

Program EVALPLOT
(Version 2021-1)

by

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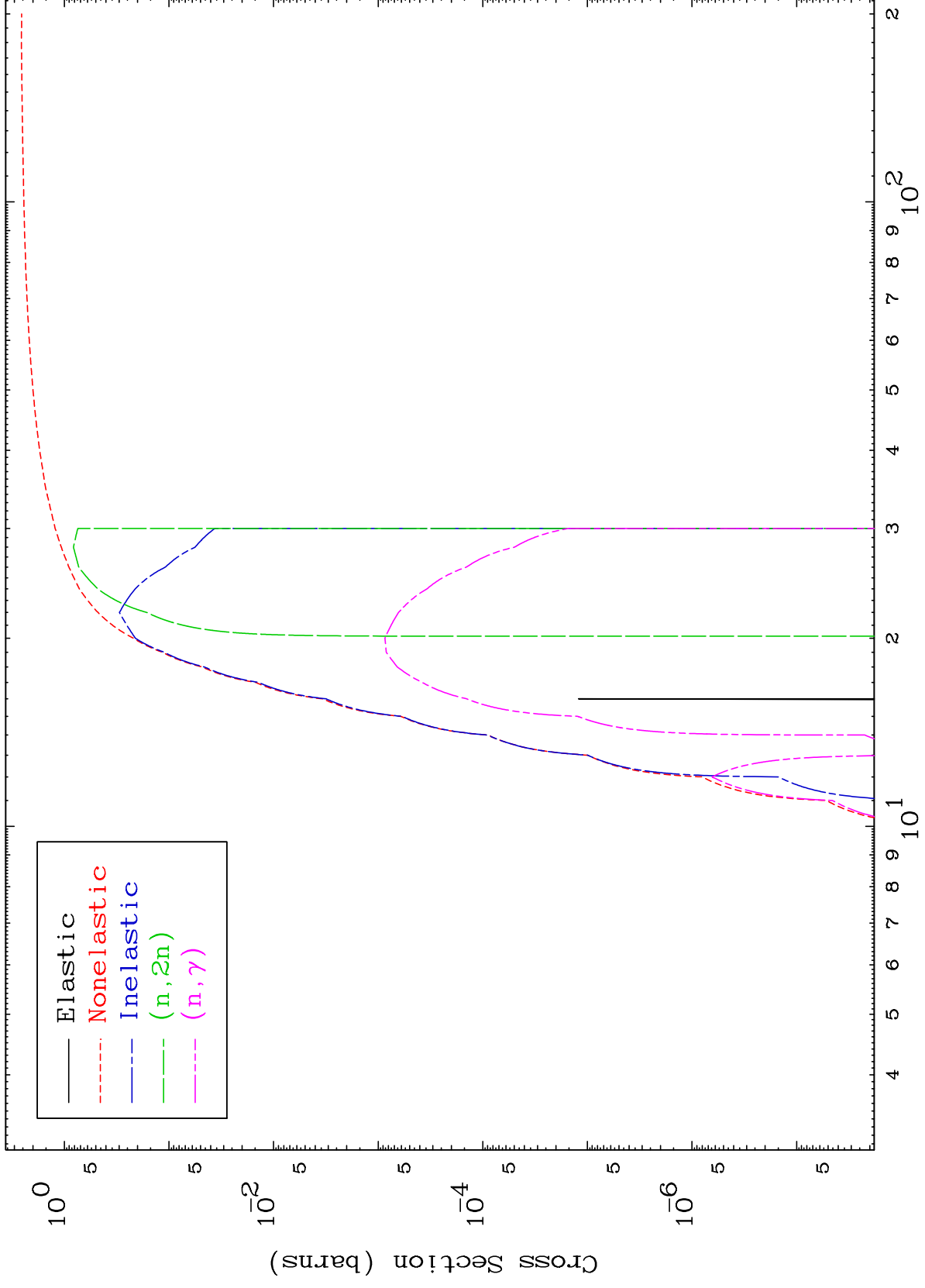
Press Mouse Button to Start

