

Program EVALPLOT  
(Version 2021-1)

by

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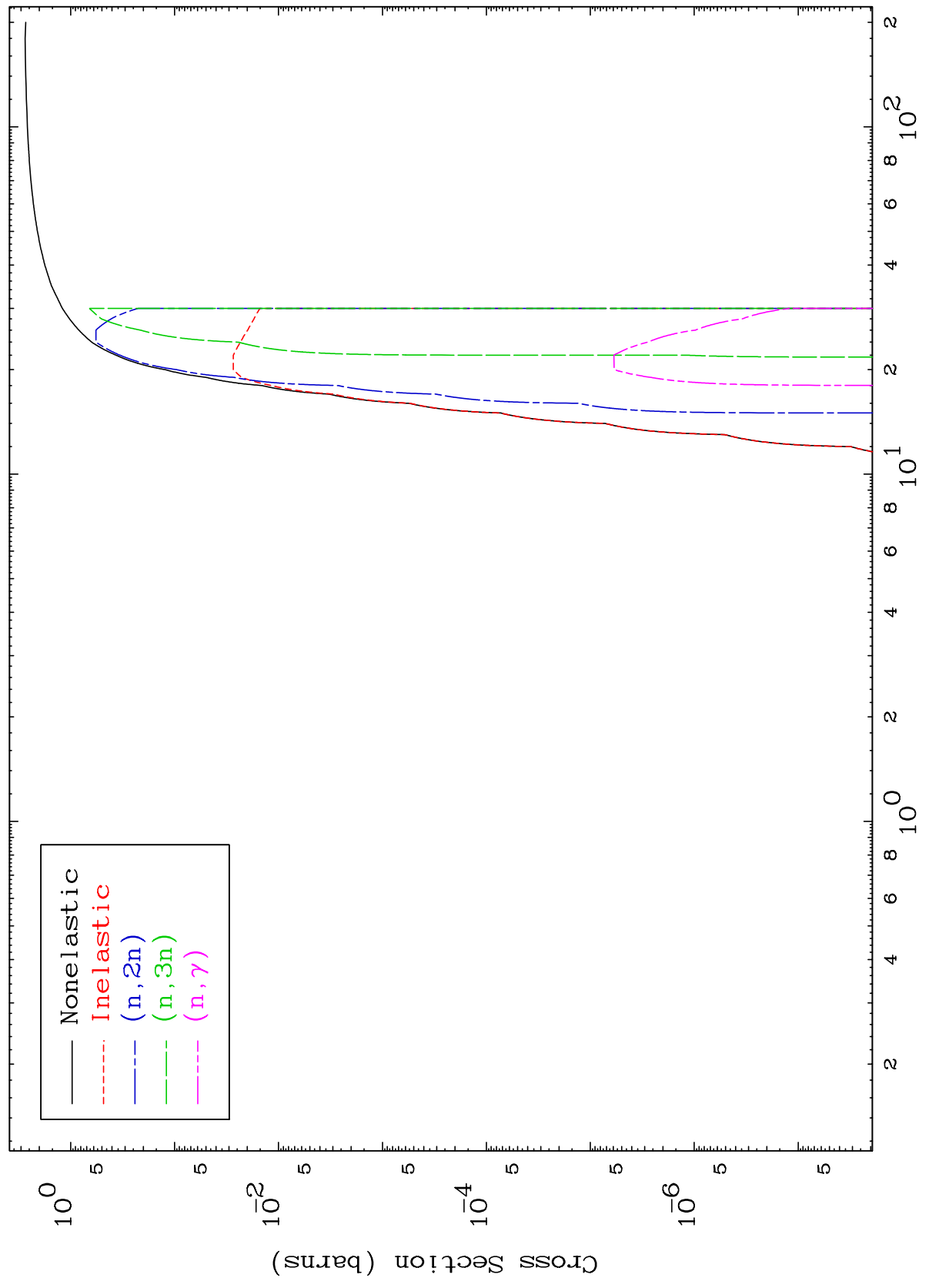
Press Mouse Button to Start

MAT 7735

0 Kelvin

$\alpha$  Major Cross Sections

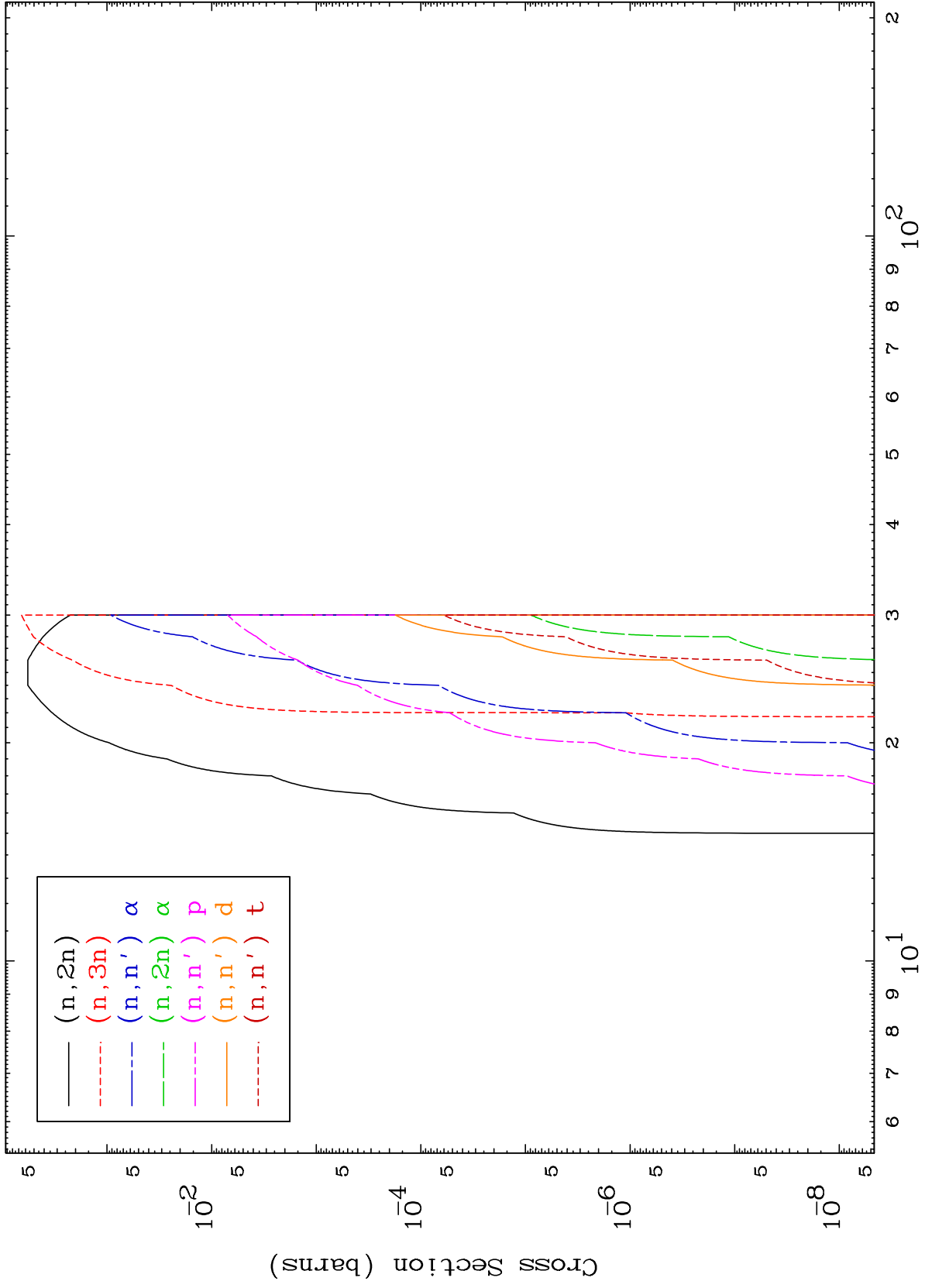
77-Ir-194m



MAT 7735

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

77-Ir-194m



2

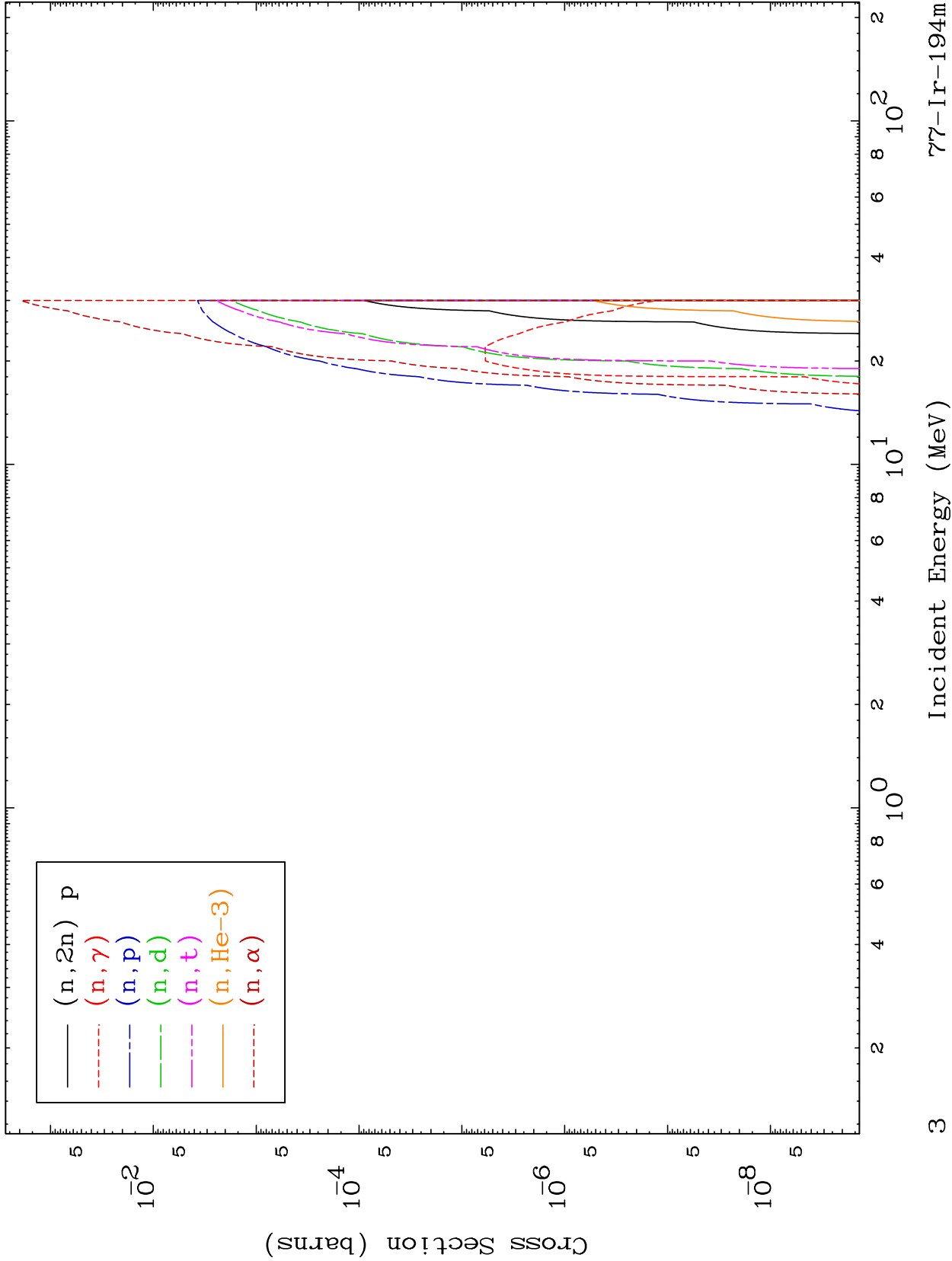
Incident Energy (MeV)

