

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

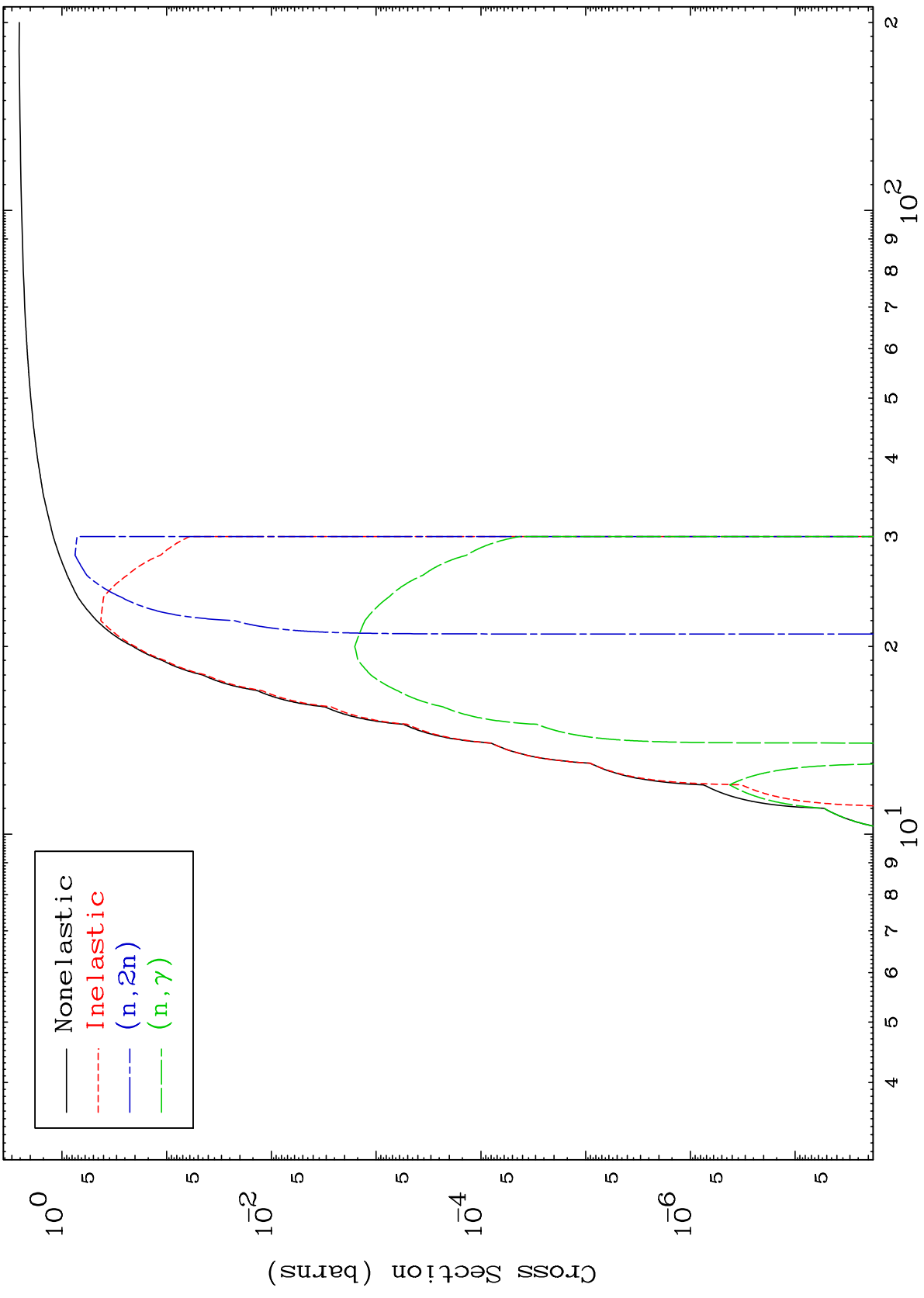
Press Mouse Button to Start

MAT 7105

$\alpha$  Major

<sup>71</sup>Lu-168m

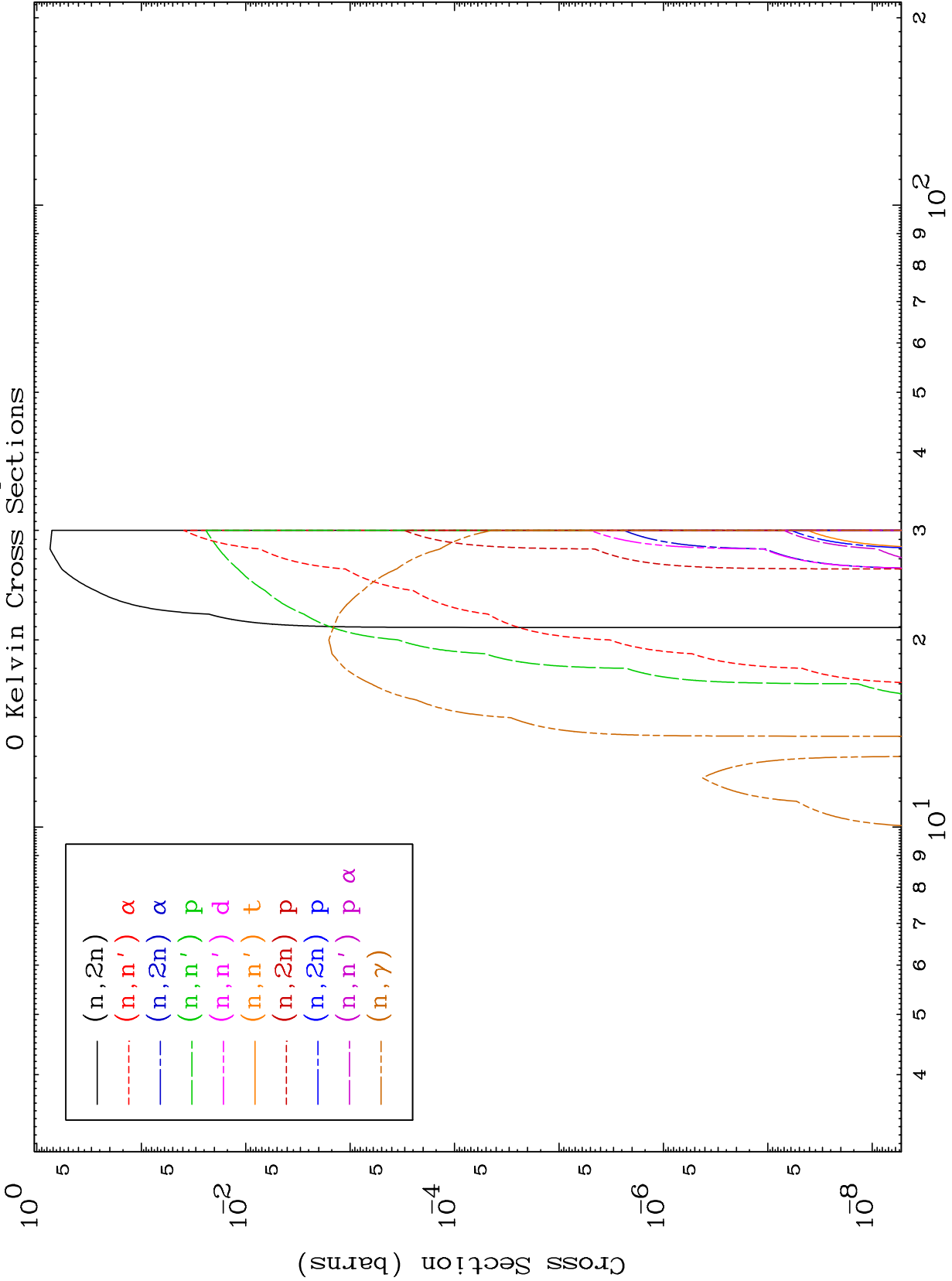
0 Kelvin Cross Sections



Incident Energy (MeV)