MAT 7110

0 Kelvin

α Major Cross Sections

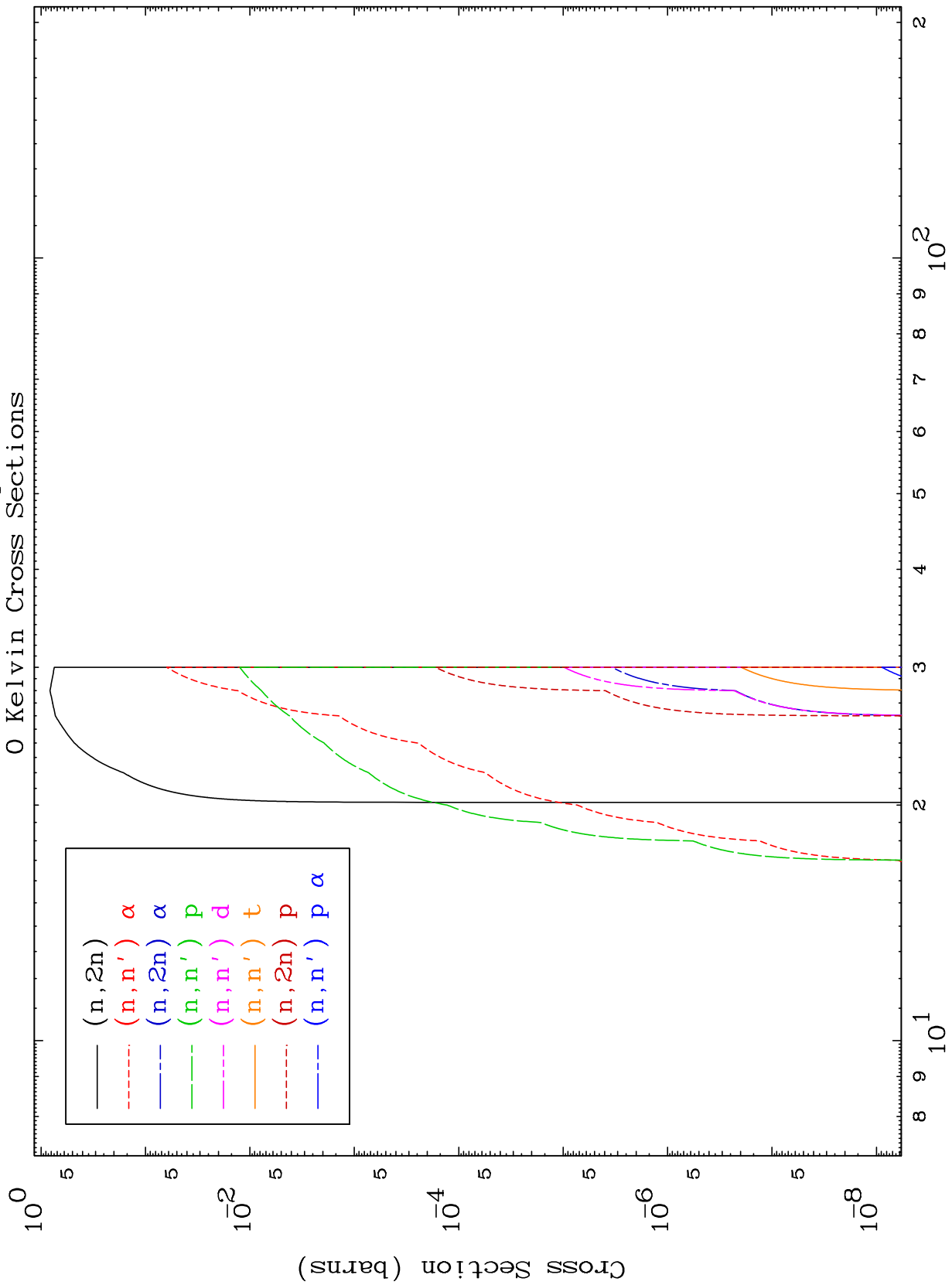
71-Lu-170



MAT 7110

α Neutron Absorption

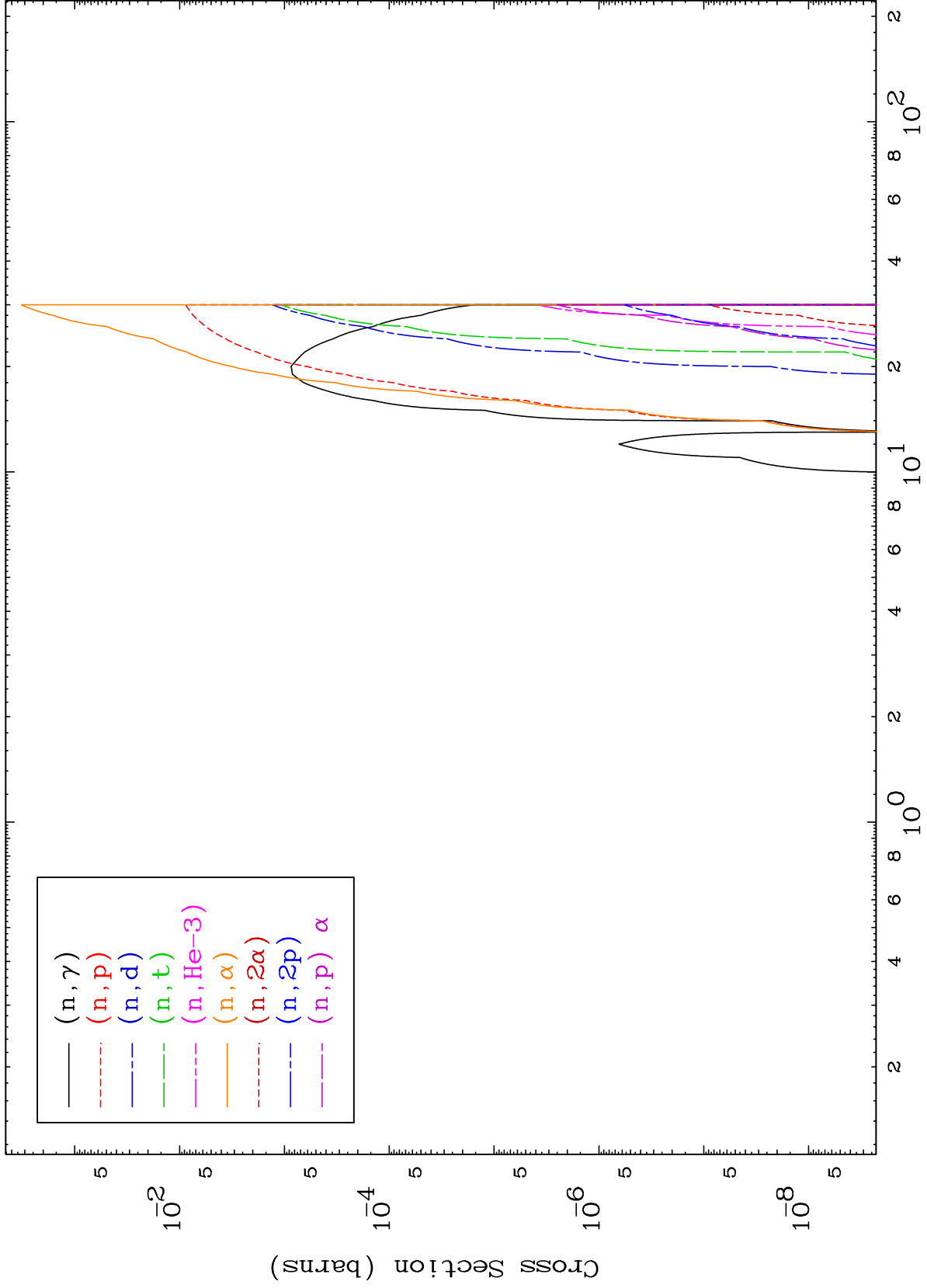