77-Ir-194m

MAT 7735

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

77-Ir-194m

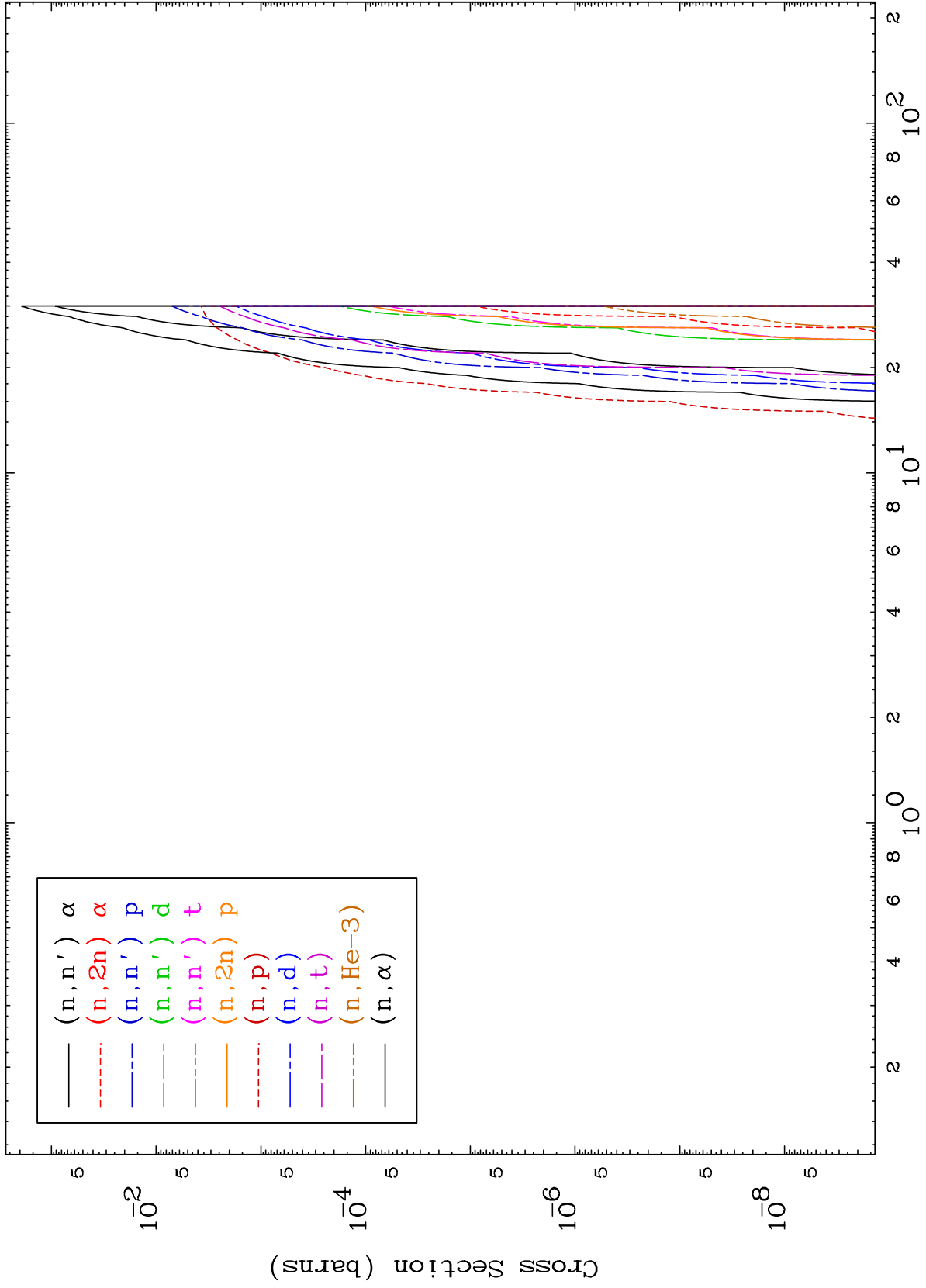


MAT 7735

$\alpha$  Charged Particle

77-Ir-194m

0 Kelvin Cross Sections



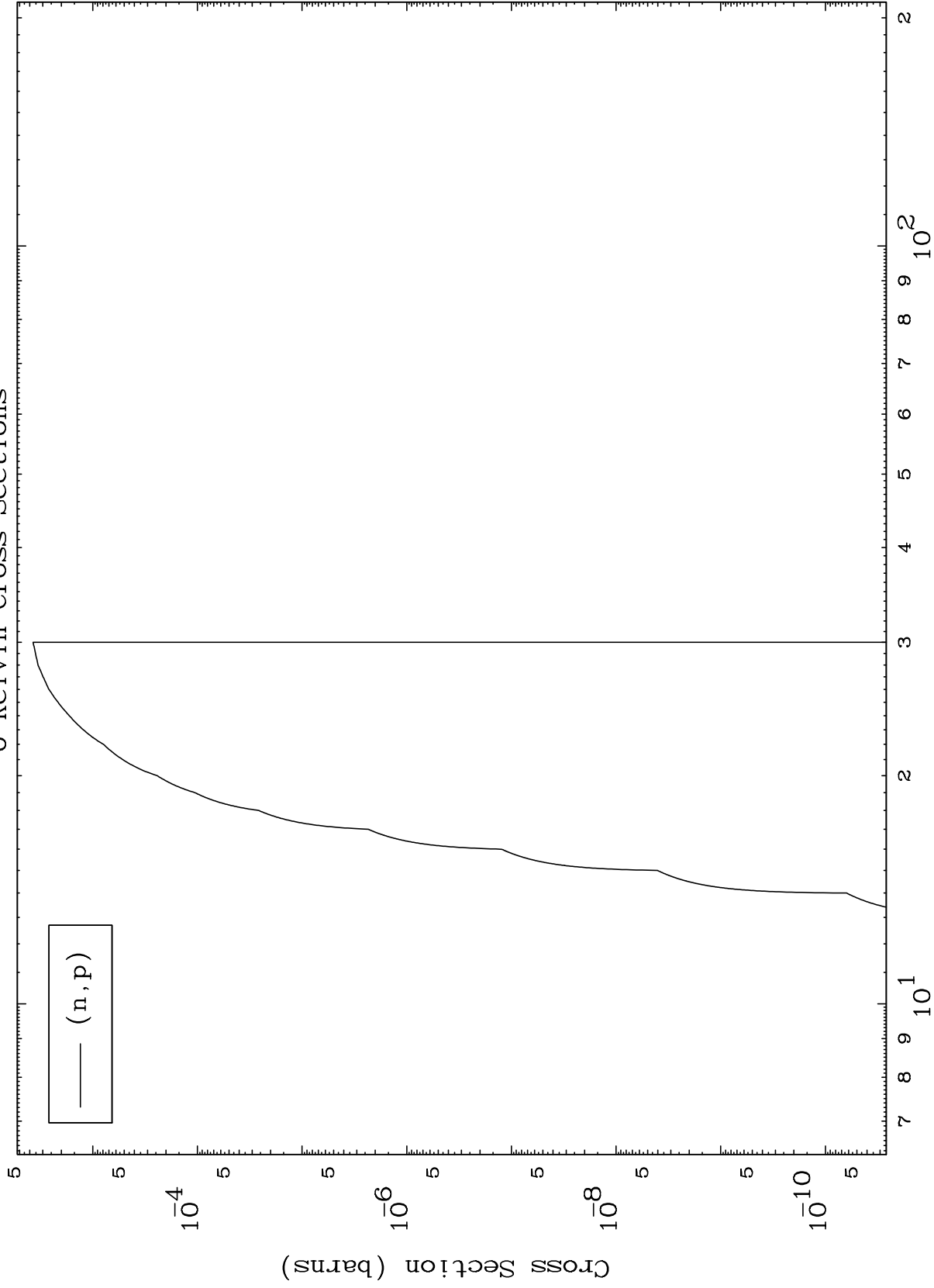
77-Ir-194m

Incident Energy (MeV)

MAT 7735

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

77-Ir-194m



5

Incident Energy (MeV)