<sup>71</sup>Lu-168m

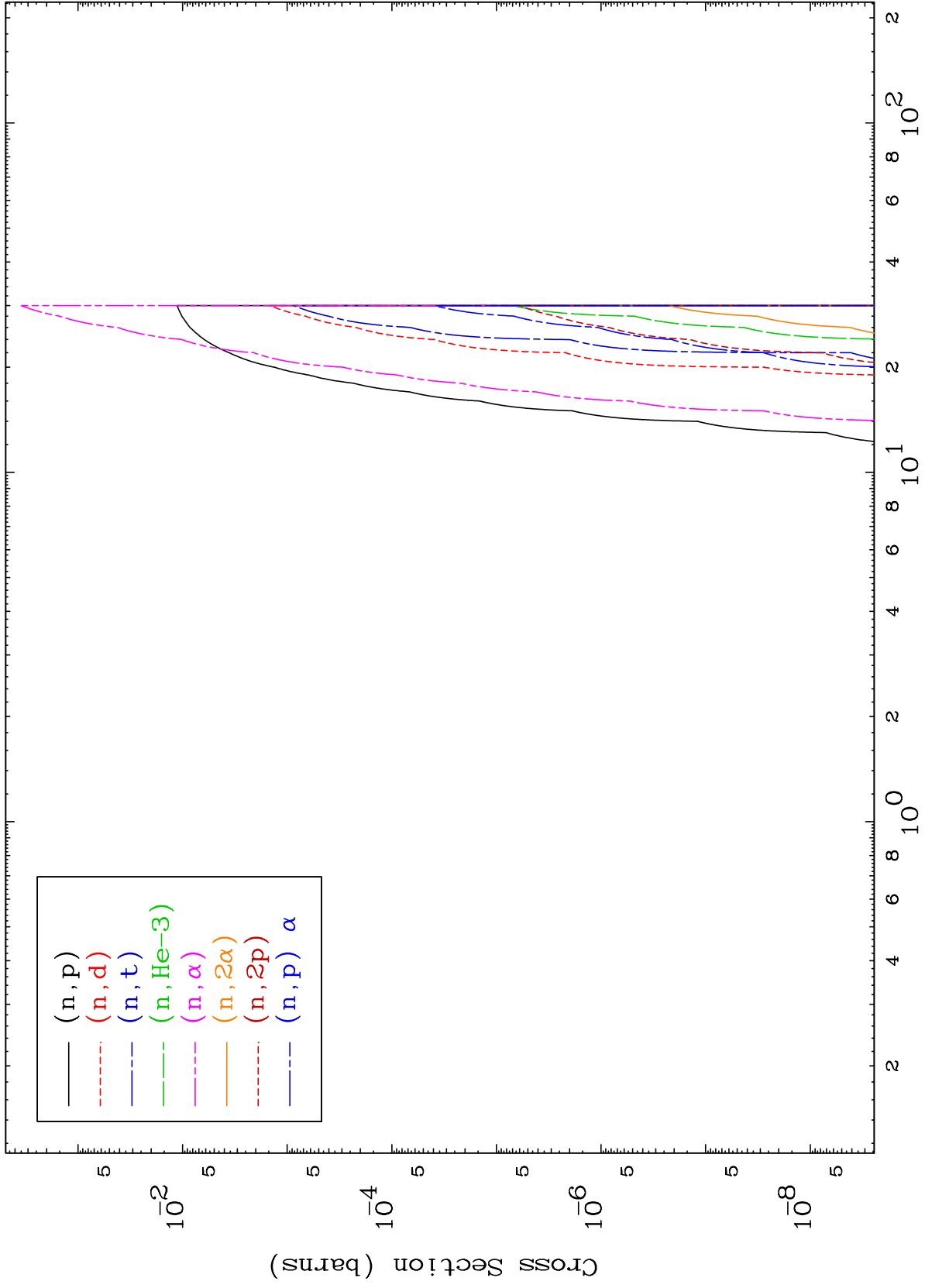
1

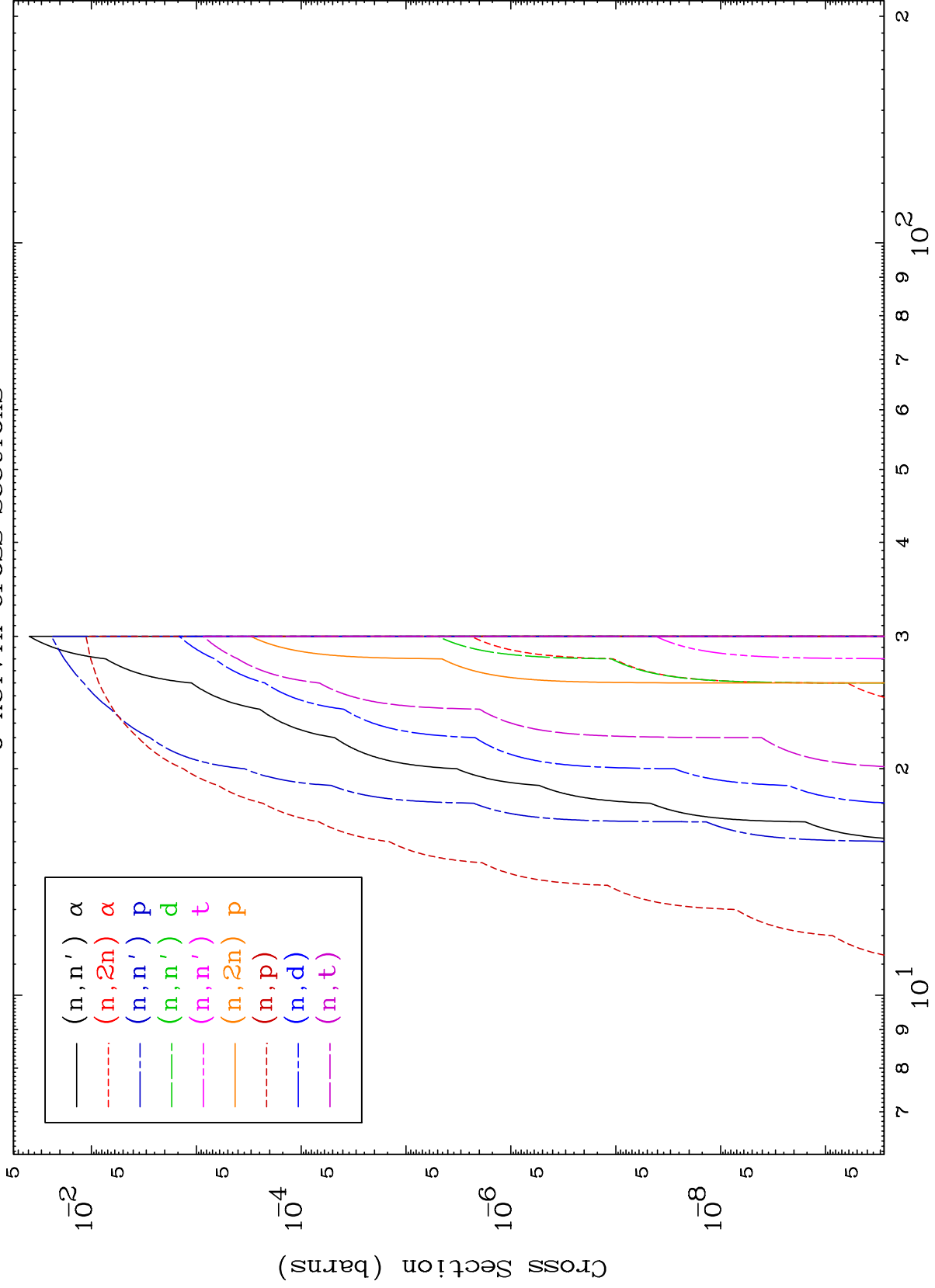


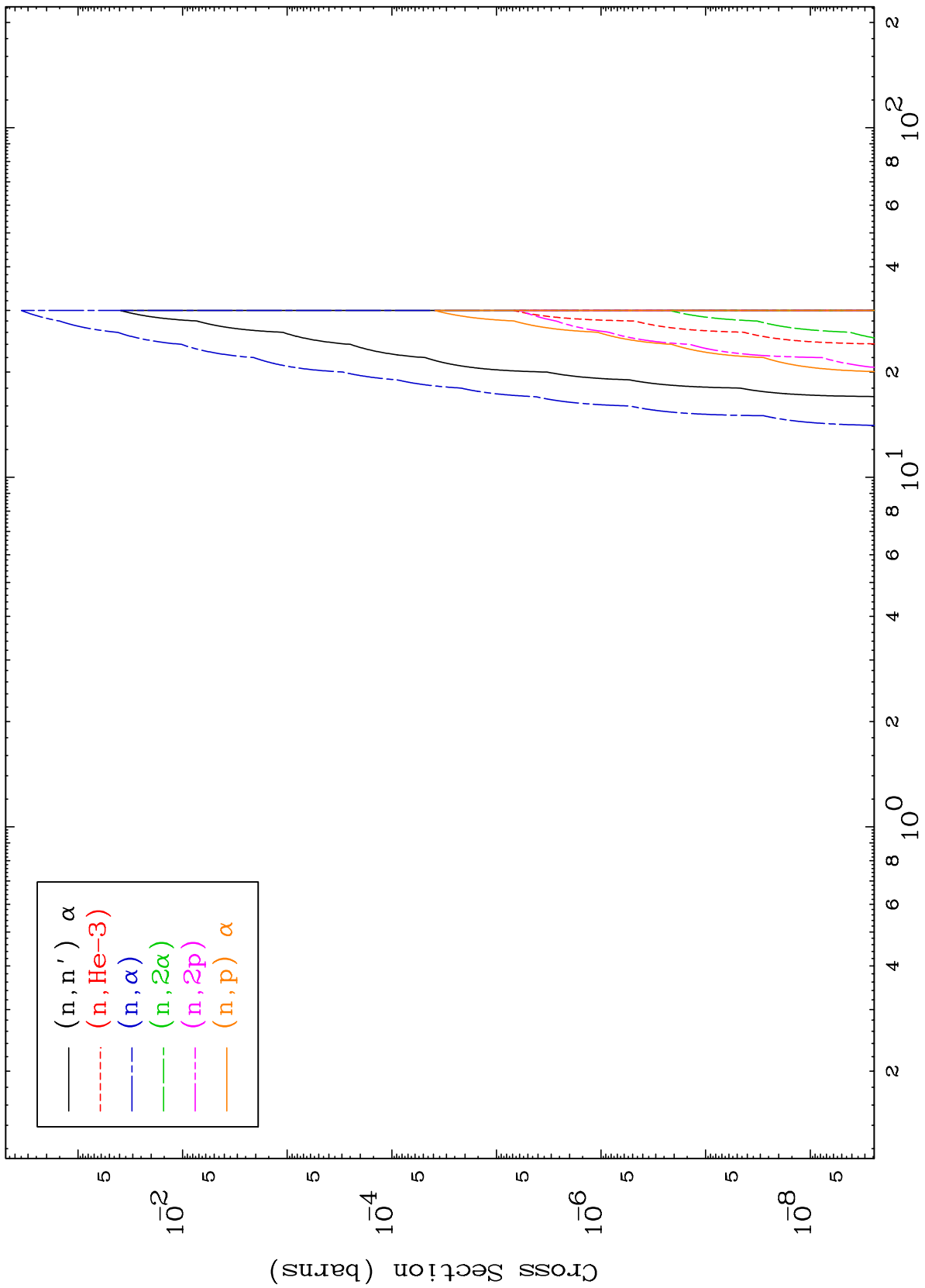
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$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