71-Lu-170

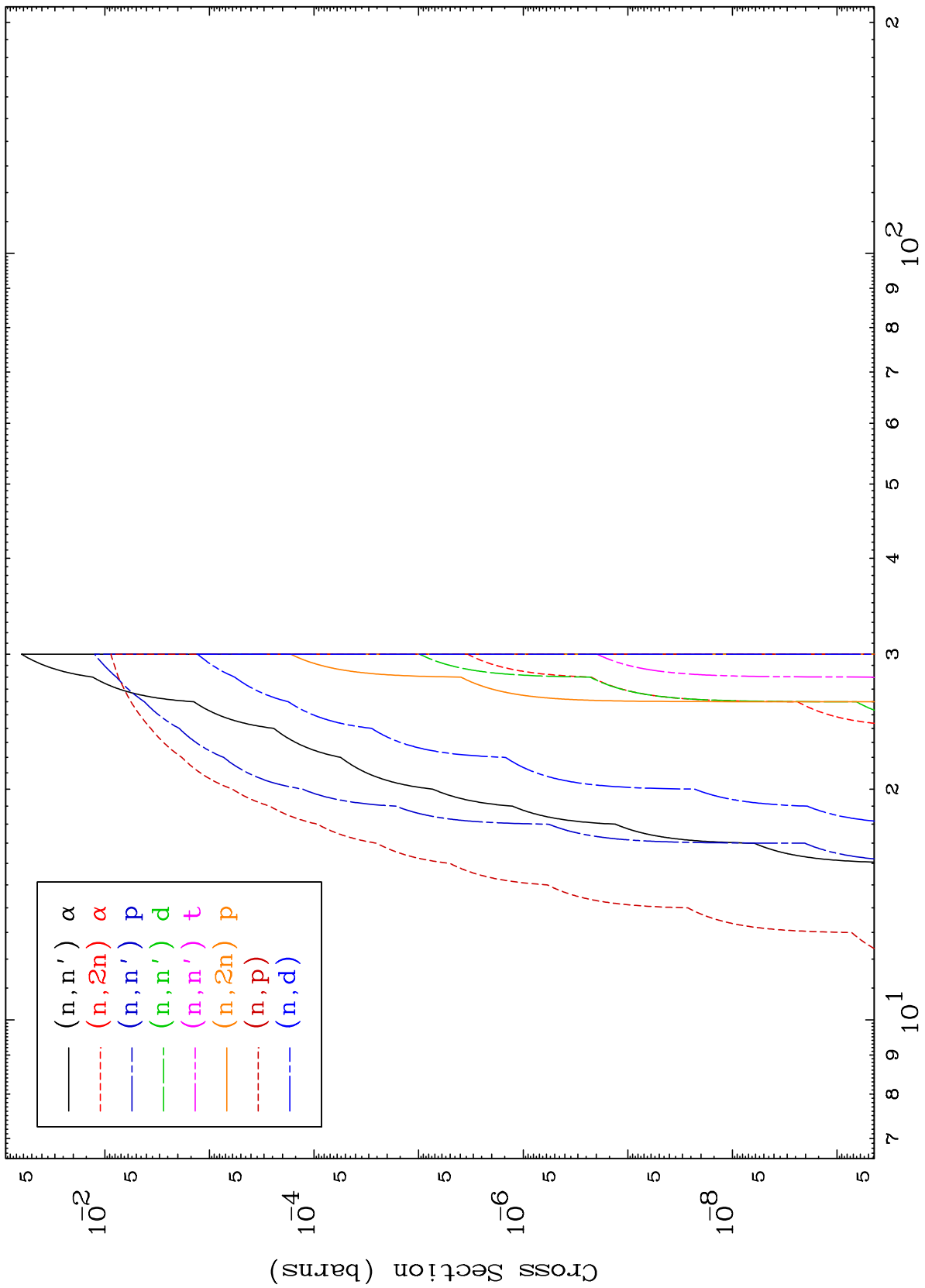


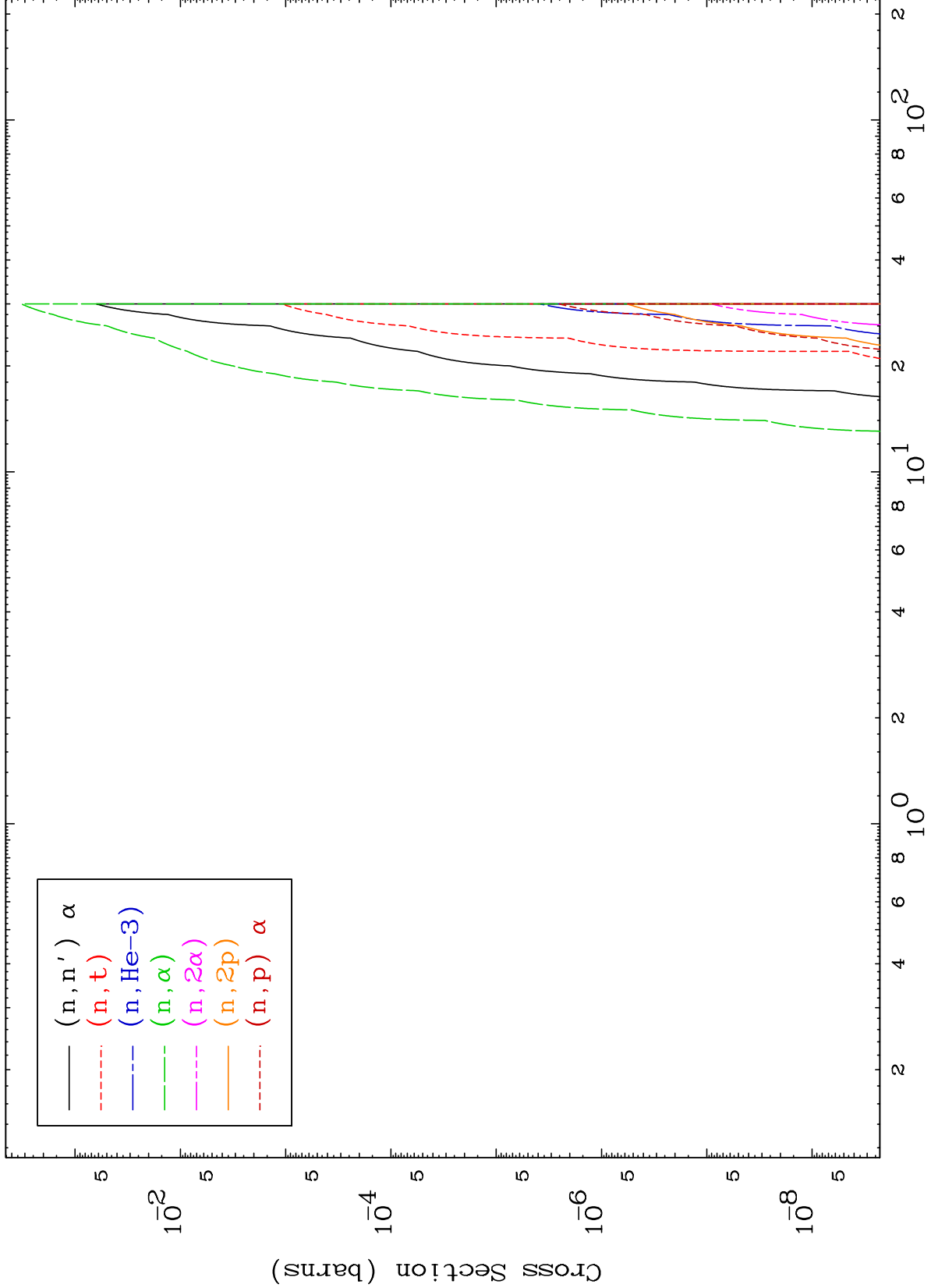
2

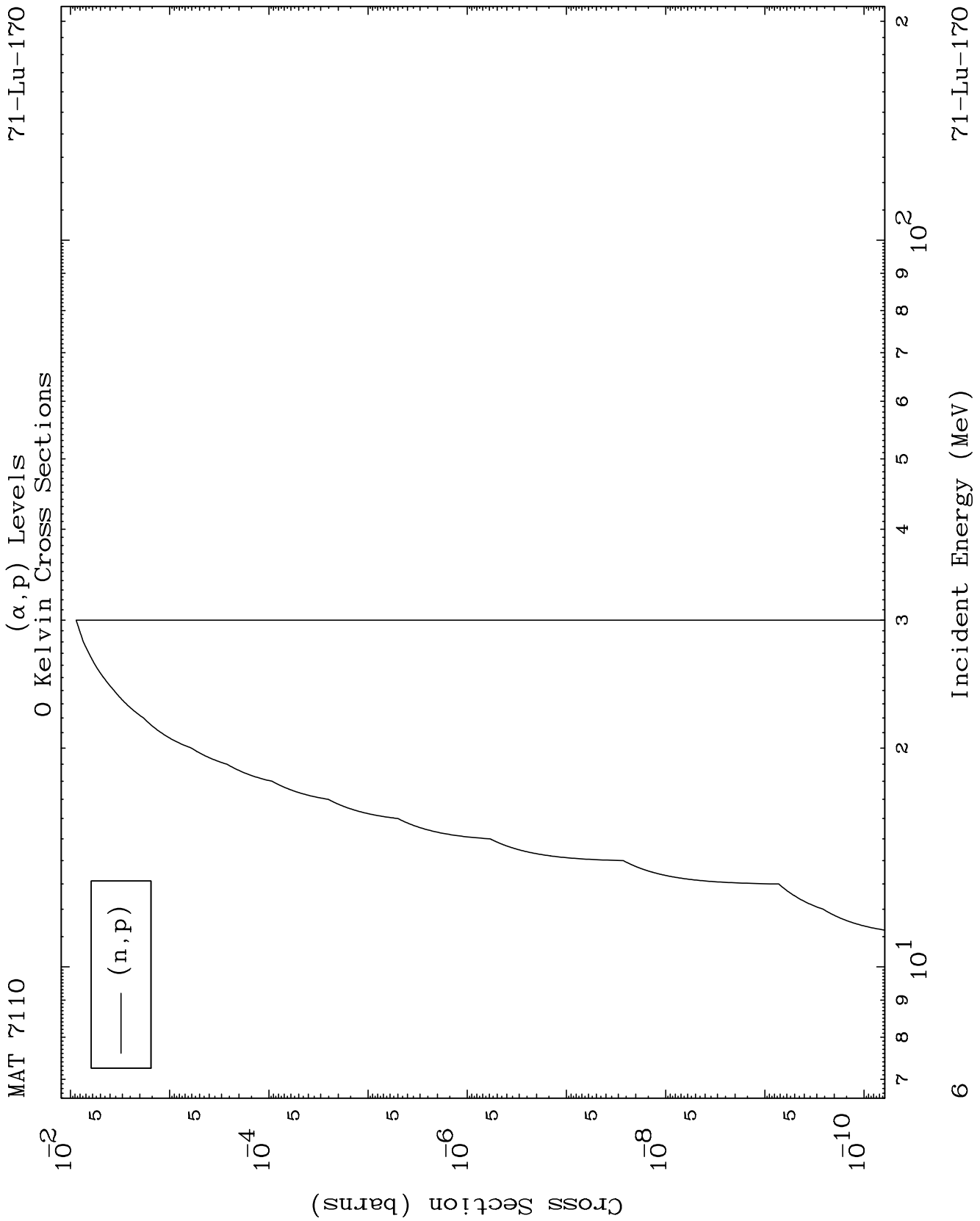
Incident Energy (MeV)