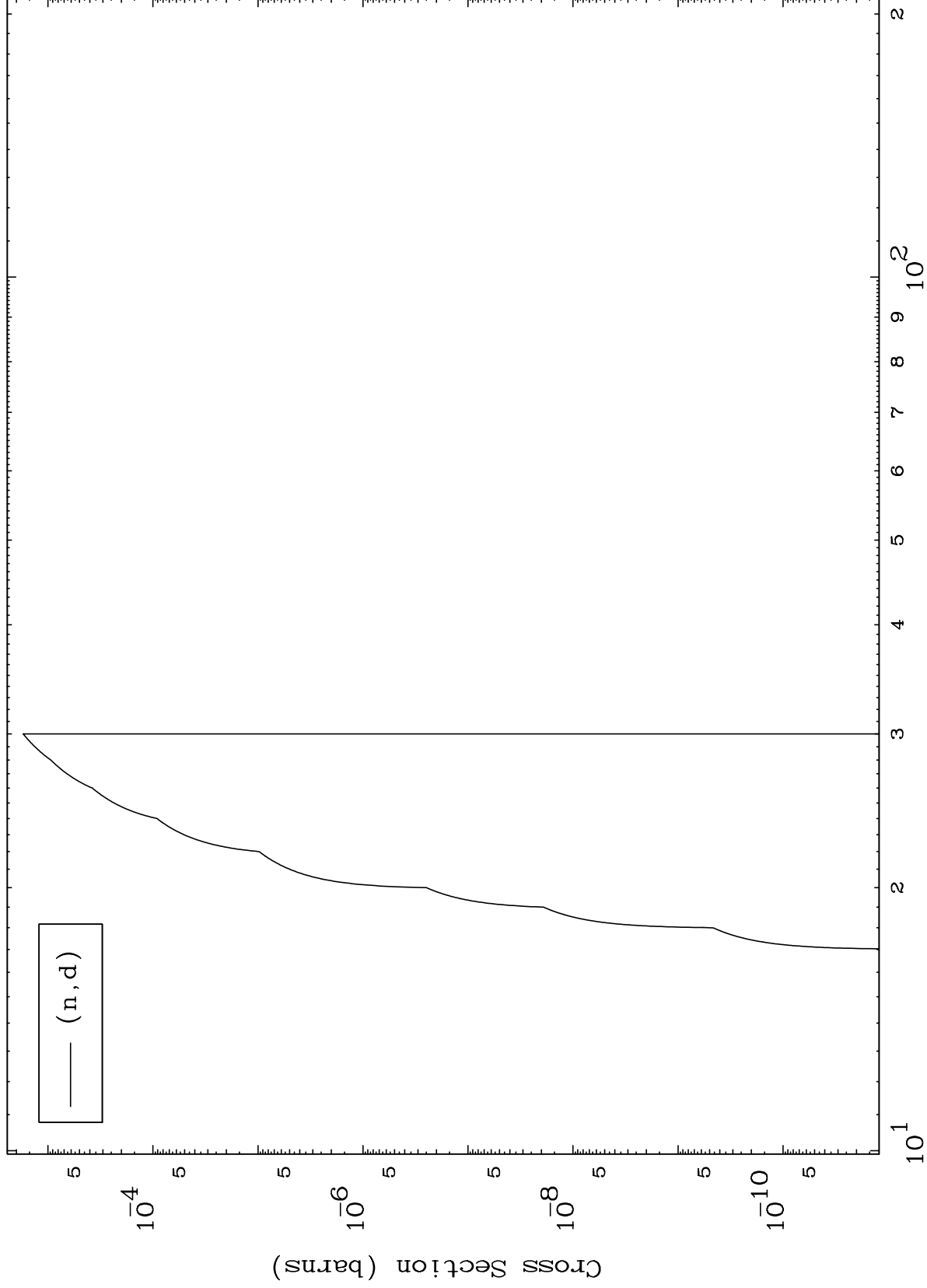
77-Ir-194m

MAT 7735

( $\alpha, d$ ) Levels

$^{77}\text{Ir}-194\text{m}$

0 Kelvin Cross Sections



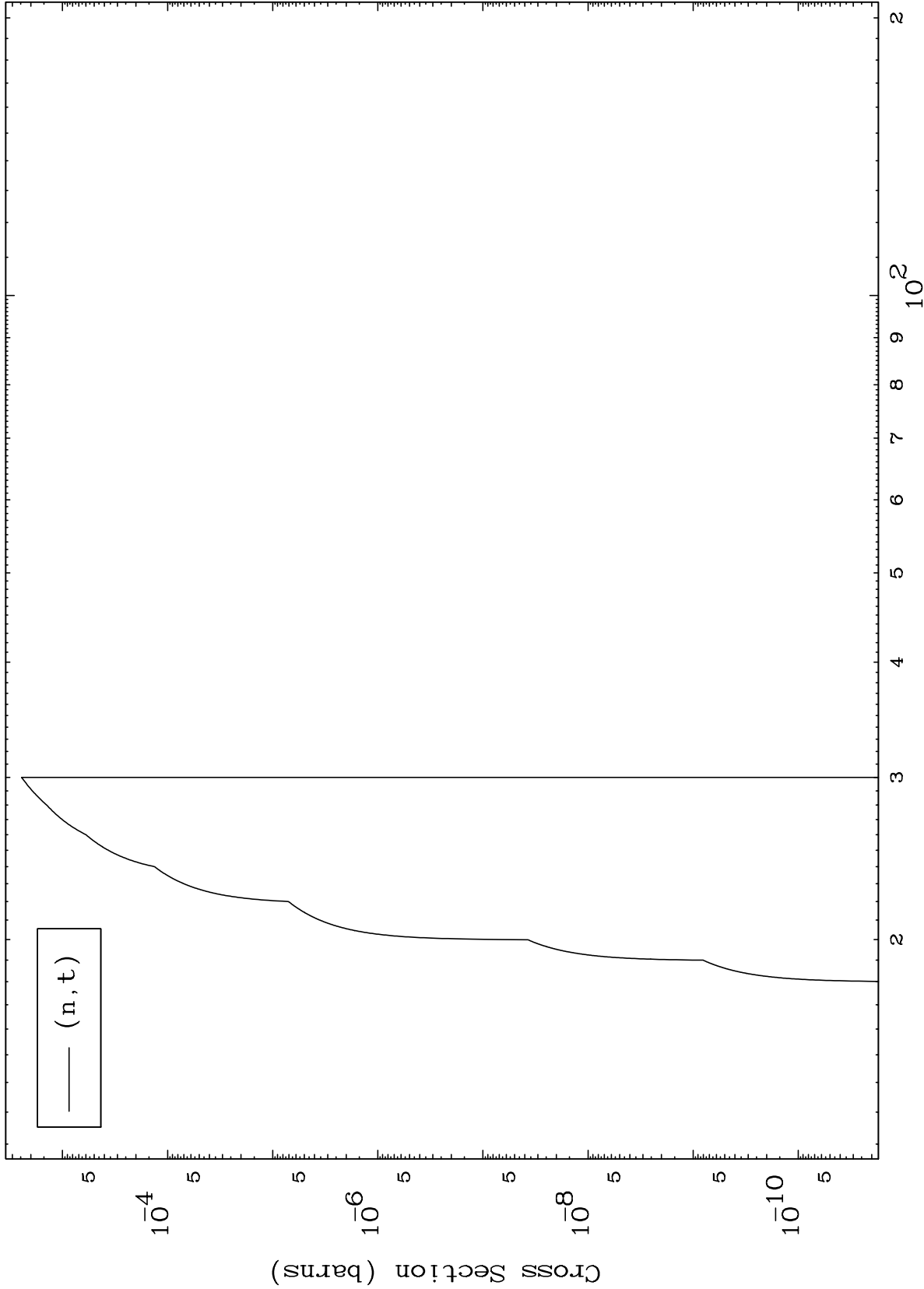
Incident Energy (MeV)

