

Program EVALPLOT  
(Version 2021-1)

by

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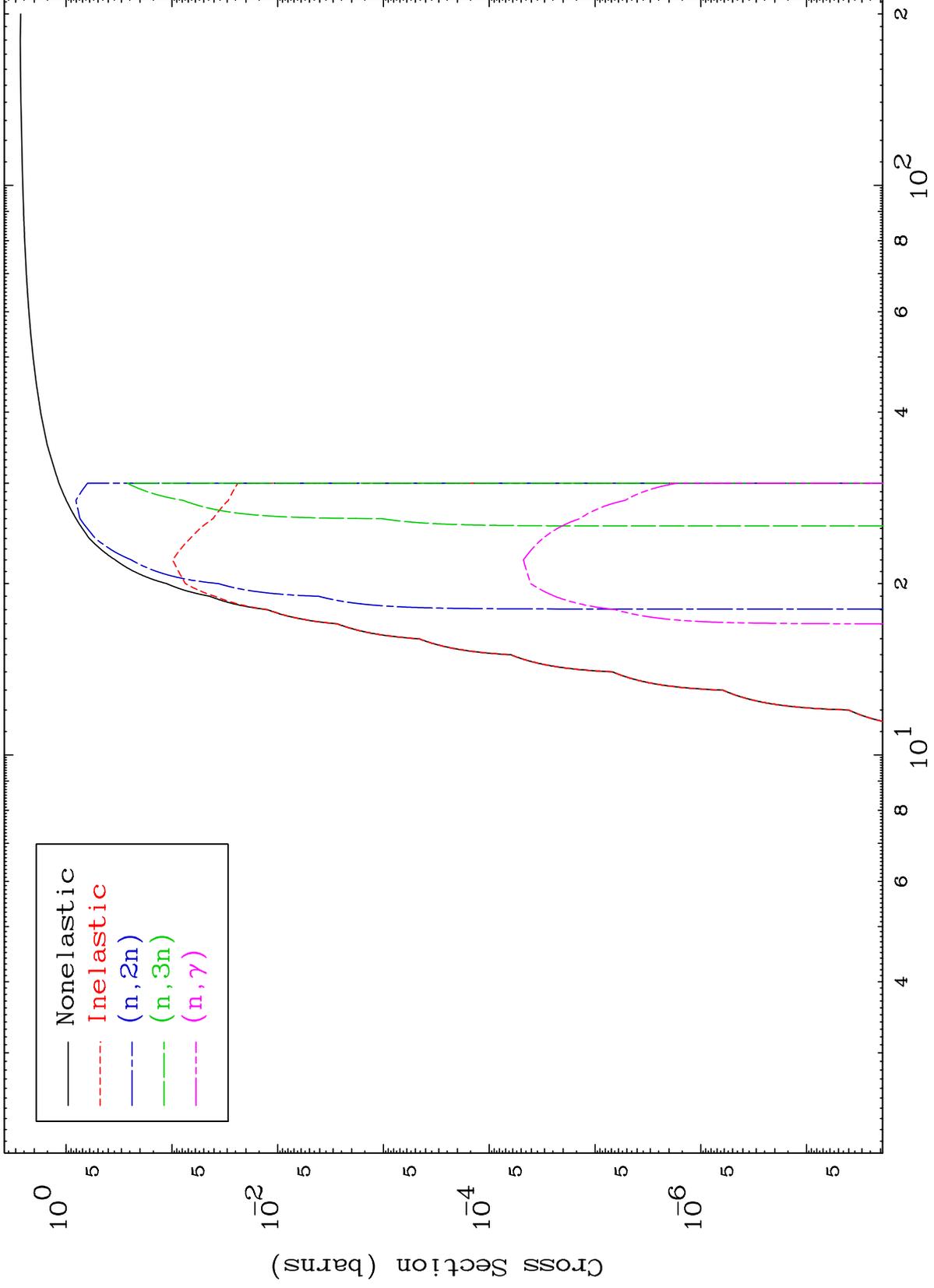
Press Mouse Button to Start

MAT 7723

0 Kelvin

Major Cross Sections

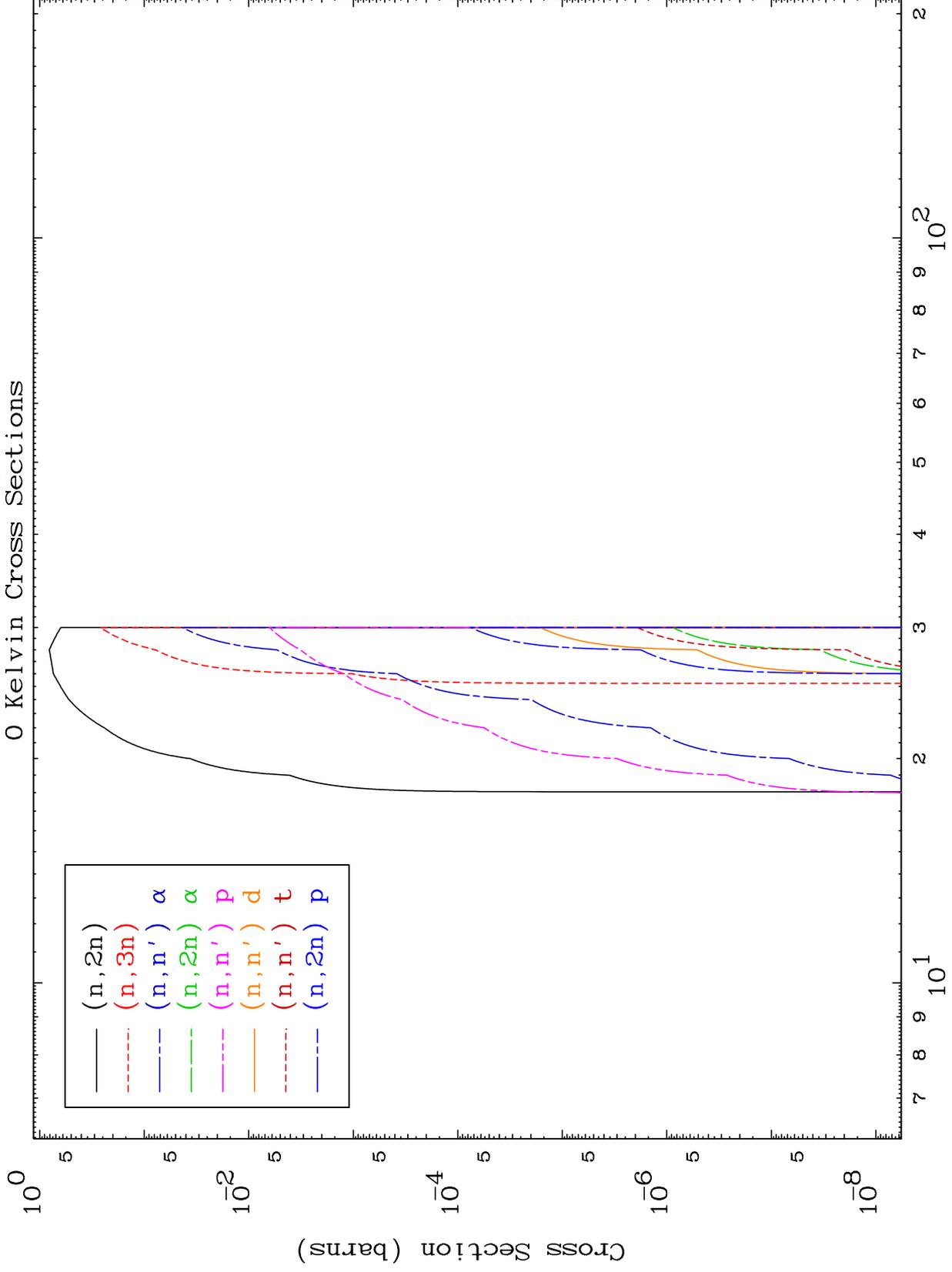
<sup>77</sup>Ir-190m



MAT 7723

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

77-Ir-190m



2

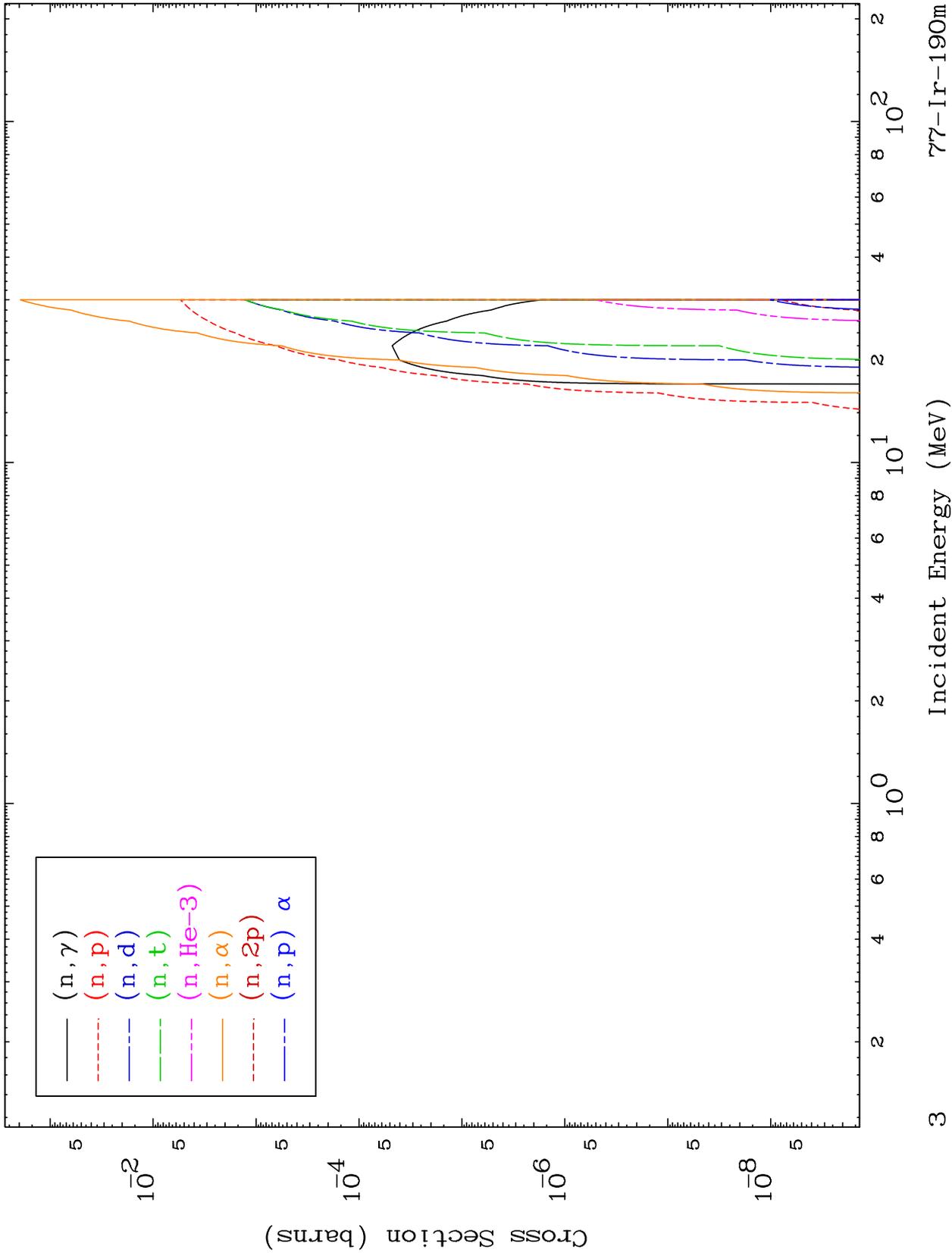
Incident Energy (MeV)