71-Lu-170





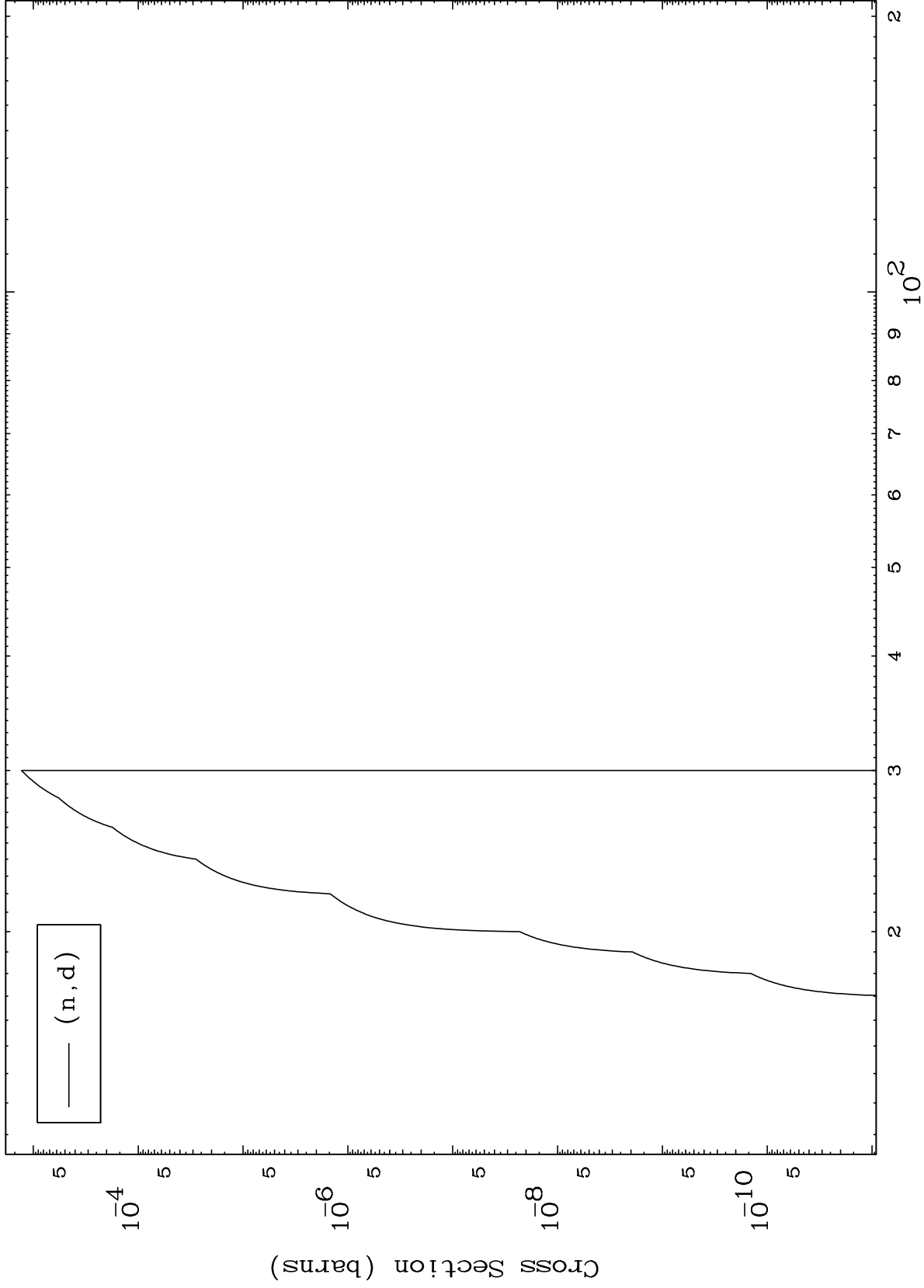


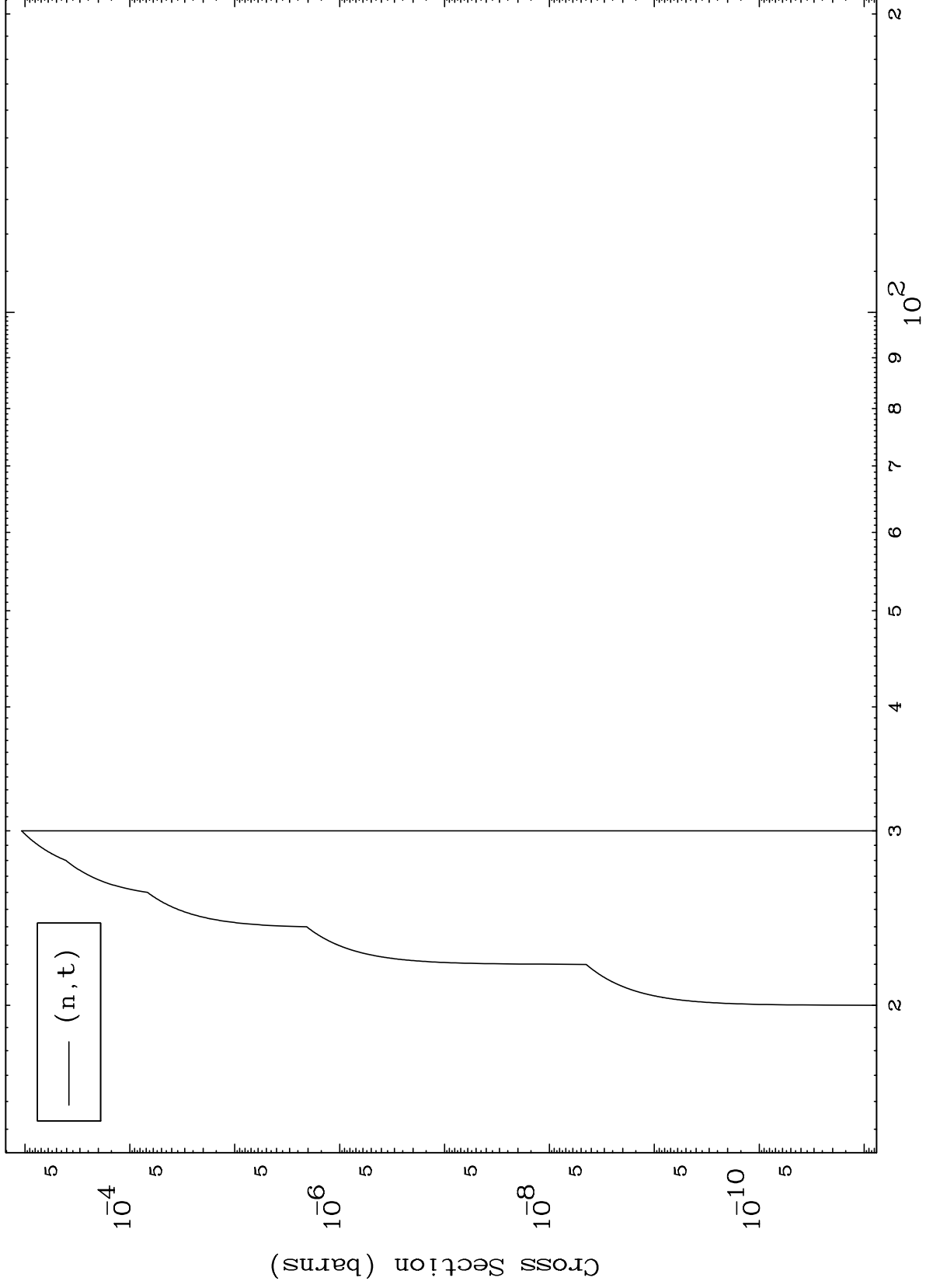


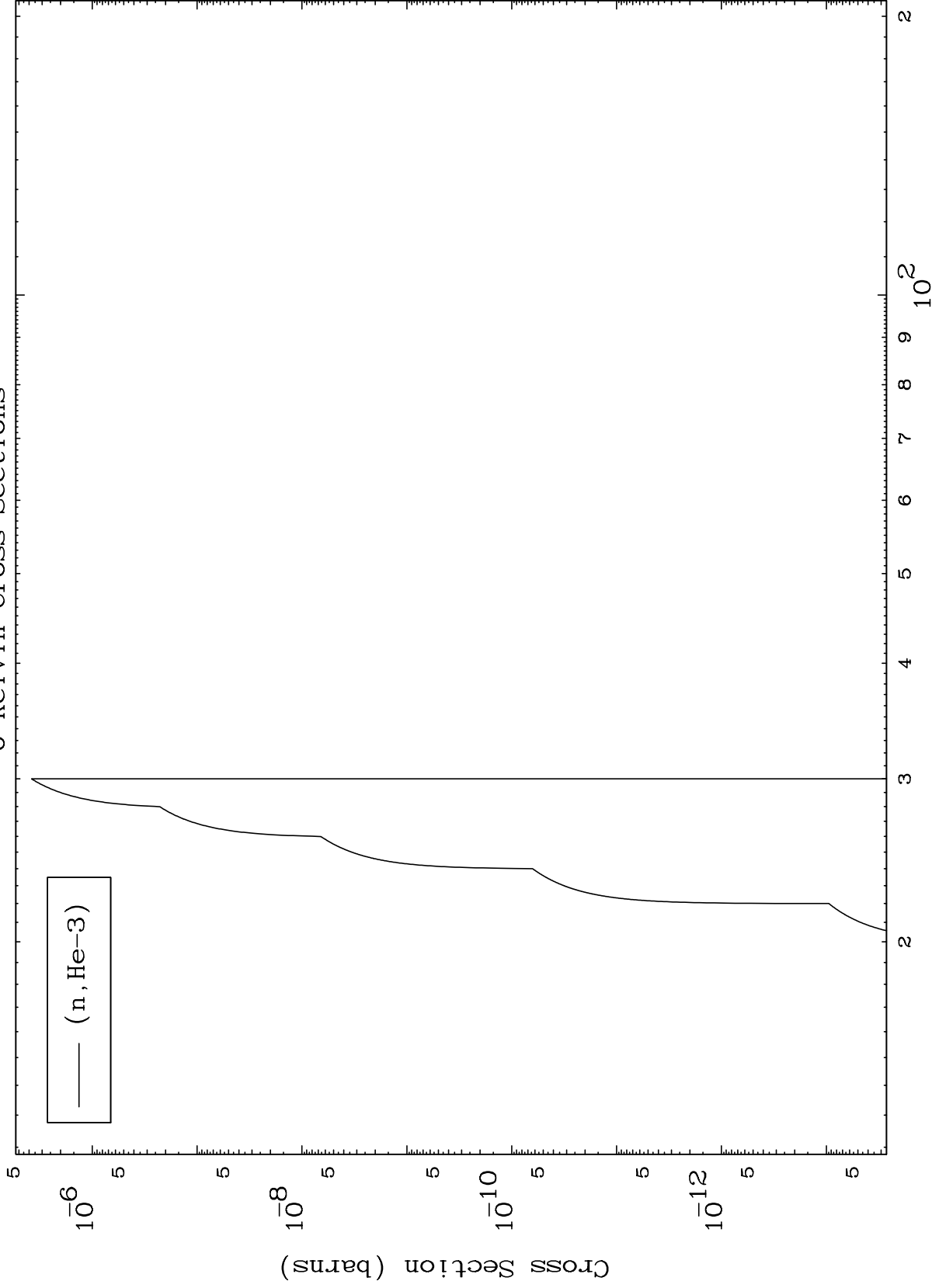
MAT 7110

71-Lu-170

(α, d) Levels
0 Kelvin Cross Sections





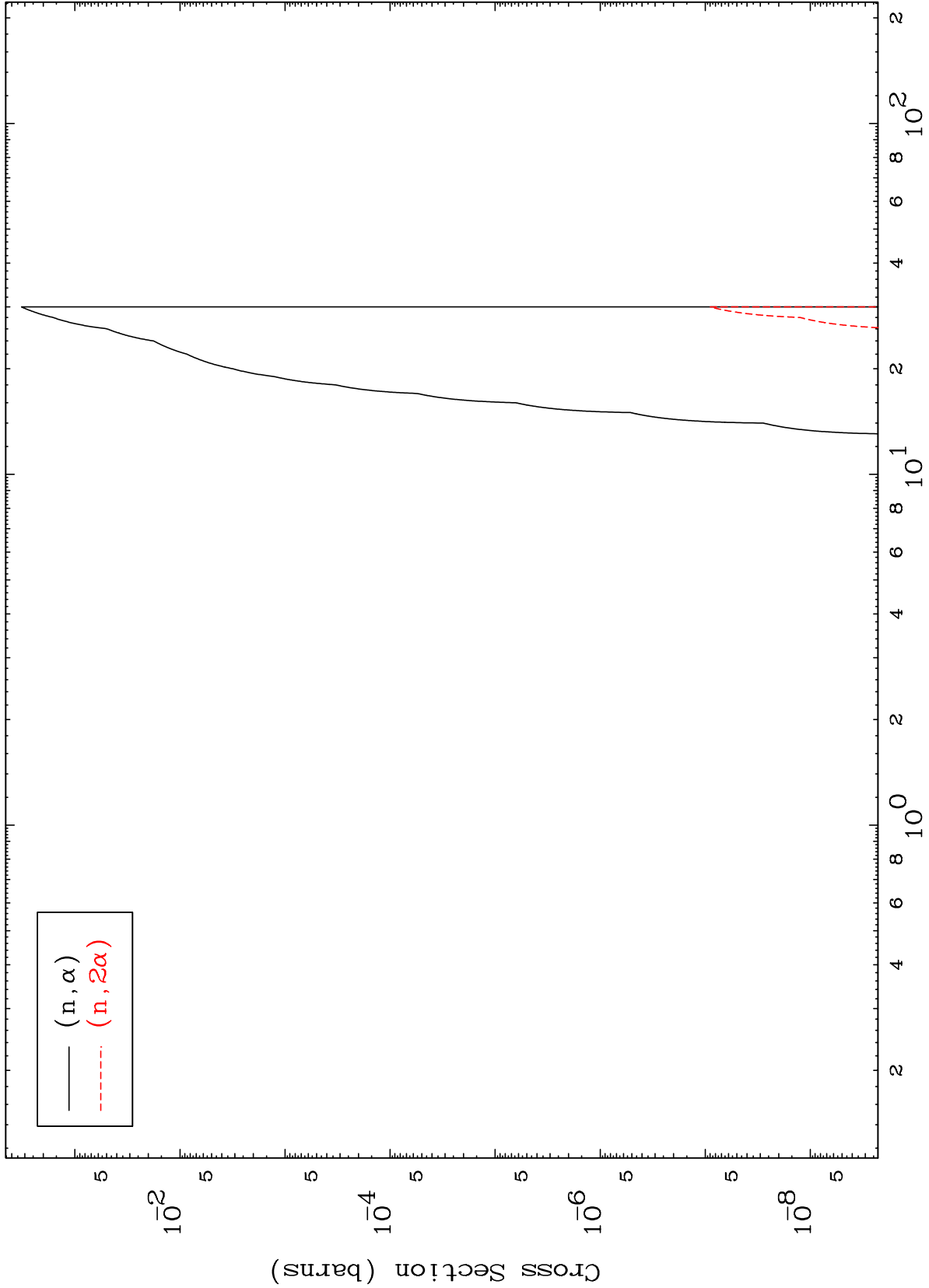


MAT 7110

(α, α) Levels

71-Lu-170

0 Kelvin Cross Sections



— (n, α)
- - - ($n, 2\alpha$)

10

Incident Energy (MeV)