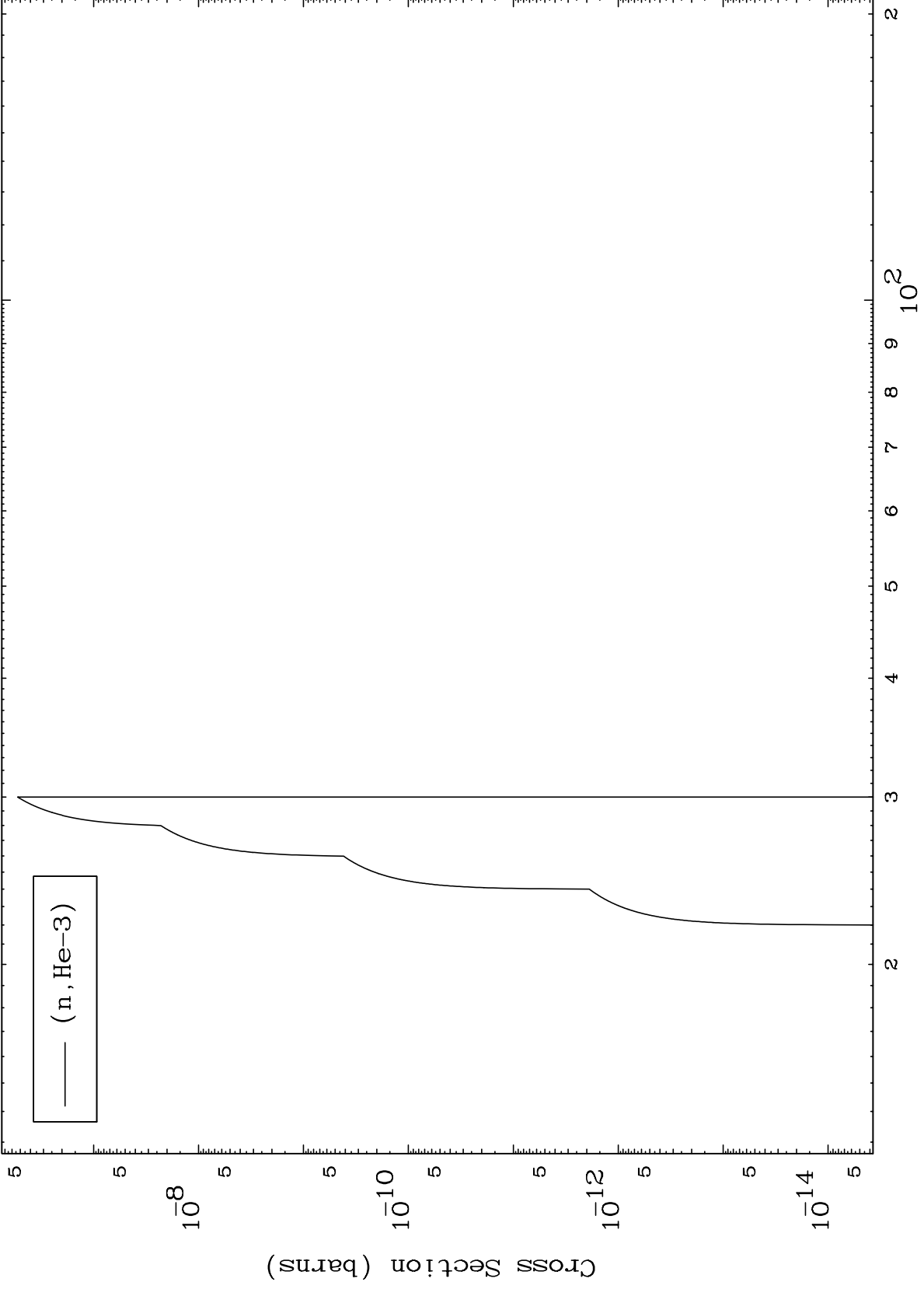
$^{77}\text{Ir}-194\text{m}$

MAT 7735

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

77-Ir-194m



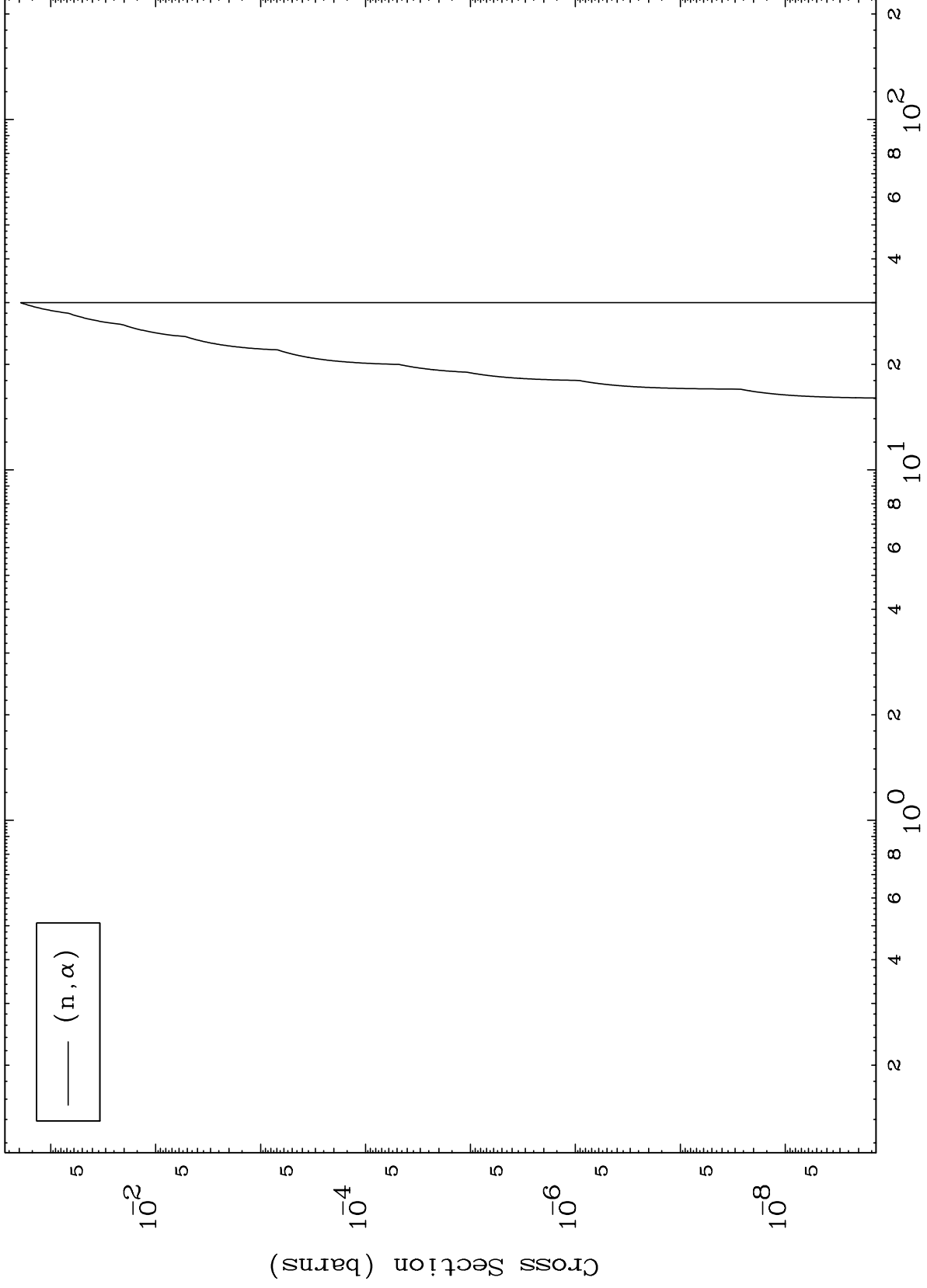


MAT 7735

( $\alpha, \alpha$ ) Levels

77-Ir-194m

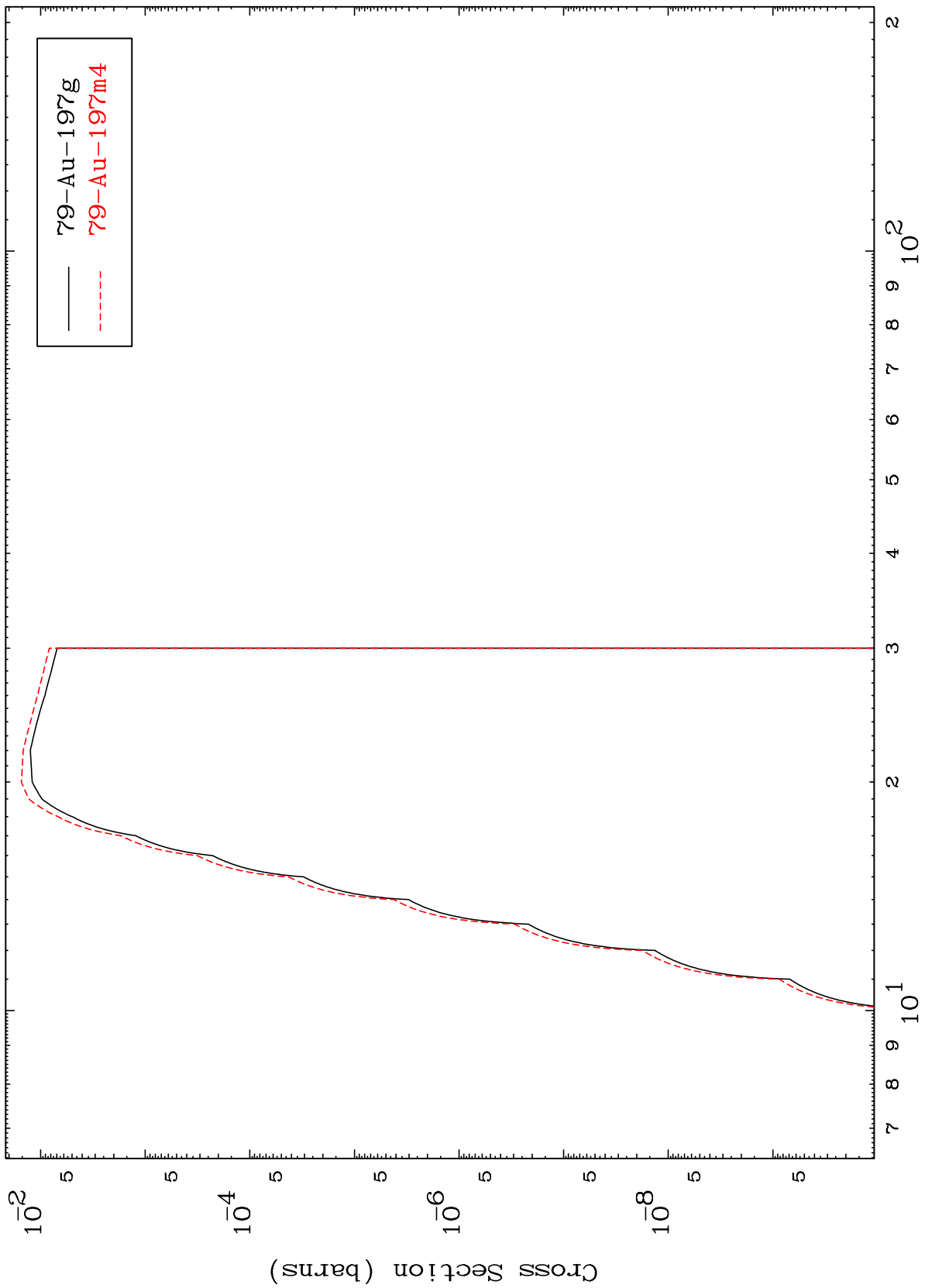
0 Kelvin Cross Sections



MAT 7735

<sup>77</sup>Ir-194m

Inelastic  
Radionuclide Production Cross Section