<sup>71</sup>Lu-168m





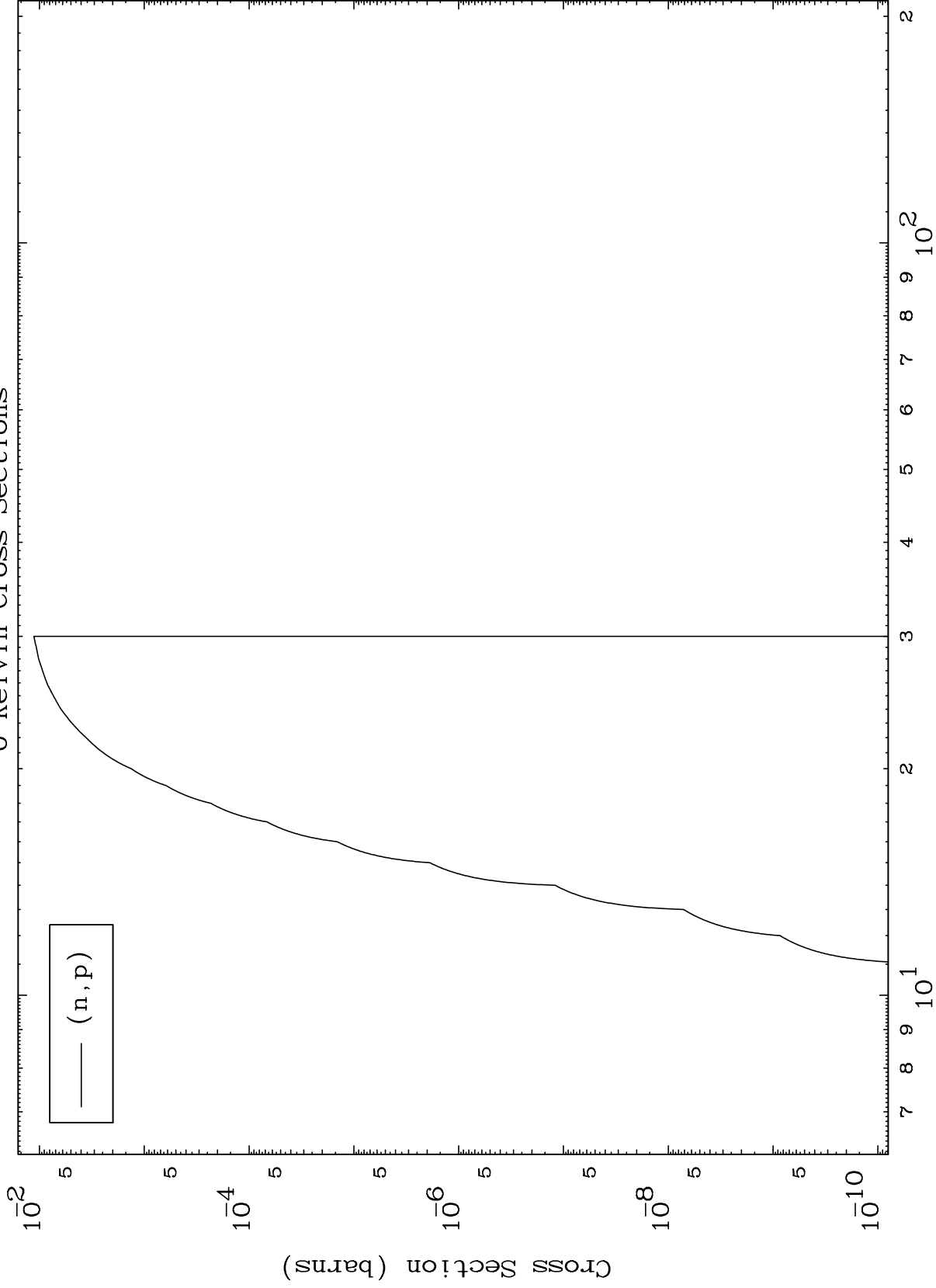


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( $\alpha, p$ ) Levels

$^{71}\text{Lu}-168\text{m}$

0 Kelvin Cross Sections



6

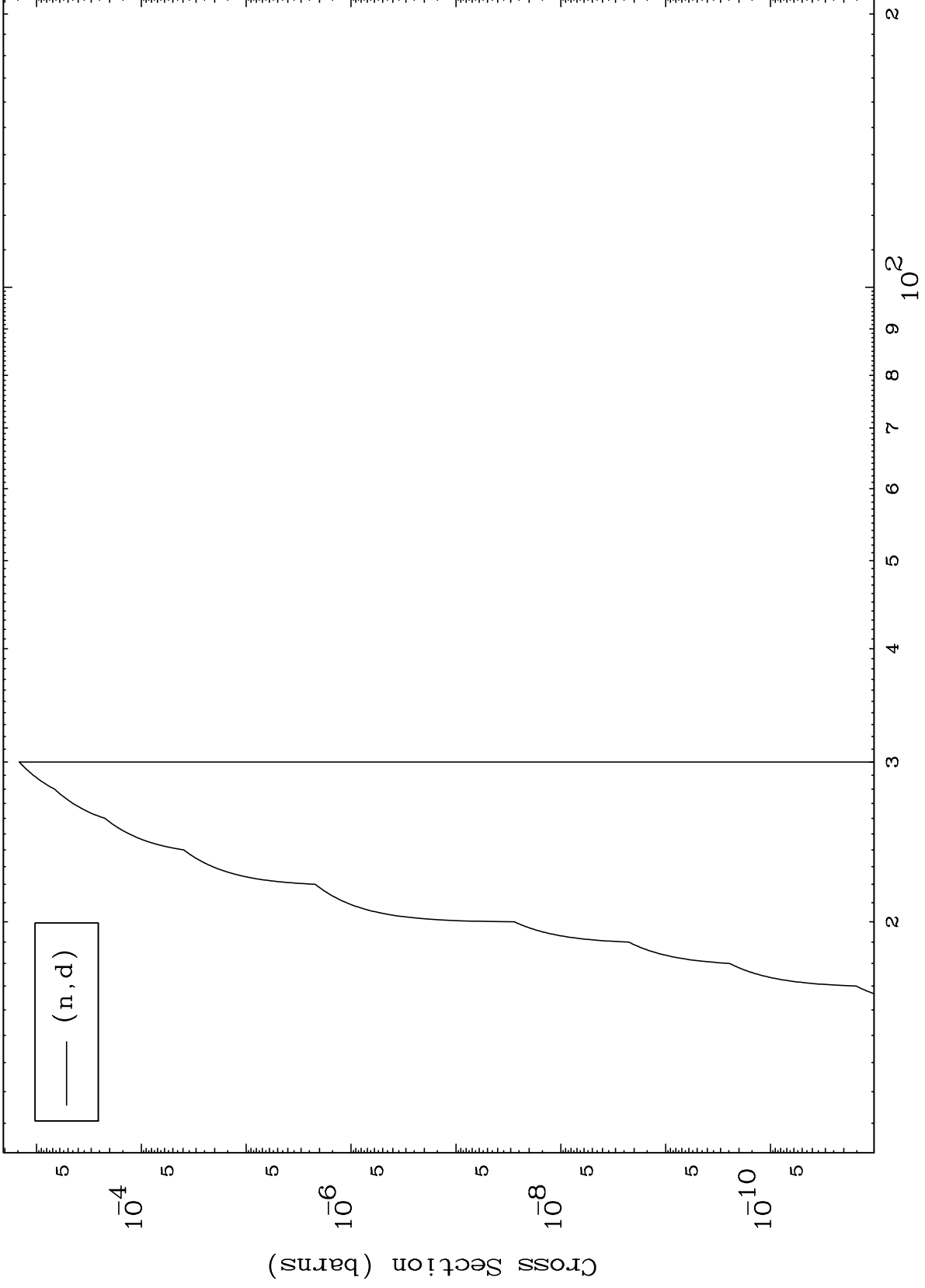
Incident Energy (MeV)

$^{71}\text{Lu}-168\text{m}$

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( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

