

Program EVALPLOT  
(Version 2021-1)

by

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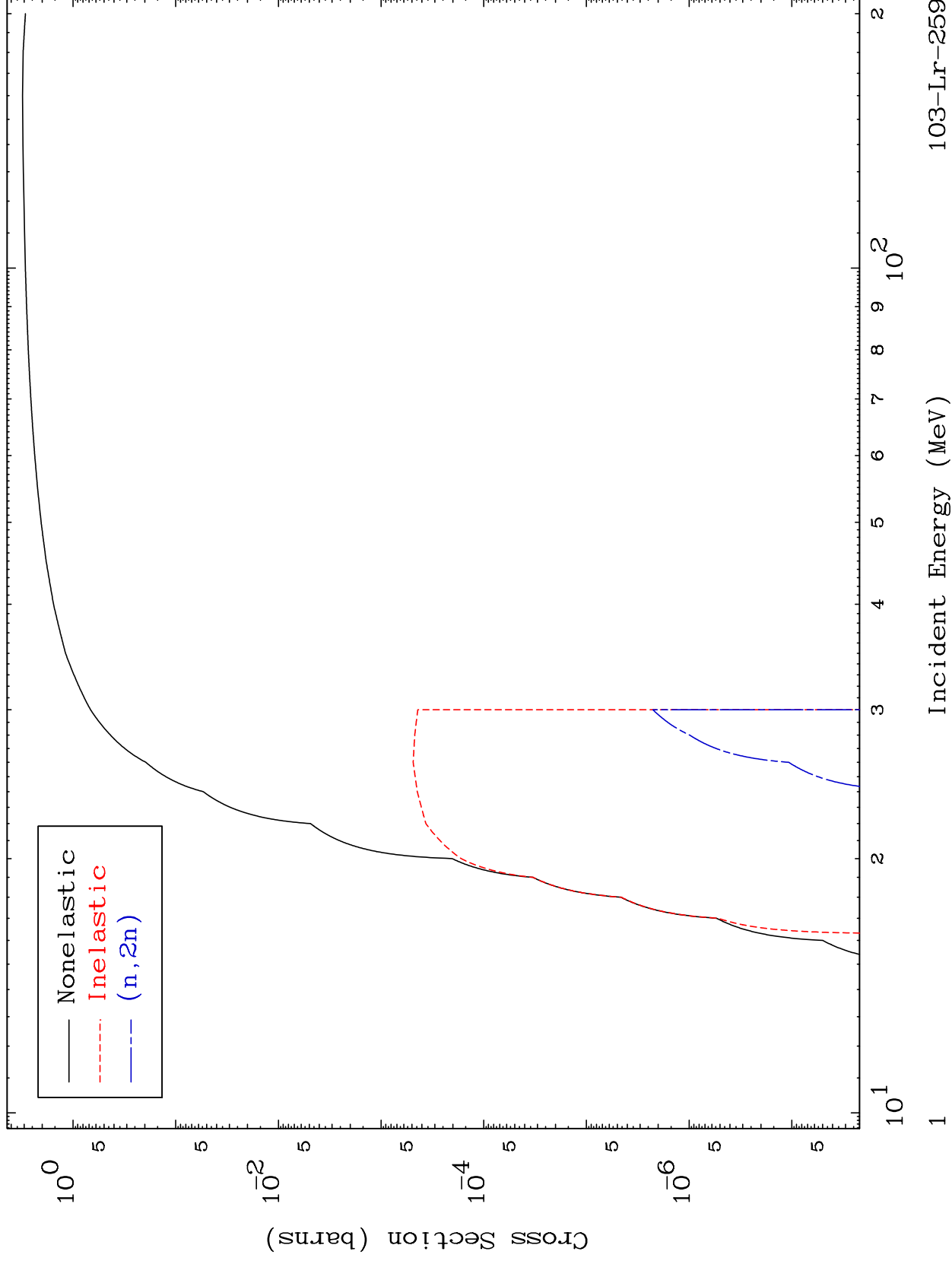
Press Mouse Button to Start