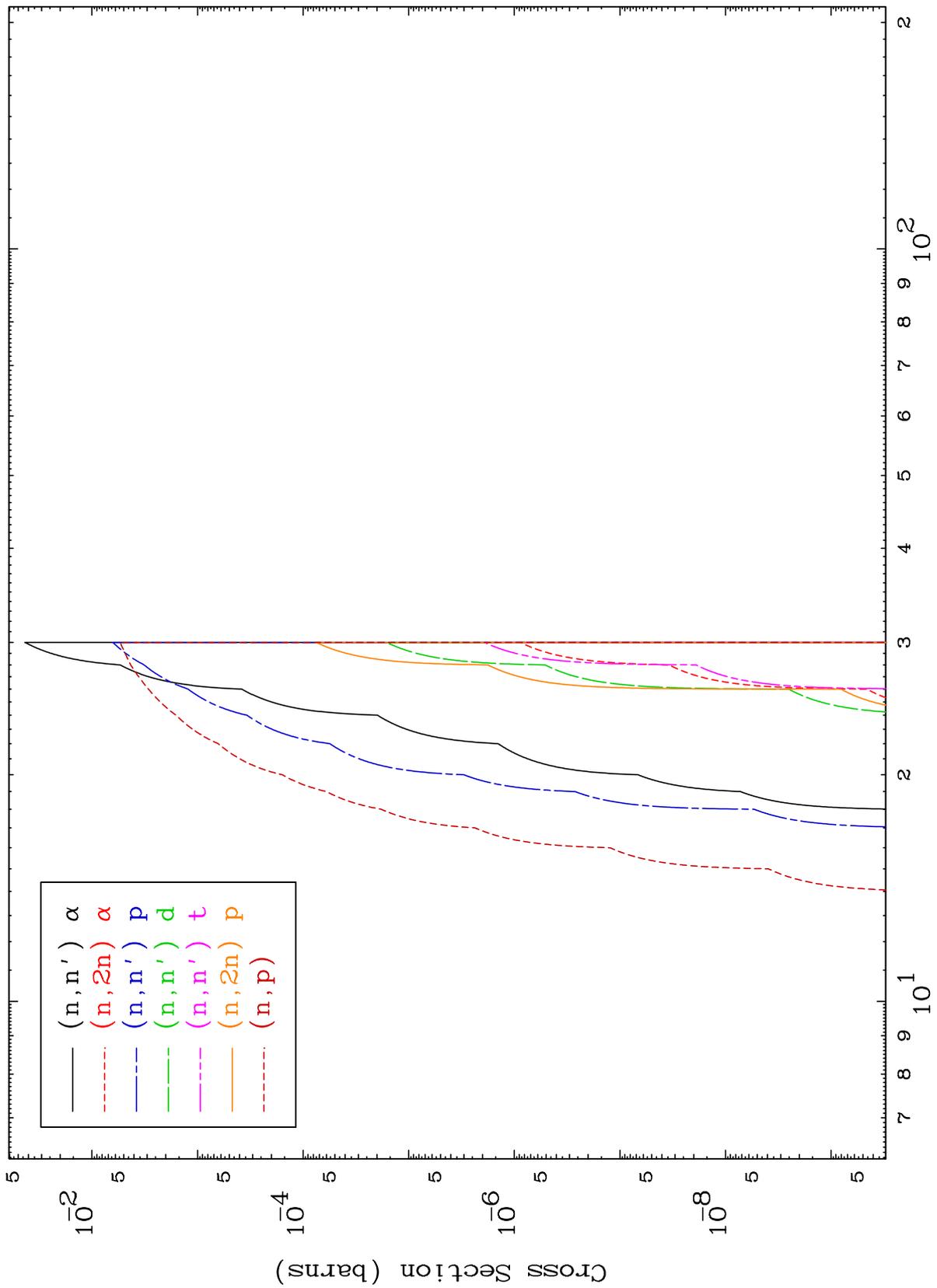
77-Ir-190m

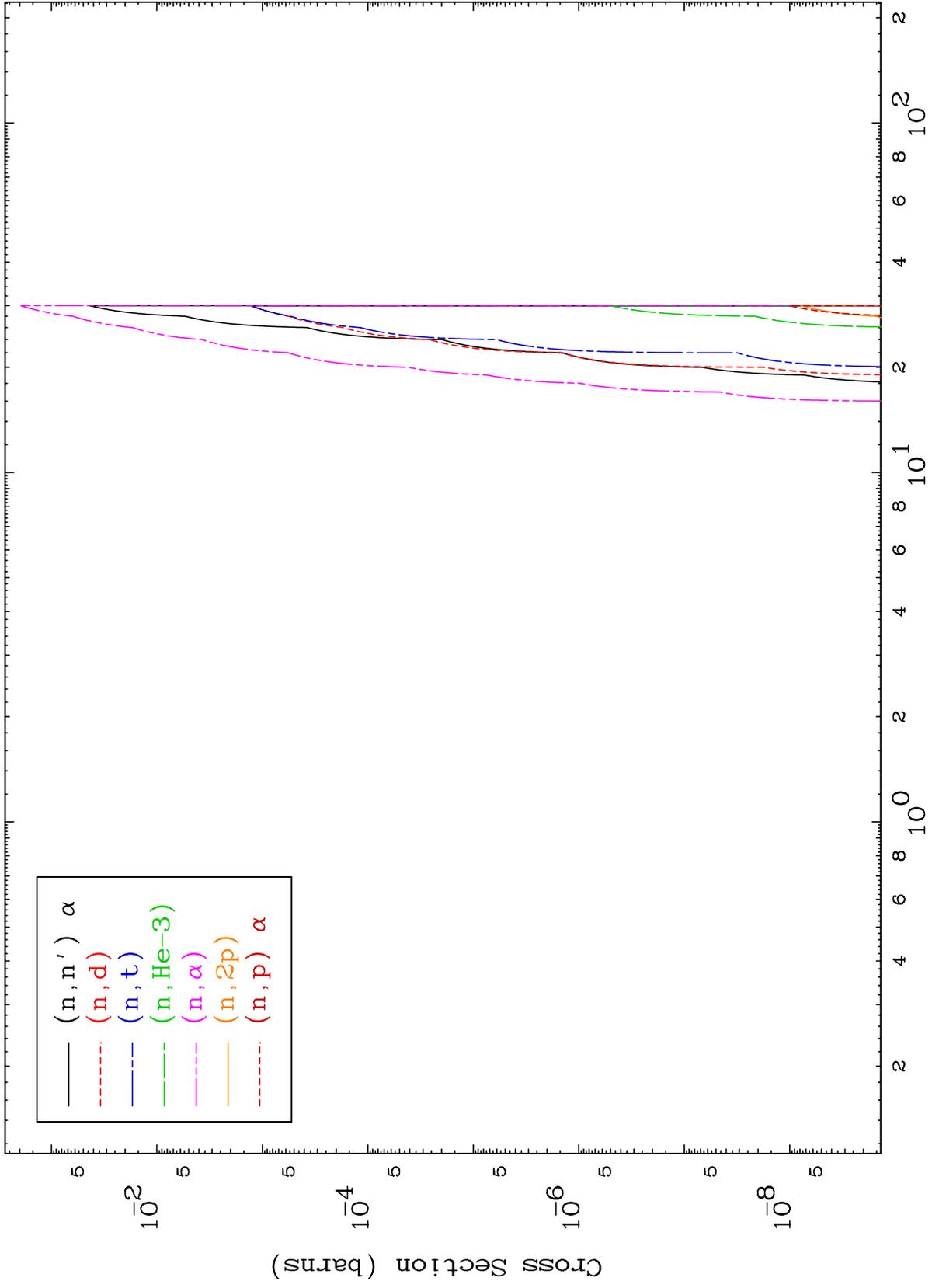
MAT 7723

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

77-Ir-190m



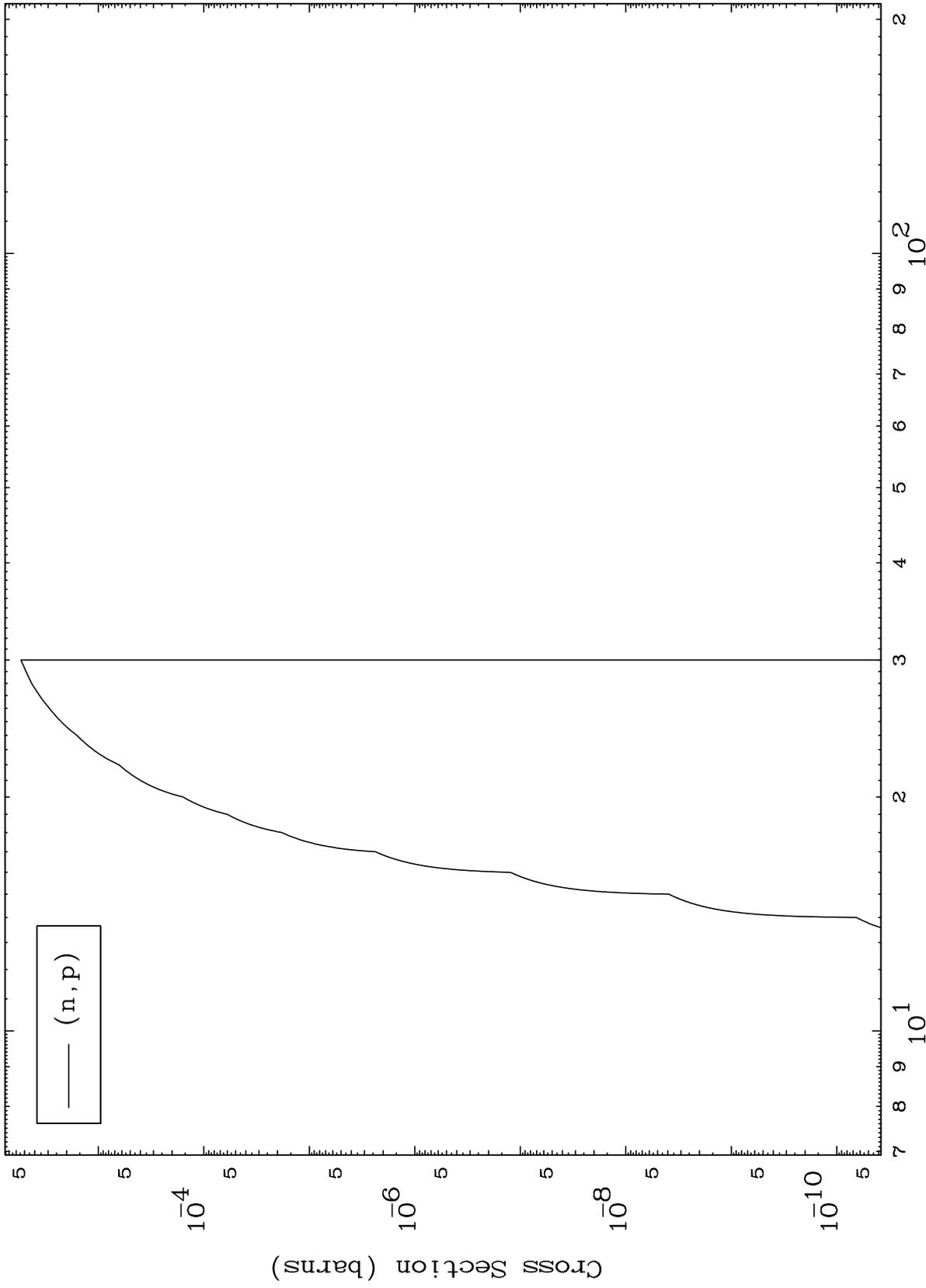




MAT 7723

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

77-Ir-190m



6

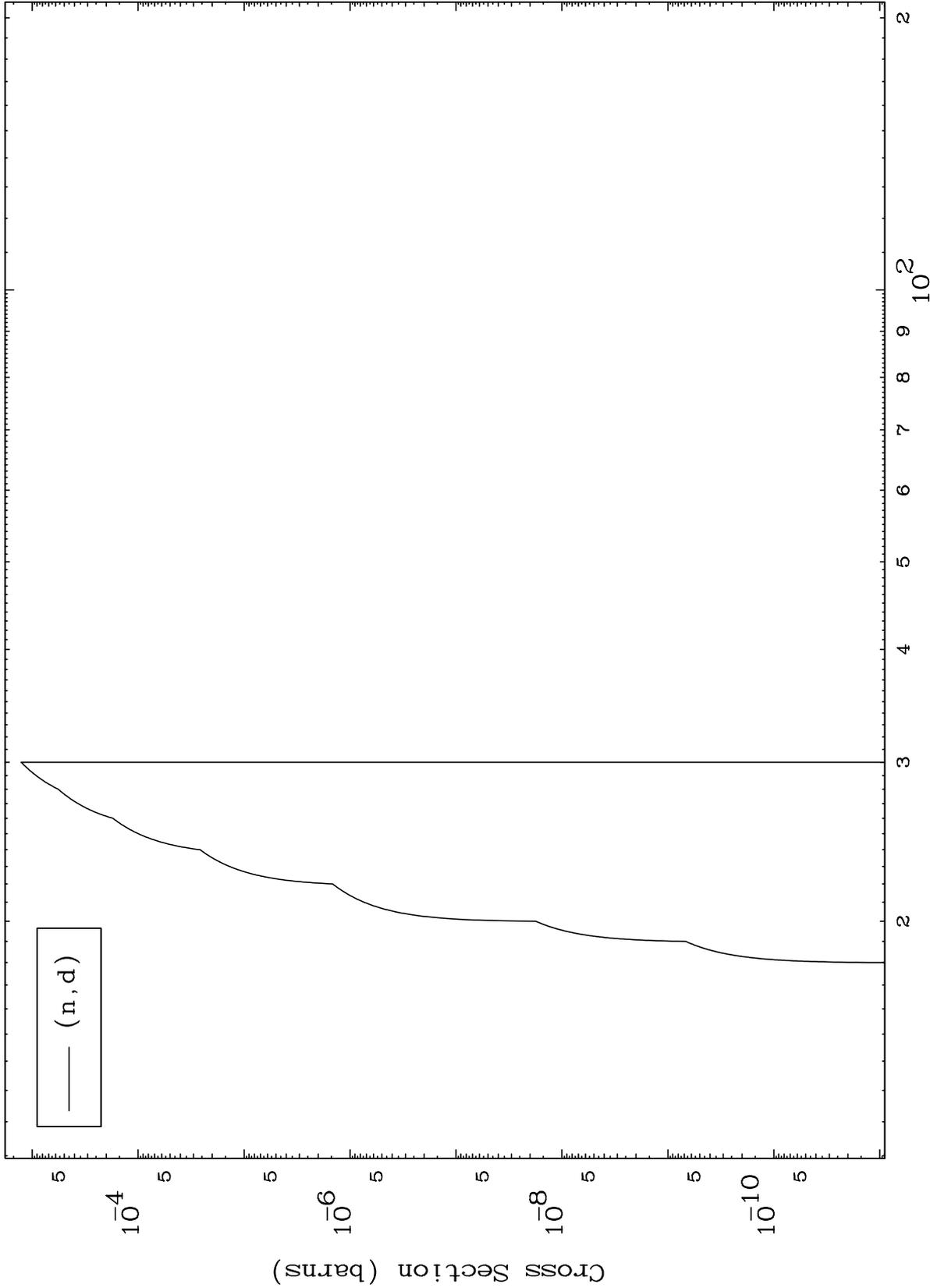
Incident Energy (MeV)

