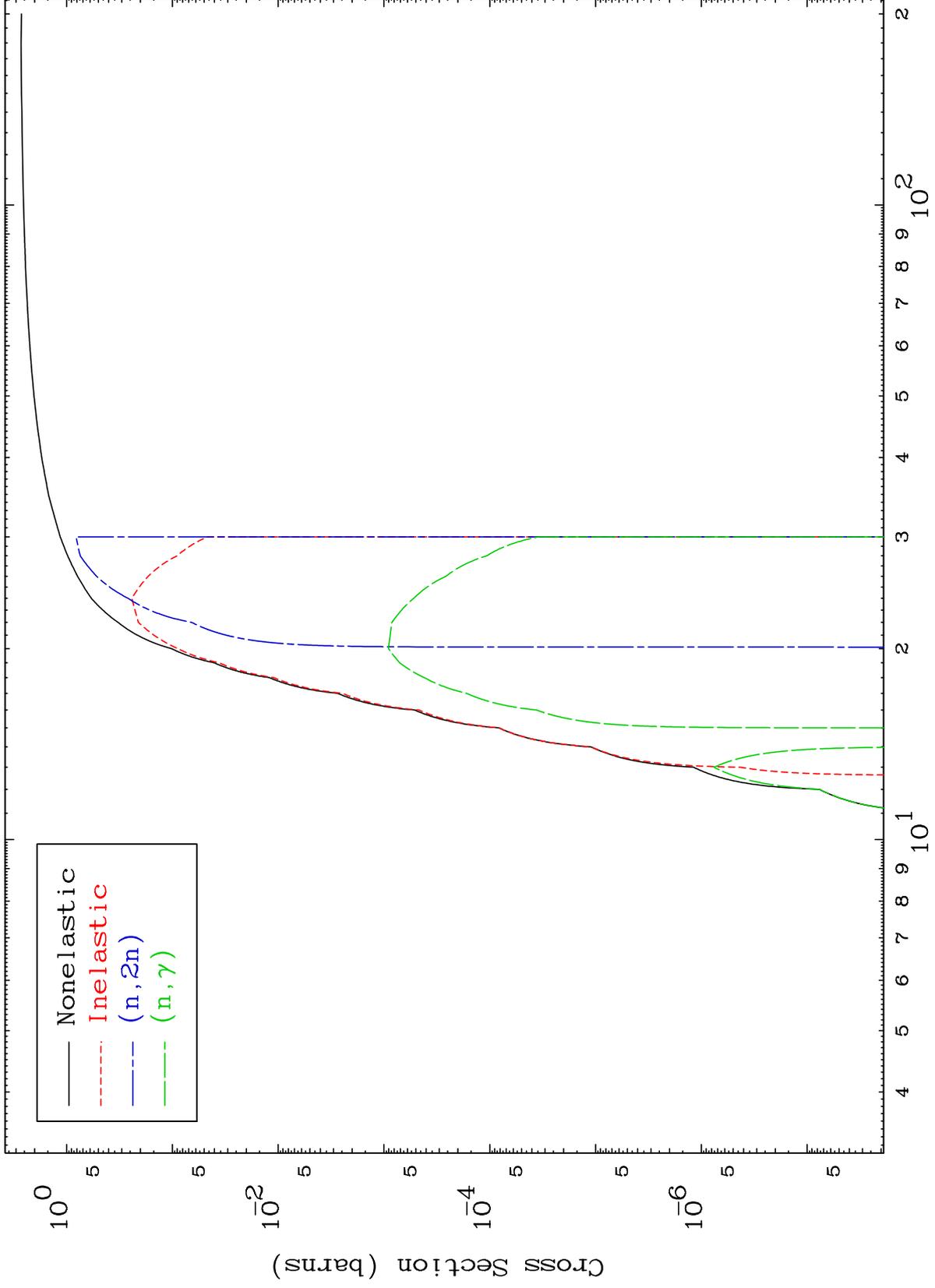
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Press Mouse Button to Start