MAT 9988

$\alpha$  Major

103-Lr-259

0 Kelvin Cross Sections

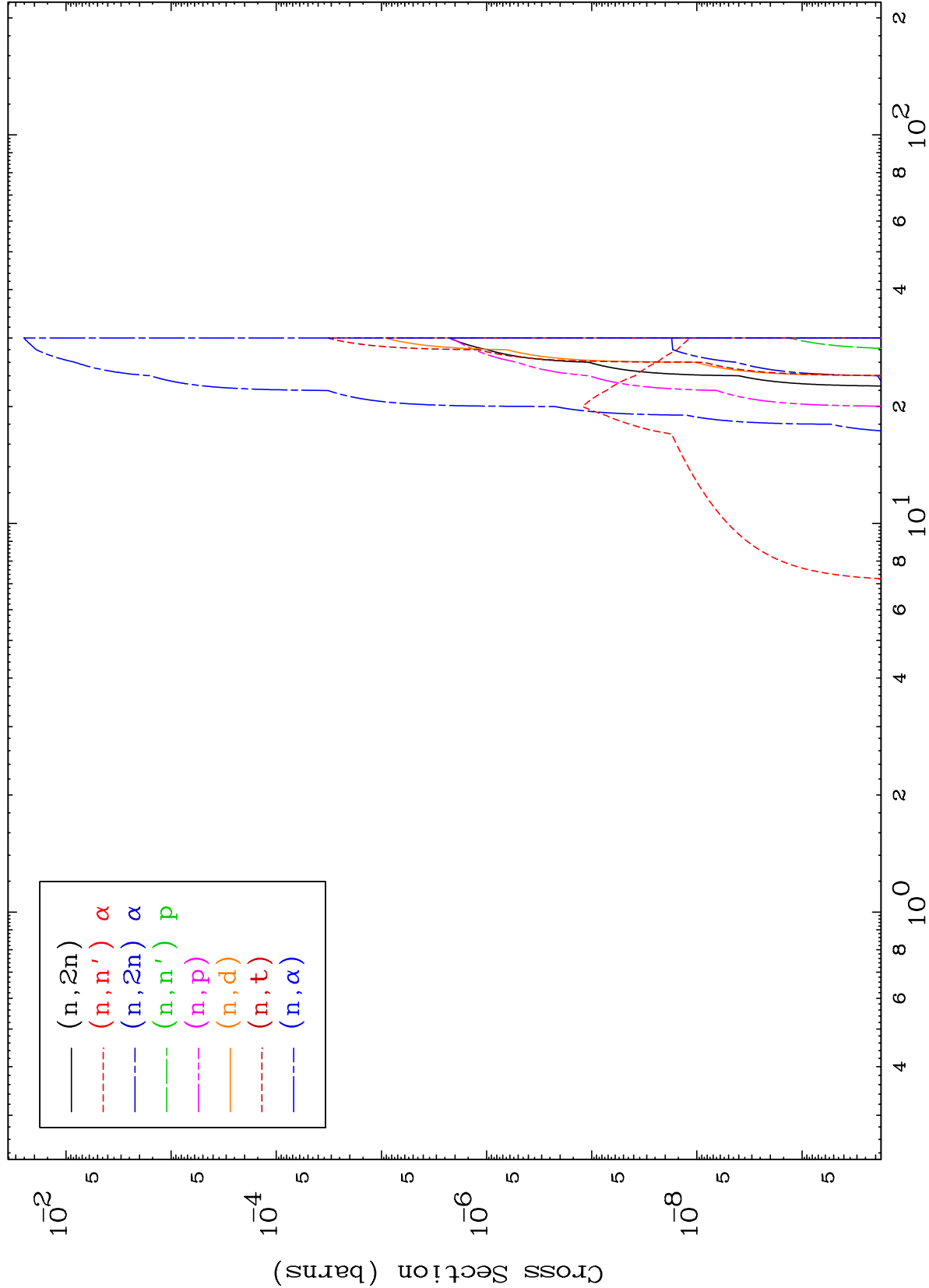


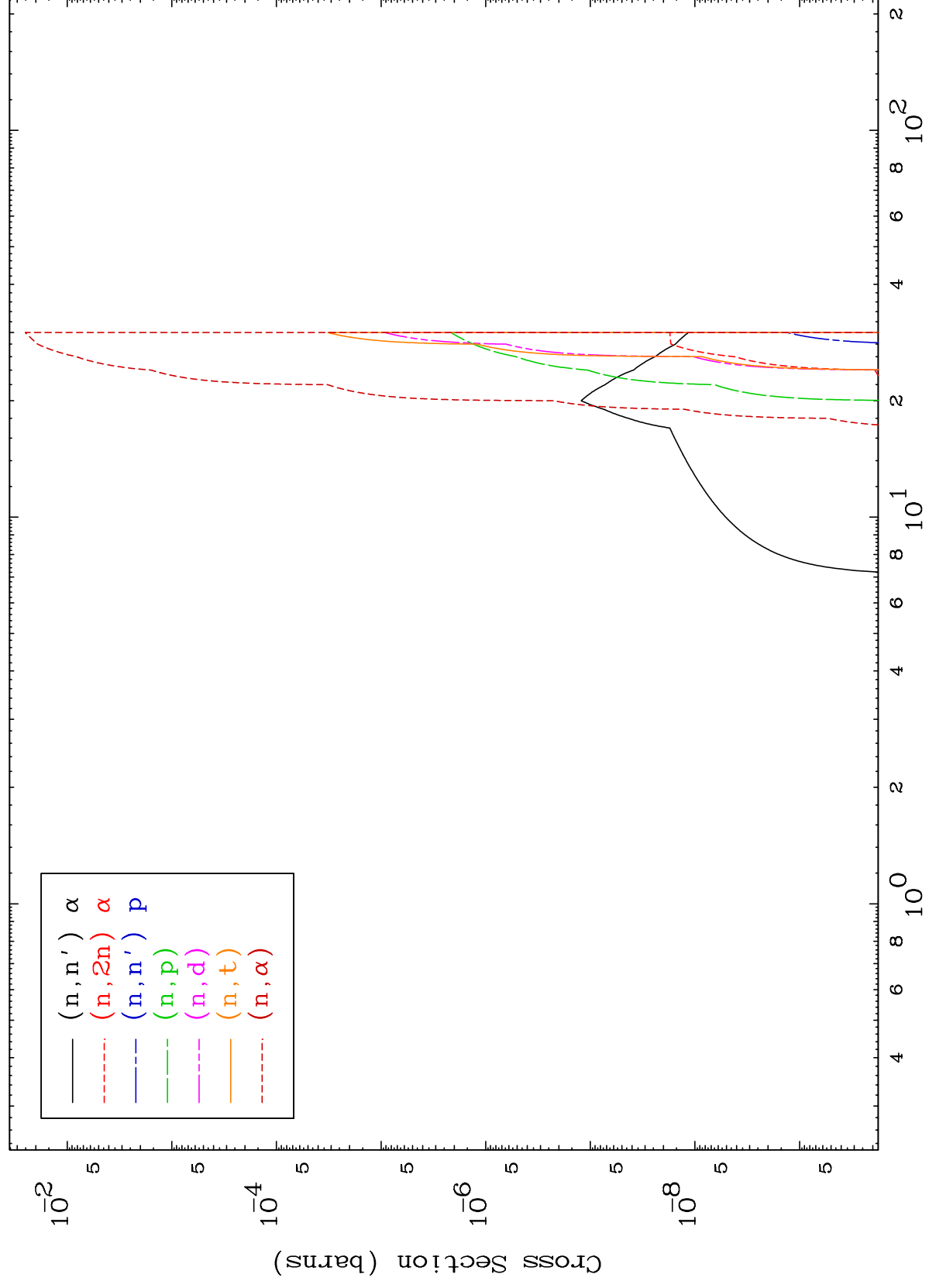
MAT 9988

