Program EVALPLOT
(Version 2021-1)
by
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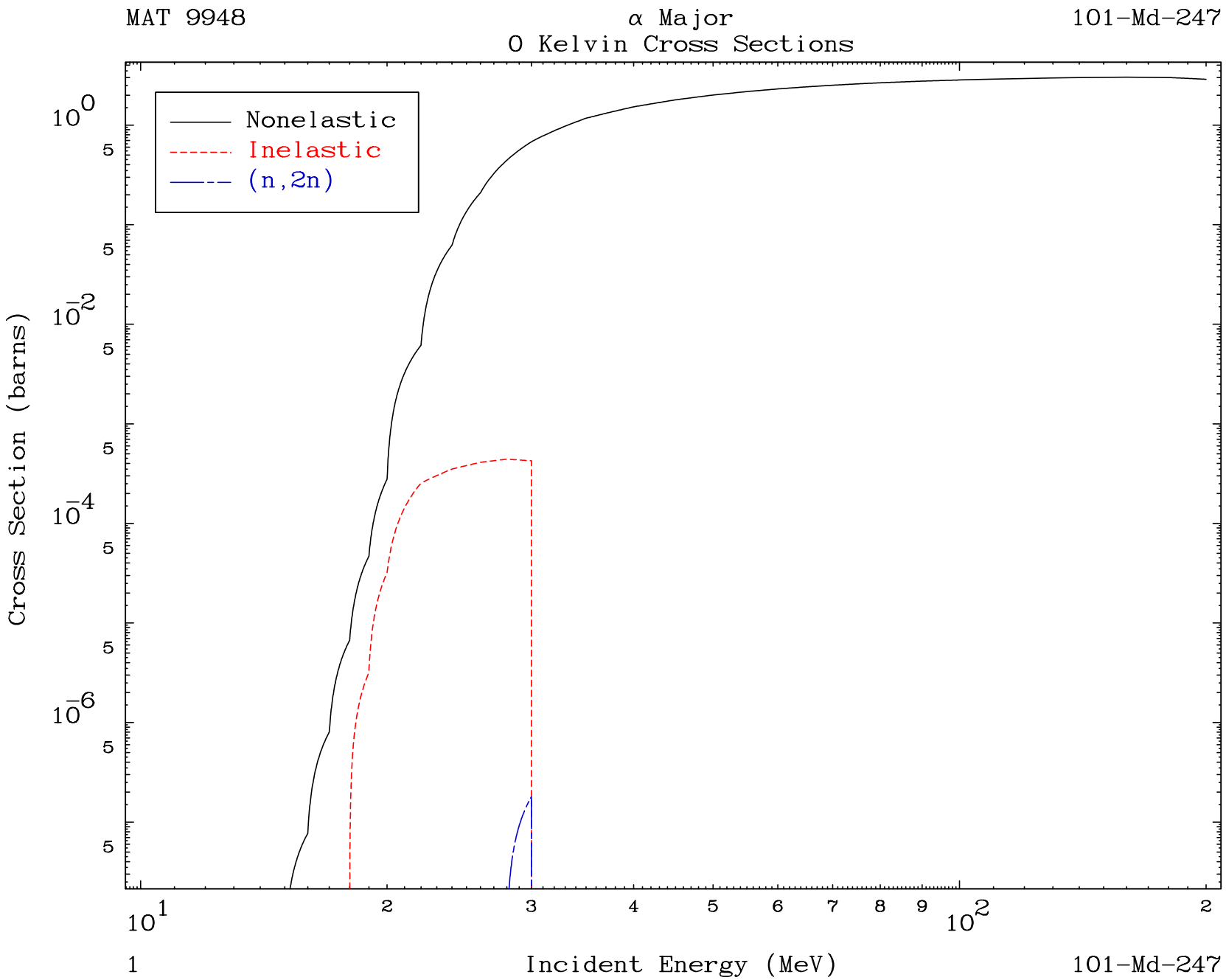