77-Ir-190m

MAT 7723

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

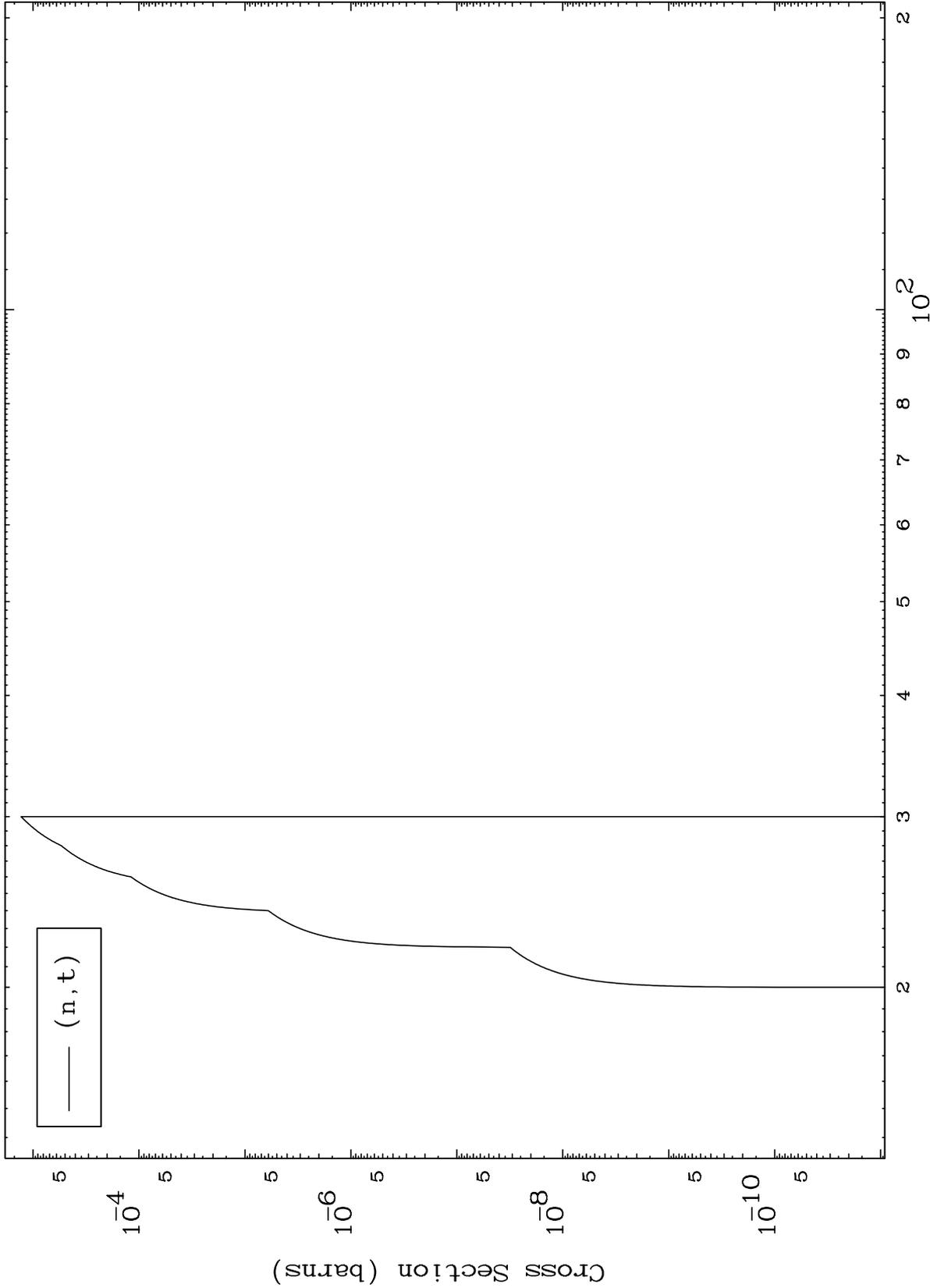
77-Ir-190m



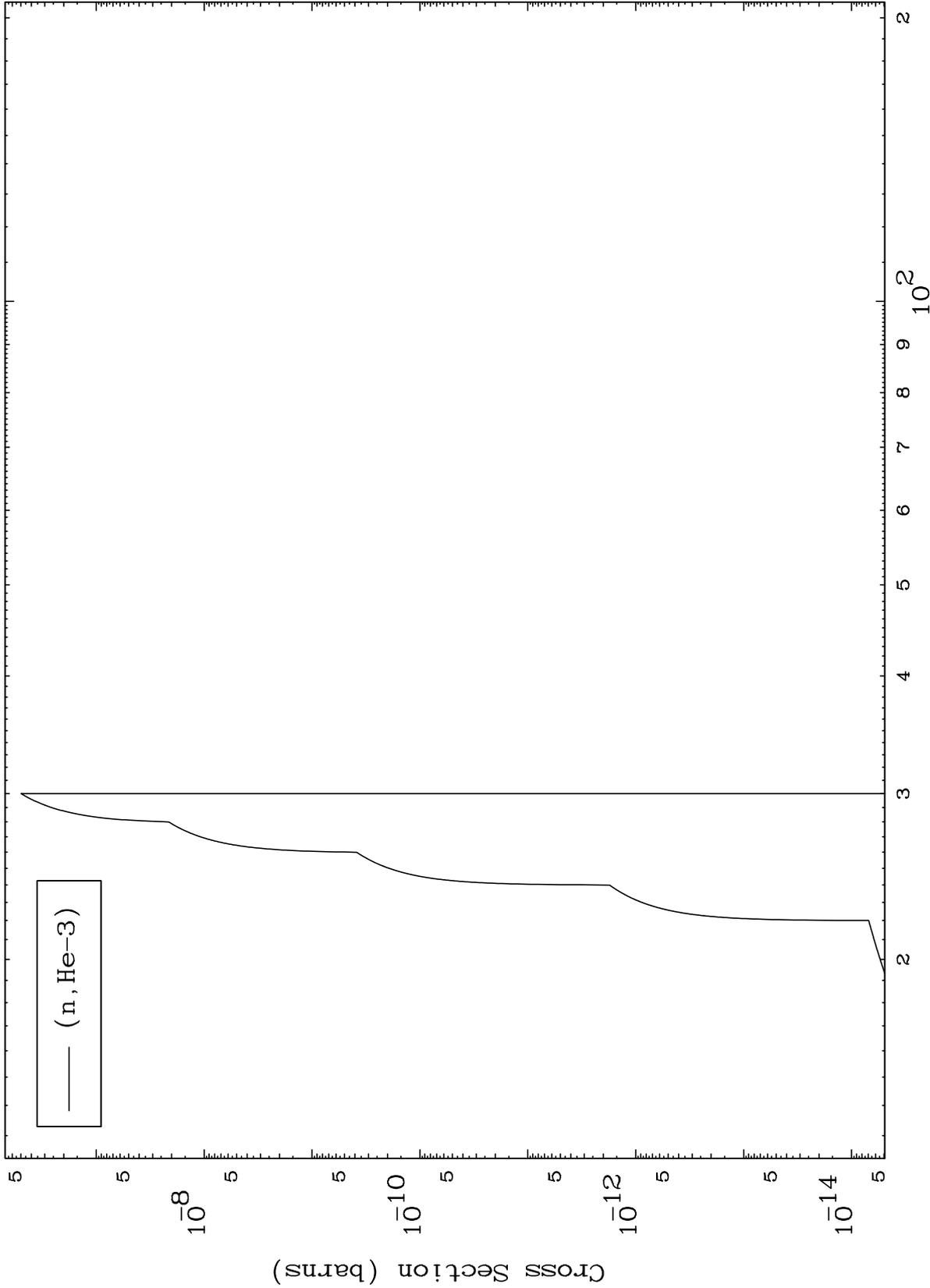
MAT 7723

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

77-Ir-190m



0 Kelvin Cross Sections

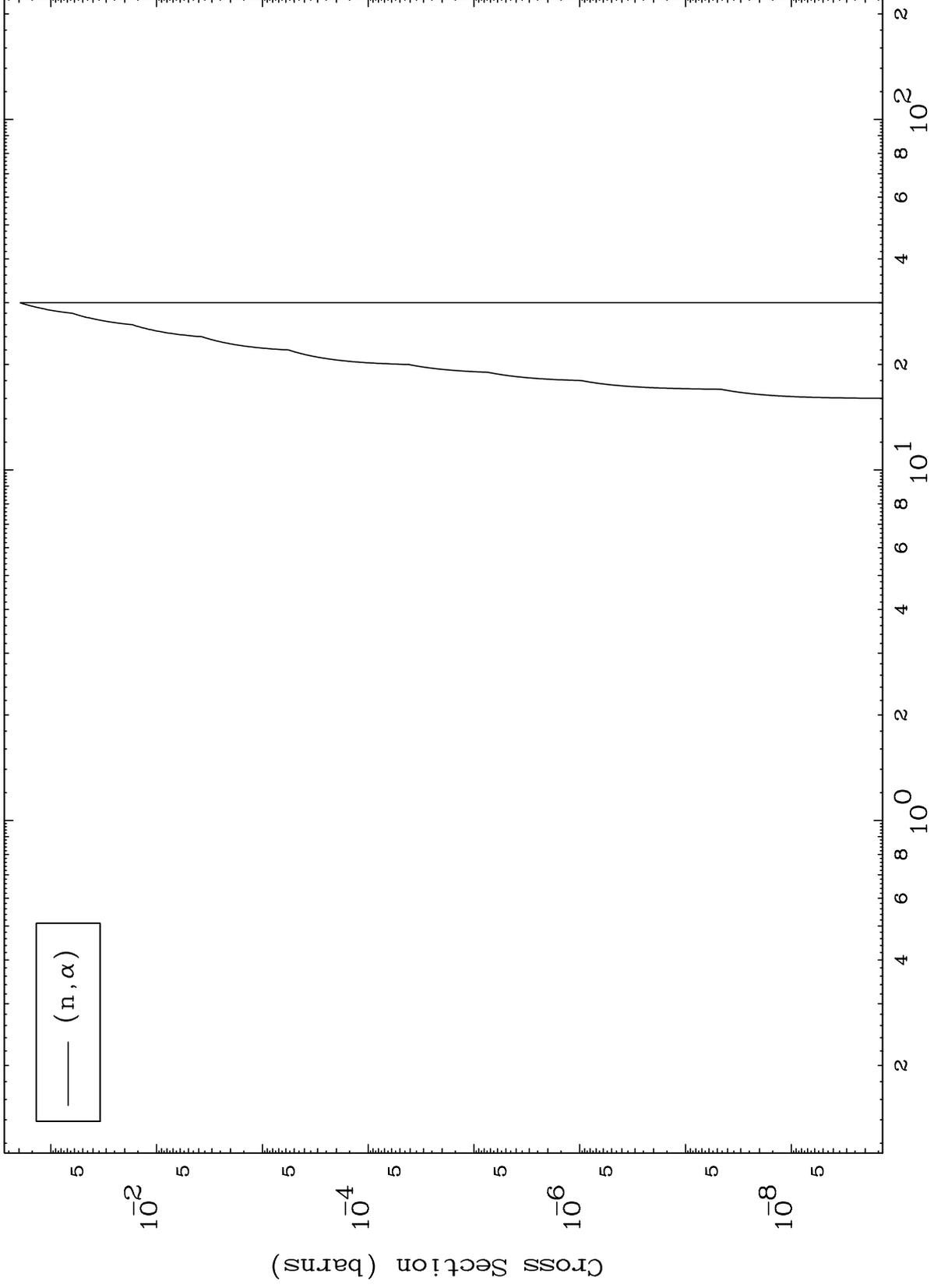


MAT 7723

( $\alpha, \alpha$ ) Levels

77-Ir-190m

0 Kelvin Cross Sections



10