$\alpha$  Neutron Absorption

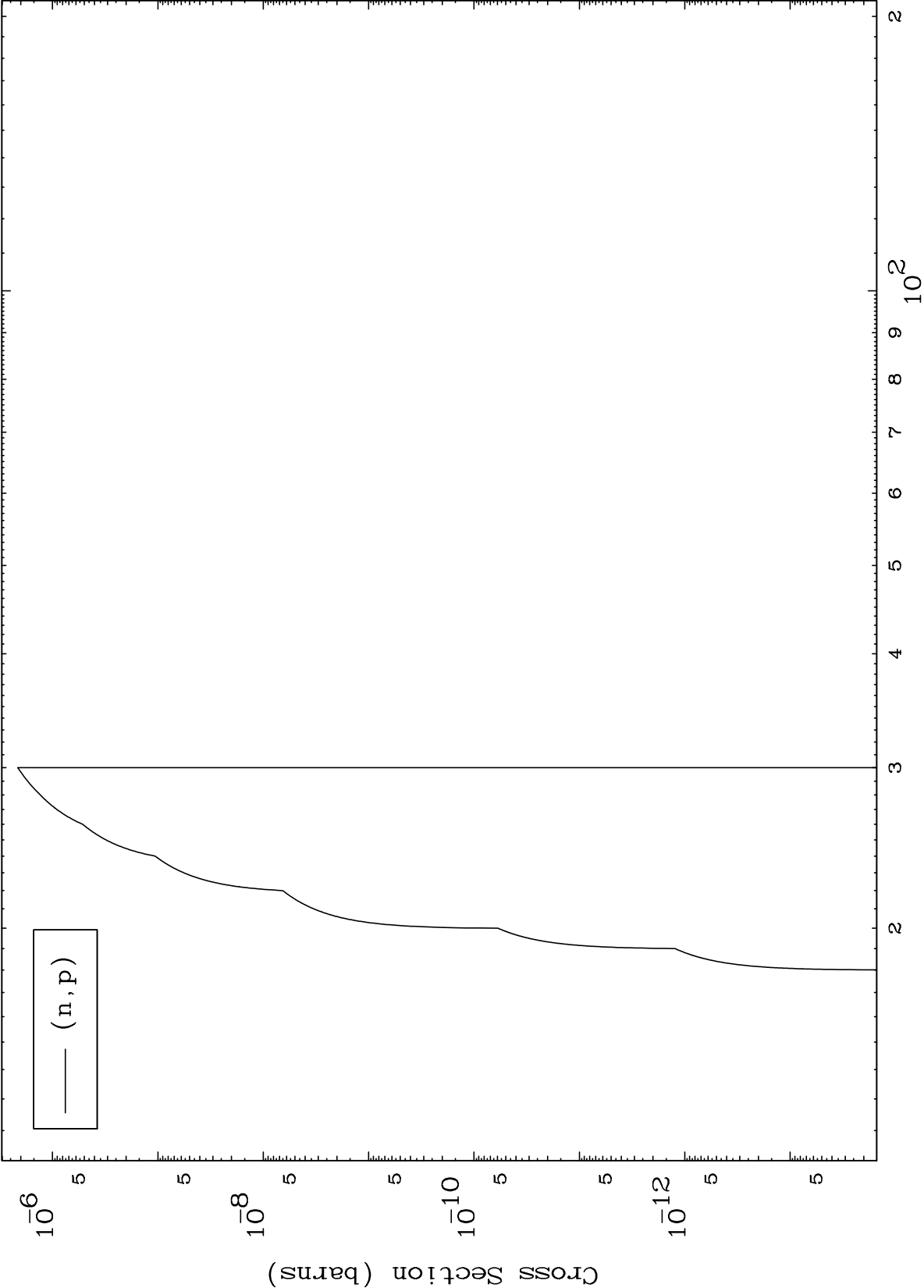
103-Lr-259

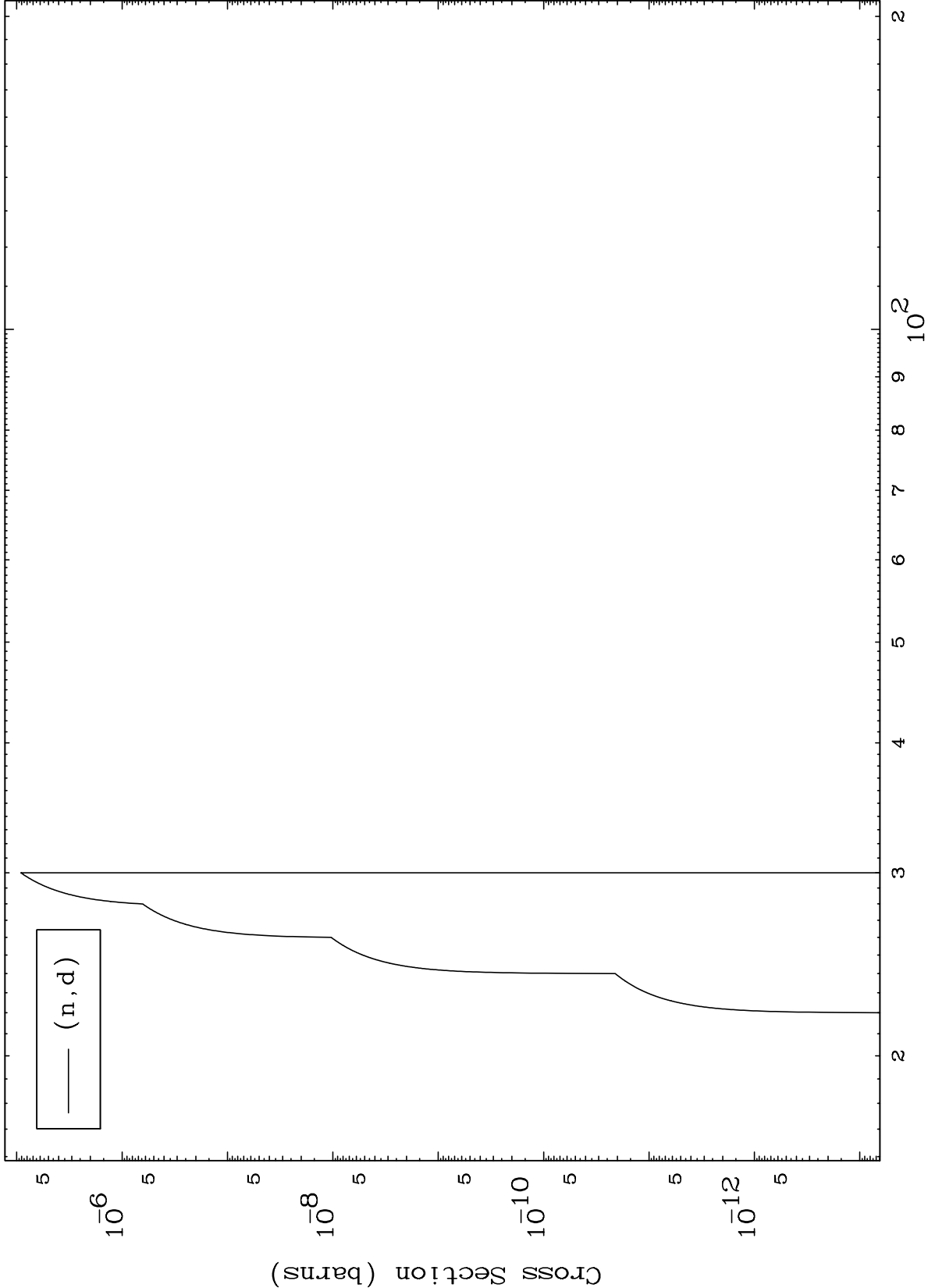
0 Kelvin Cross Sections





0 Kelvin Cross Sections