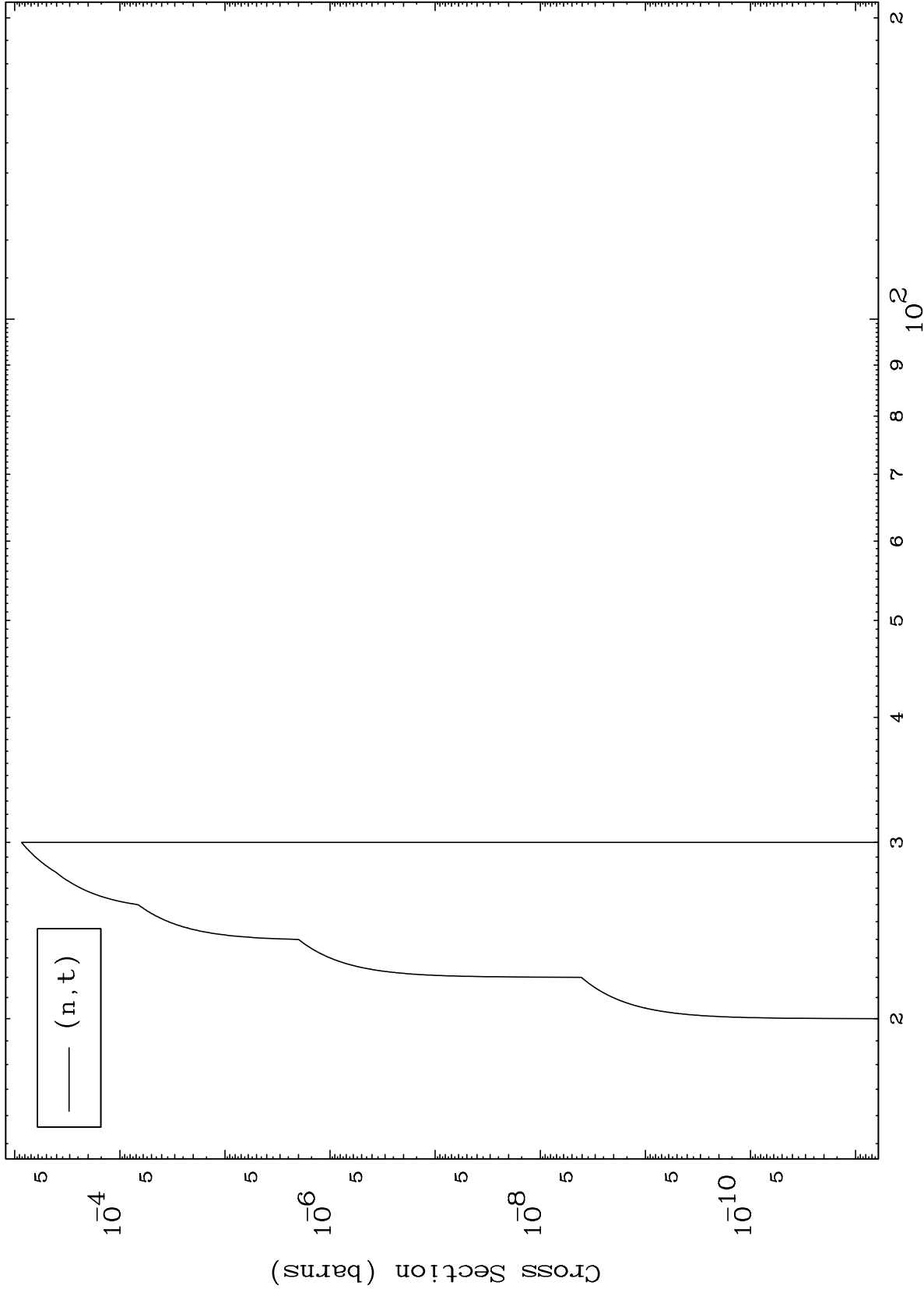
<sup>71</sup>Lu-168m

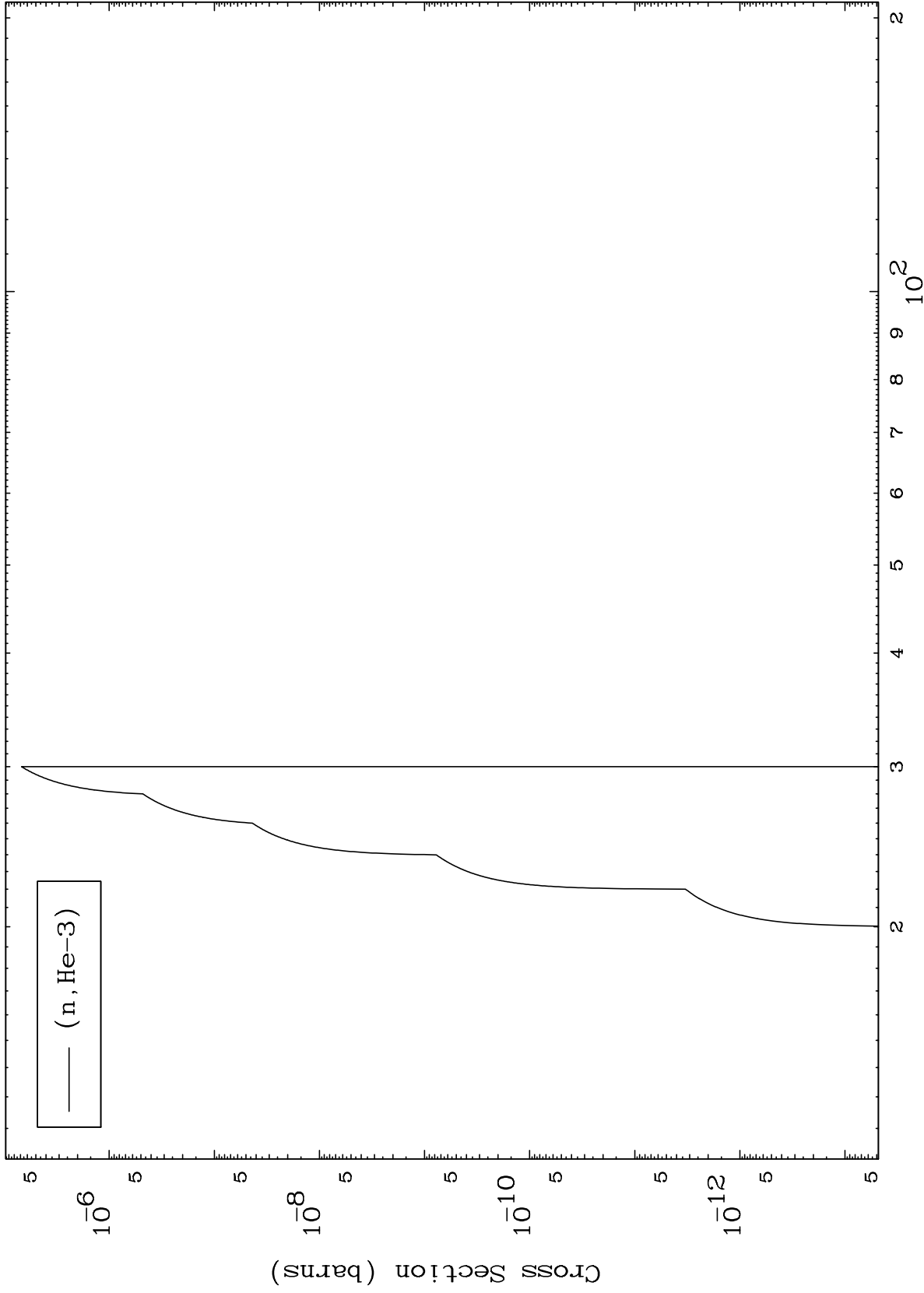


7

Incident Energy (MeV)