MAT 7713

0 Kelvin Cross Sections

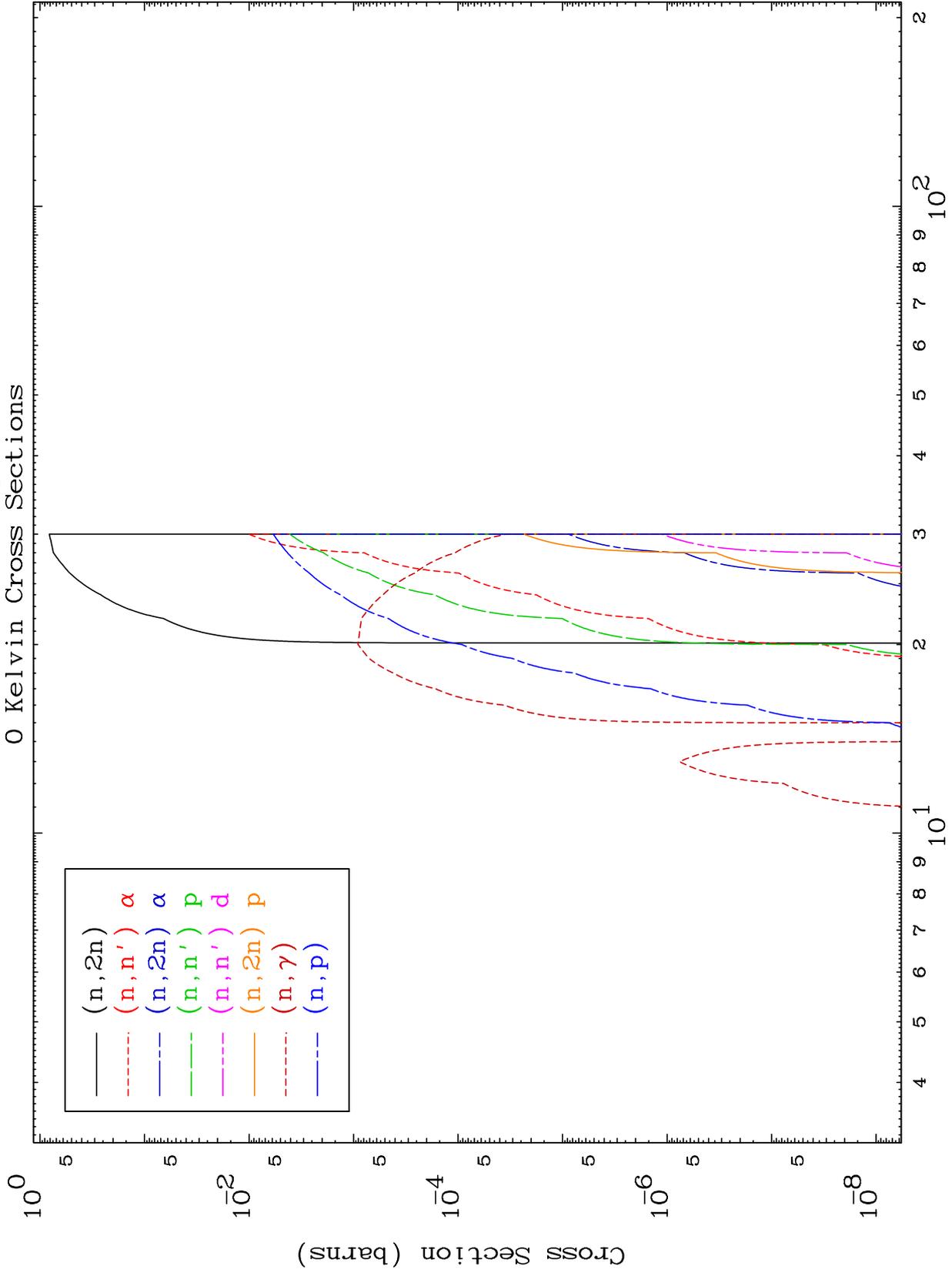
77-Ir-187

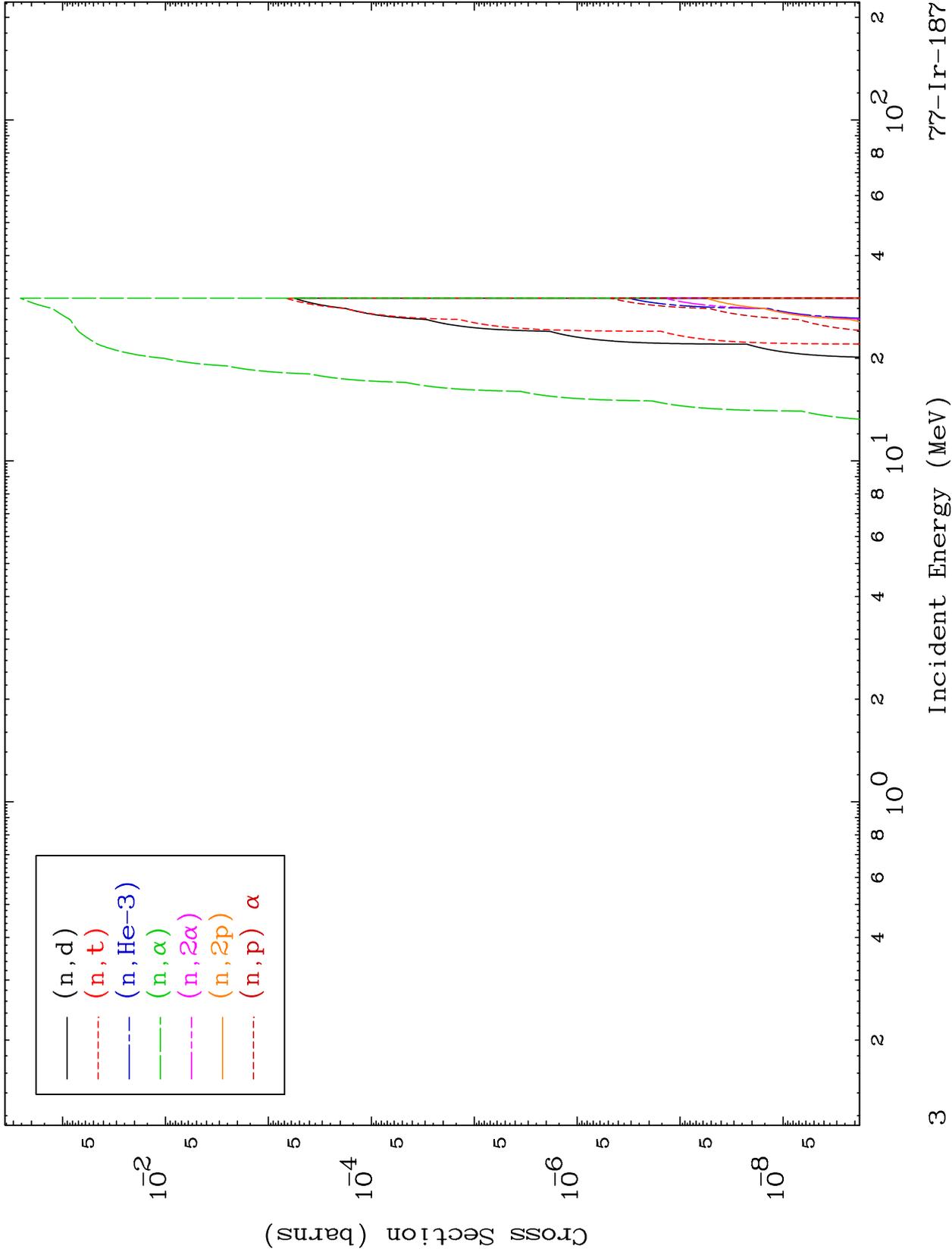