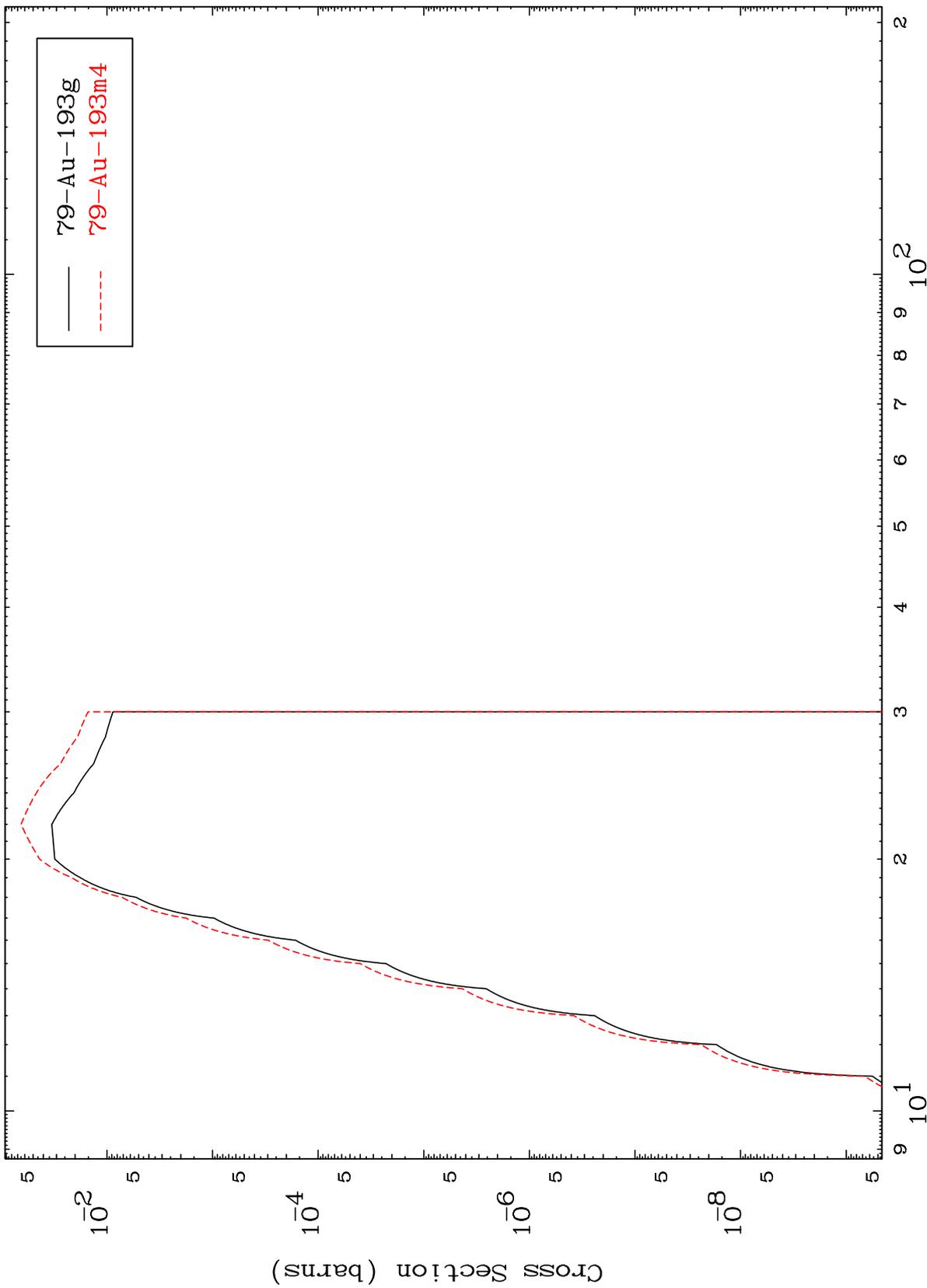
Incident Energy (MeV)

77-Ir-190m

MAT 7723

<sup>77</sup>Ir-190m

Inelastic  
Radionuclide Production Cross Section



<sup>77</sup>Ir-190m

Incident Energy (MeV)

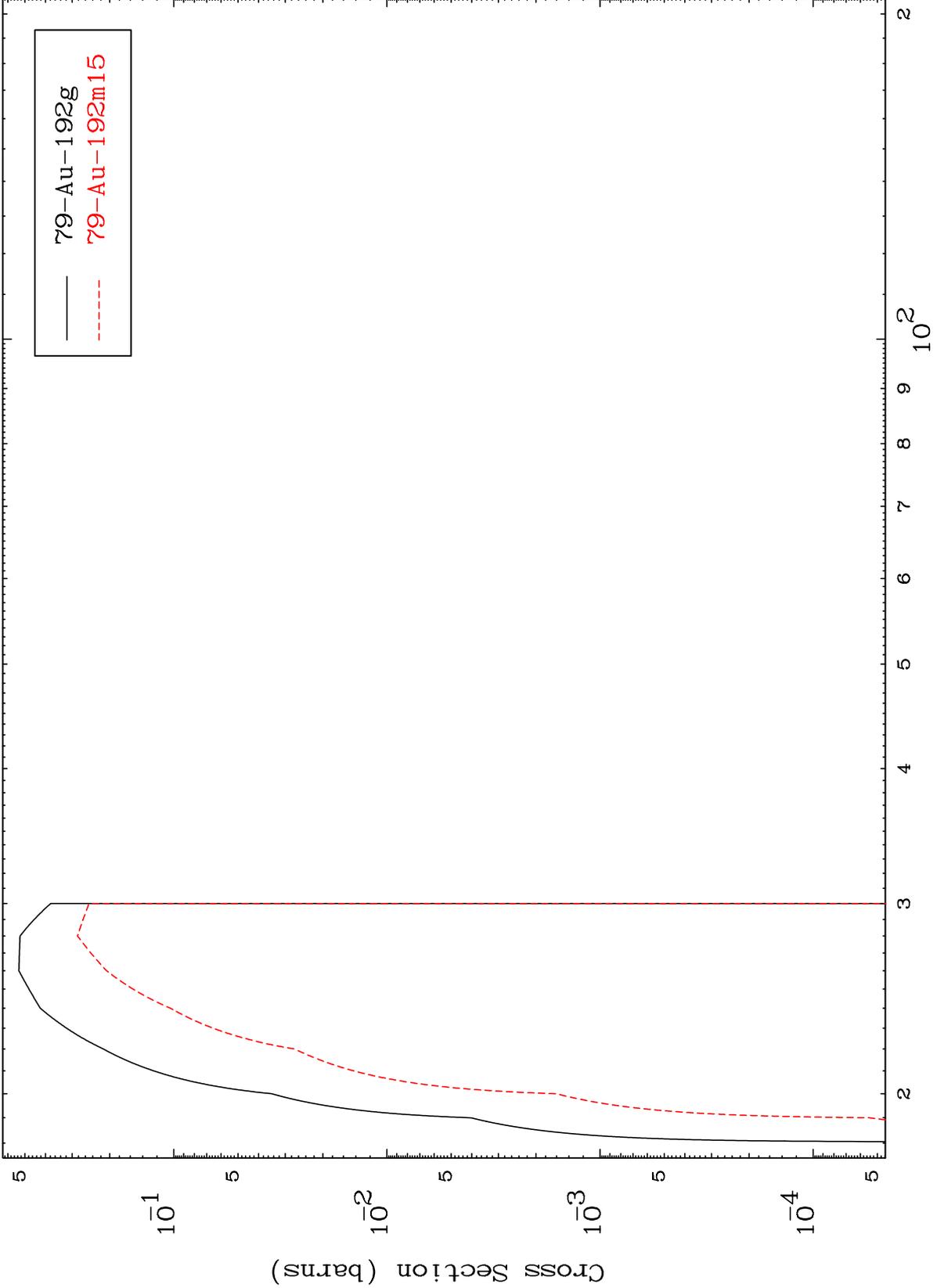
11

MAT 7723

(n,2n)

<sup>77</sup>Ir-190m

Radionuclide Production Cross Section



— <sup>79</sup>Au-192g  
- - - <sup>79</sup>Au-192m15

12

Incident Energy (MeV)