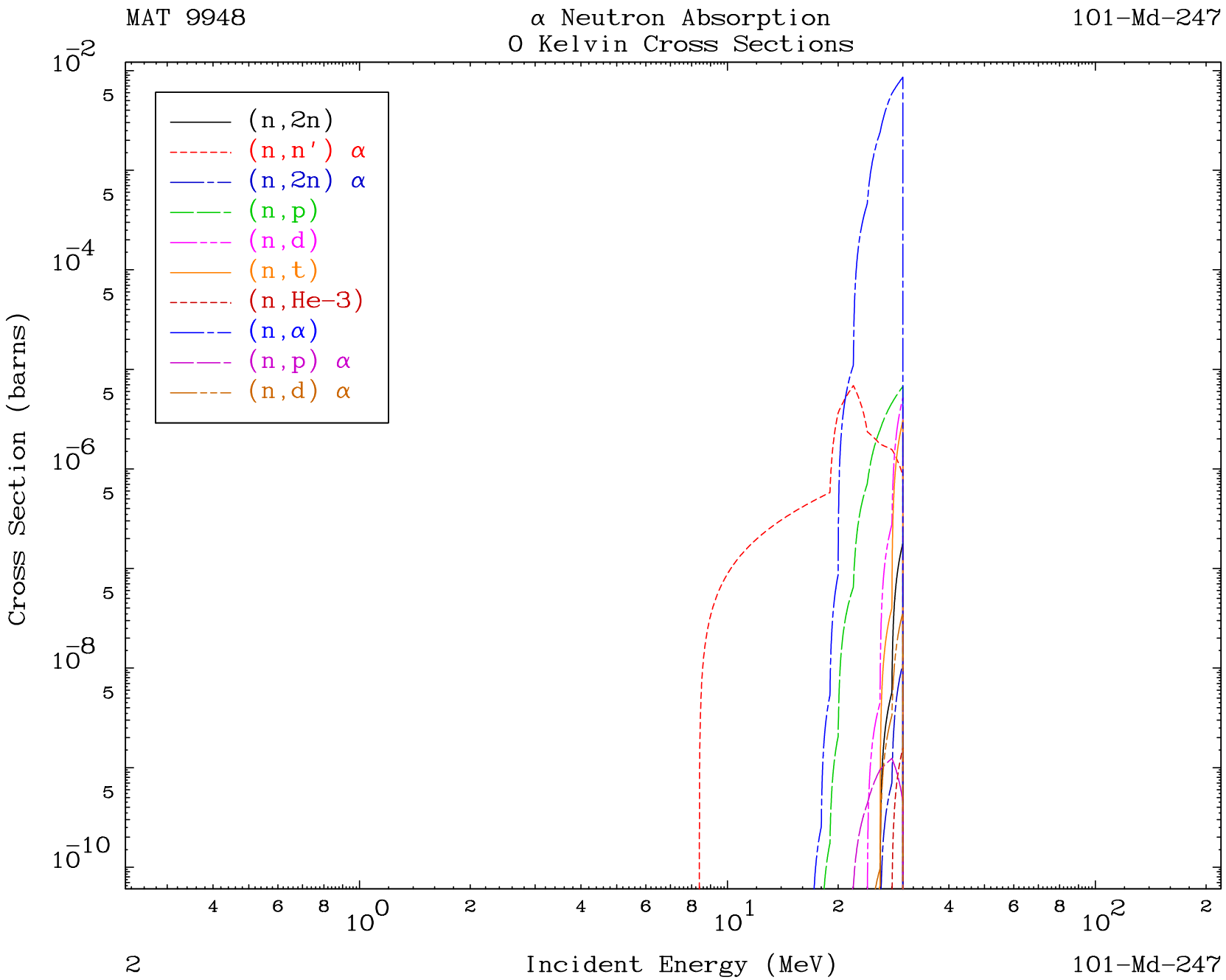
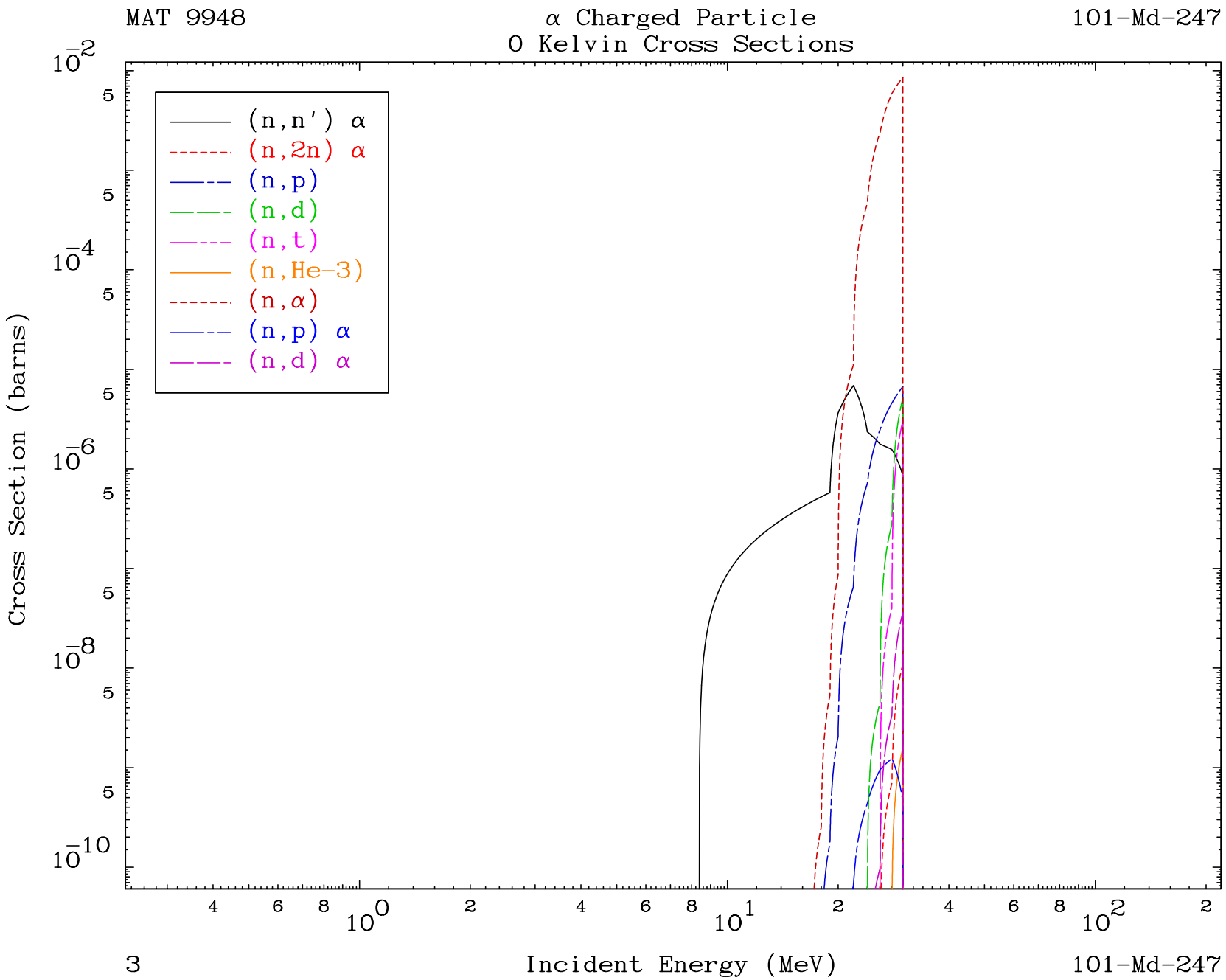
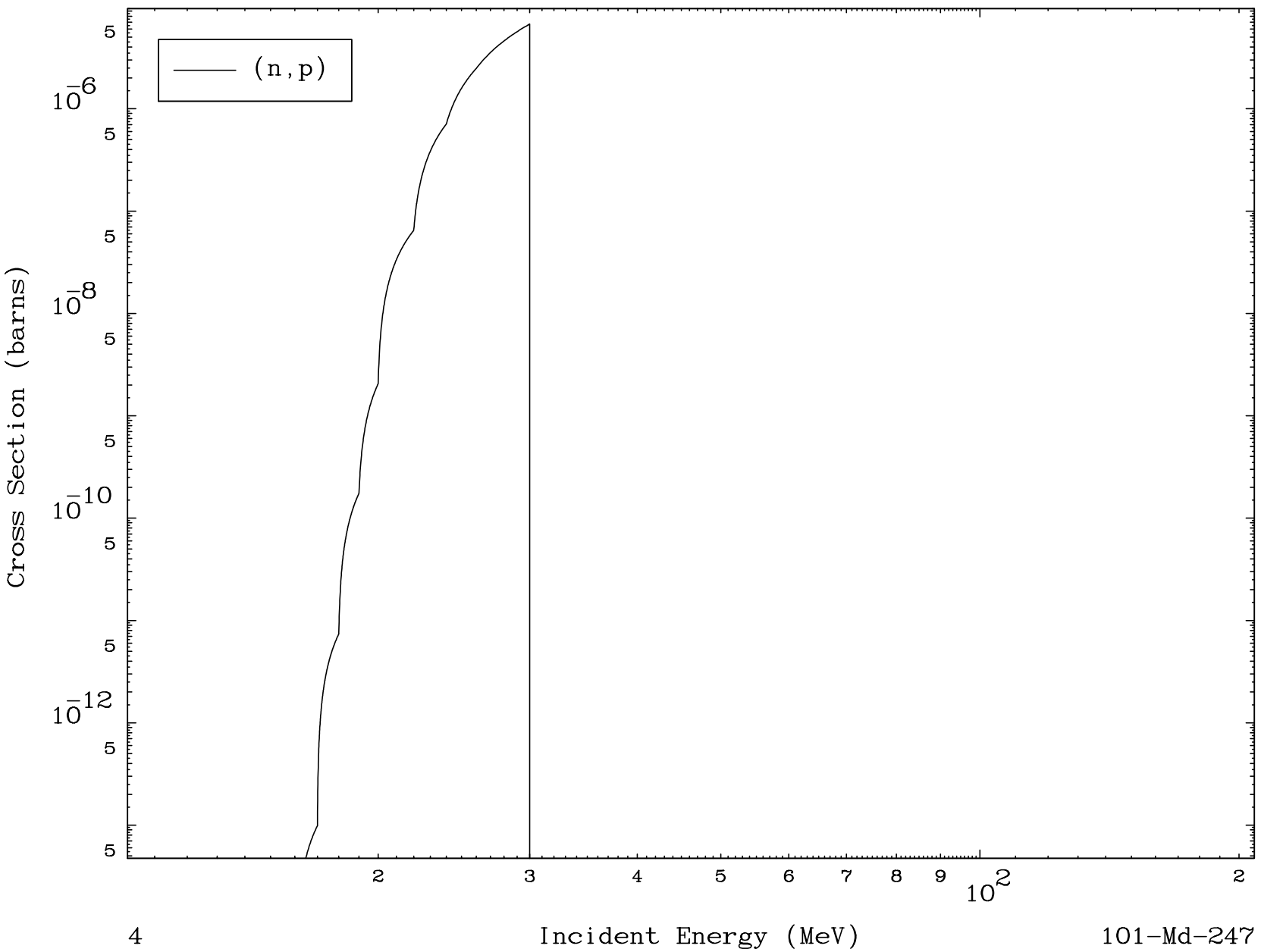