<sup>71</sup>Lu-168m



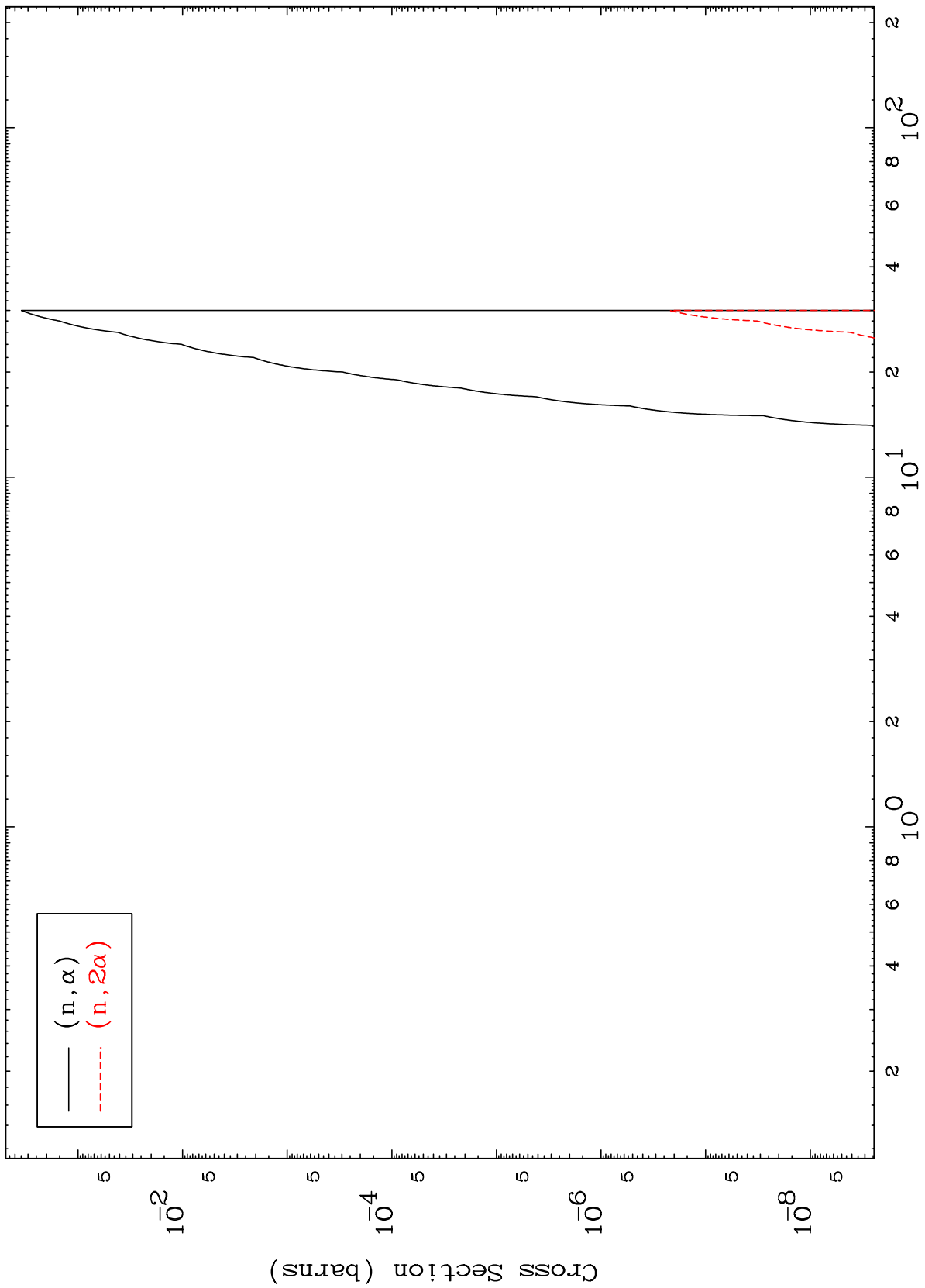


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( $\alpha, \alpha$ ) Levels

71-Lu-168m

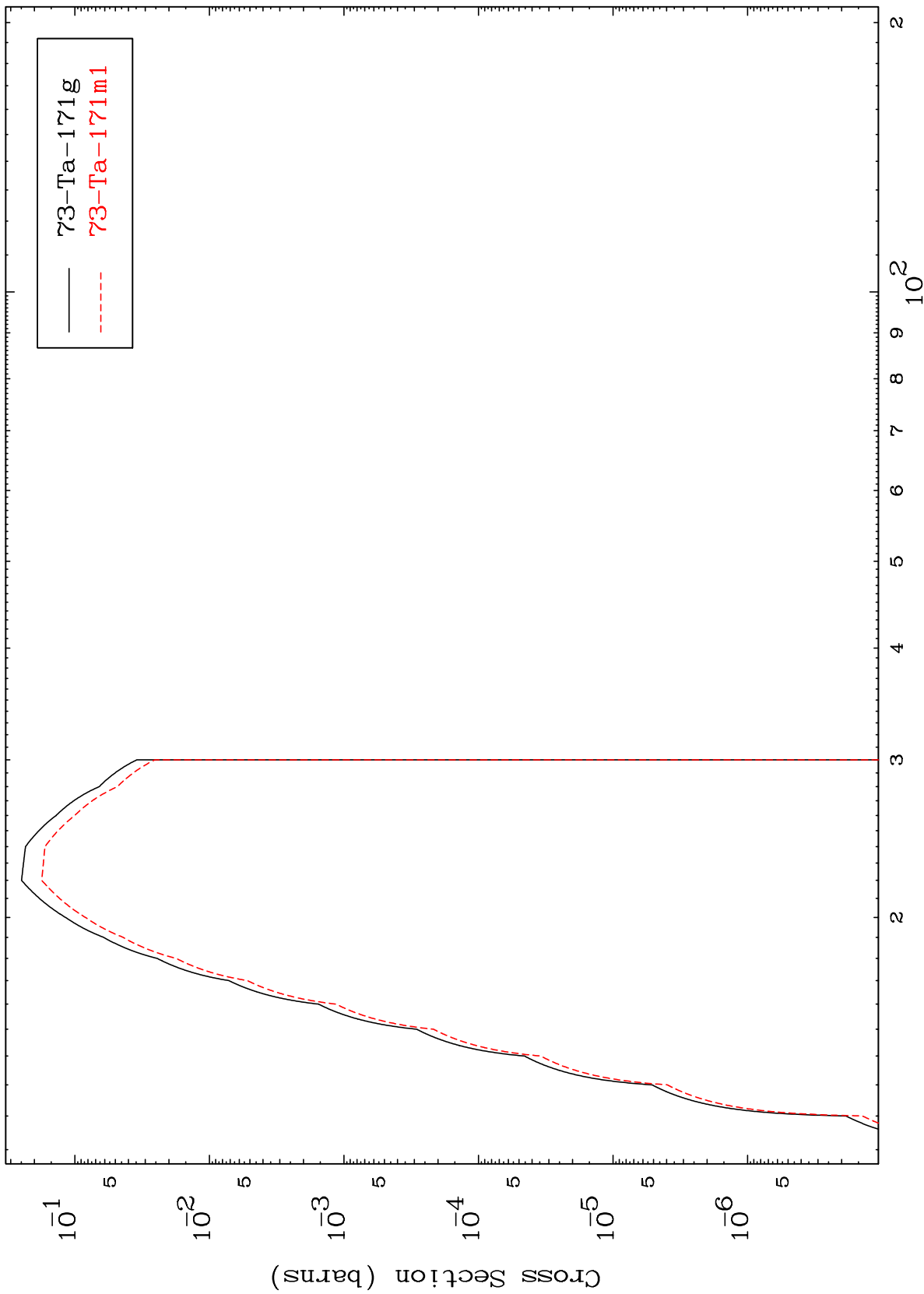
0 Kelvin Cross Sections



10

Incident Energy (MeV)

71-Lu-168m

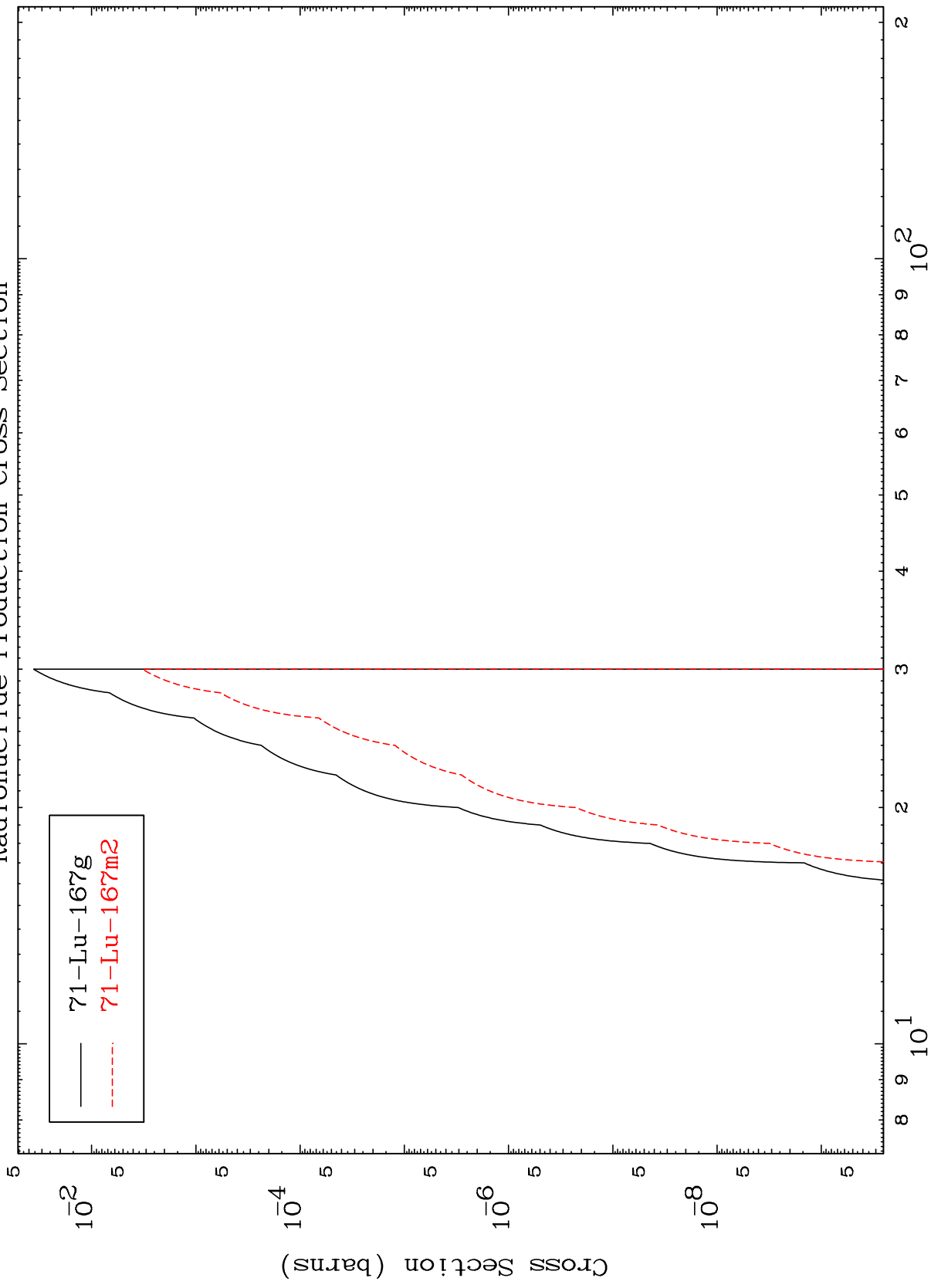


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$(n, n') \alpha$

$^{71}\text{Lu-168m}$

Radionuclide Production Cross Section



12

Incident Energy (MeV)