<sup>77</sup>Ir-194m

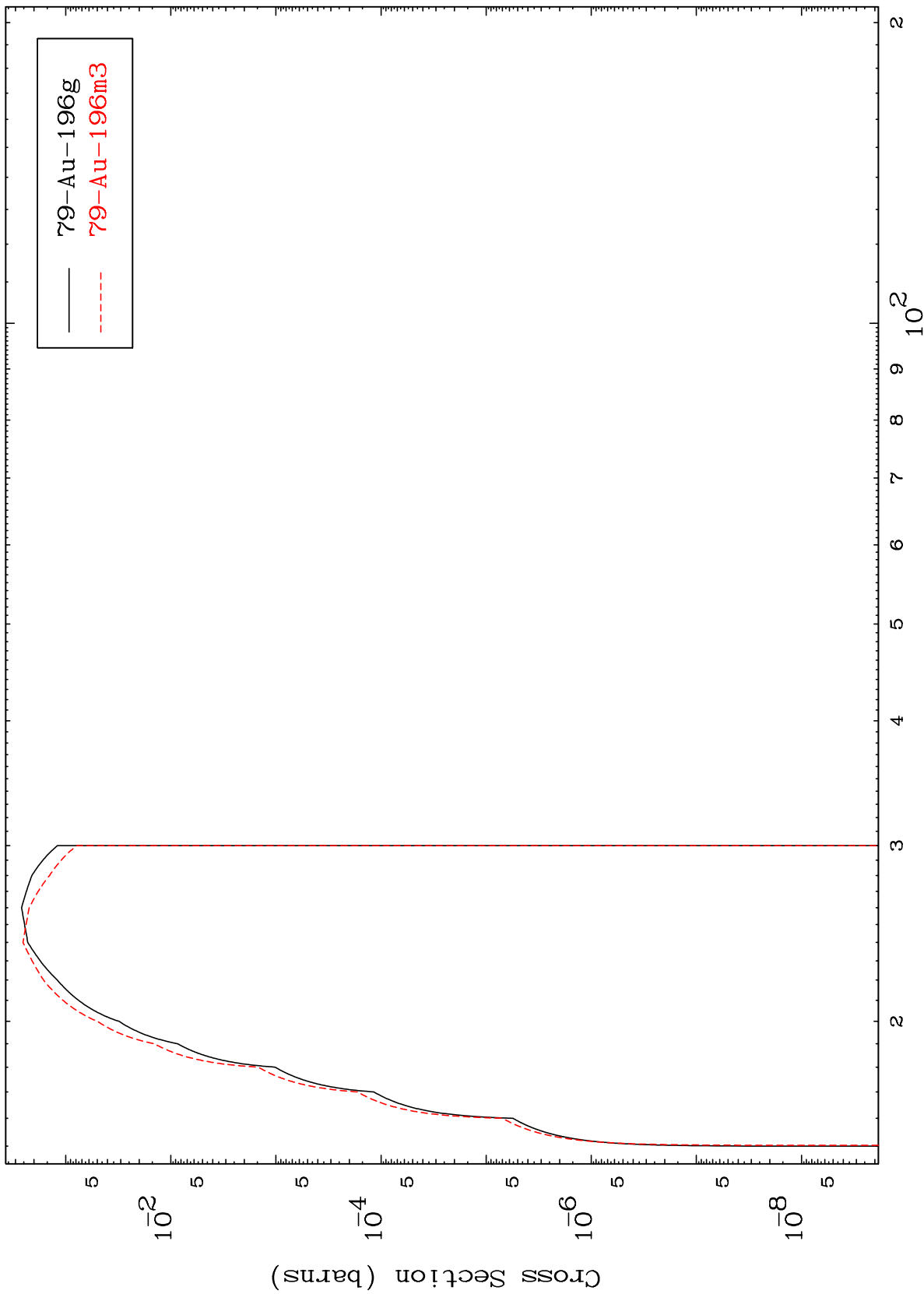
Incident Energy (MeV)

10

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<sup>77</sup>Ir-194m

(n,2n)  
Radionuclide Production Cross Section



<sup>77</sup>Ir-194m

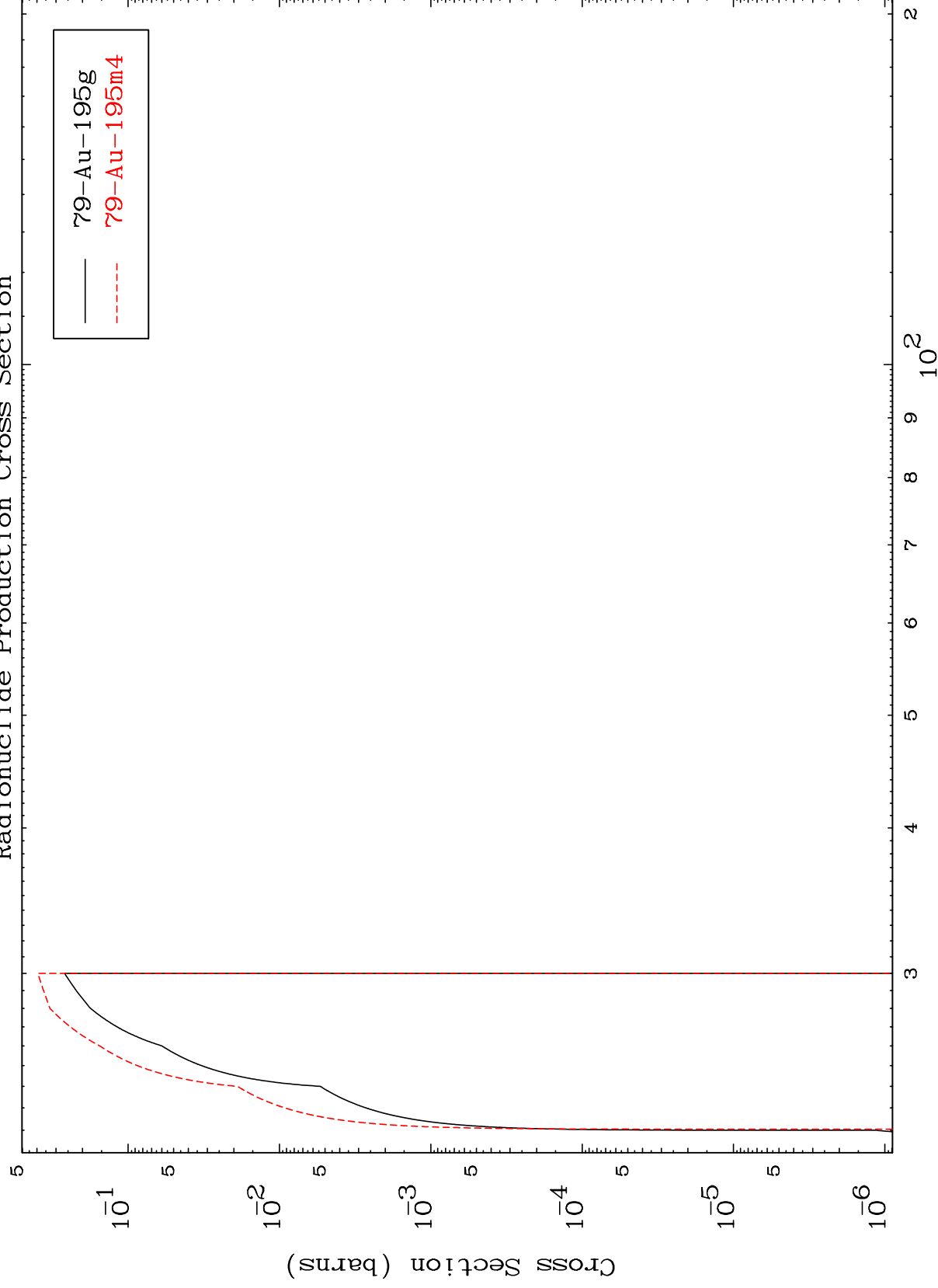
Incident Energy (MeV)

MAT 7735

(n,3n)

<sup>77</sup>Ir-194m

Radionuclide Production Cross Section



12

Incident Energy (MeV)