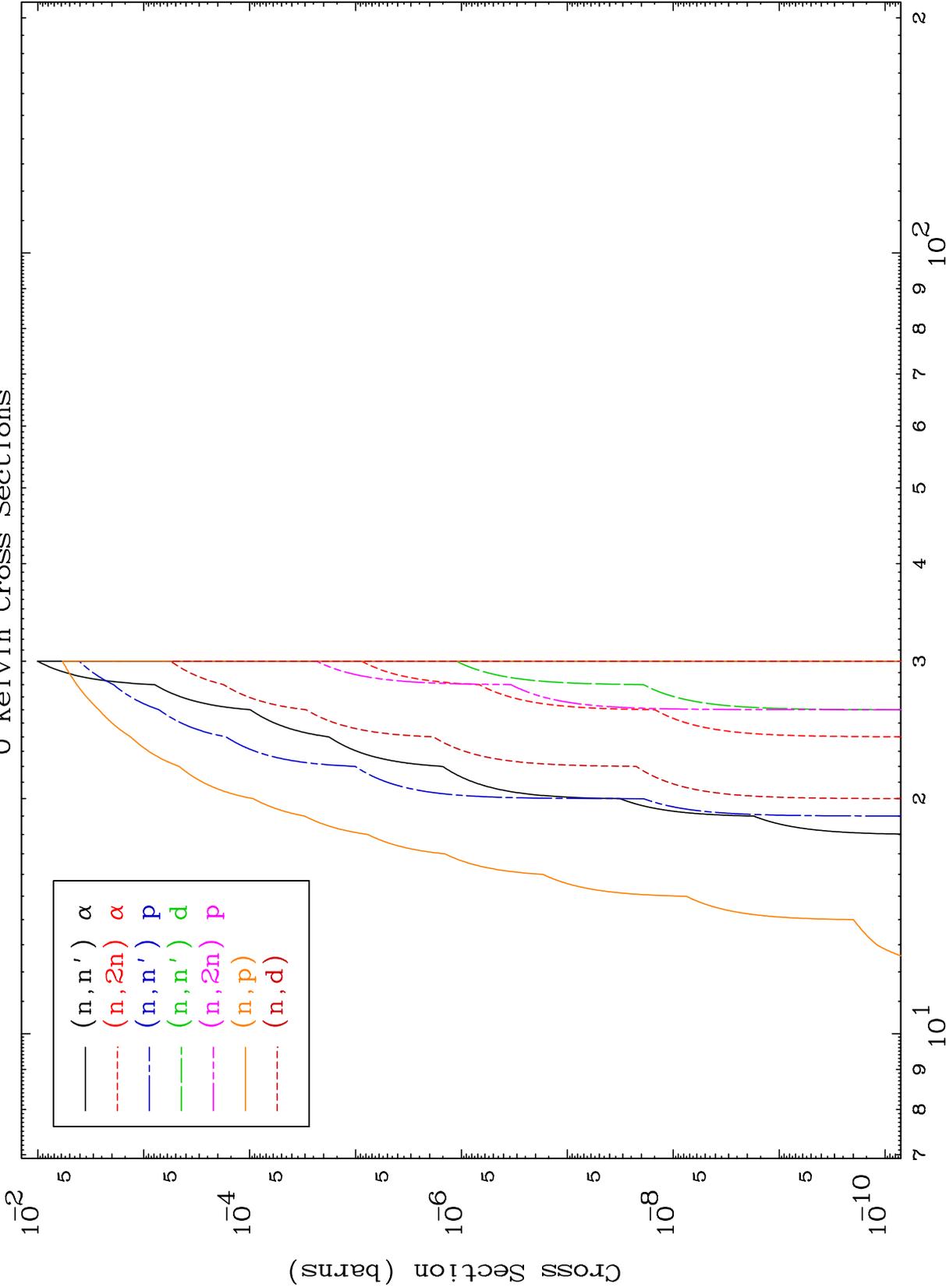
MAT 7713

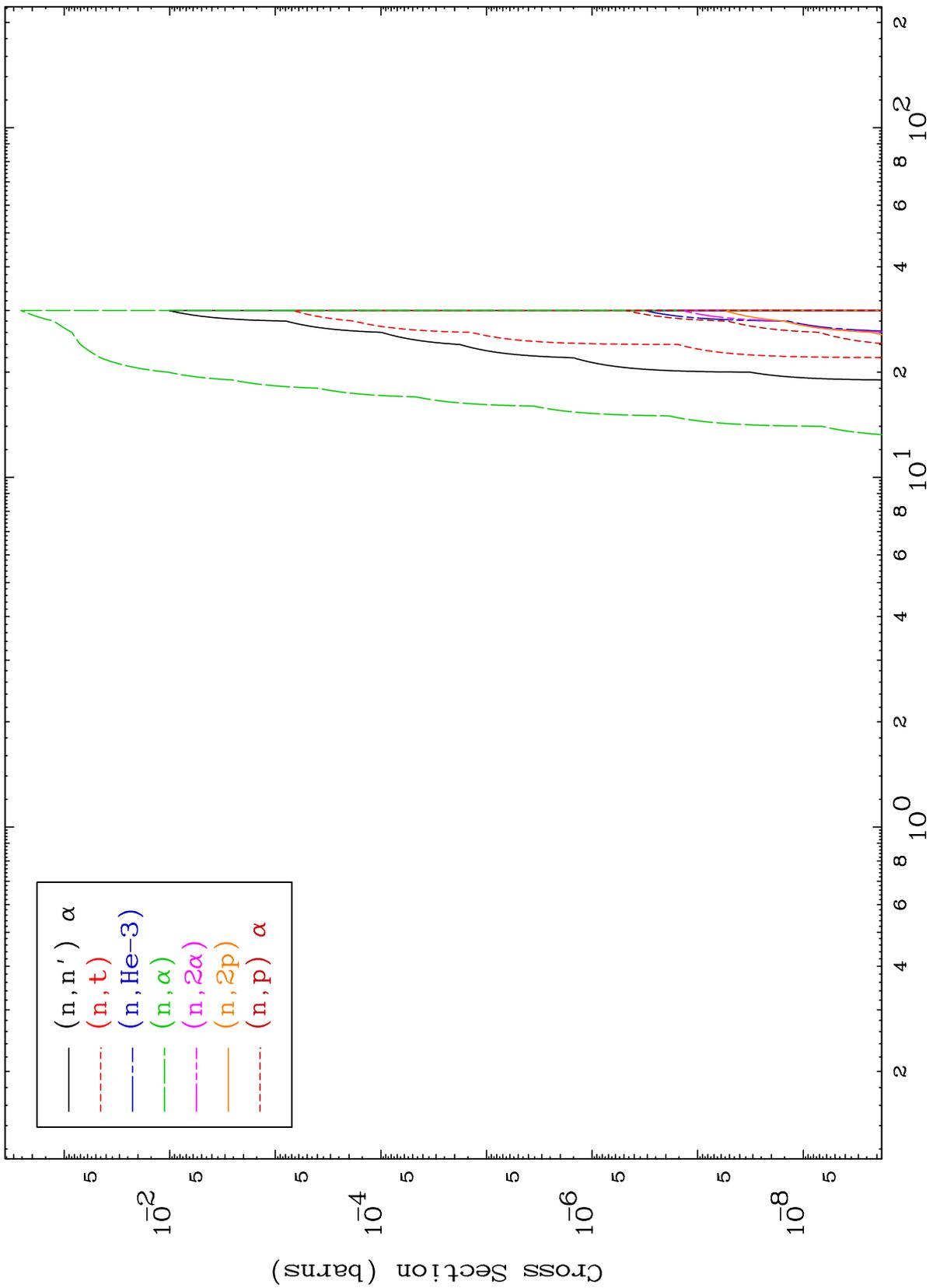
α Neutron Absorption
0 Kelvin Cross Sections