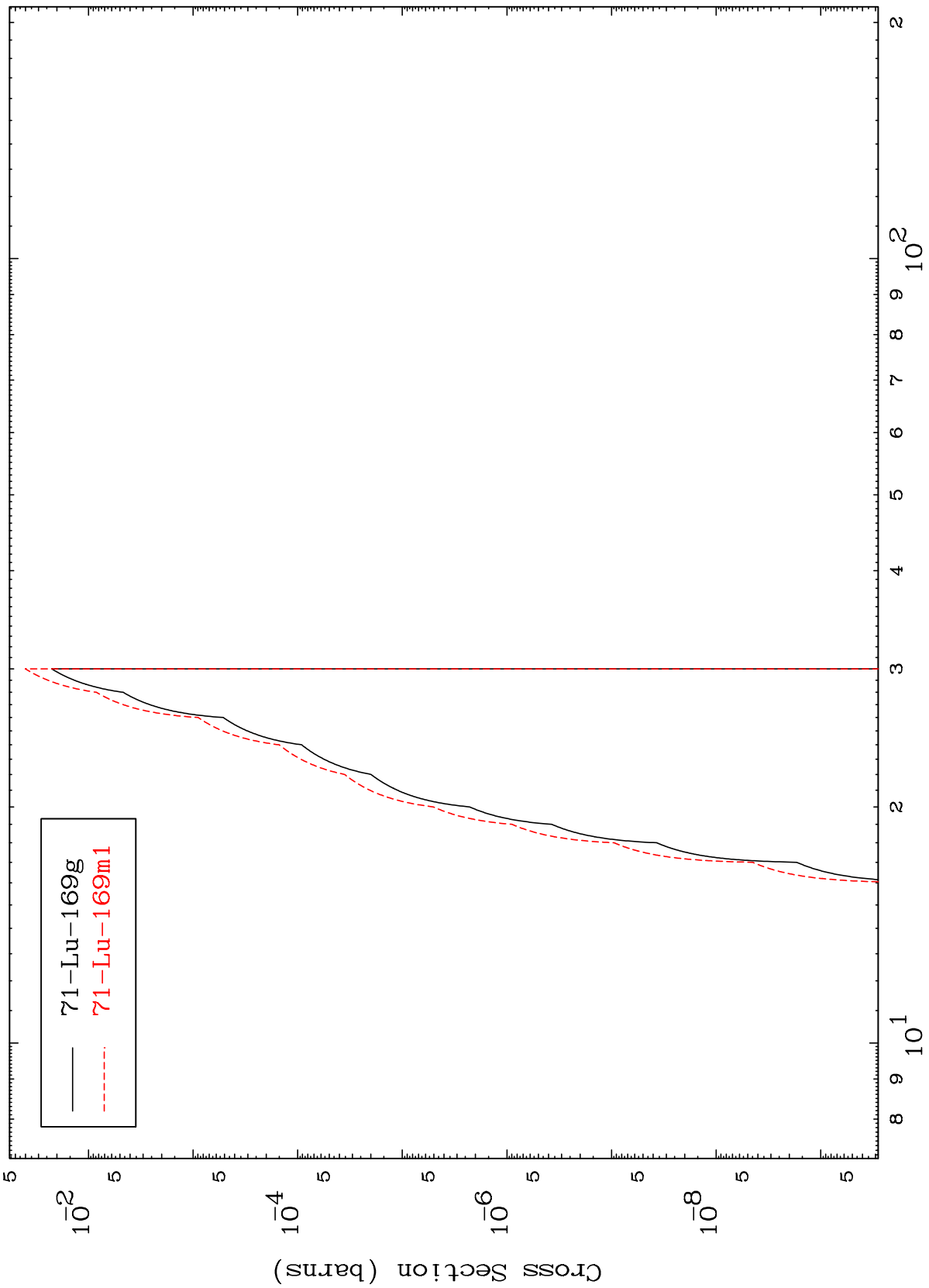
71-Lu-170

MAT 7110

$(n, n') \alpha$

$^{71}\text{Lu-170}$

Radionuclide Production Cross Section



11

Incident Energy (MeV)

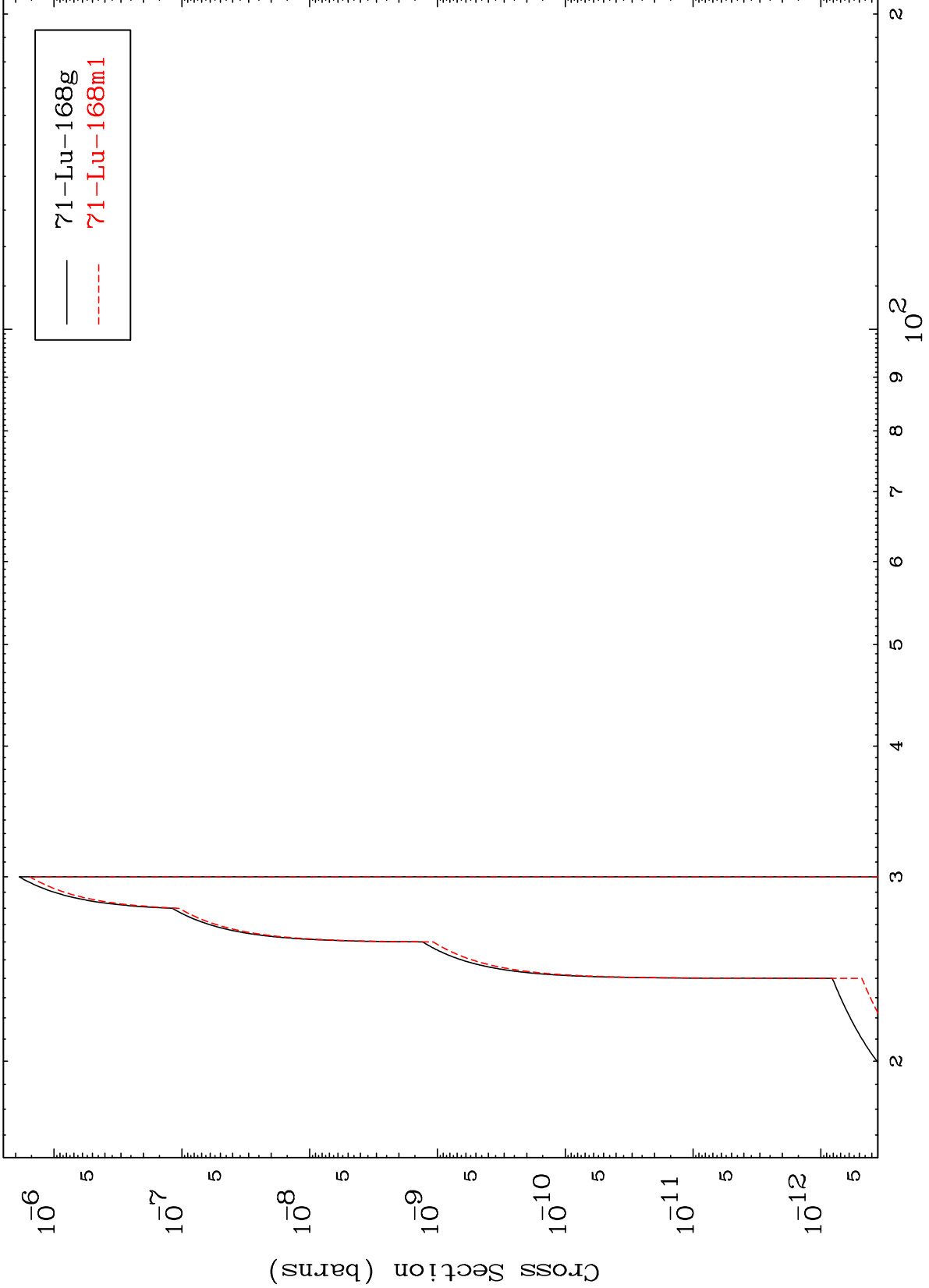
$^{71}\text{Lu-170}$

MAT 7110

$(n,2n) \alpha$

$^{71}\text{Lu-170}$

Radionuclide Production Cross Section



12

Incident Energy (MeV)