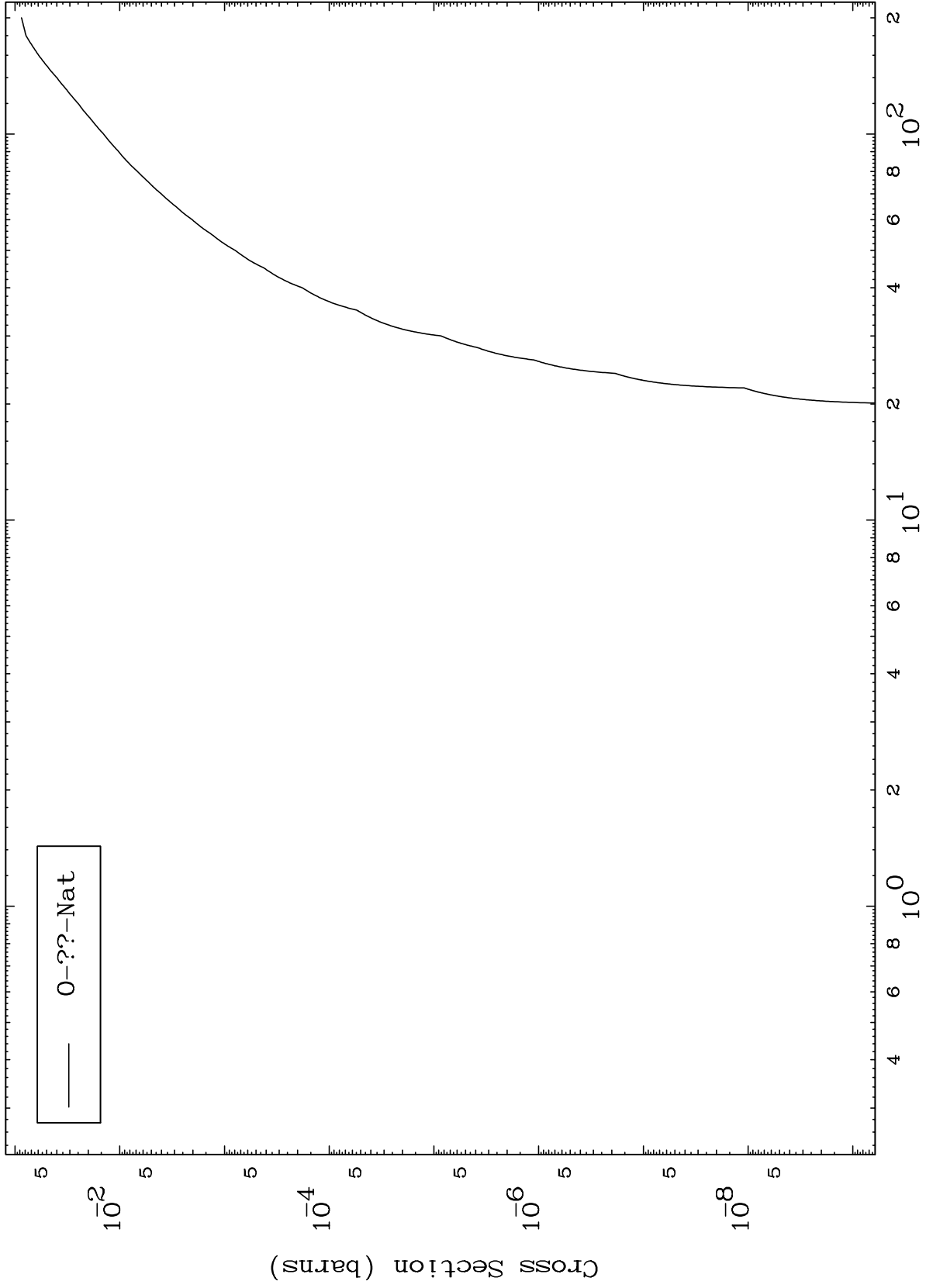
<sup>77</sup>Ir-194m

MAT 7735

Fission

<sup>77</sup>Ir-194m

Radionuclide Production Cross Section

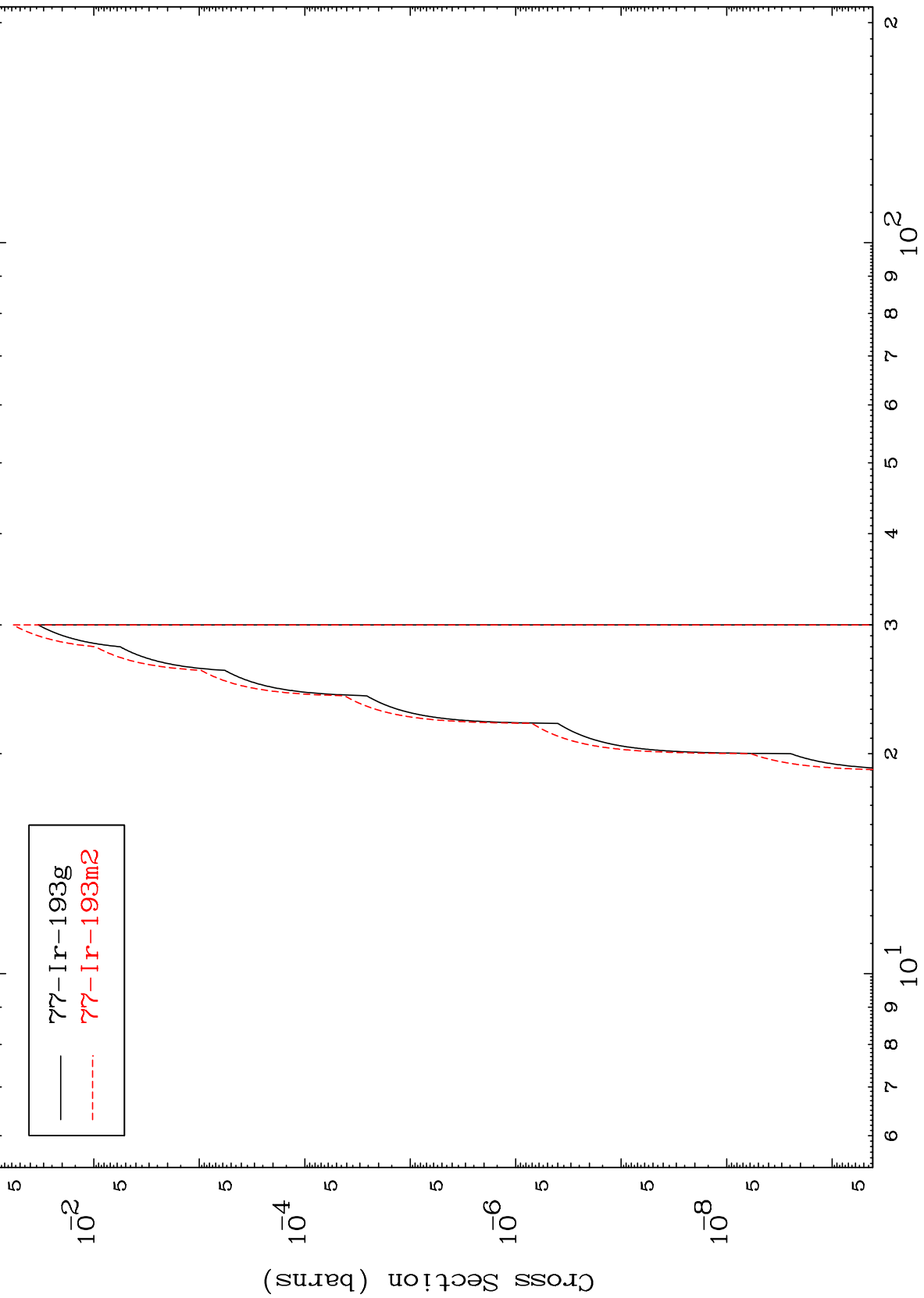


MAT 7735

$(n, n') \alpha$

$^{77}\text{Ir}-194\text{m}$

Radionuclide Production Cross Section



—  $^{77}\text{Ir}-193\text{g}$   
- - -  $^{77}\text{Ir}-193\text{m}2$

14

Incident Energy (MeV)

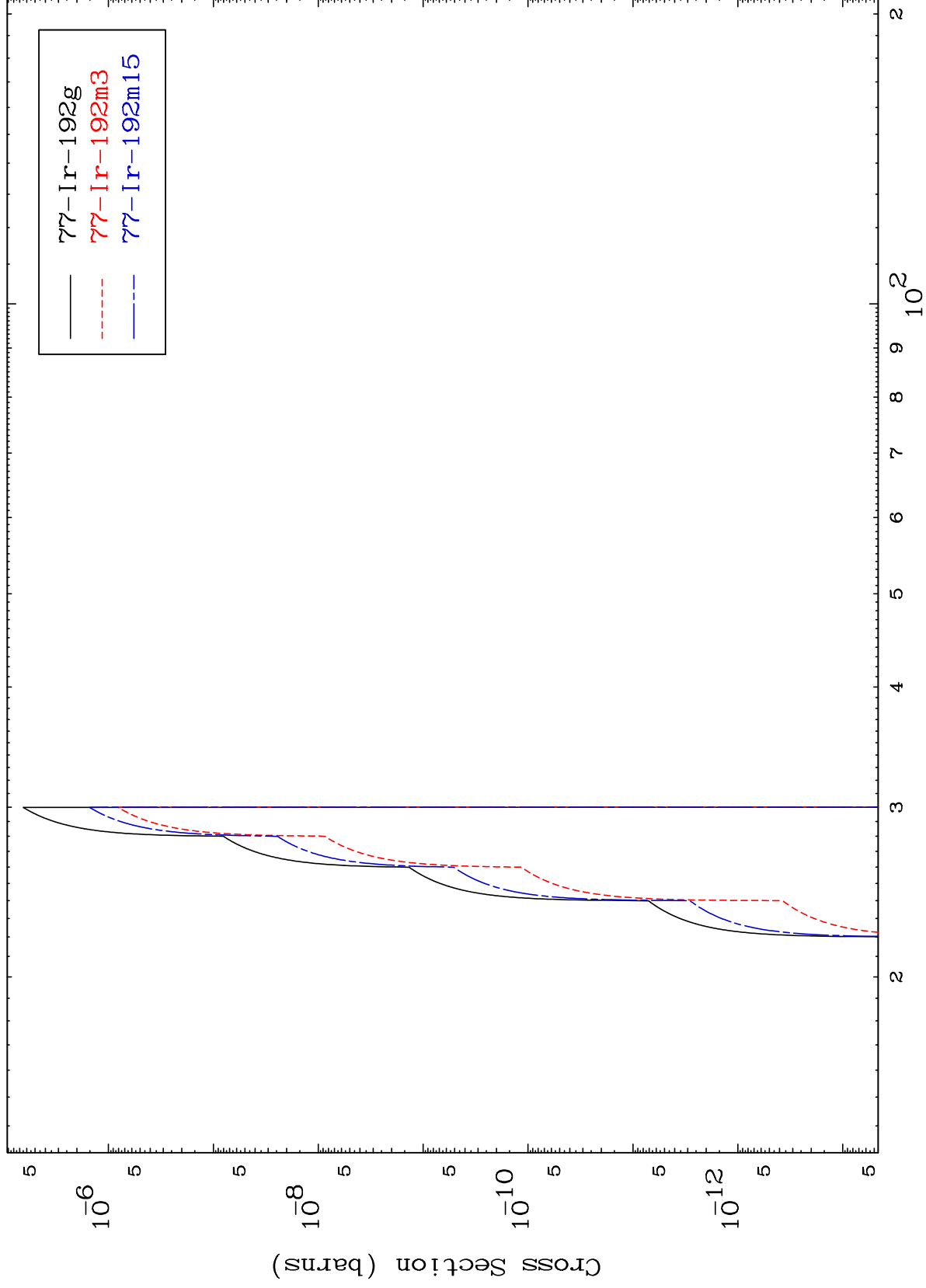
$^{77}\text{Ir}-194\text{m}$

MAT 7735

$(n,2n) \alpha$

$^{77}\text{Ir}-194\text{m}$

Radionuclide Production Cross Section



15

Incident Energy (MeV)