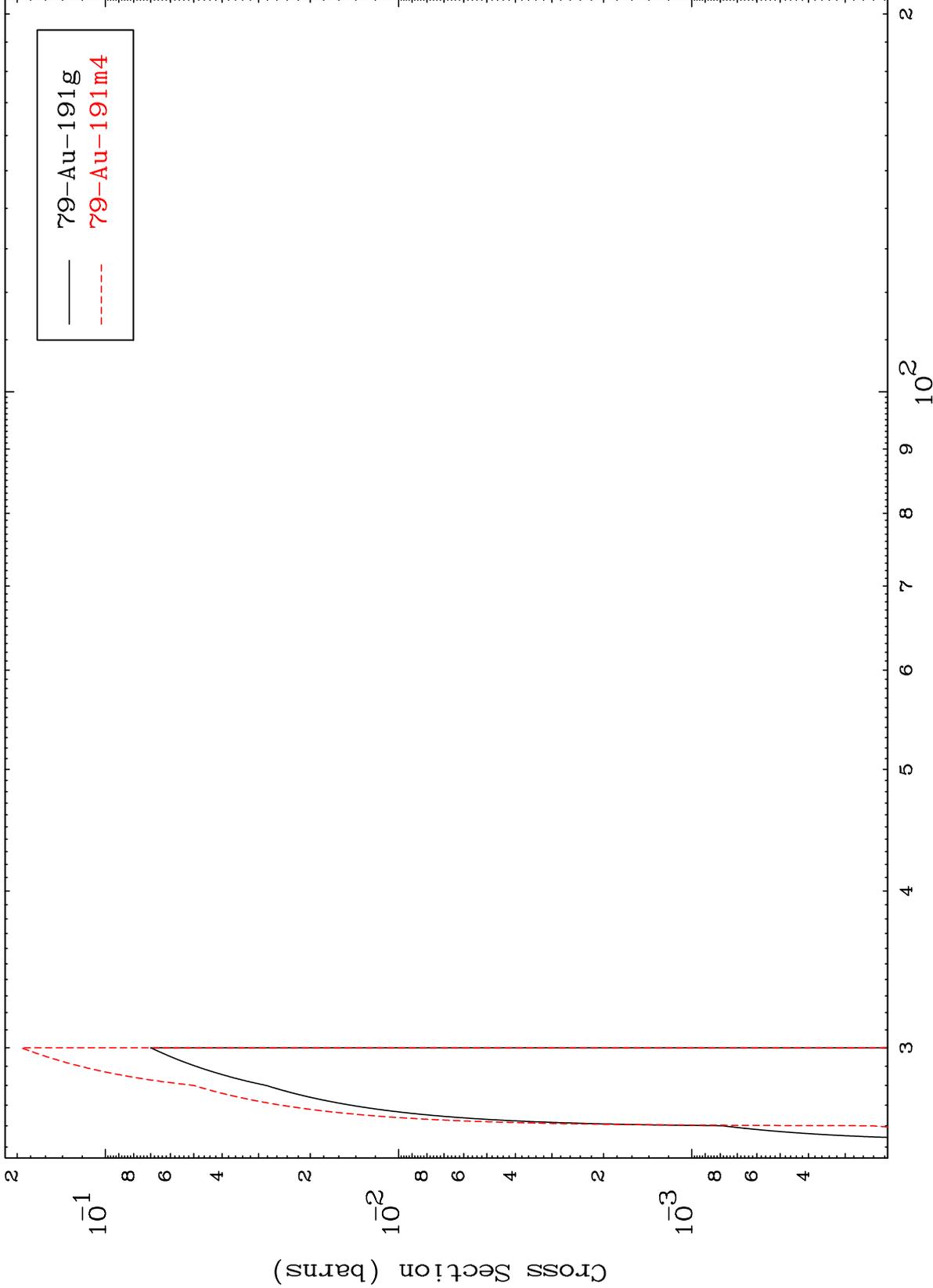
<sup>77</sup>Ir-190m

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(n,3n)

<sup>77</sup>Ir-190m

Radionuclide Production Cross Section



— <sup>79</sup>Au-191g  
- - - <sup>79</sup>Au-191m4

13

Incident Energy (MeV)