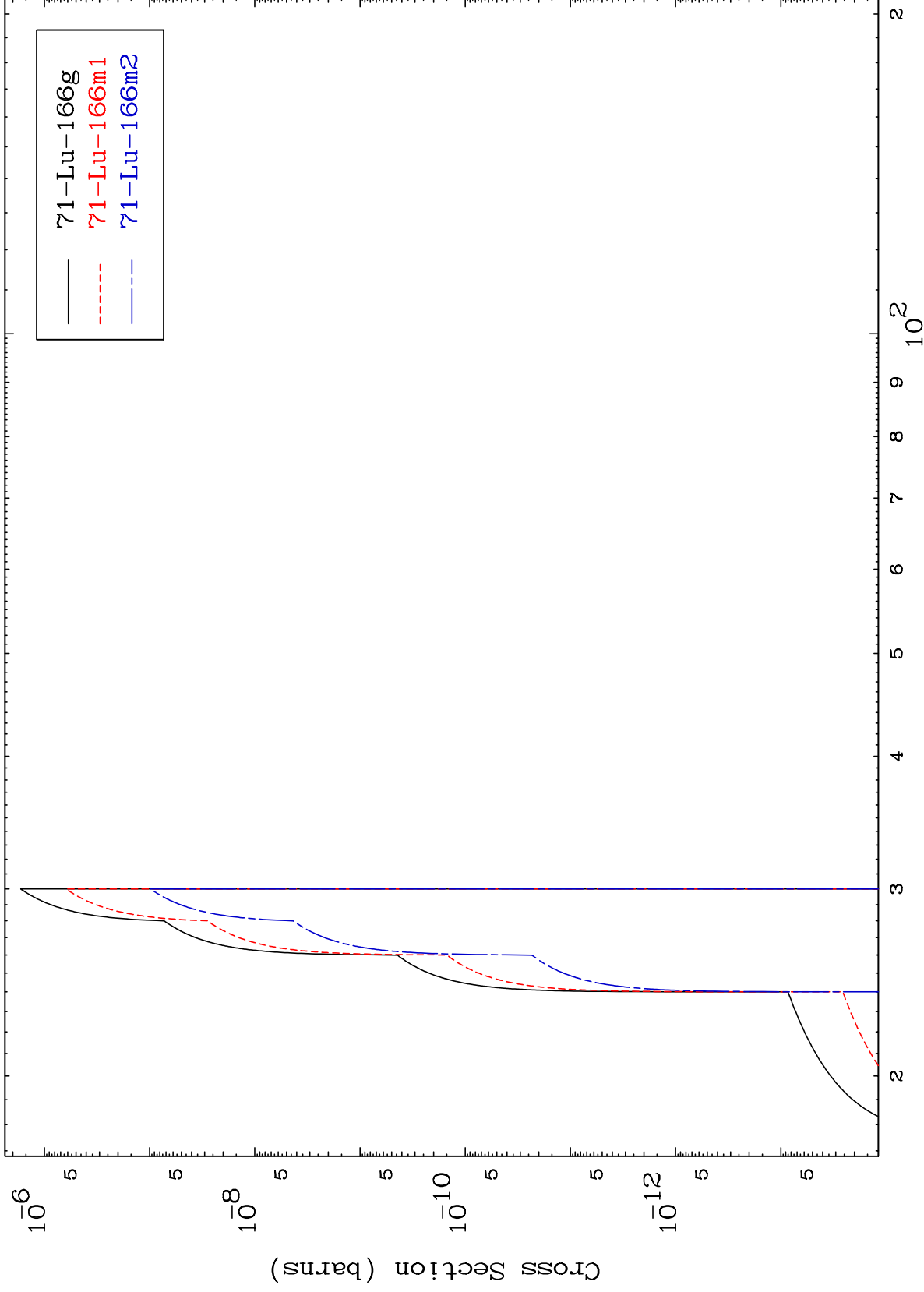
$^{71}\text{Lu-168m}$

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$(n,2n) \alpha$

$^{71}\text{Lu-168m}$

Radionuclide Production Cross Section



13

Incident Energy (MeV)

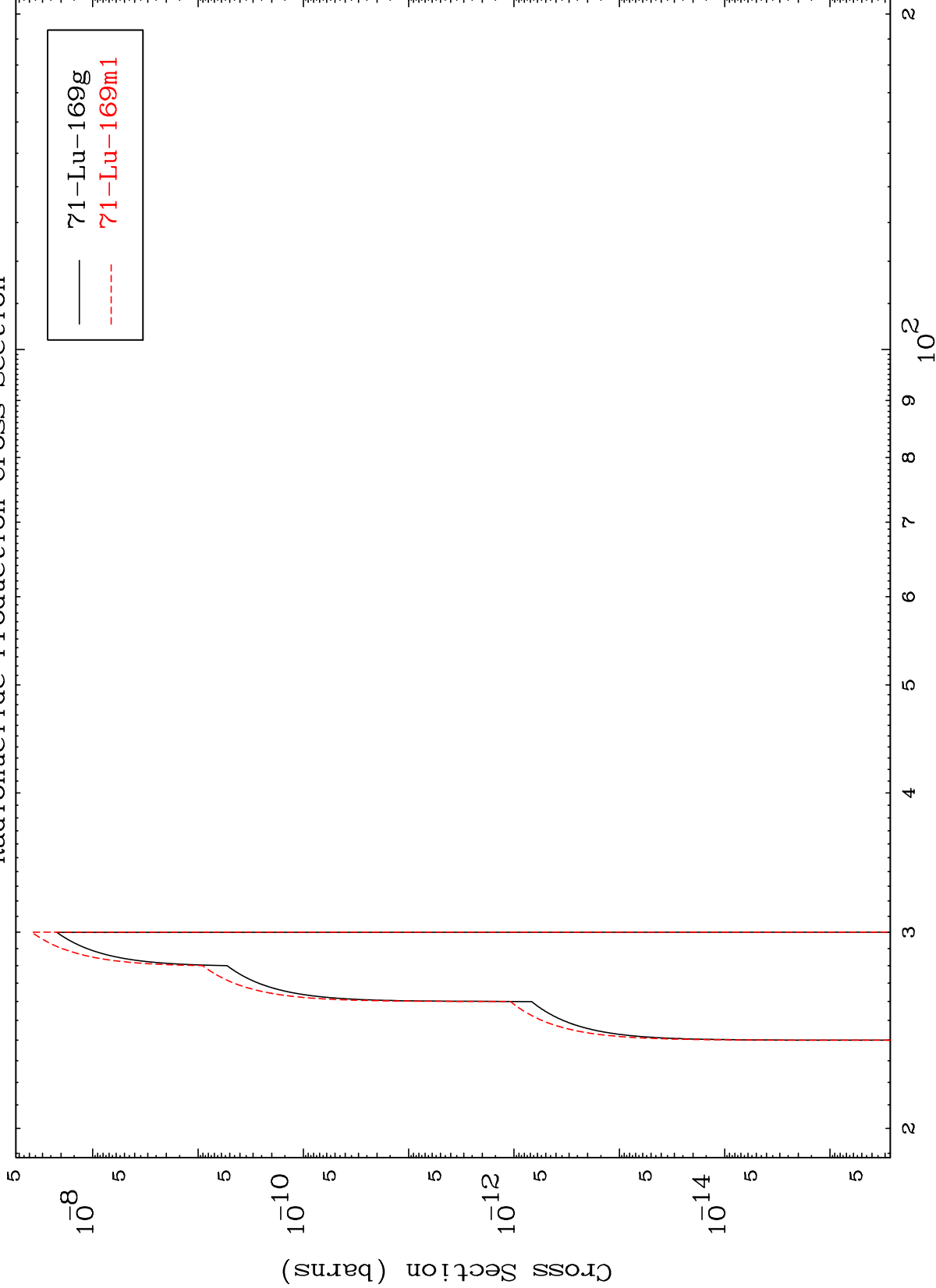
$^{71}\text{Lu-168m}$

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<sup>71</sup>Lu-168m

(n,2n) p

Radionuclide Production Cross Section



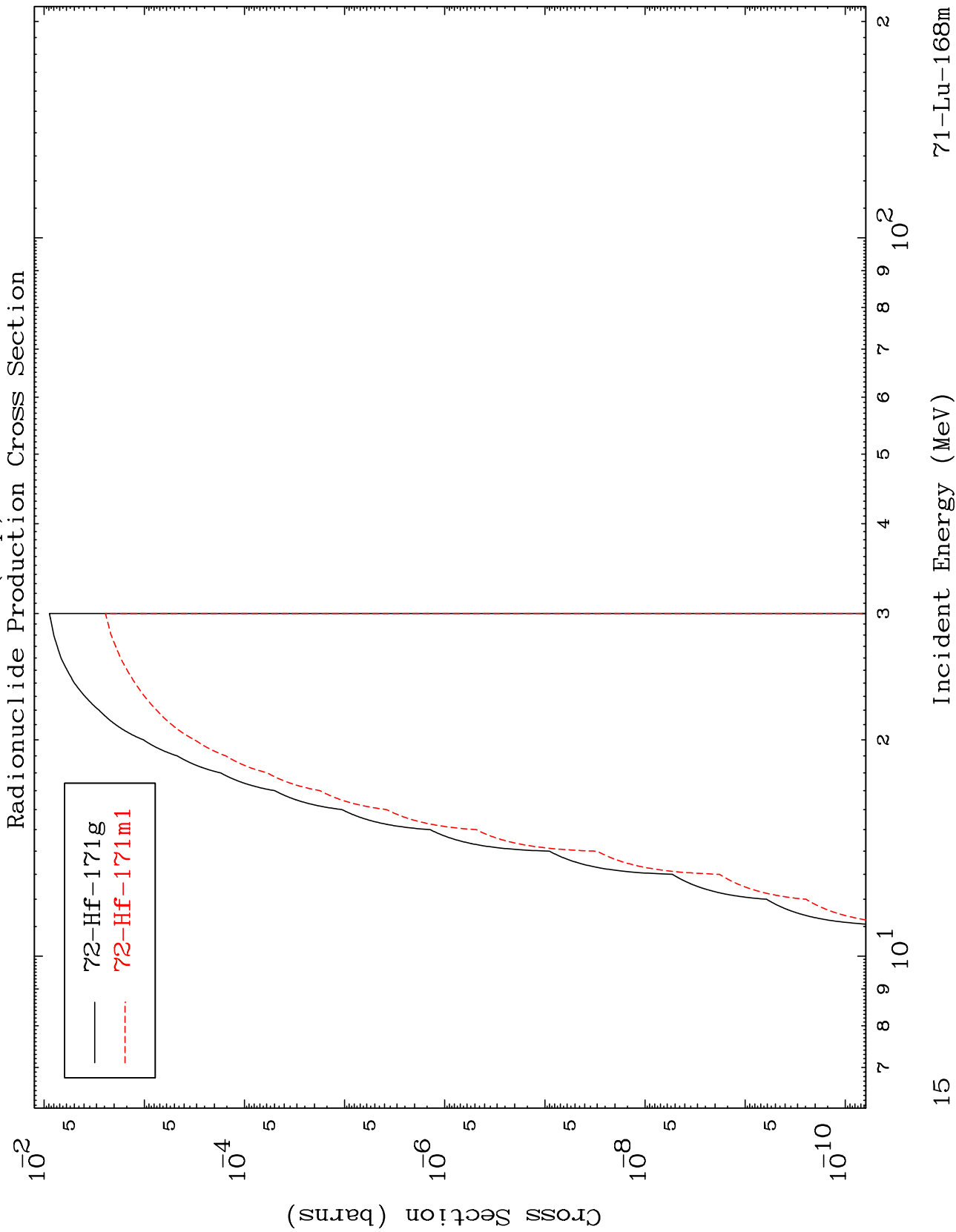
14

Incident Energy (MeV)