77-Ir-187





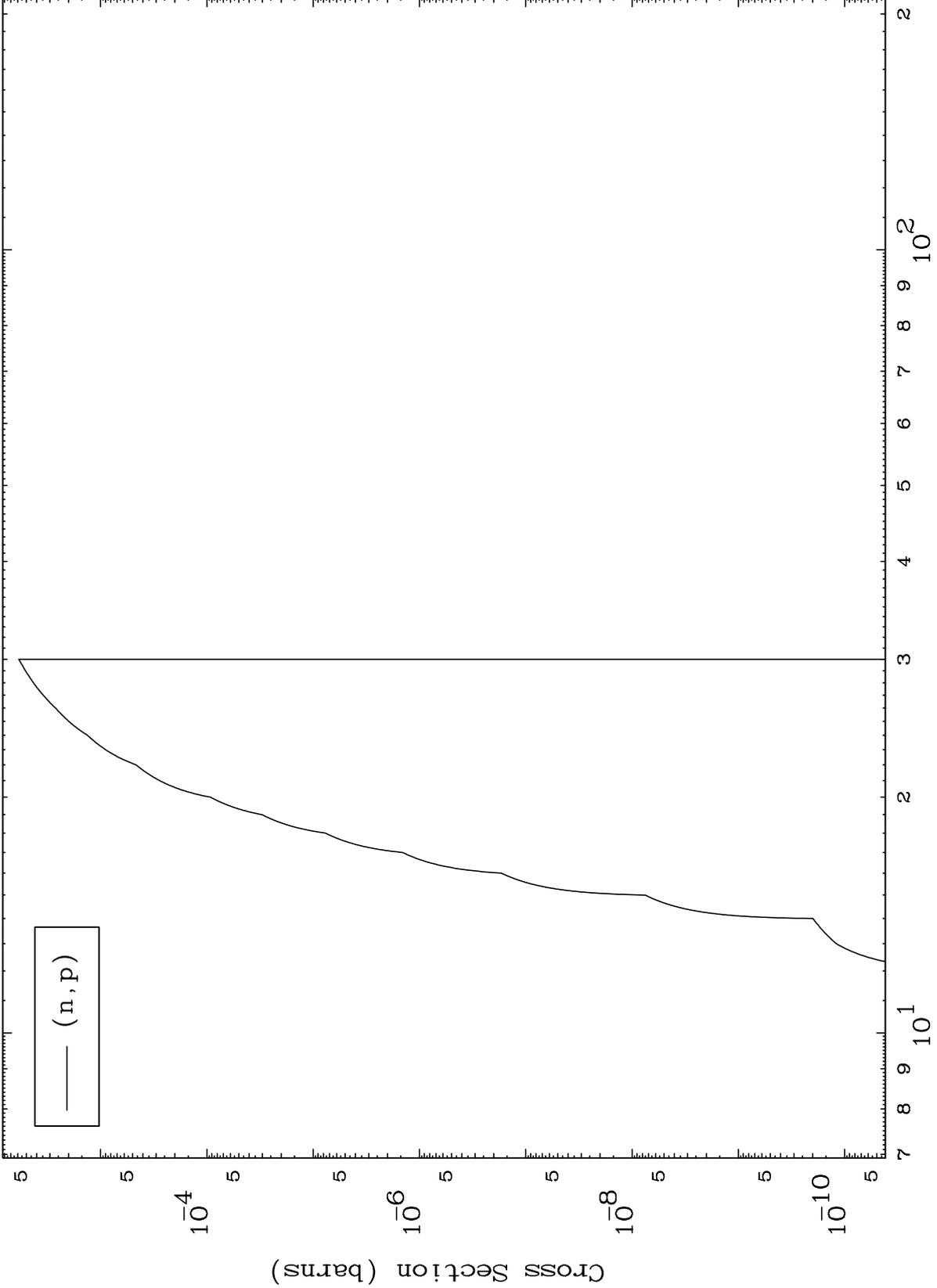




MAT 7713

(α, p) Levels
0 Kelvin Cross Sections

77-Ir-187



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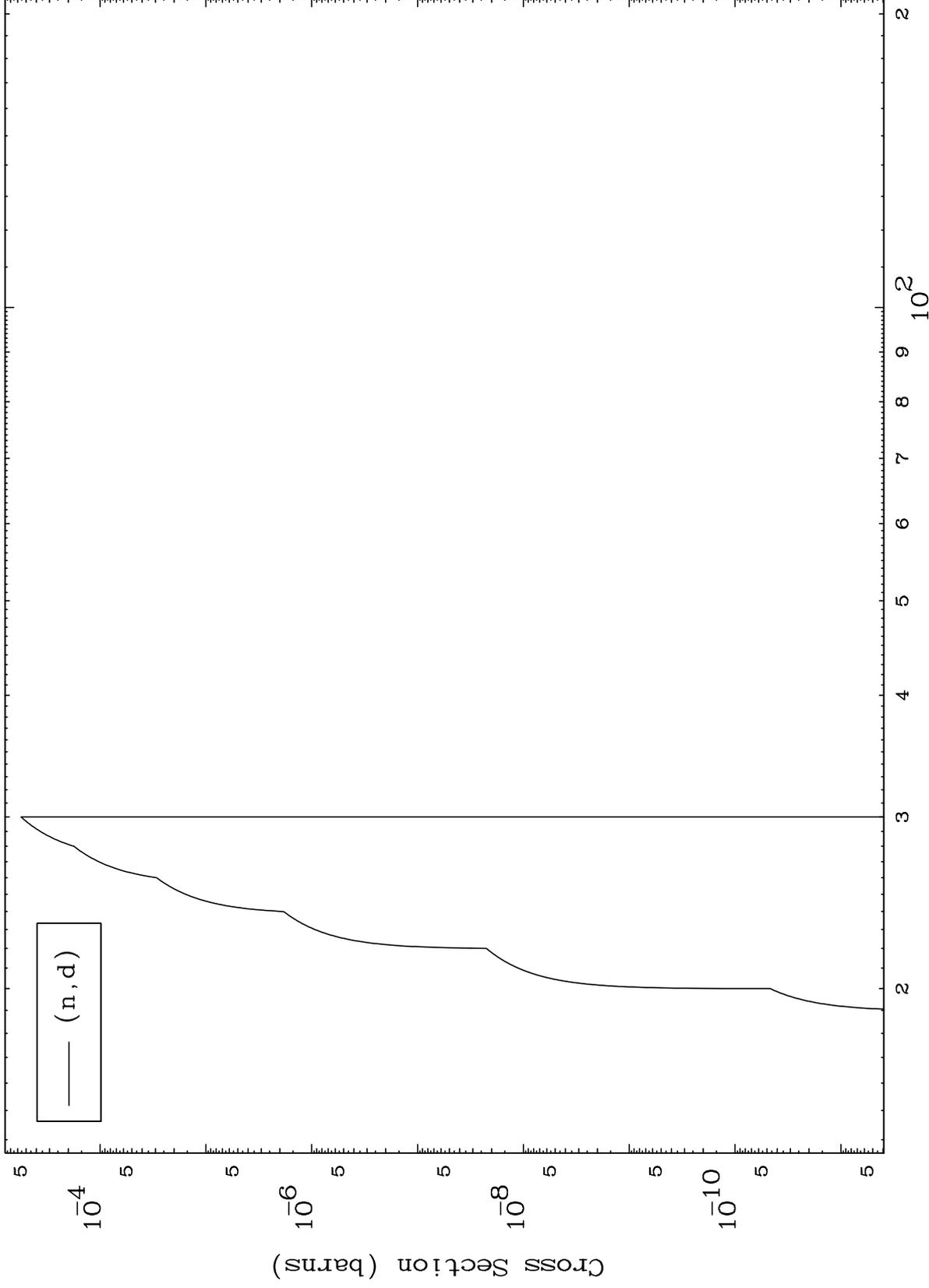
Incident Energy (MeV)