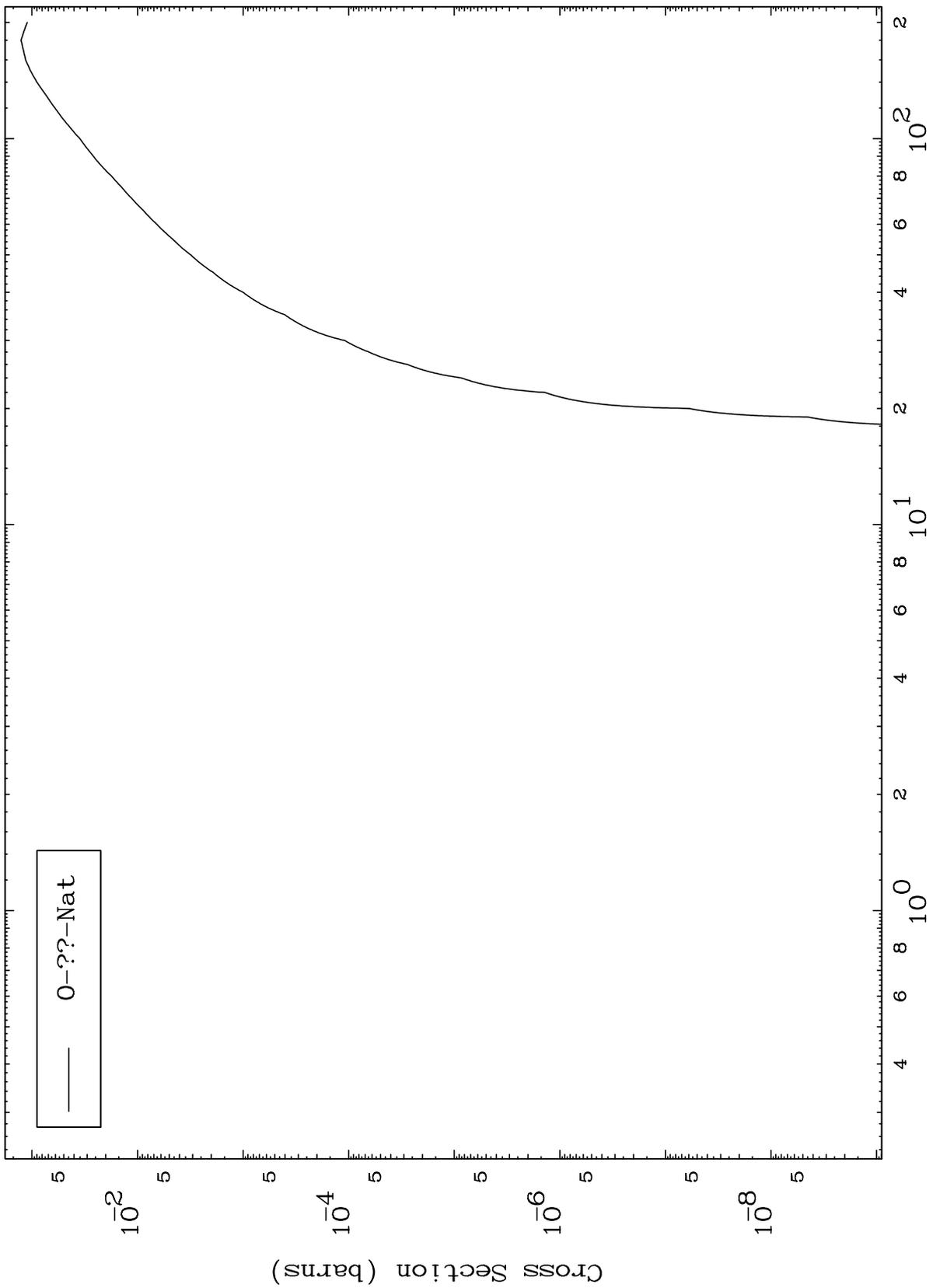
<sup>77</sup>Ir-190m

MAT 7723

Fission

<sup>77</sup>Ir-190m

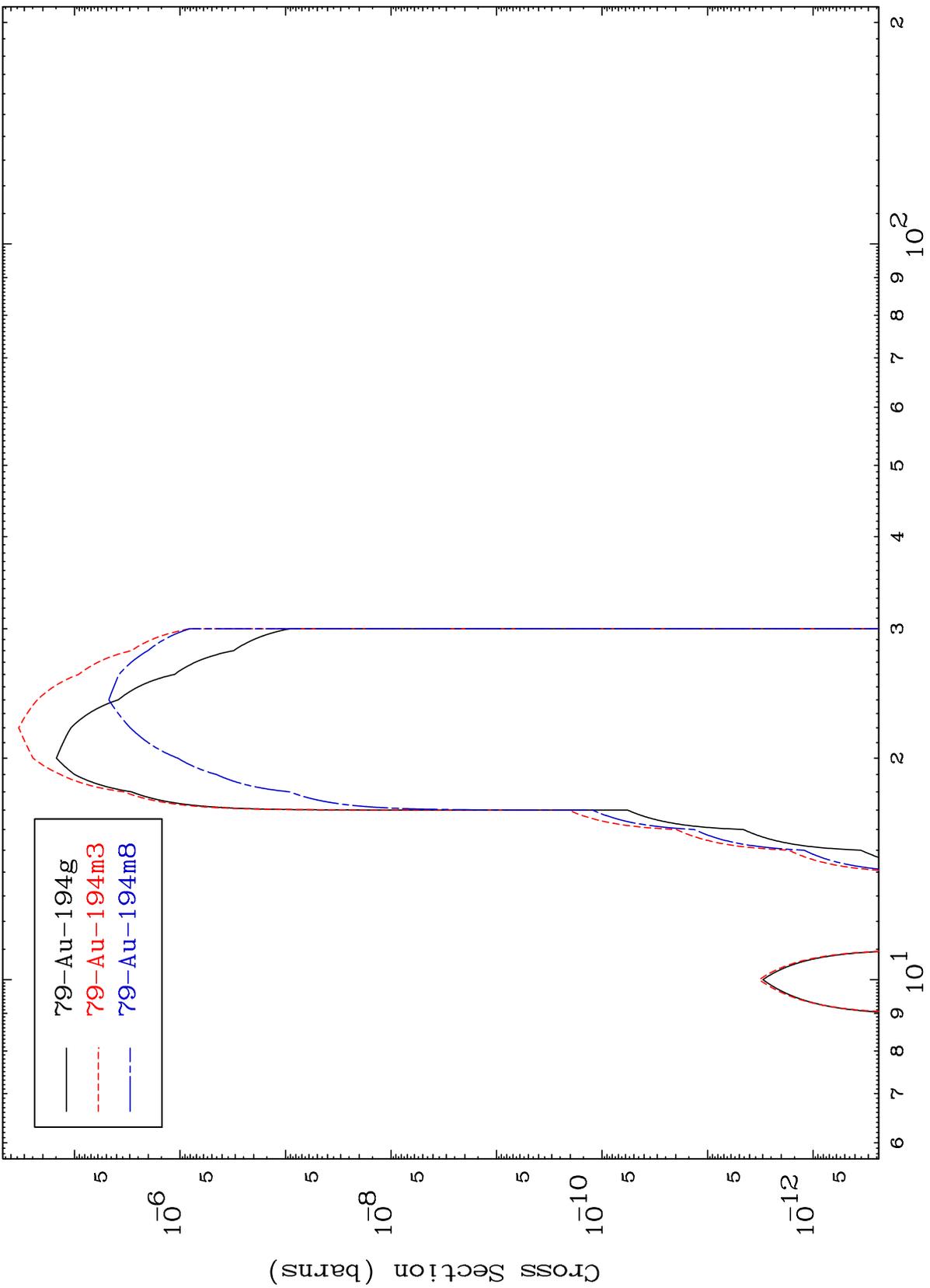
Radionuclide Production Cross Section



MAT 7723

<sup>77</sup>Ir-190m