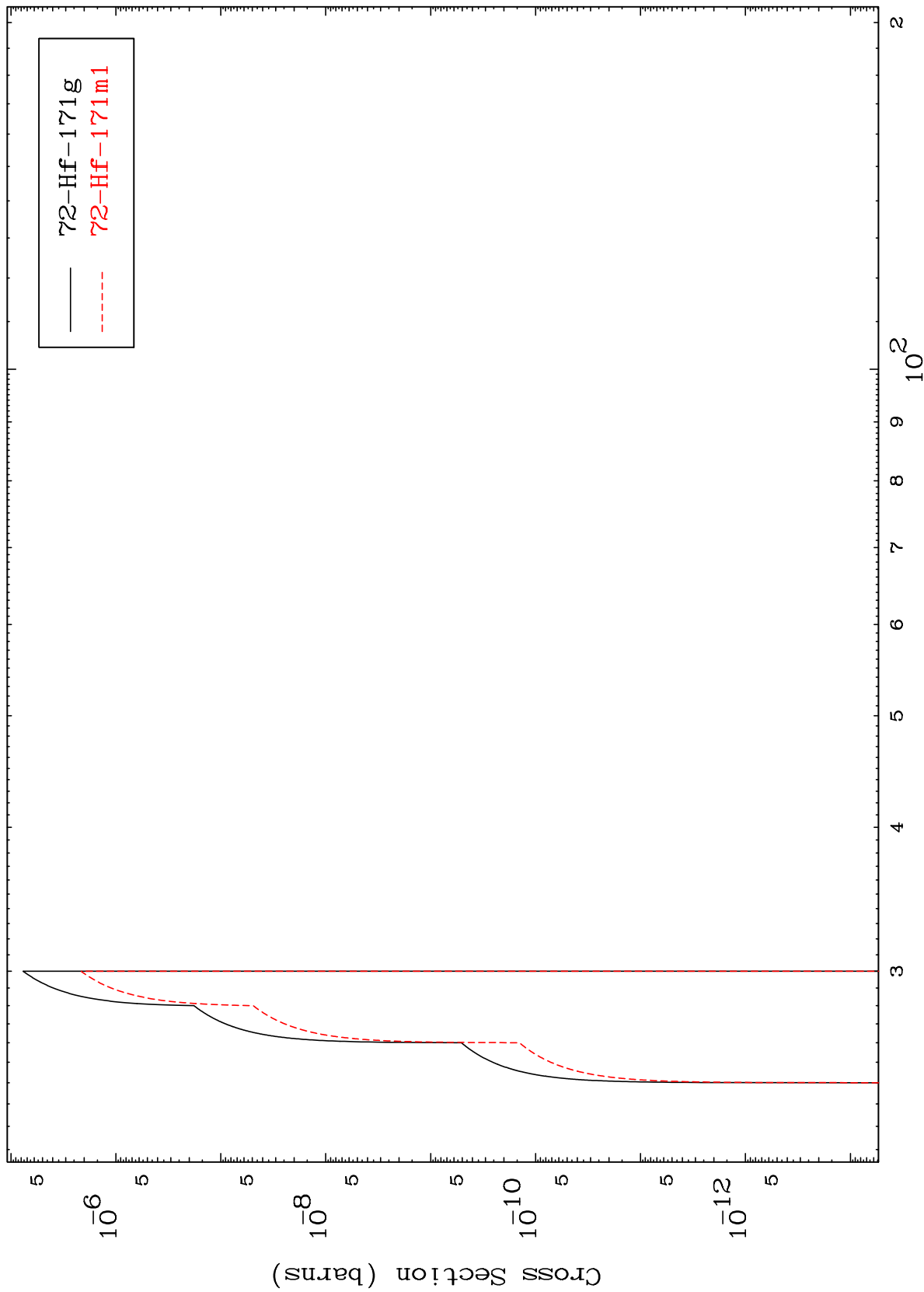
$^{71}\text{Lu-170}$

MAT 7110

(n,n') d

71-Lu-170

Radionuclide Production Cross Section



13

Incident Energy (MeV)

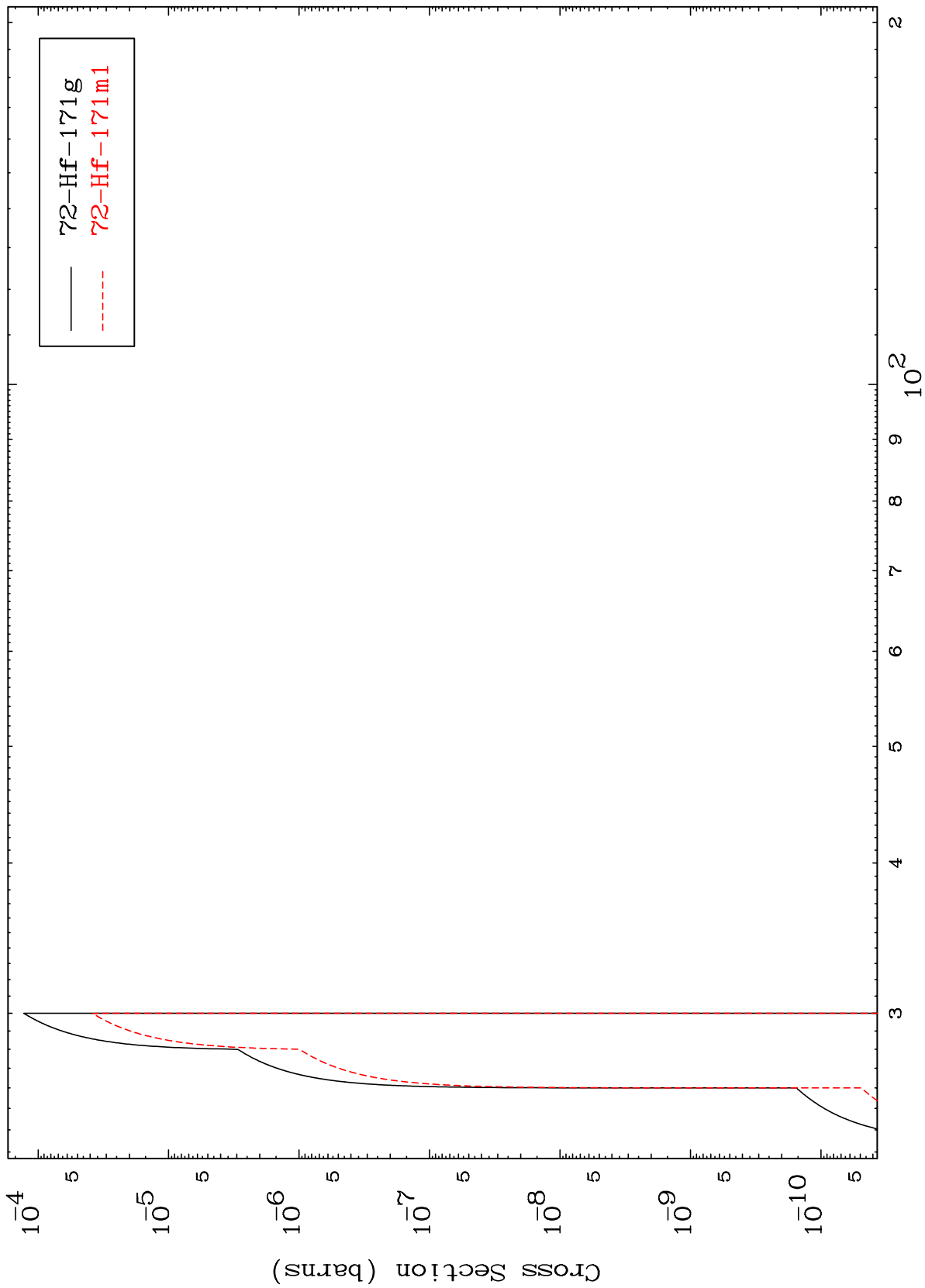
71-Lu-170

MAT 7110

(n,2n) p

71-Lu-170

Radionuclide Production Cross Section



14

Incident Energy (MeV)

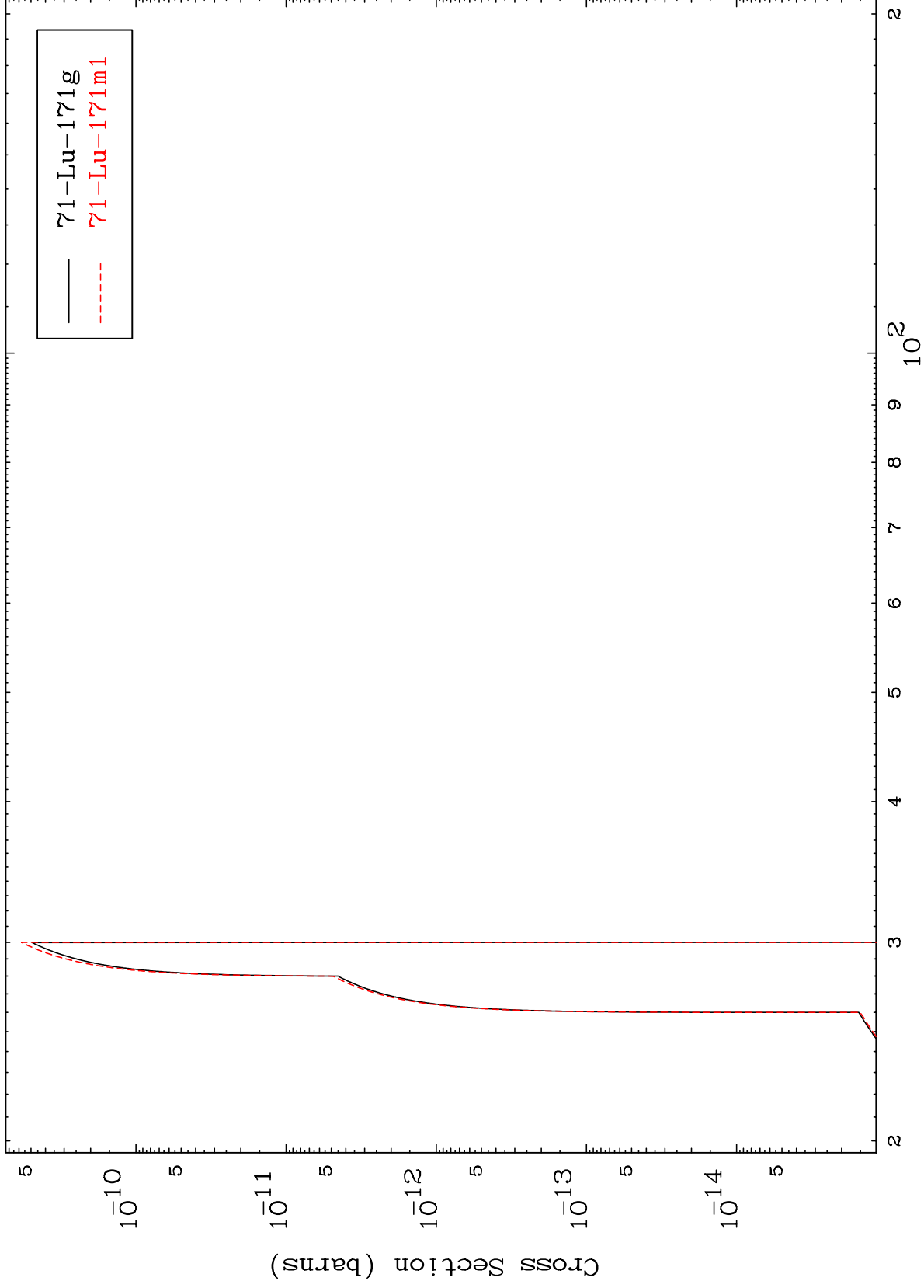
71-Lu-170

MAT 7110

(n,2n) p

71-Lu-170

Radionuclide Production Cross Section



15

Incident Energy (MeV)