<sup>71</sup>Lu-168m

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71-Lu-168m

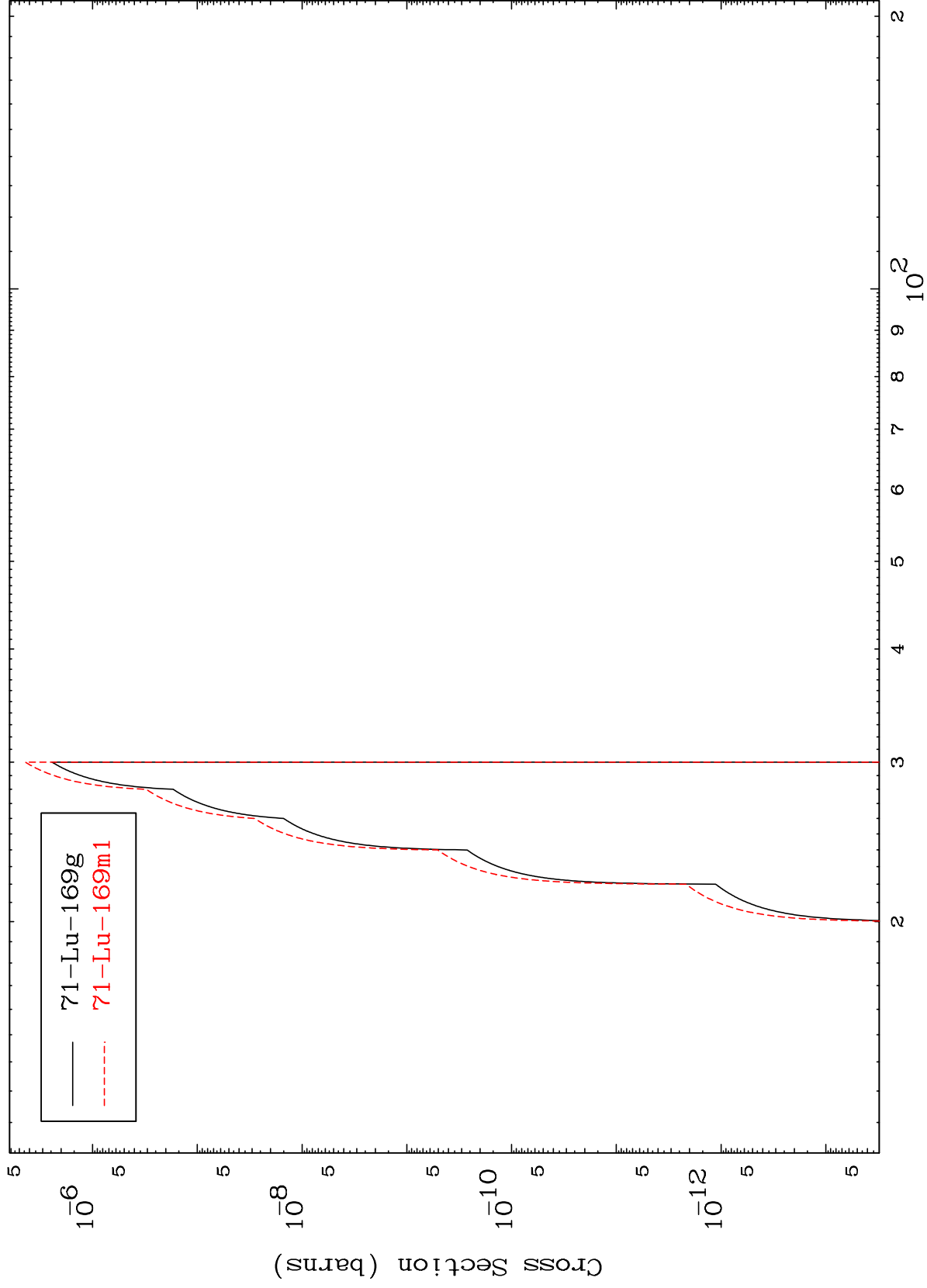


15

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<sup>71</sup>Lu-168m

Radionuclide Production Cross Section  
(n,He-3)



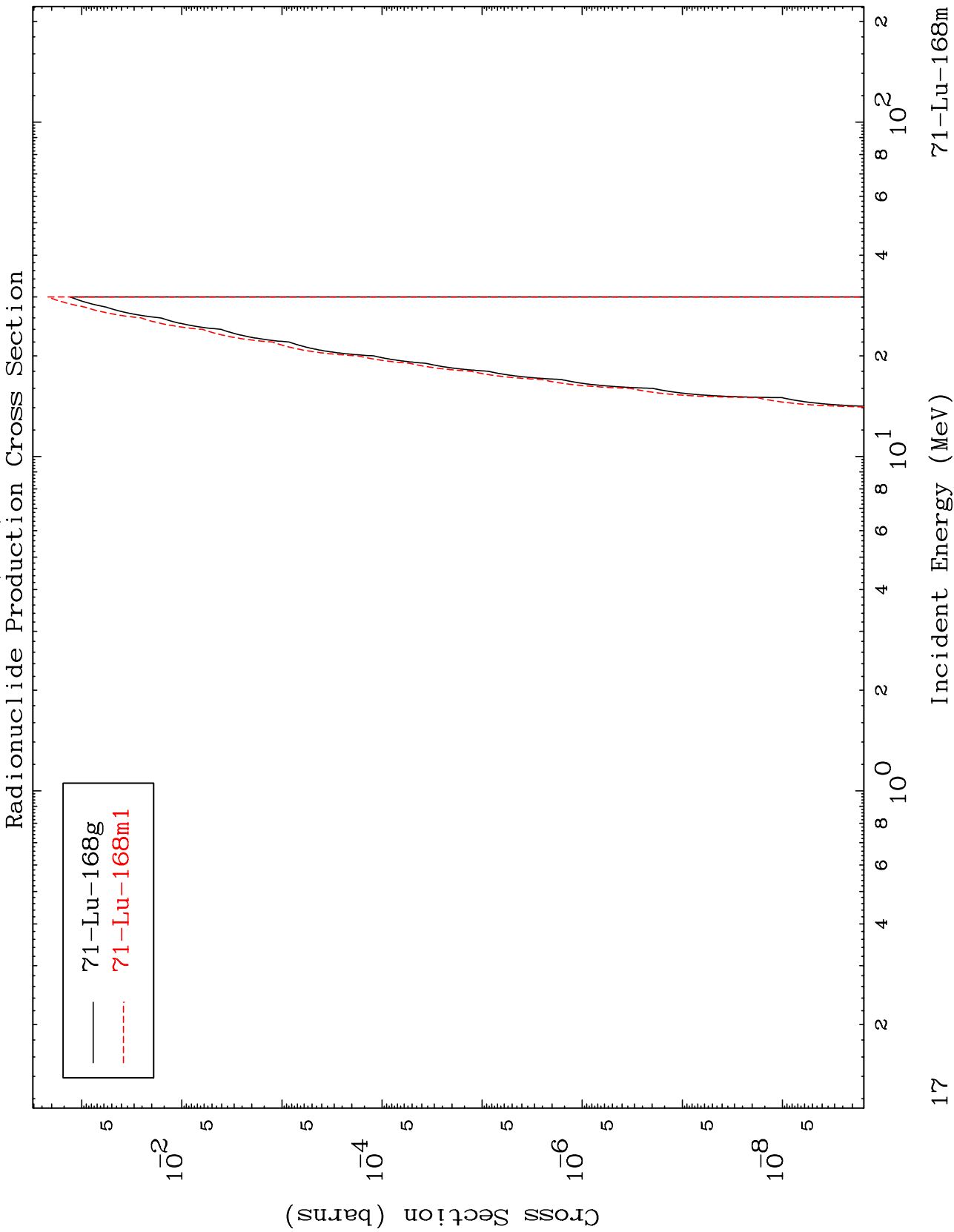
<sup>71</sup>Lu-168m

Incident Energy (MeV)