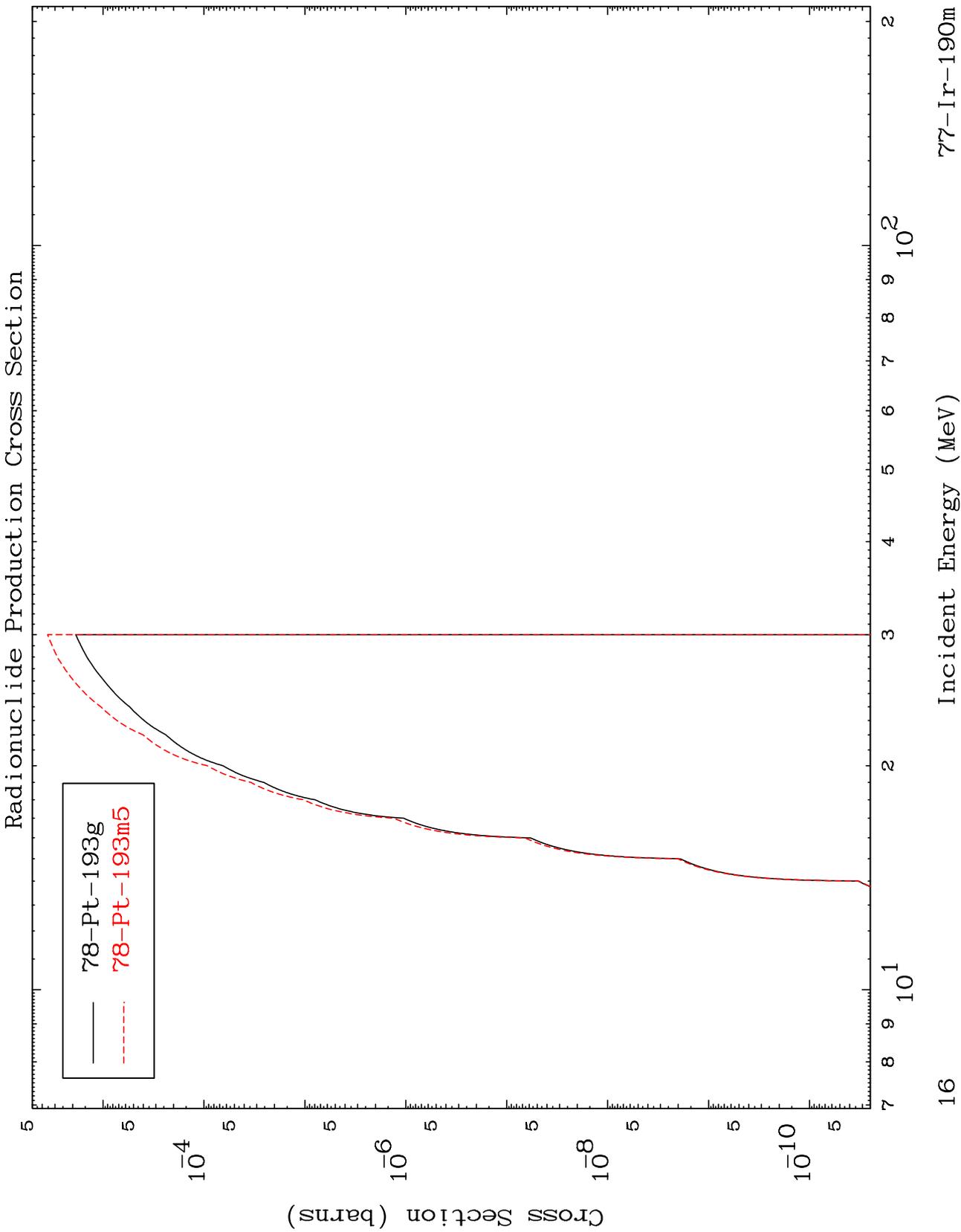
Radionuclide Production Cross Section (n,γ)



79-Au-194g  
79-Au-194m3  
79-Au-194m8

MAT 7723

<sup>77</sup>Ir-190m



16

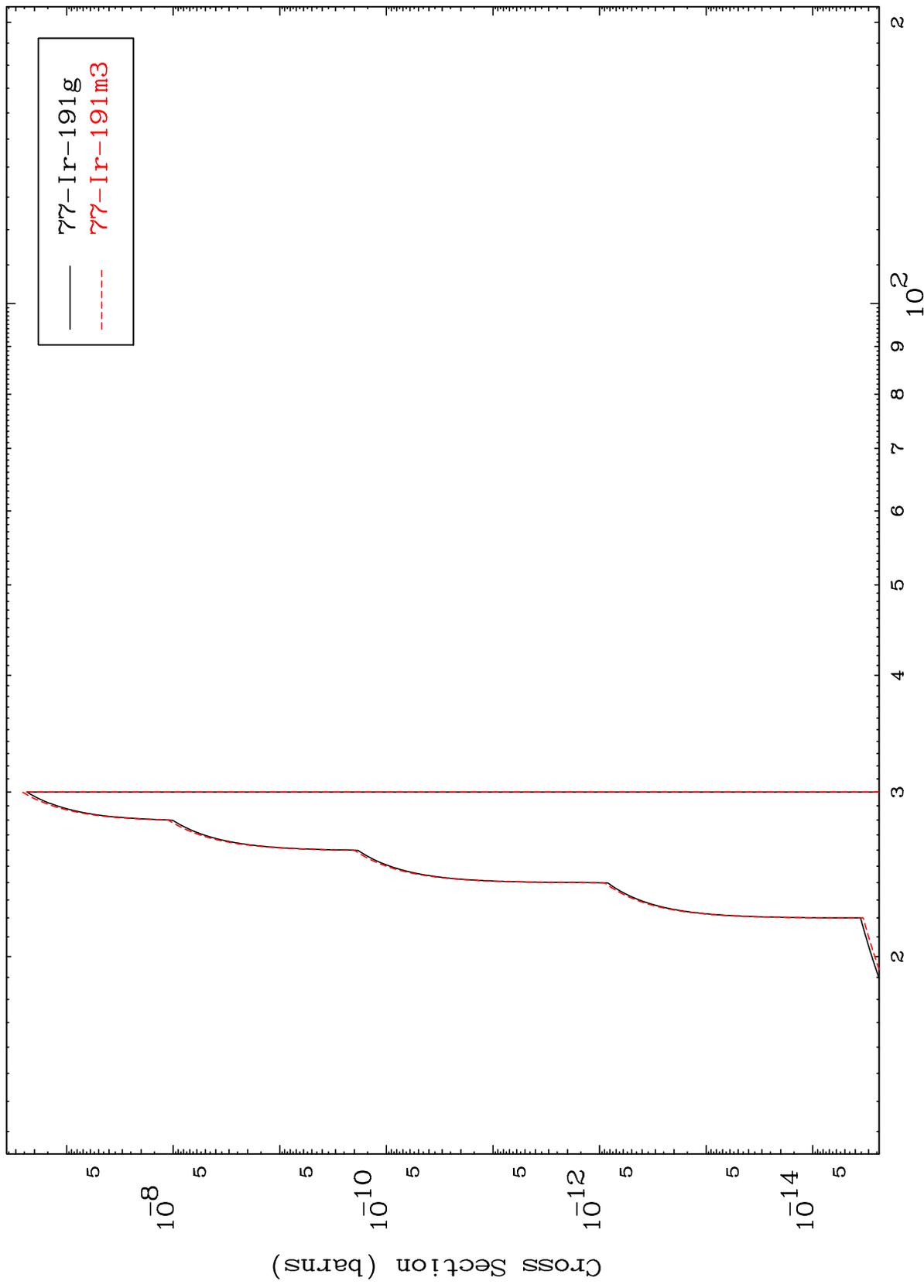
Incident Energy (MeV)