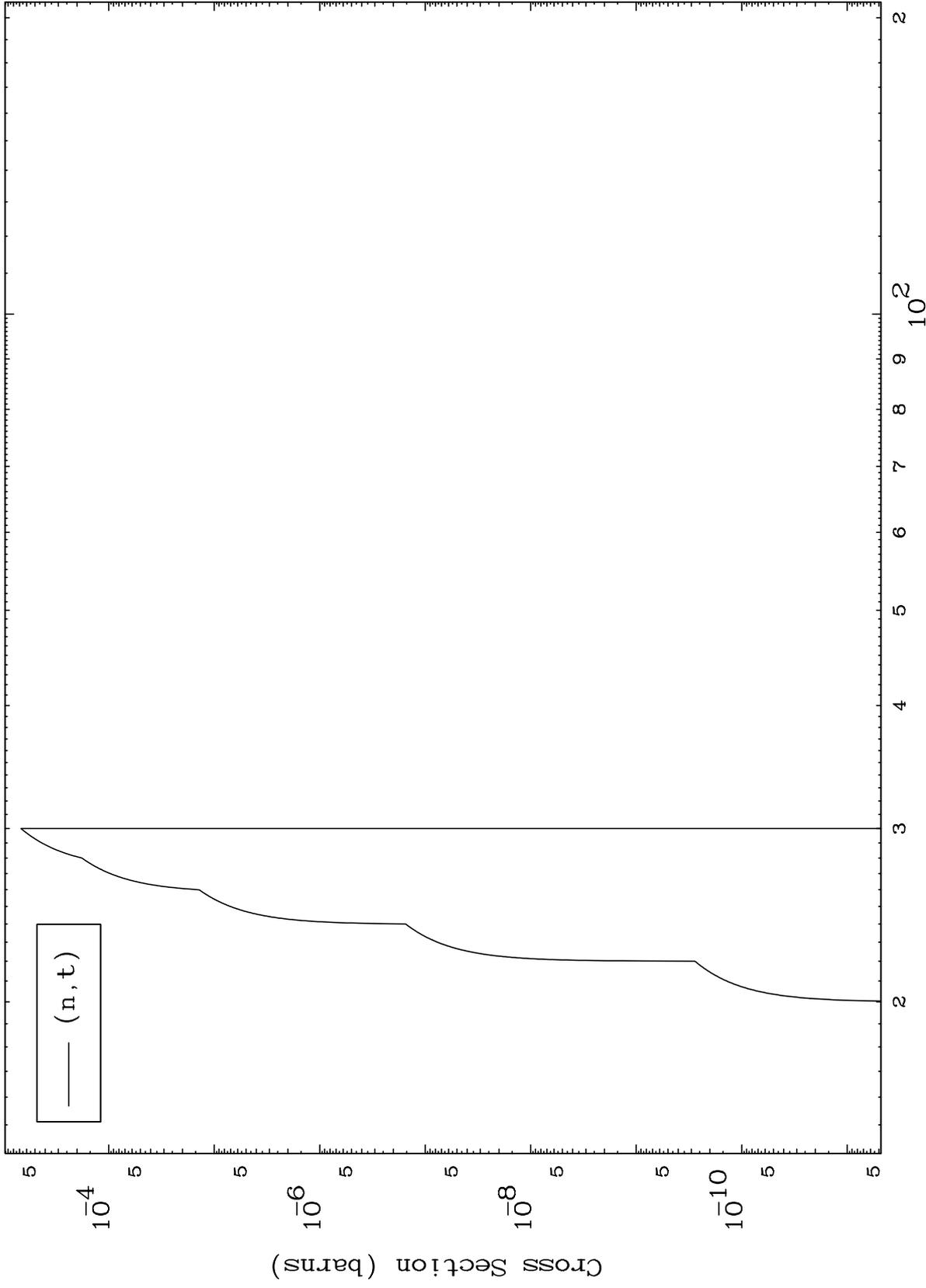
77-Ir-187

MAT 7713

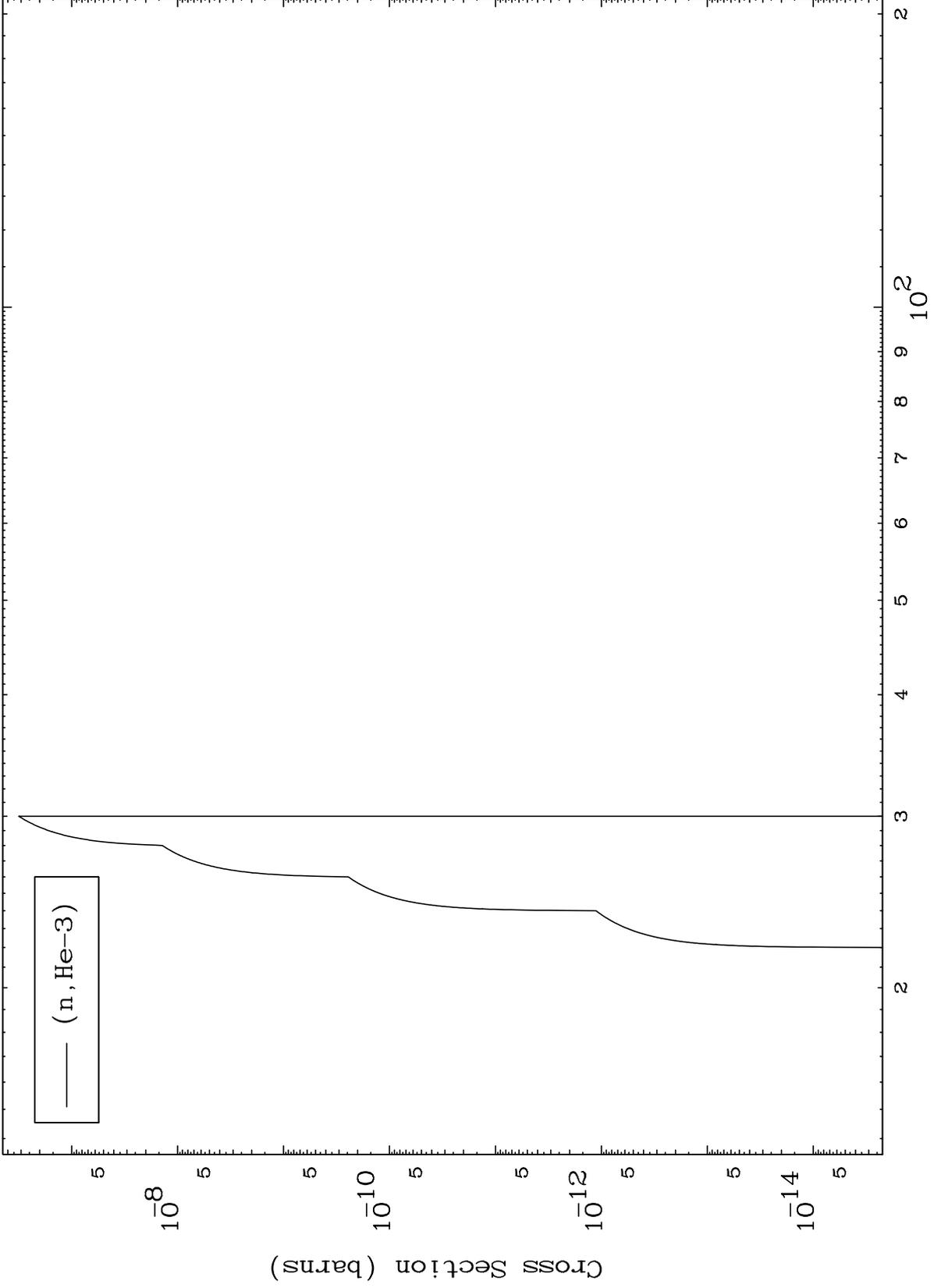
(α, d) Levels
0 Kelvin Cross Sections

77-Ir-187





($\alpha, \text{He}3$) Levels
0 Kelvin Cross Sections

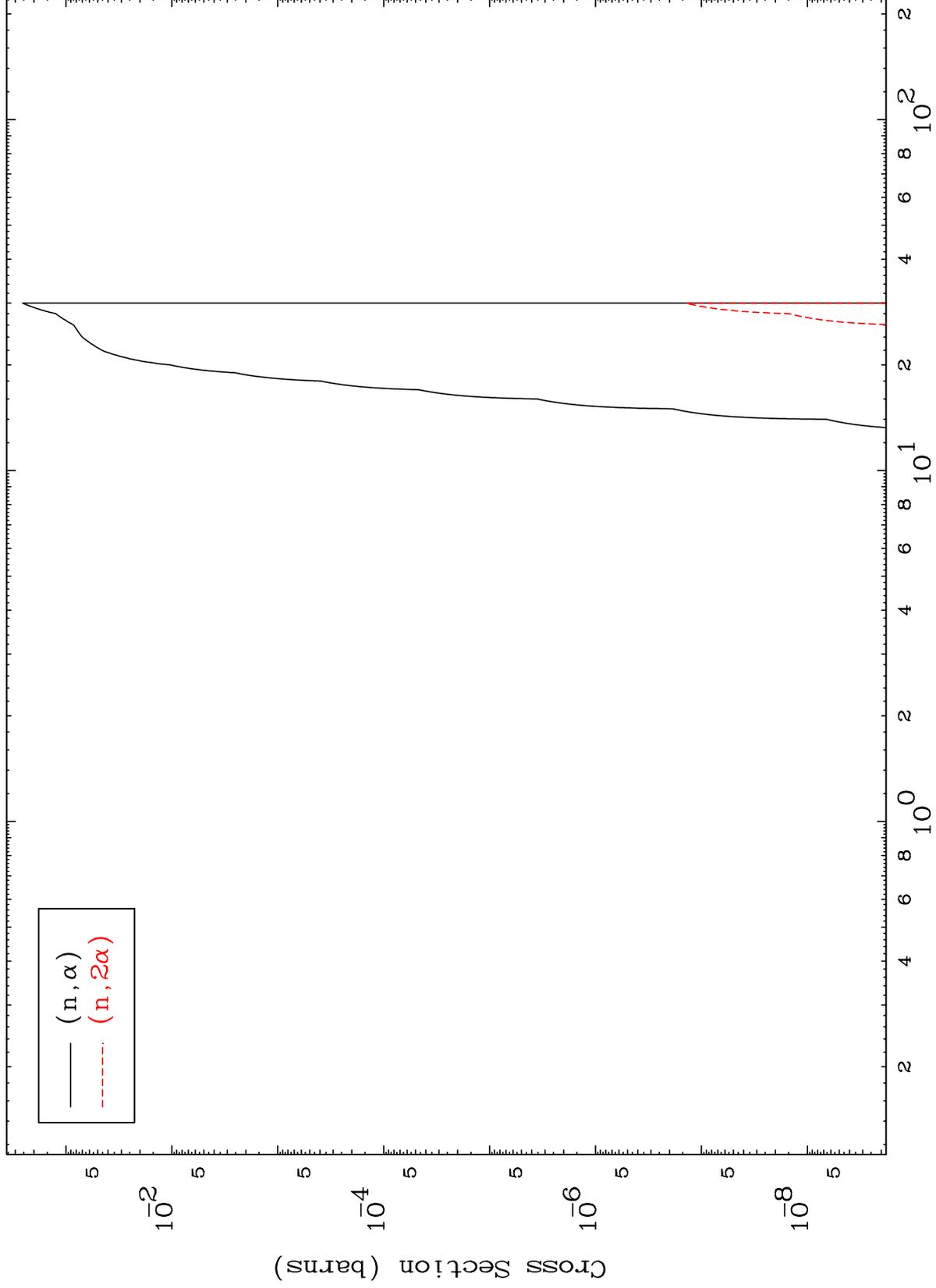