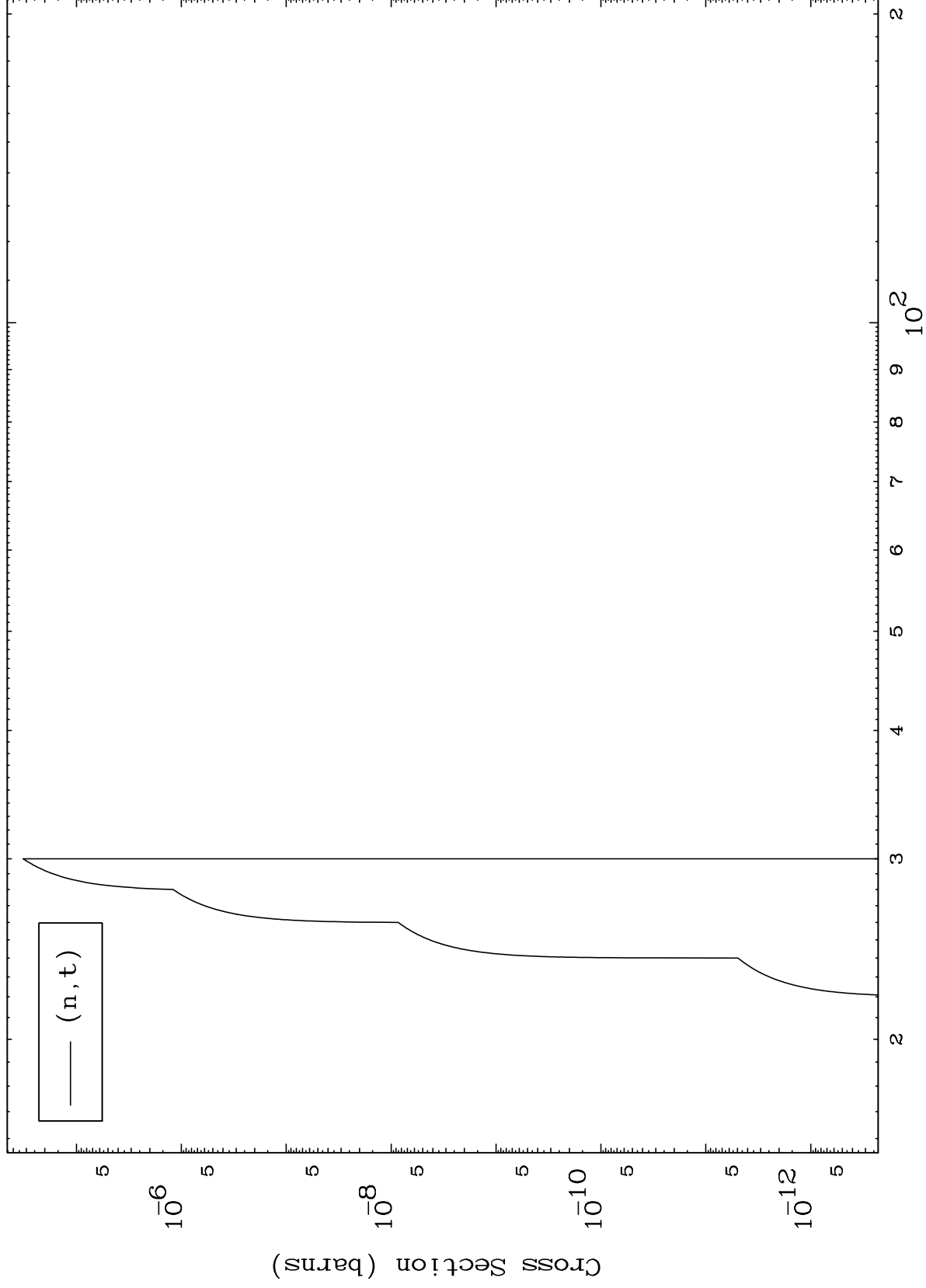




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103-Lr-259

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections



103-Lr-259

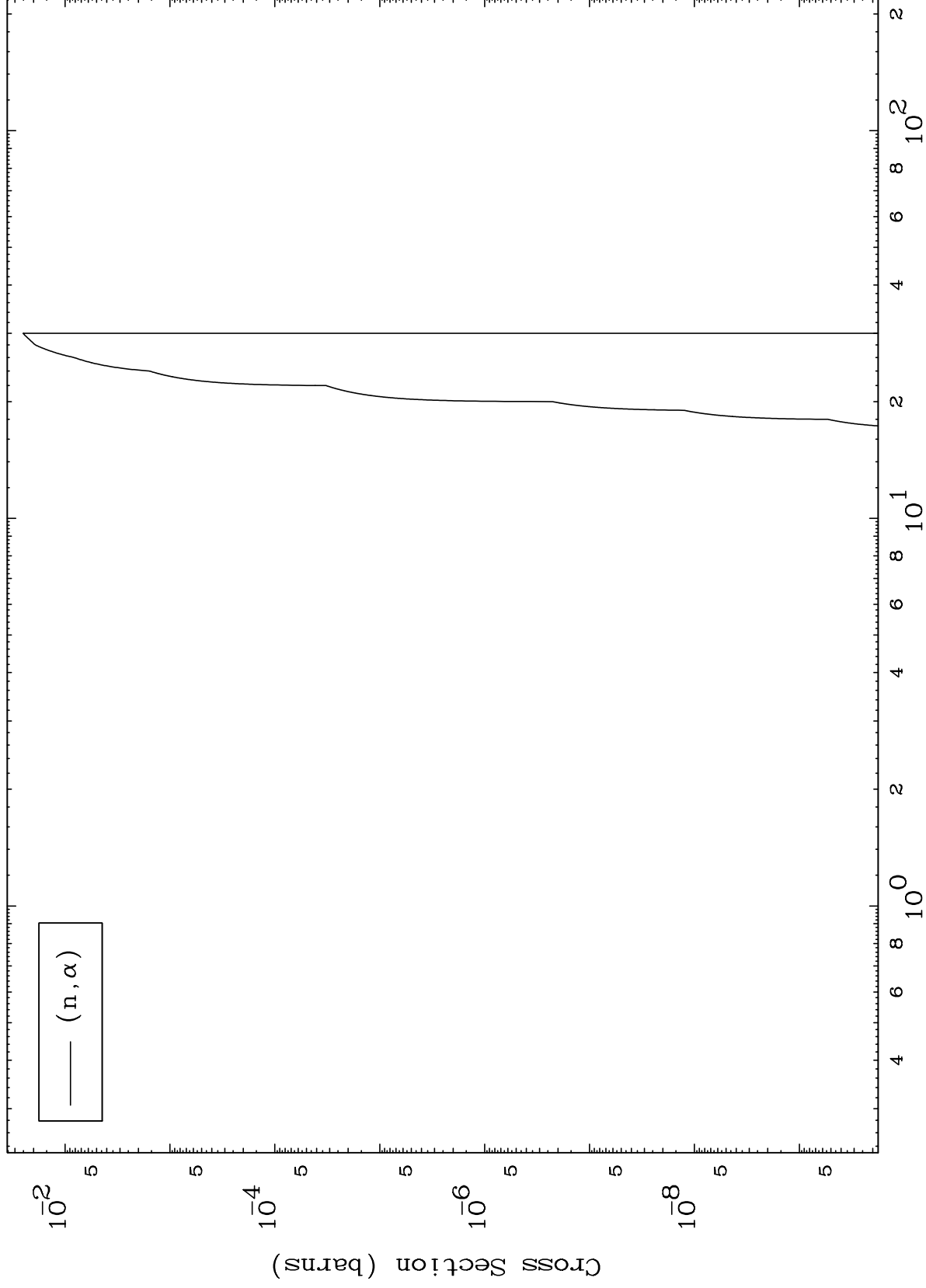
Incident Energy (MeV)

6

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103-Lr-259

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections



103-Lr-259

Incident Energy (MeV)



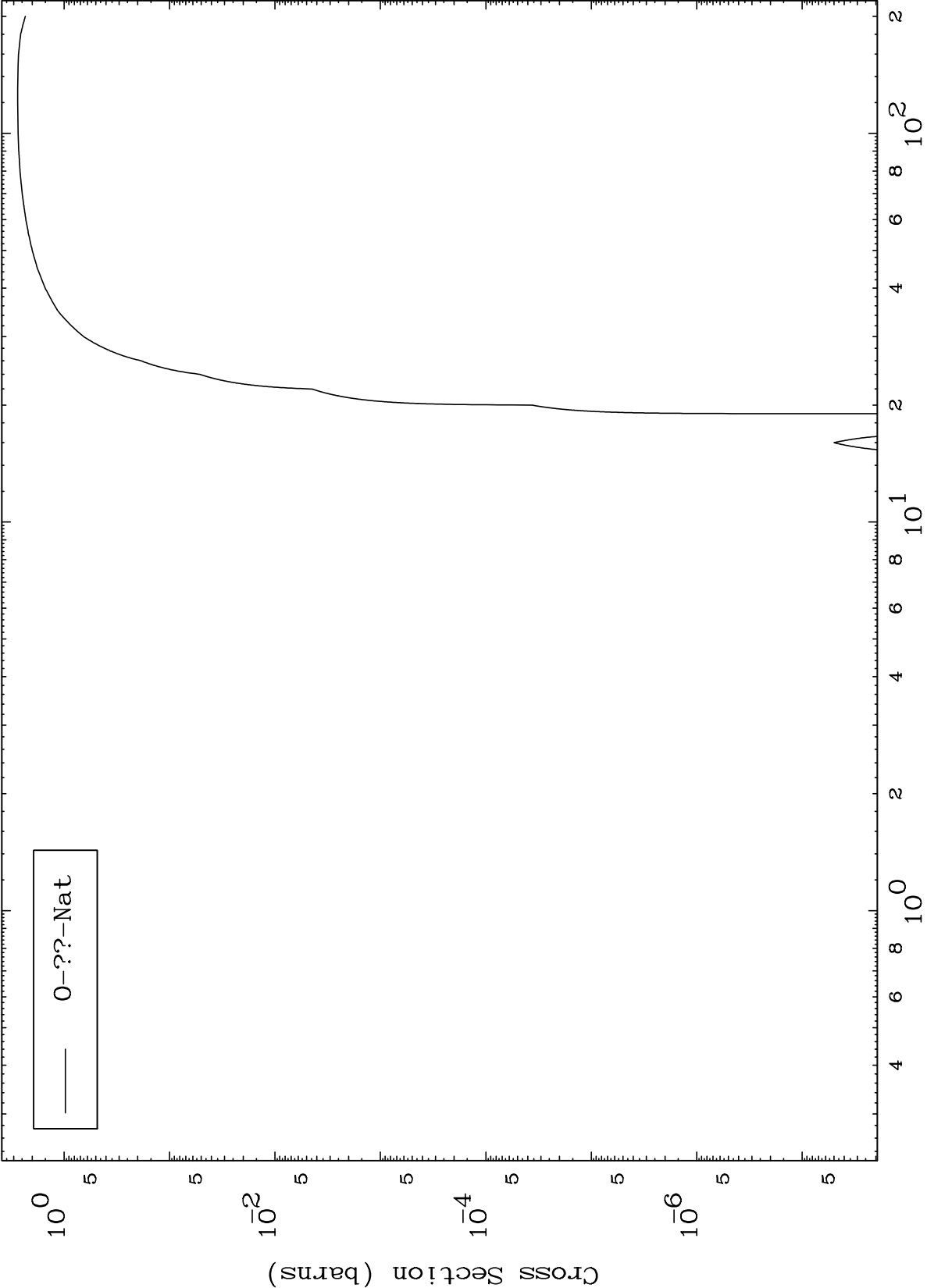
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Fission