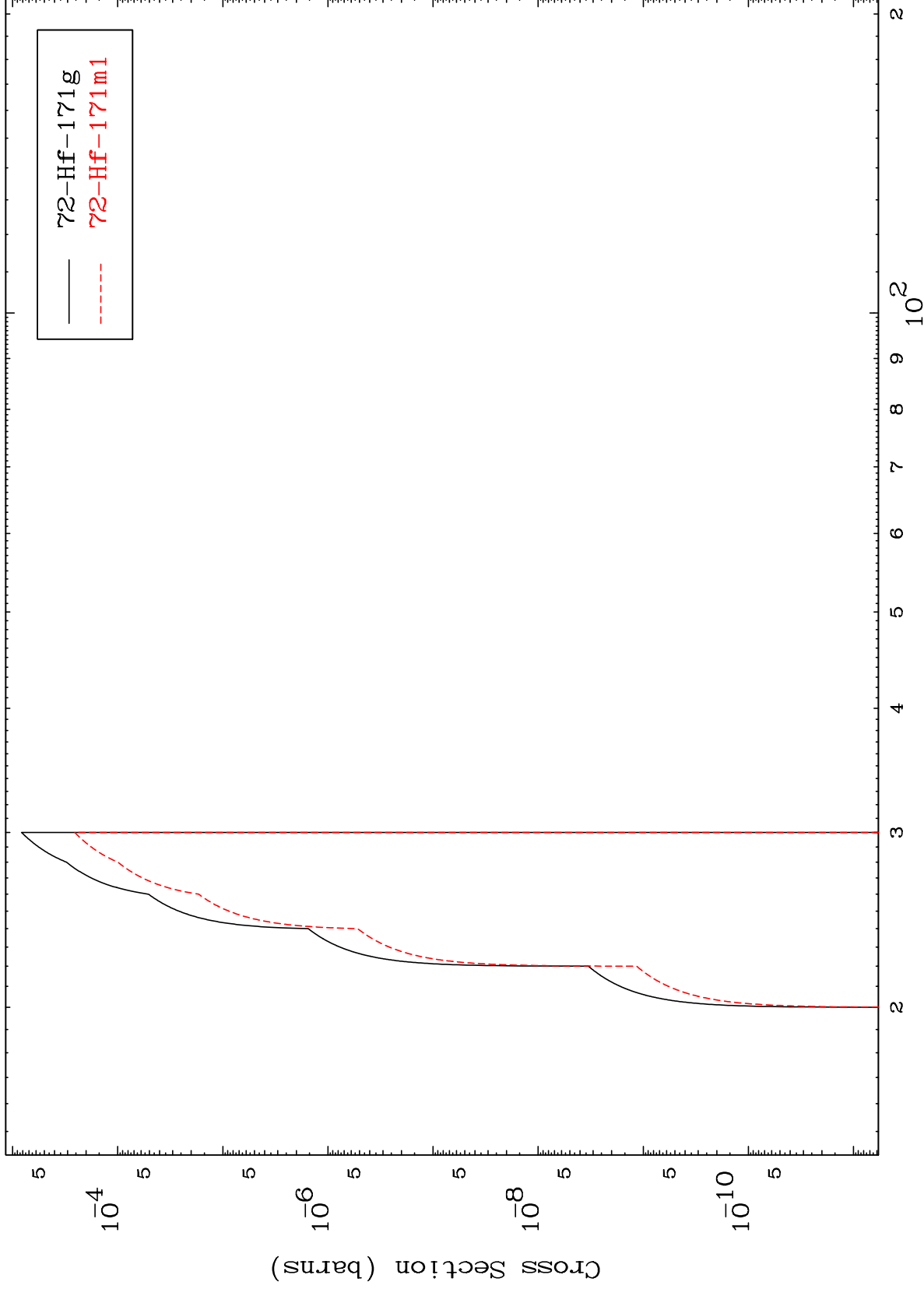
71-Lu-170

MAT 7110

(n, t)

71-Lu-170

Radionuclide Production Cross Section



16

Incident Energy (MeV)

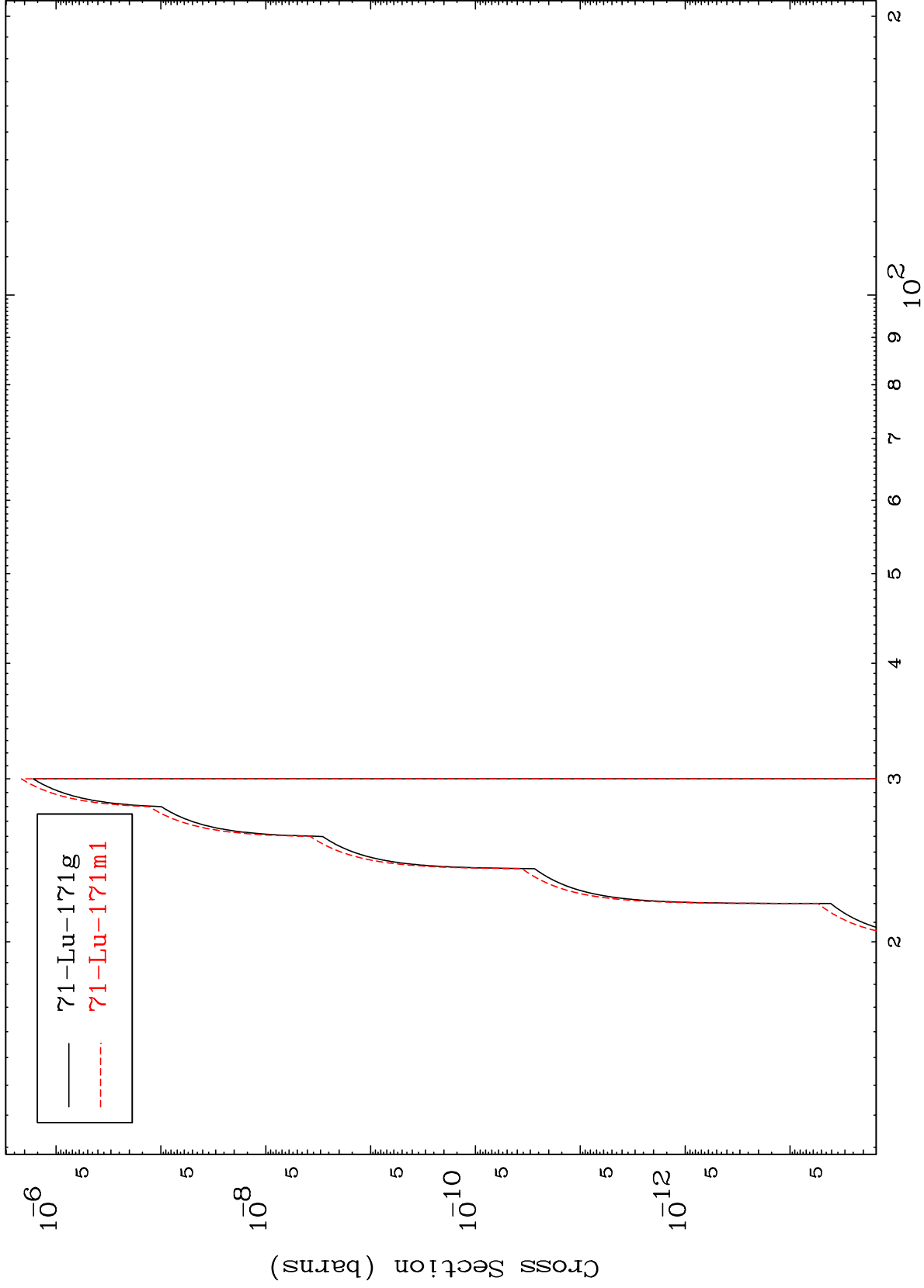
71-Lu-170

MAT 7110

(n,He-3)

71-Lu-170

Radionuclide Production Cross Section



17

Incident Energy (MeV)