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(α, α) Levels

77-Ir-187

0 Kelvin Cross Sections



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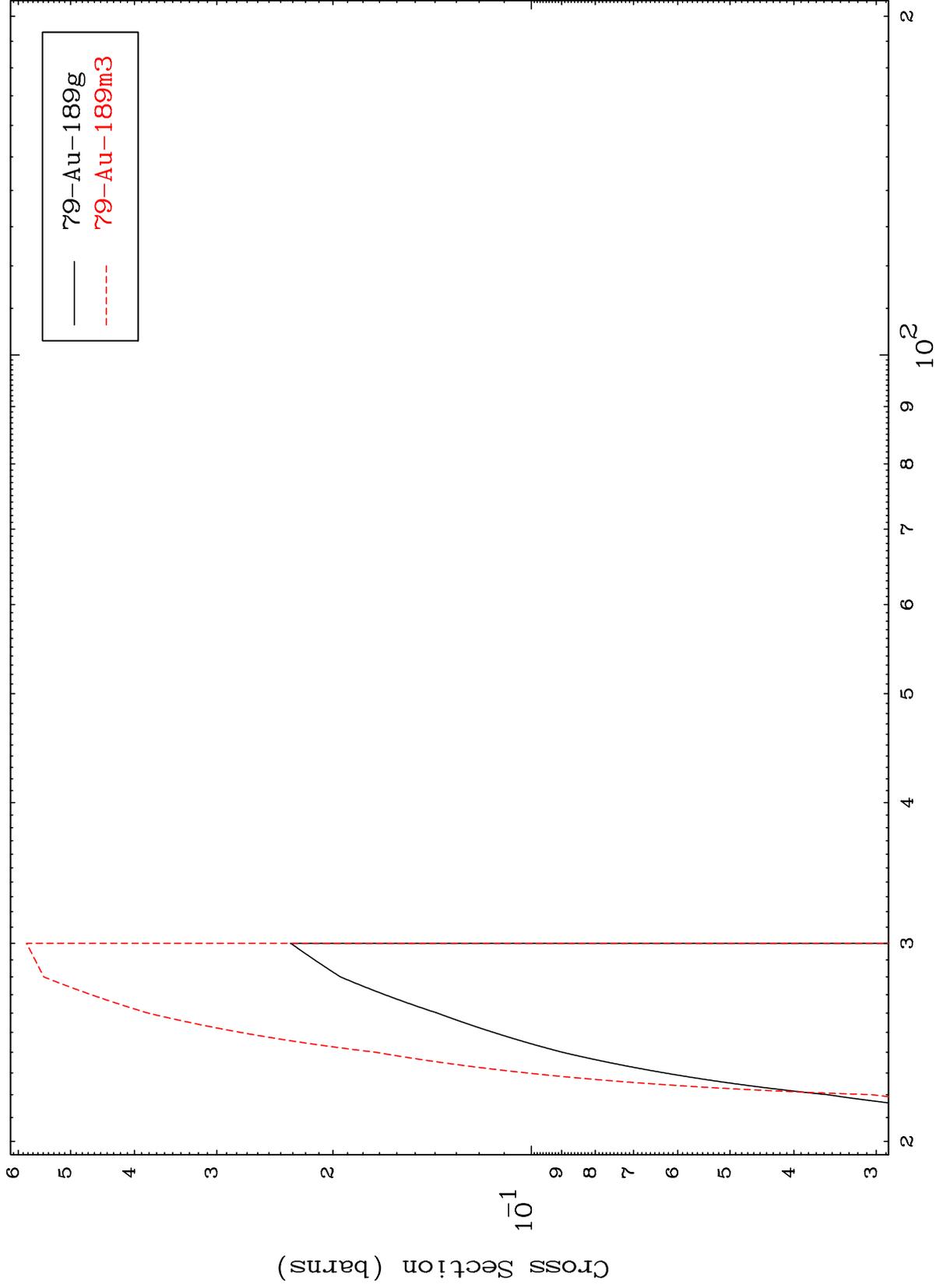
Incident Energy (MeV)

77-Ir-187

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77-Ir-187

(n,2n)
Radionuclide Production Cross Section



77-Ir-187

Incident Energy (MeV)

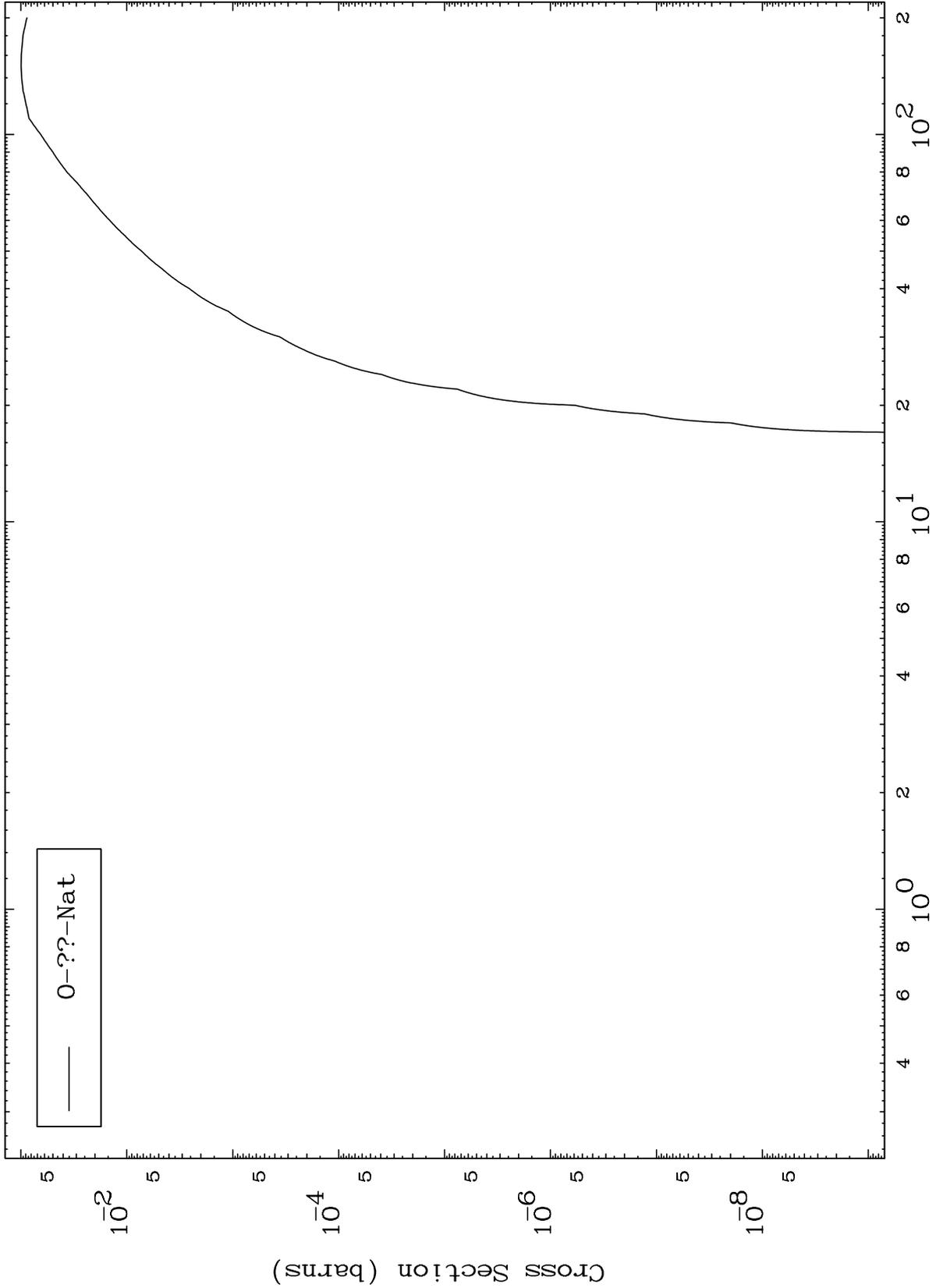
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MAT 7713