103-Lr-259

Radionuclide Production Cross Section

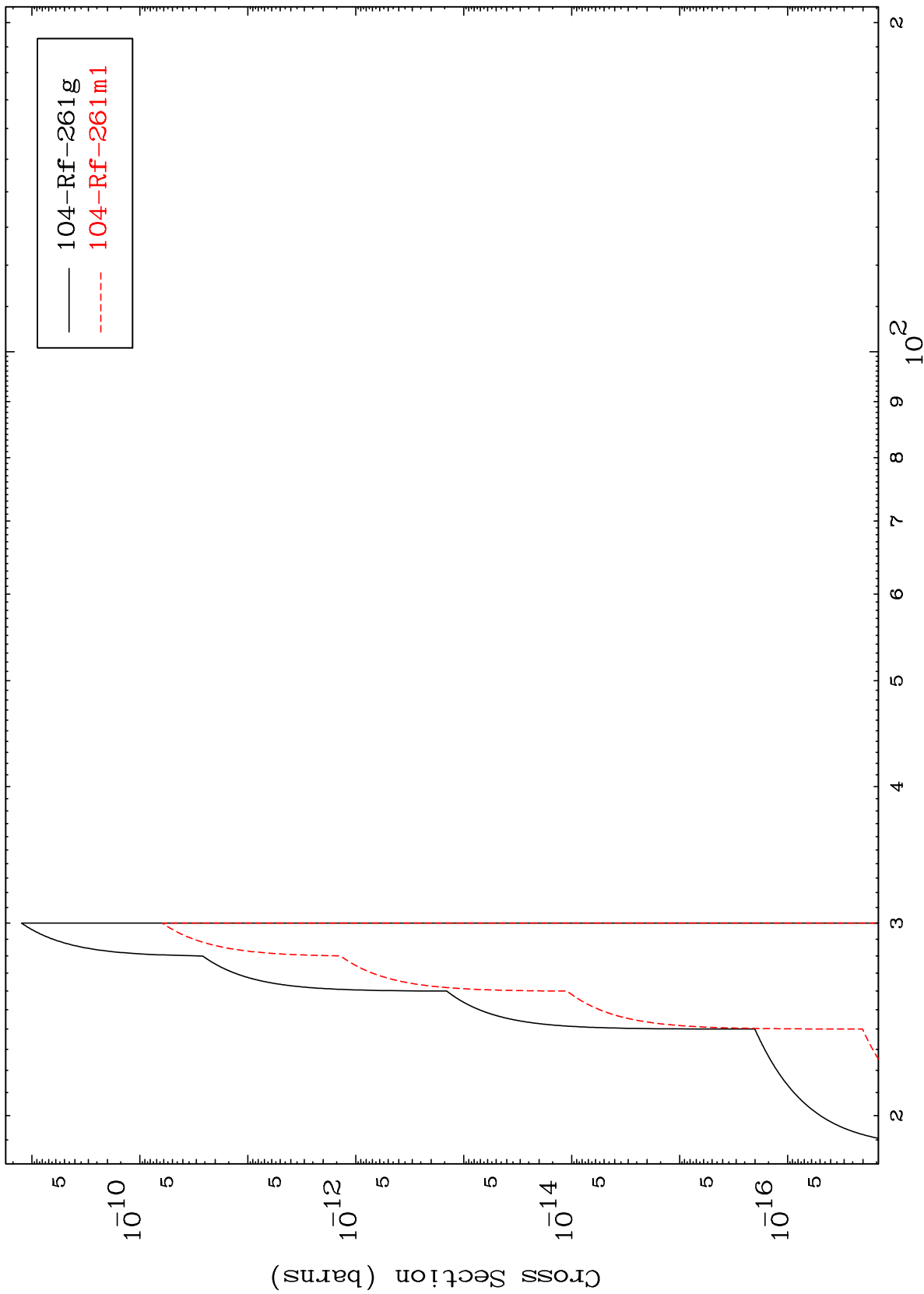
0-??-Nat



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103-Lr-259

(n,n') p  
Radionuclide Production Cross Section