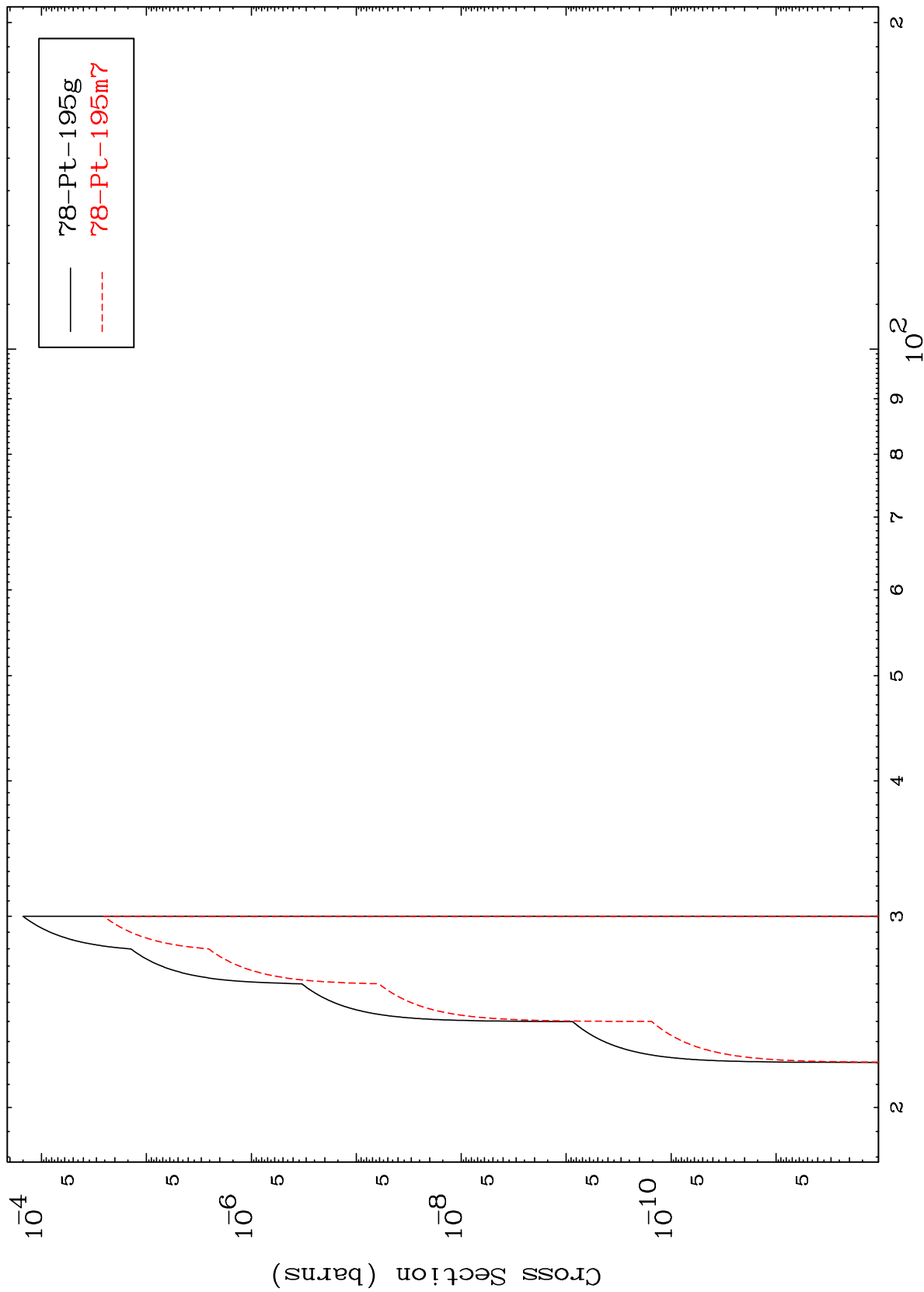
$^{77}\text{Ir}-194\text{m}$

MAT 7735

77-Ir-194m

(n,n') d

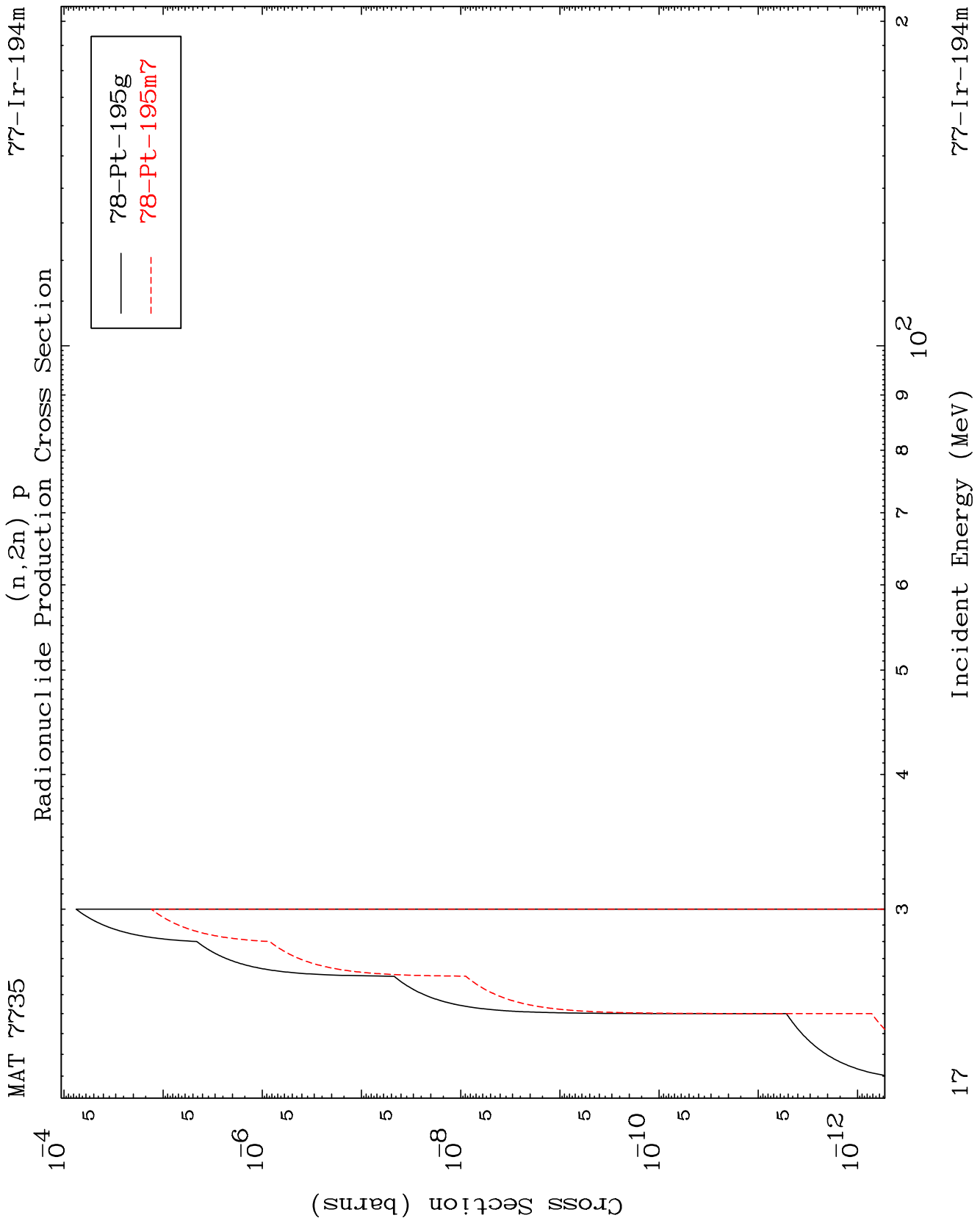
Radionuclide Production Cross Section



16

Incident Energy (MeV)