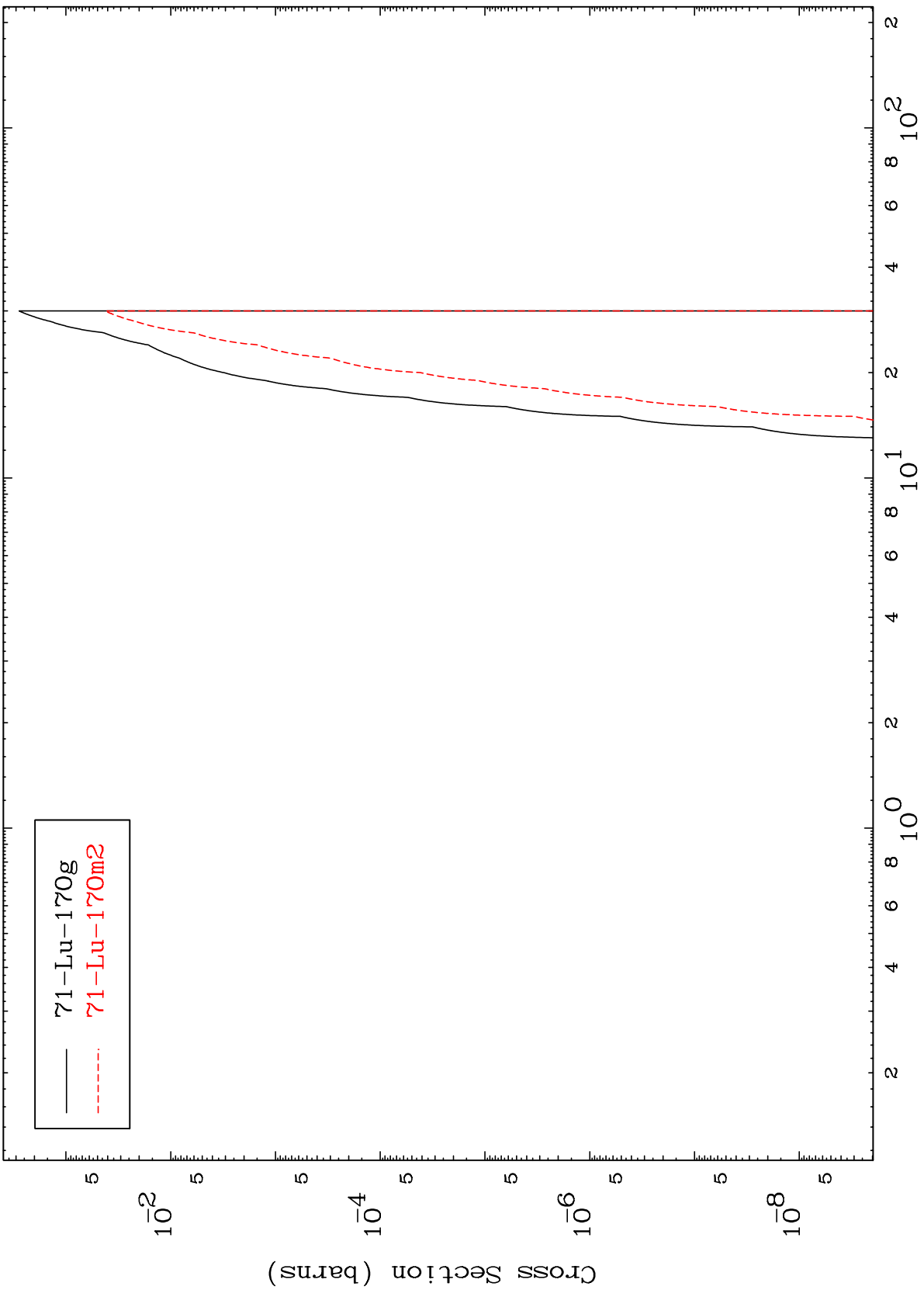
71-Lu-170

MAT 7110

(n, α)

⁷¹Lu-170

Radionuclide Production Cross Section

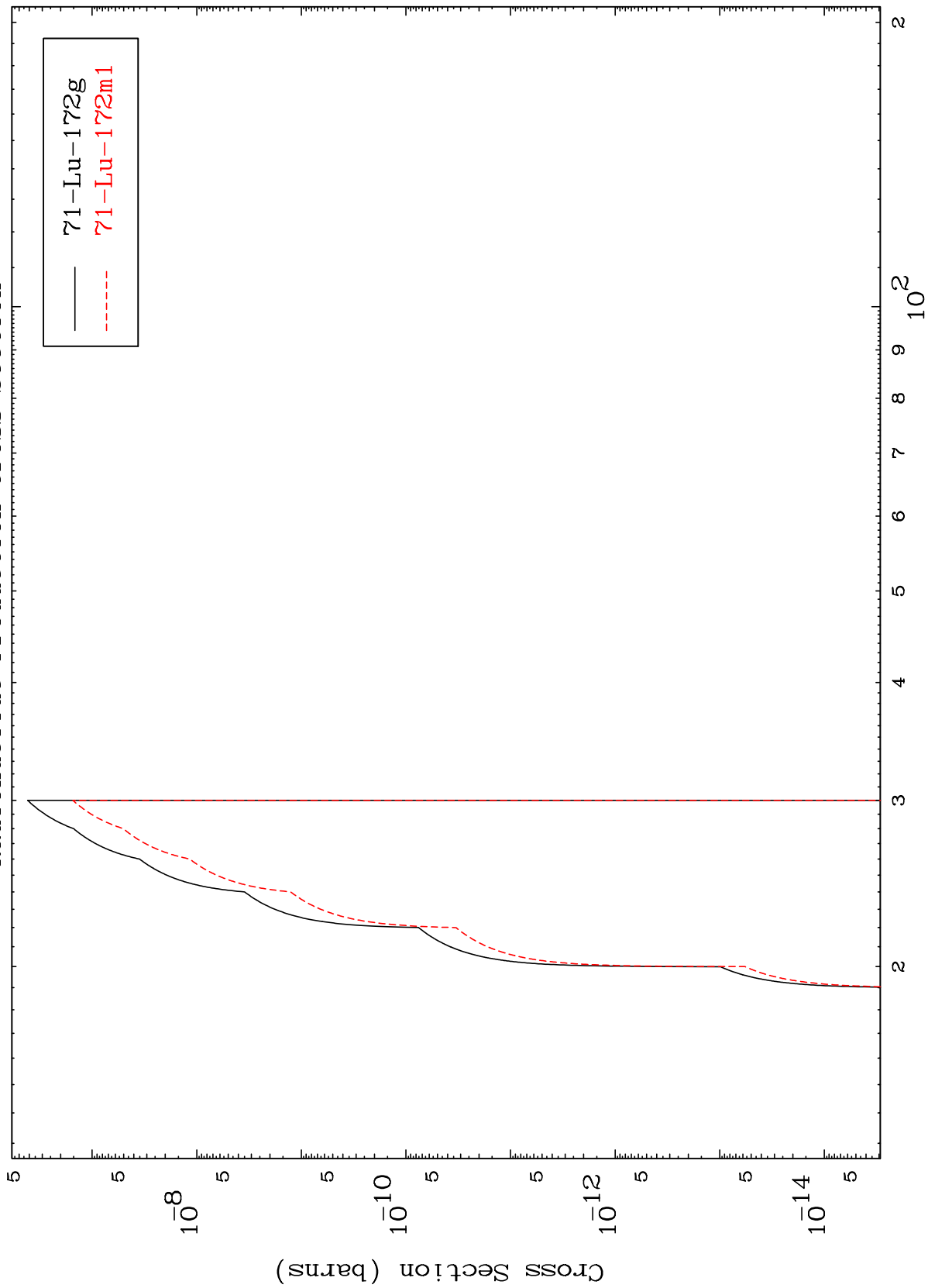


— ⁷¹Lu-170g
- - - ⁷¹Lu-170m2

MAT 7110

71-Lu-170

(n,2p)
Radionuclide Production Cross Section



19

Incident Energy (MeV)

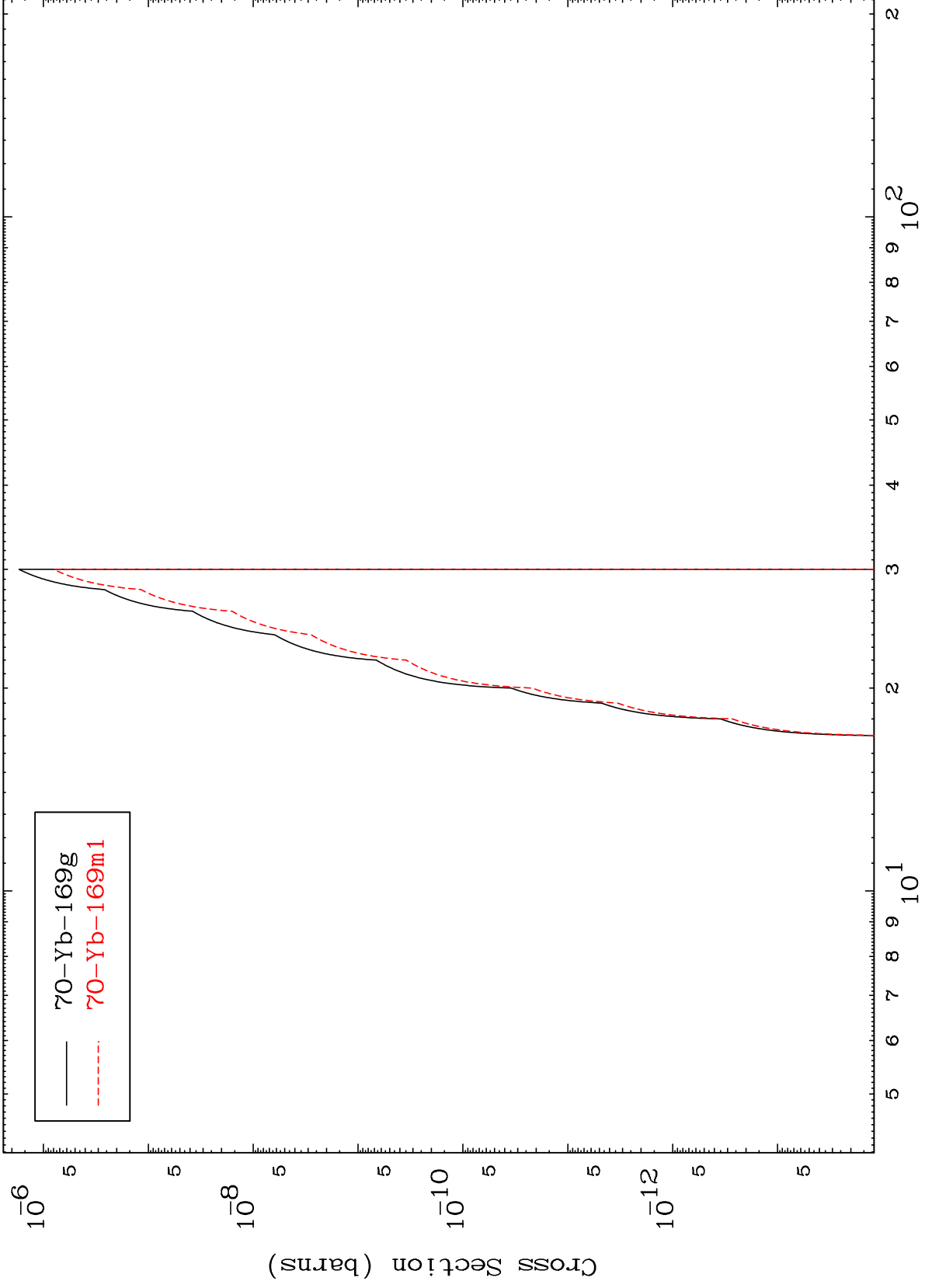
71-Lu-170

MAT 7110

$(n,p) \alpha$

$^{71}\text{Lu-170}$

Radionuclide Production Cross Section



20

Incident Energy (MeV)

$^{71}\text{Lu-170}$