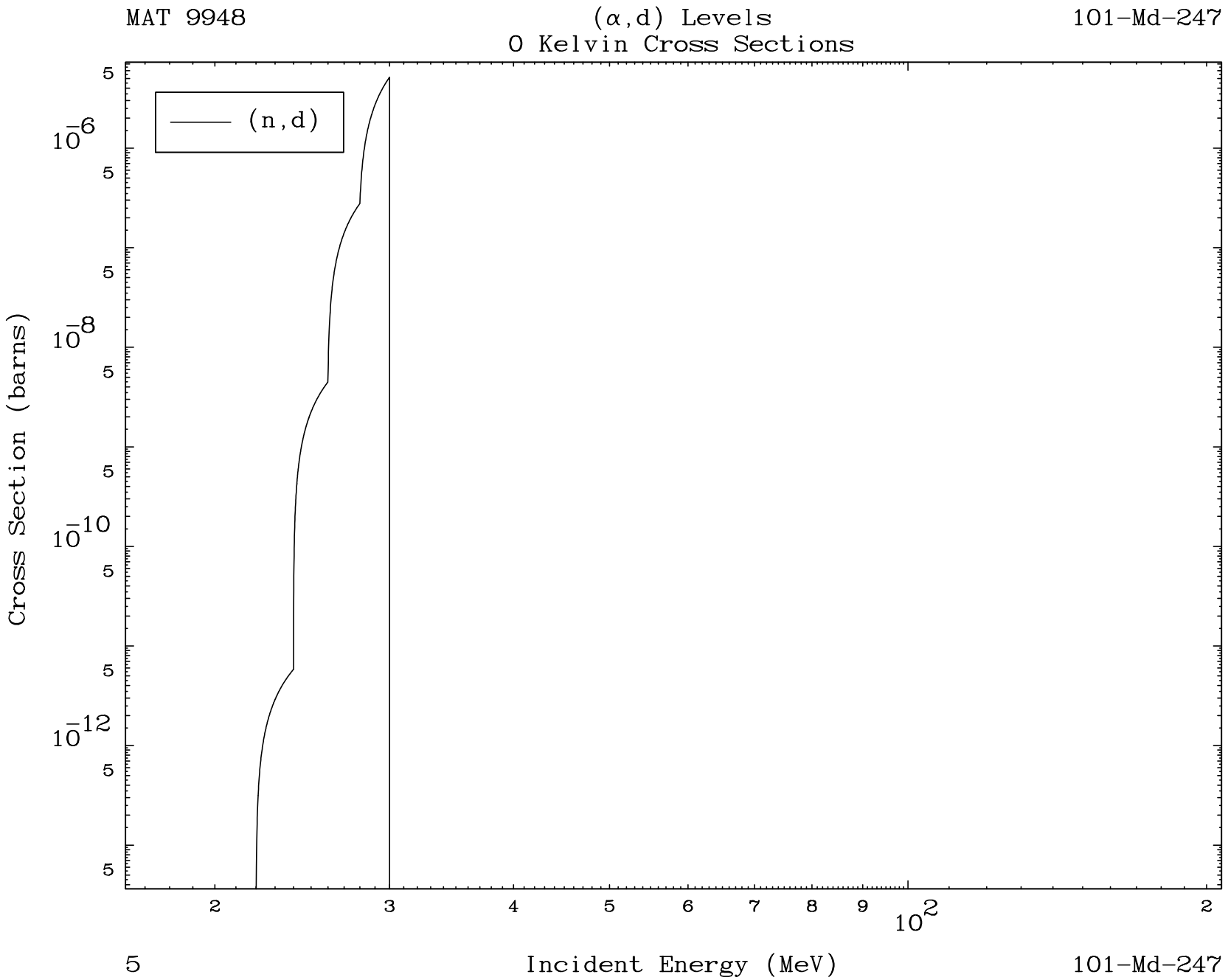
Press Mouse Button to Start


MAT 9948
α Major
101-Md-247
0 Kelvin Cross Sections
Nonelastic
Inelastic