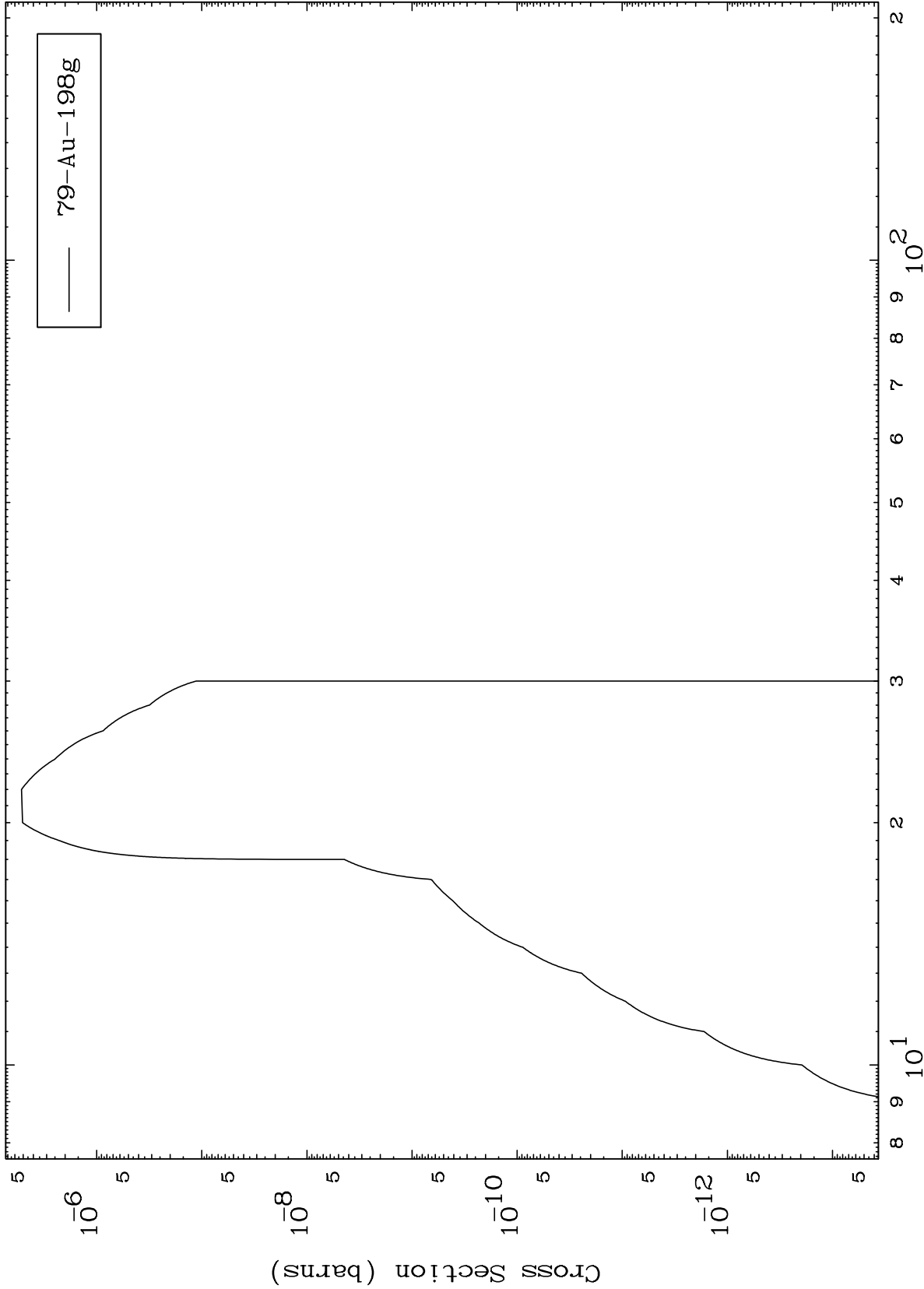
77-Ir-194m



MAT 7735

<sup>77</sup>Ir-194m

(n,γ)  
Radionuclide Production Cross Section



<sup>77</sup>Ir-194m

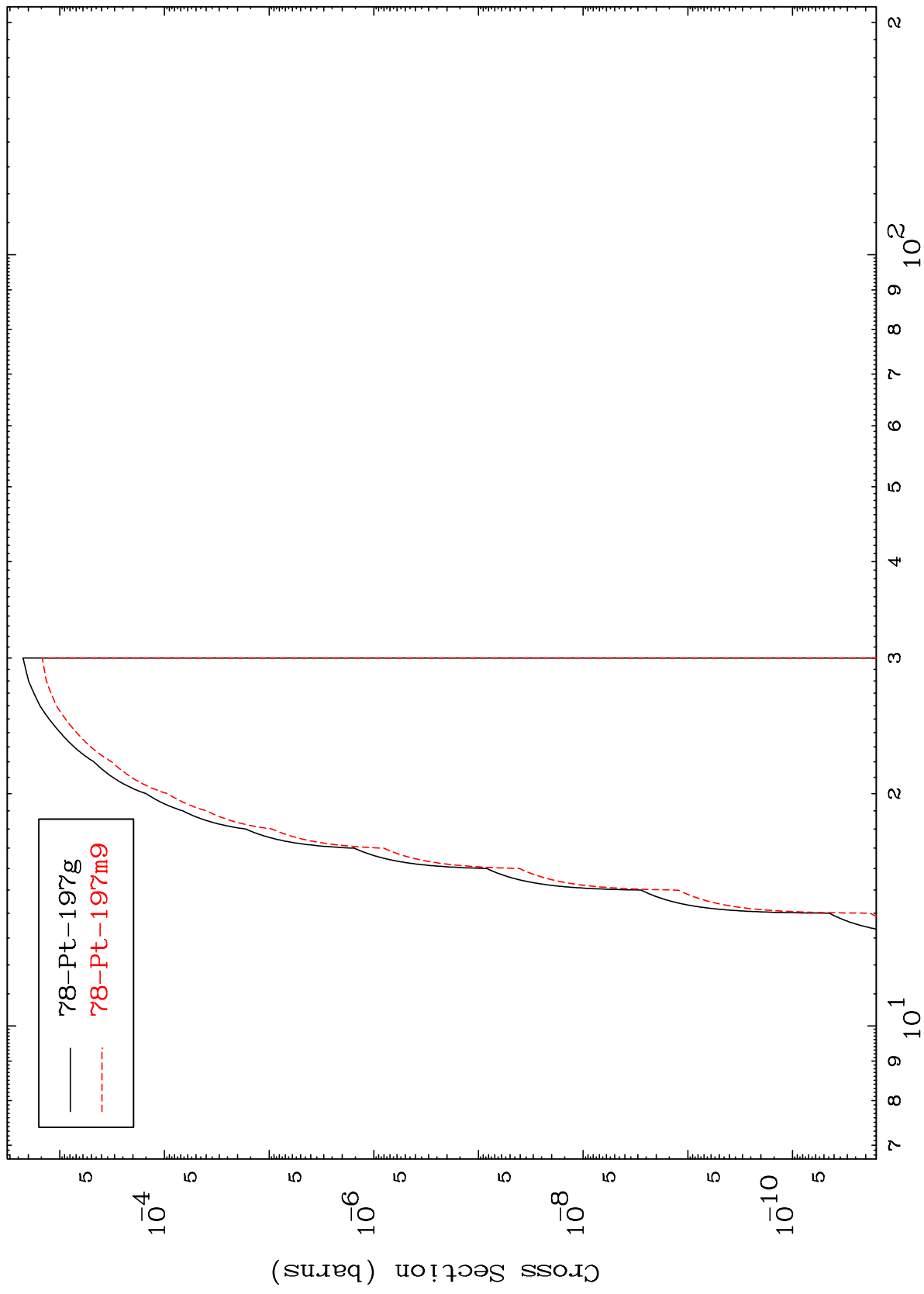
Incident Energy (MeV)

18

MAT 7735

<sup>77</sup>Ir-194m

Radionuclide Production Cross Section (n,p)



19

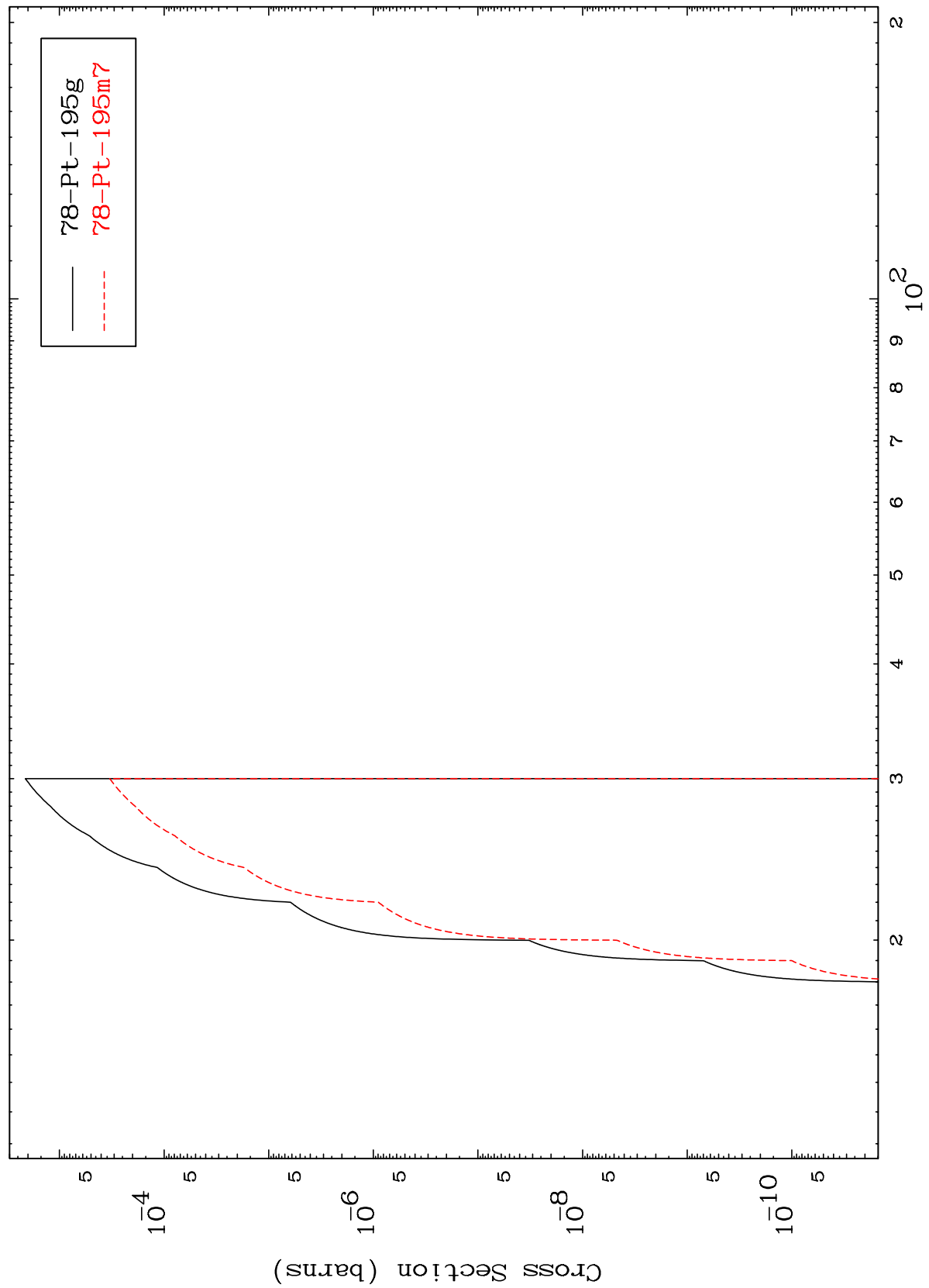
Incident Energy (MeV)

<sup>77</sup>Ir-194m

MAT 7735

77-Ir-194m

(n, t)  
Radionuclide Production Cross Section



77-Ir-194m

Incident Energy (MeV)

20

Radionuclide Production Cross Section