<sup>77</sup>Ir-190m

MAT 7723

<sup>77</sup>Ir-190m

(n,He-3)  
Radionuclide Production Cross Section



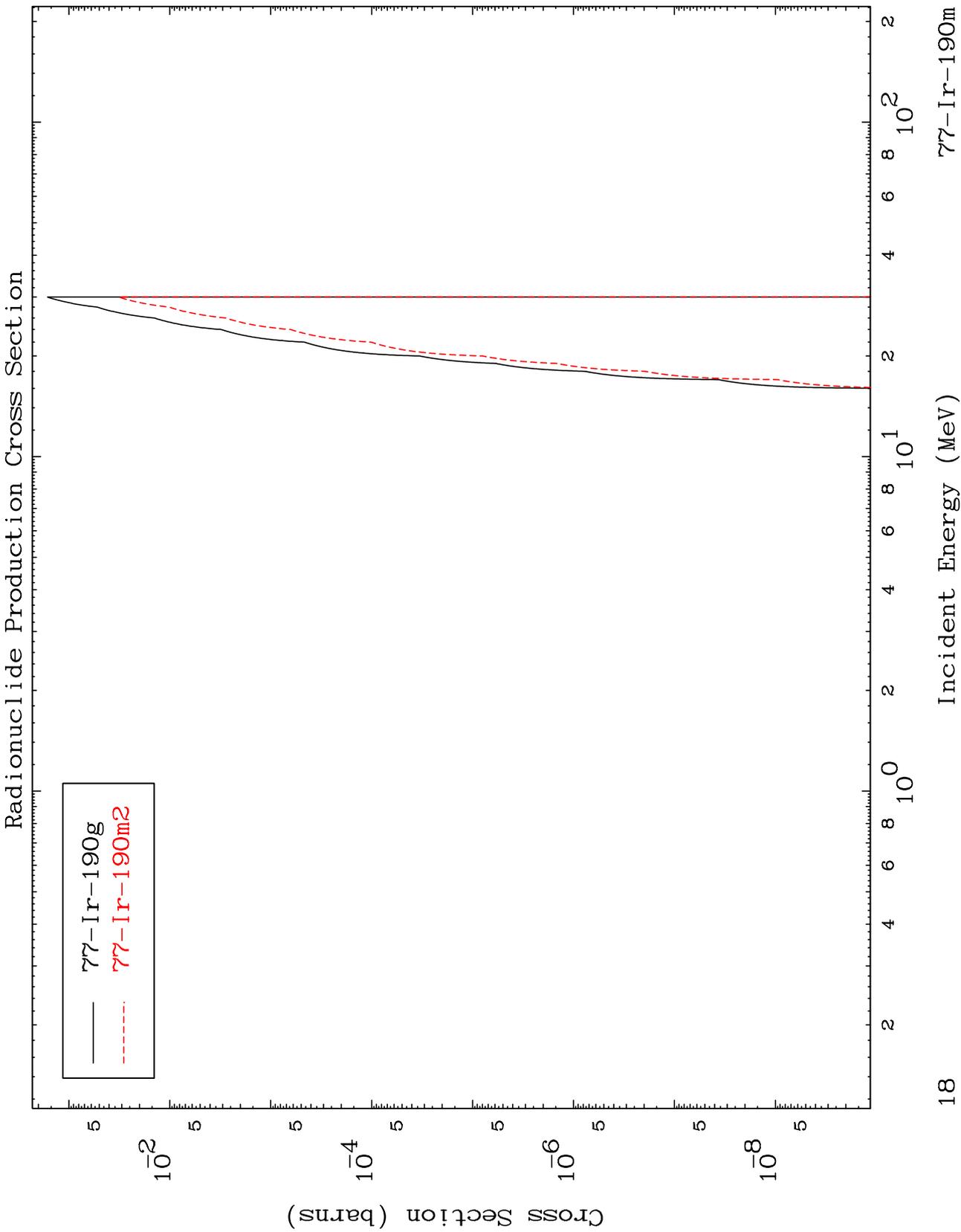
17

<sup>77</sup>Ir-190m

Incident Energy (MeV)

MAT 7723

<sup>77</sup>Ir-190m



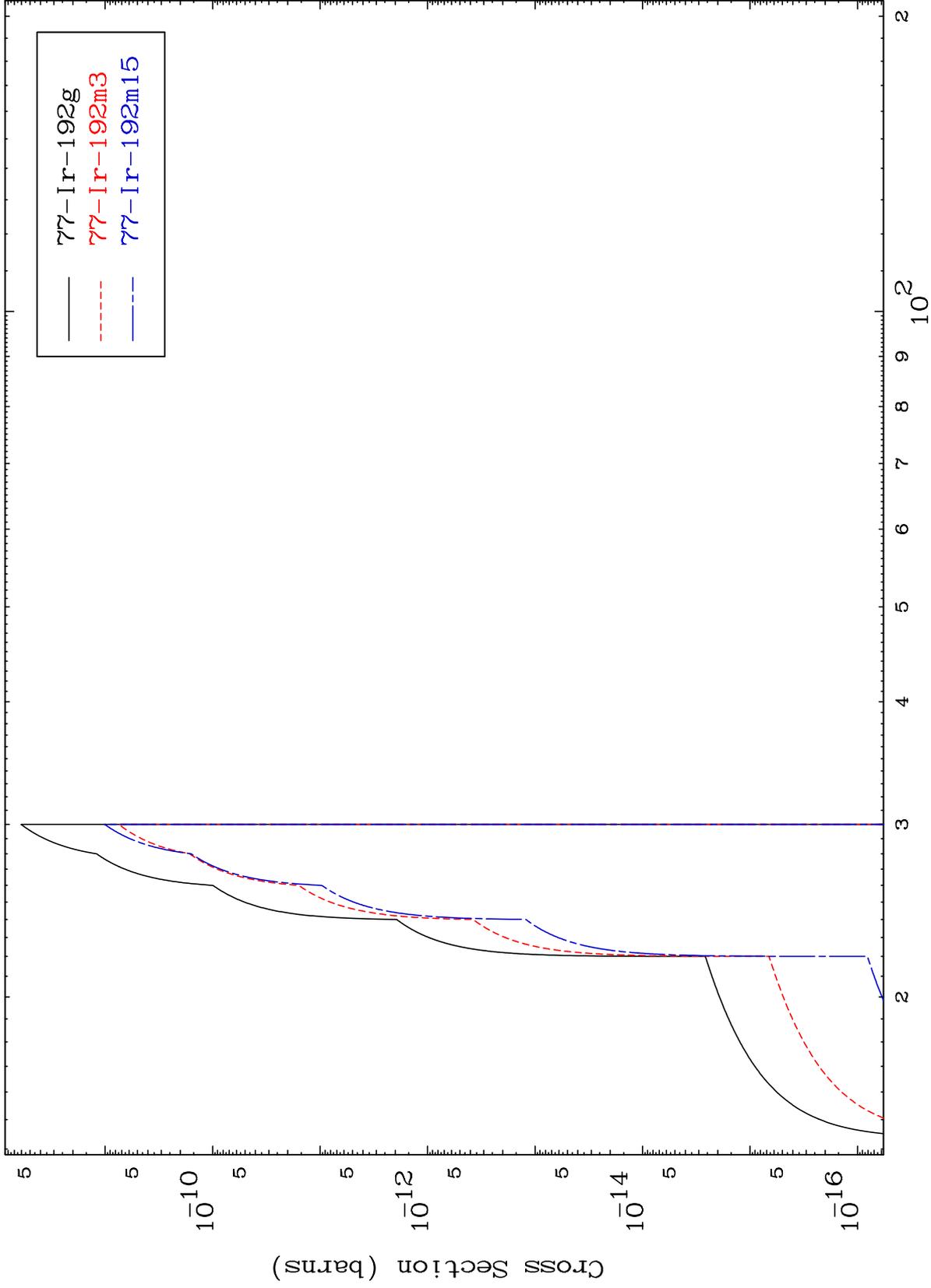
<sup>77</sup>Ir-190m

Incident Energy (MeV)

MAT 7723

77-Ir-190m

(n,2p)  
Radionuclide Production Cross Section



19

Incident Energy (MeV)

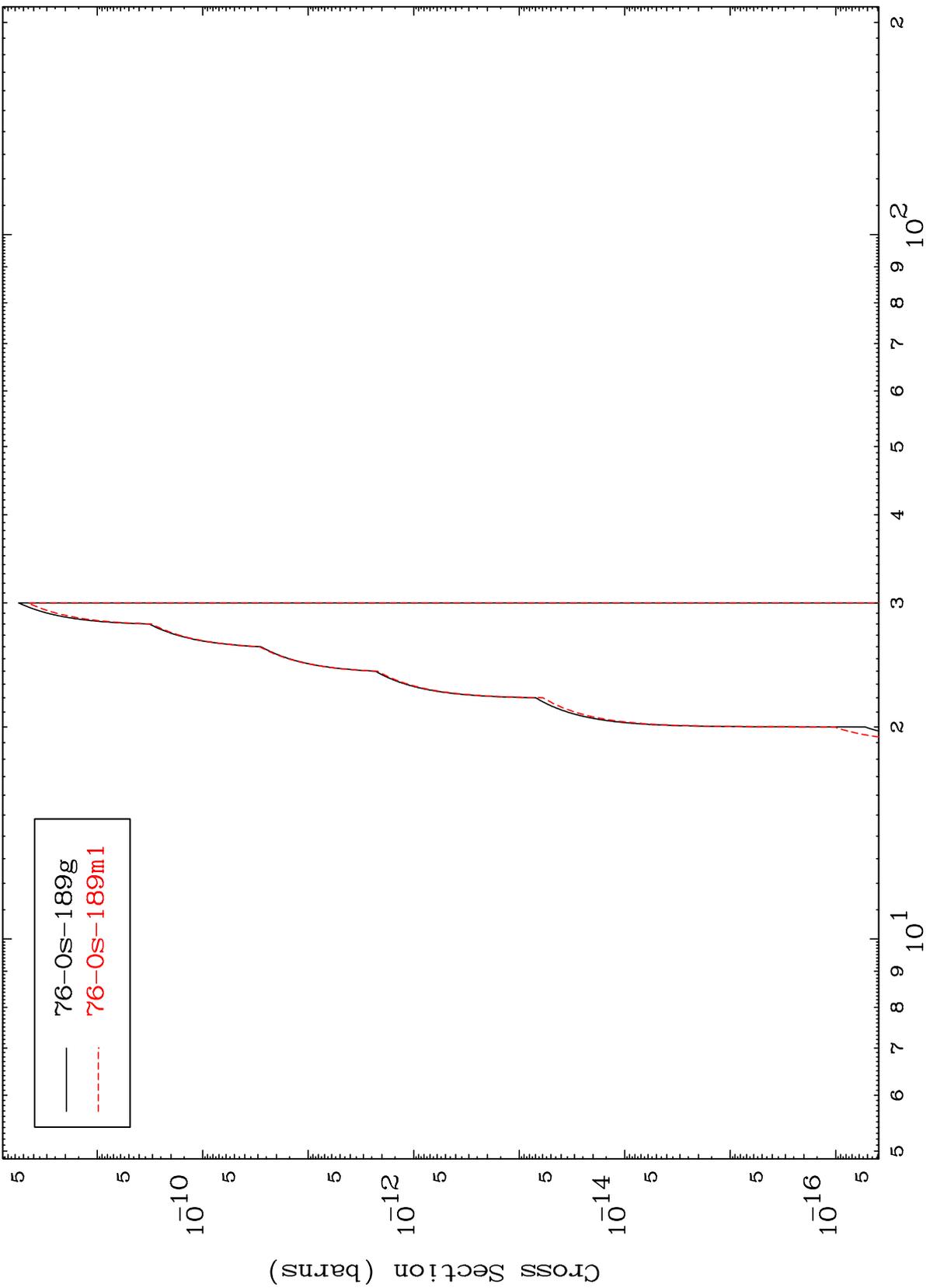
77-Ir-190m

MAT 7723

(n,p)  $\alpha$

77-Ir-190m

Radionuclide Production Cross Section



20

Incident Energy (MeV)

77-Ir-190m