(n,2n)
Cross Section (barns)
Incident Energy (MeV)
101-Md-247

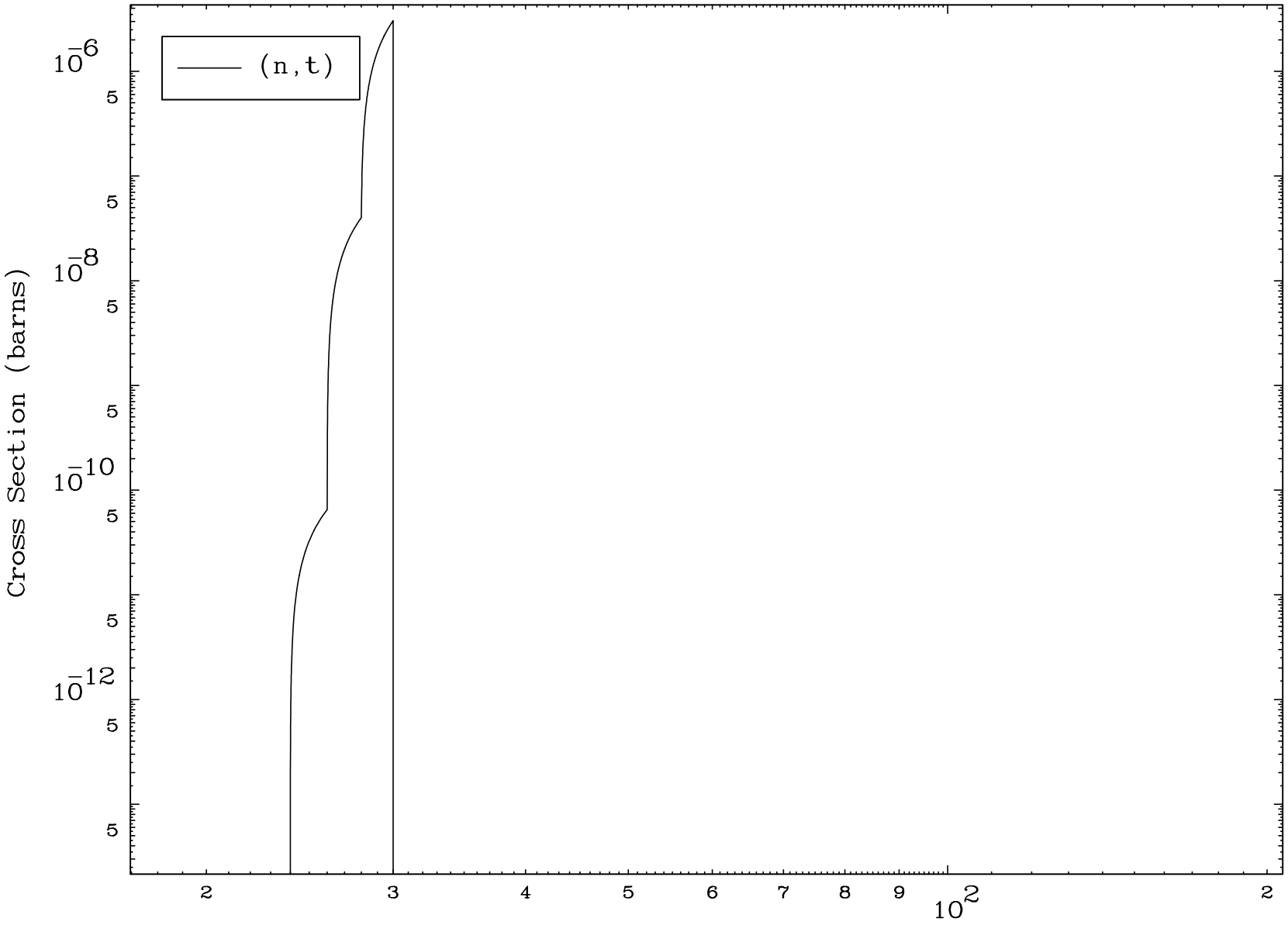

MAT 9948
α Neutron Absorption