16

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<sup>71</sup>Lu-168m

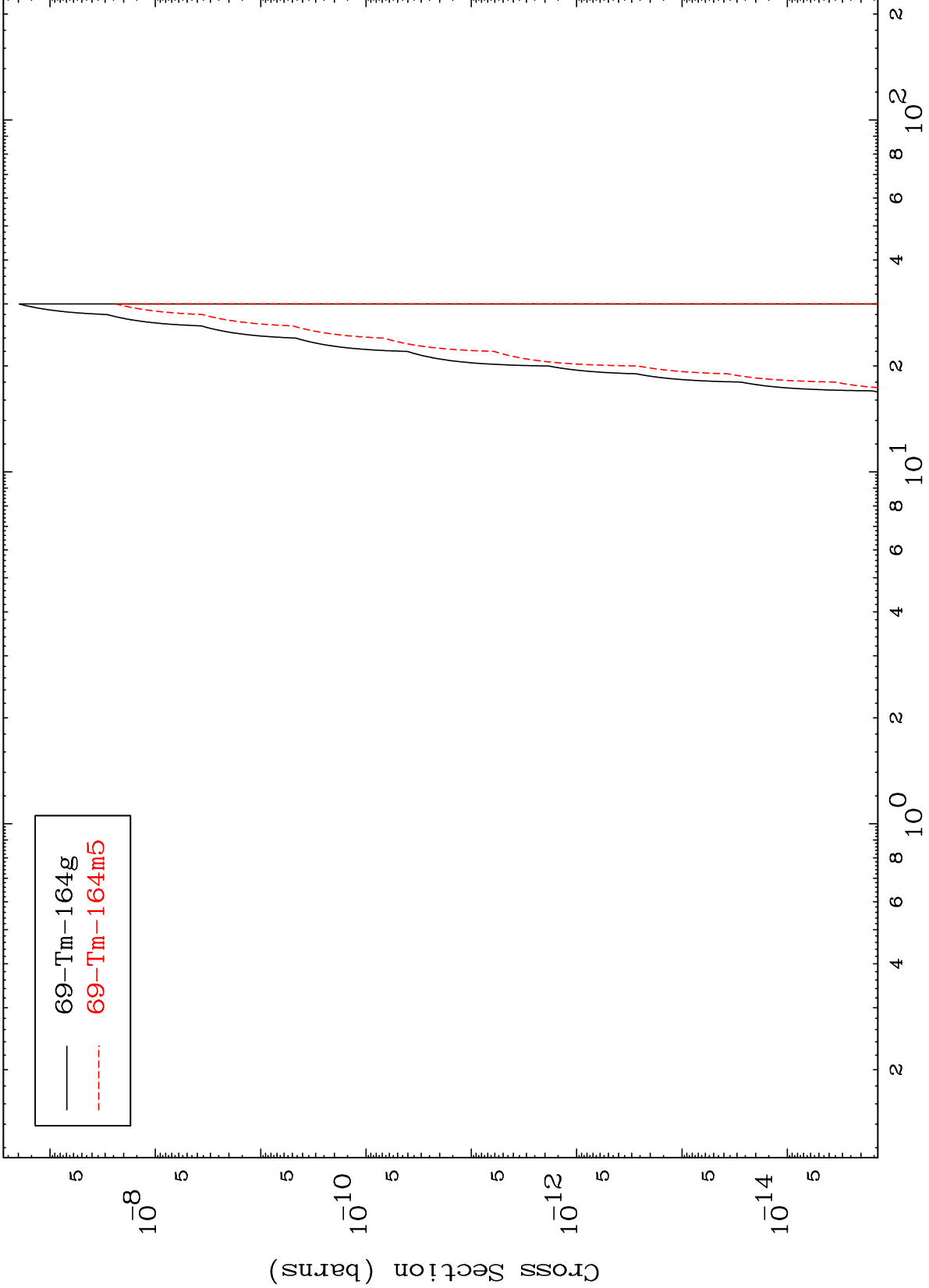


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(n,2α)

71-Lu-168m

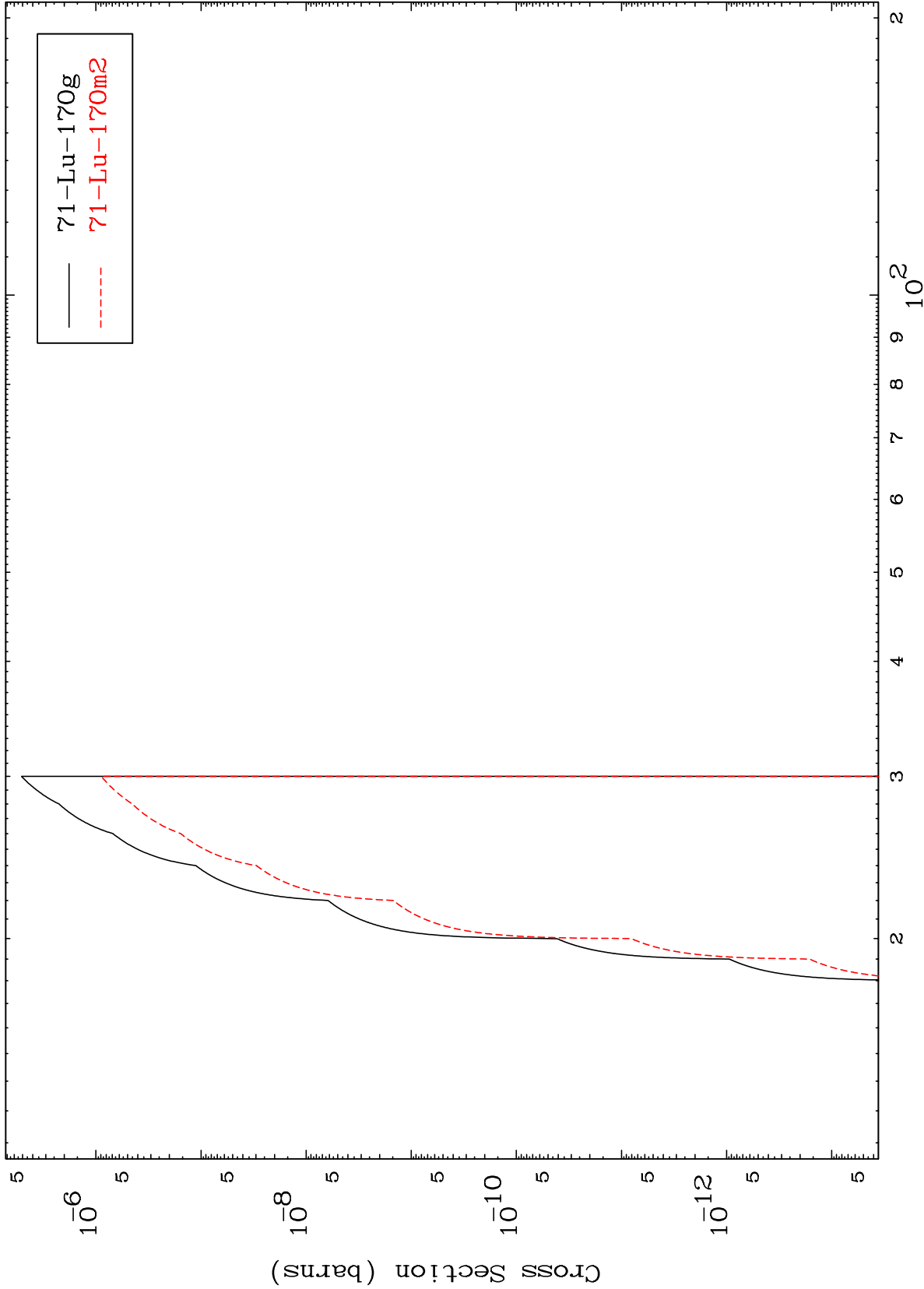
Radionuclide Production Cross Section



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<sup>71</sup>Lu-168m

(n,2p)  
Radionuclide Production Cross Section

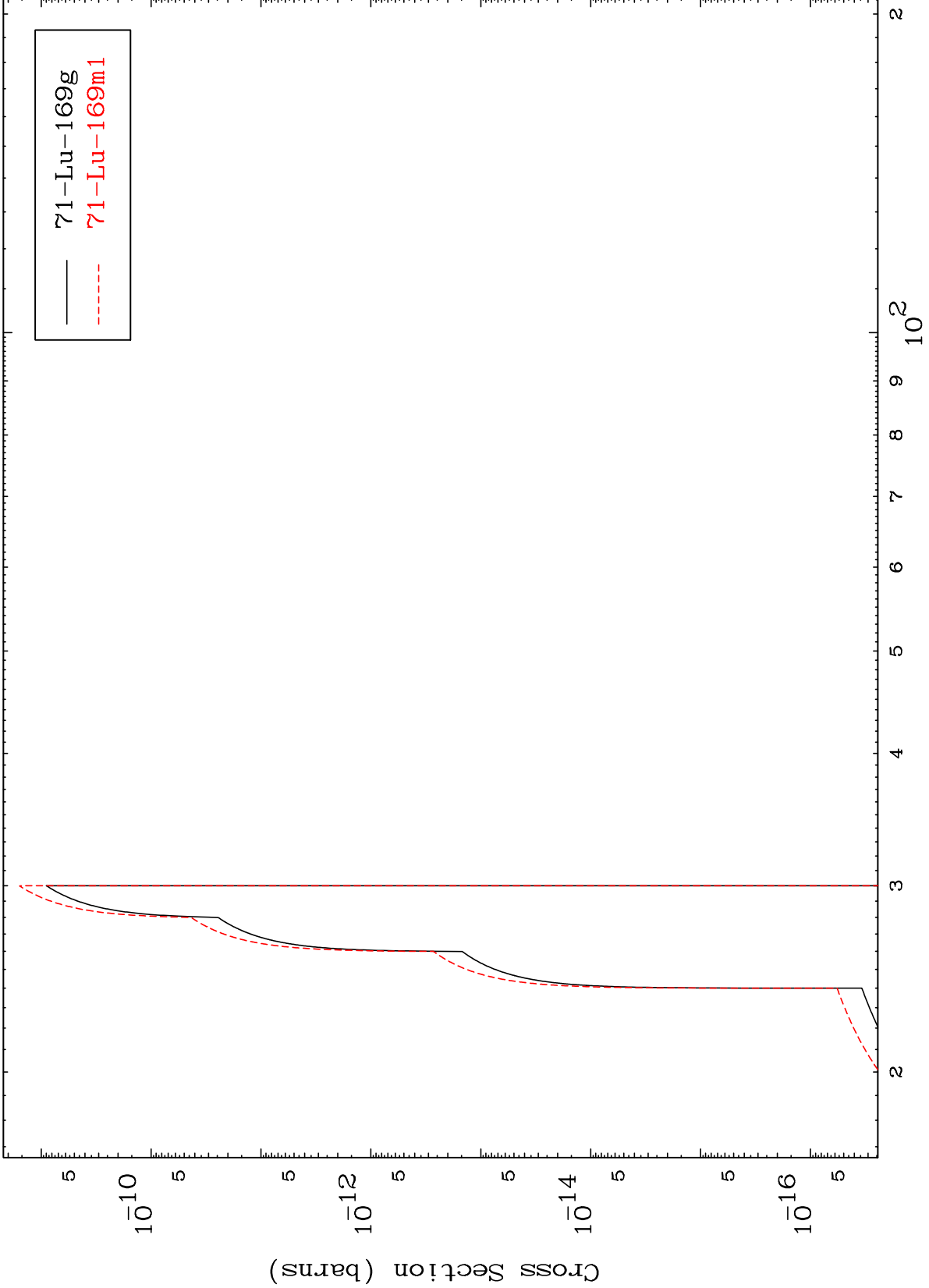


MAT 7105

(n,p) d

71-Lu-168m

Radionuclide Production Cross Section



71-Lu-169g  
71-Lu-169m1

20

Incident Energy (MeV)

71-Lu-168m