Fission

⁷⁷Ir-187

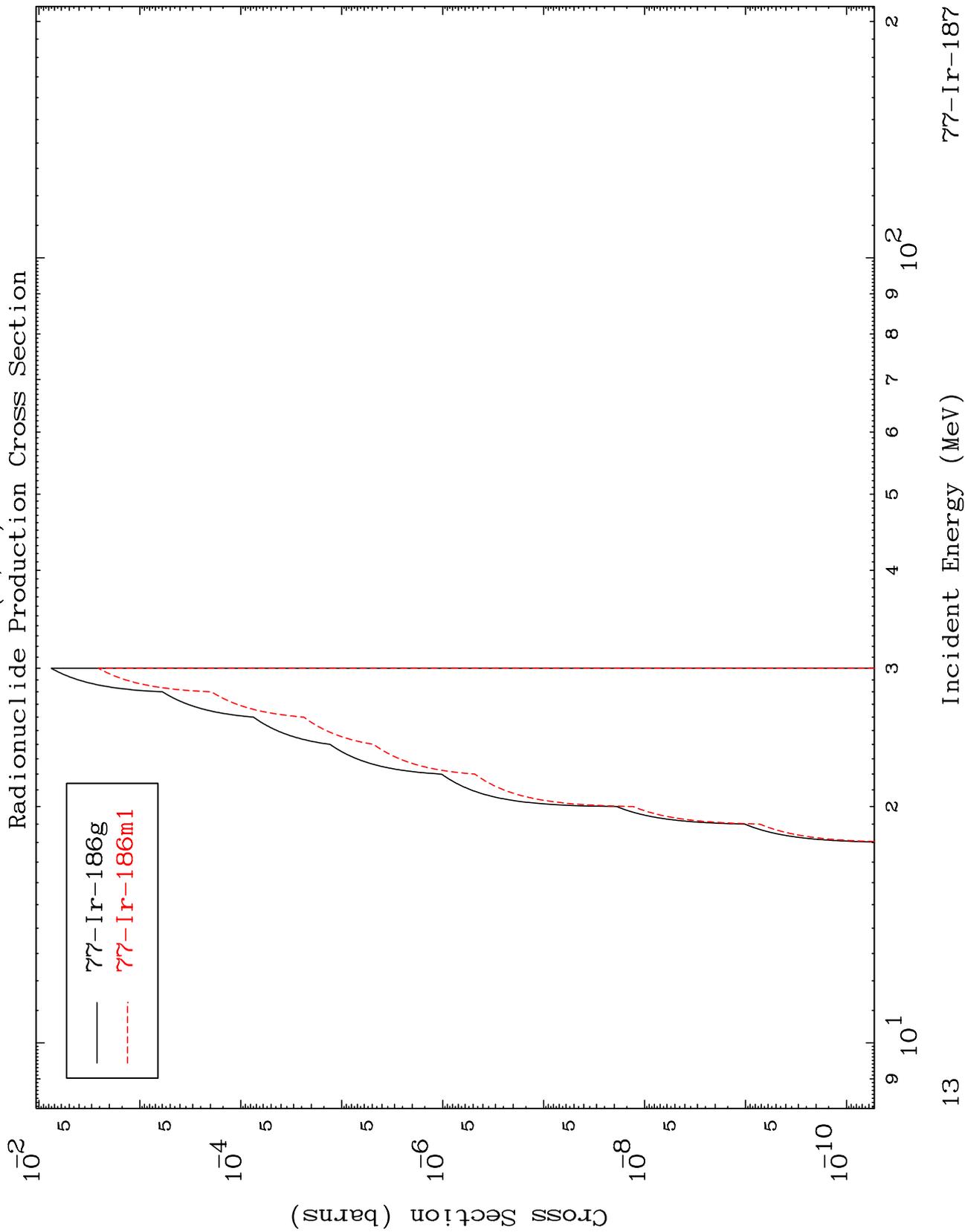
Radionuclide Production Cross Section



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$(n, n') \alpha$

$^{77}\text{Ir-187}$



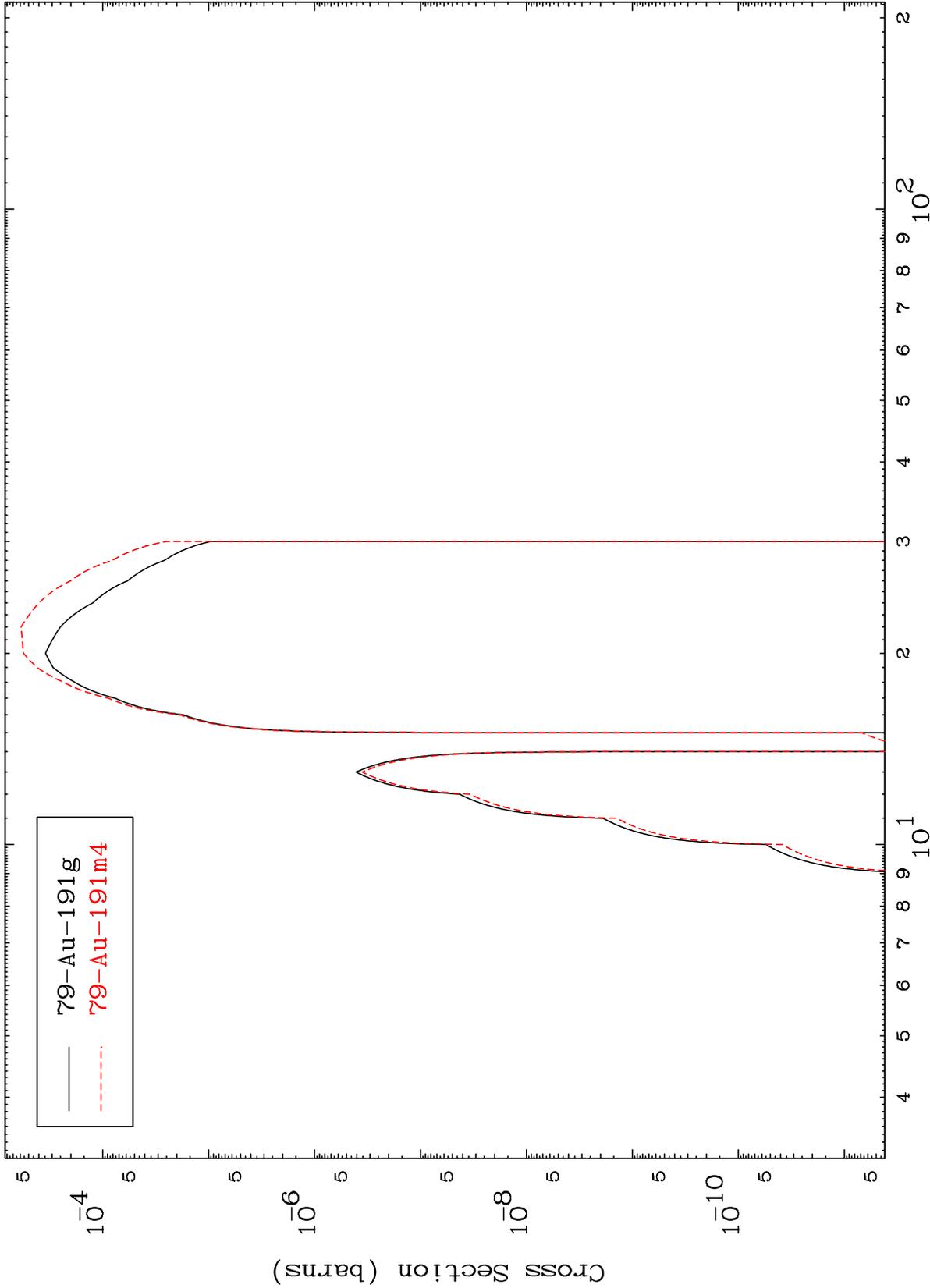
13

$^{77}\text{Ir-187}$

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⁷⁷Ir-187

(n,γ)
Radionuclide Production Cross Section



⁷⁷Ir-187

Incident Energy (MeV)

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