103-Lr-259

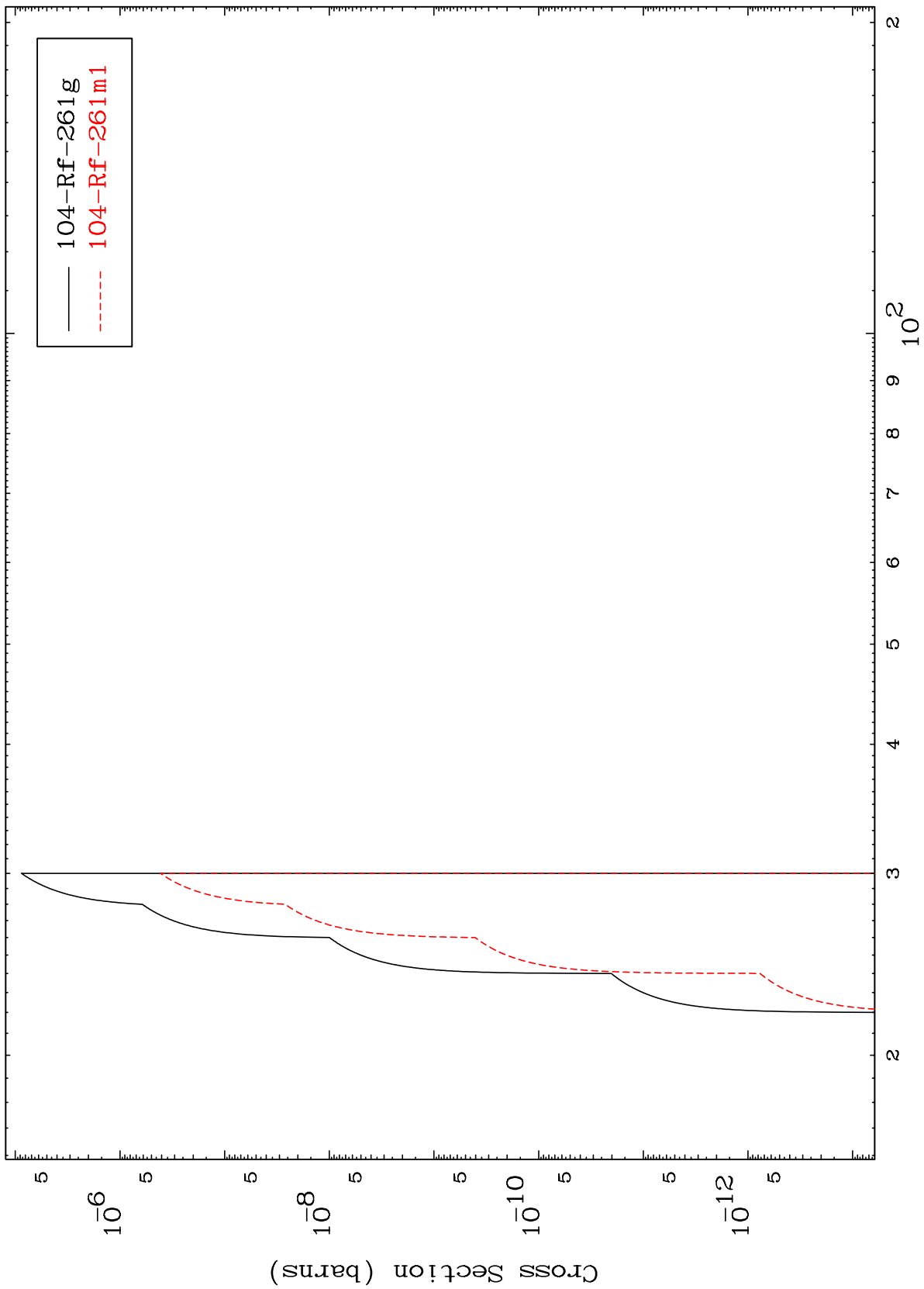
Incident Energy (MeV)

9

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103-Lr-259

(n,d)  
Radionuclide Production Cross Section



103-Lr-259

Incident Energy (MeV)

10