0 Kelvin Cross Sections
101-Md-247
(n,2n)
(n,n') α
(n,2n) α
(n,p)
(n,d)
(n,t)
(n,He-3)
(n,α)
(n,p) α
(n,d) α
Cross Section (barns)
Incident Energy (MeV)
101-Md-247
2

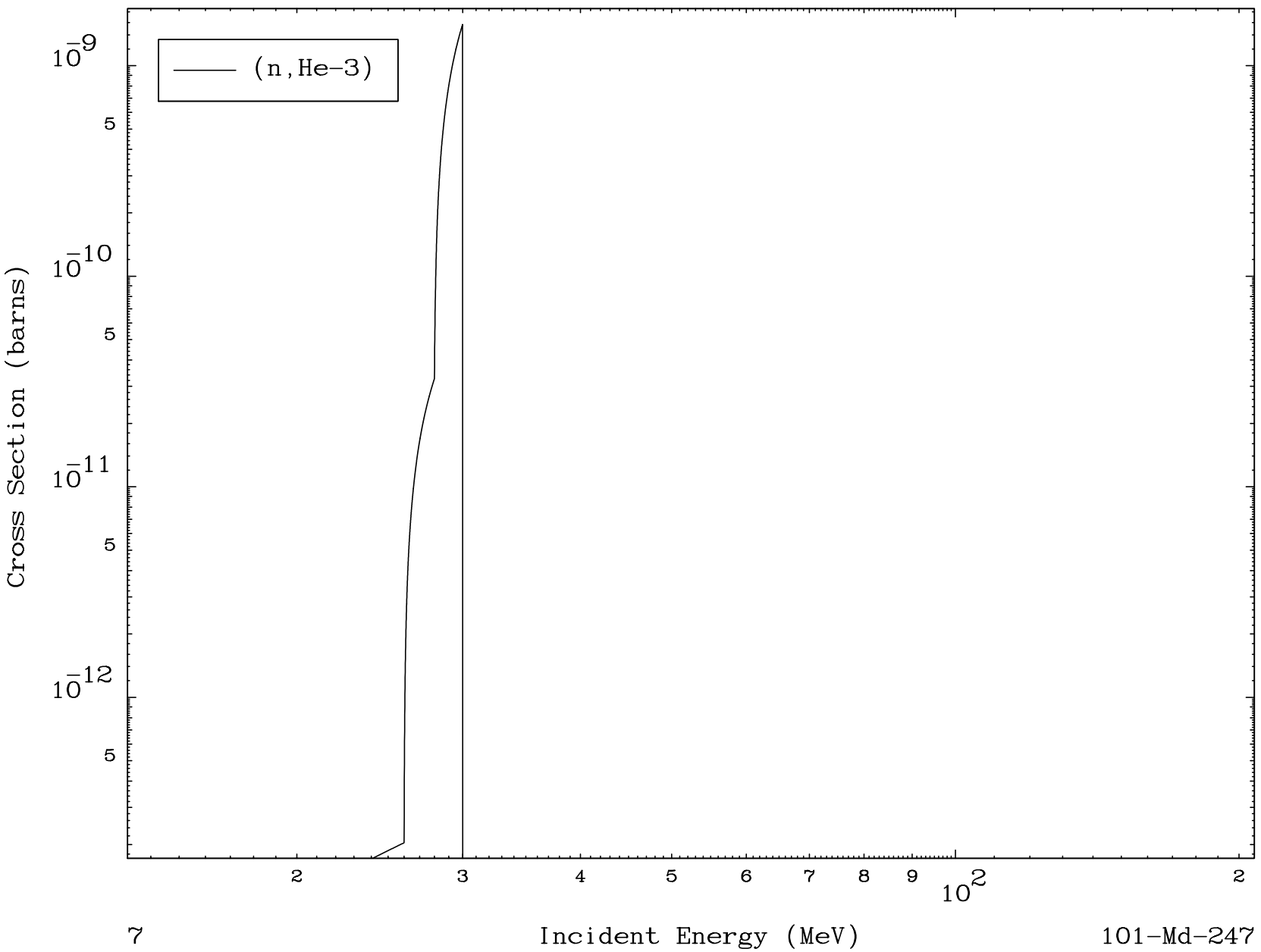

MAT 9948
α Charged Particle
0 Kelvin Cross Sections
101-Md-247
(n,n') α
(n,2n) α
(n,p)
(n,d)
(n,t)
(n,He-3)
(n,α)
(n,p) α
(n,d) α
Cross Section (barns)
Incident Energy (MeV)
3
101-Md-247

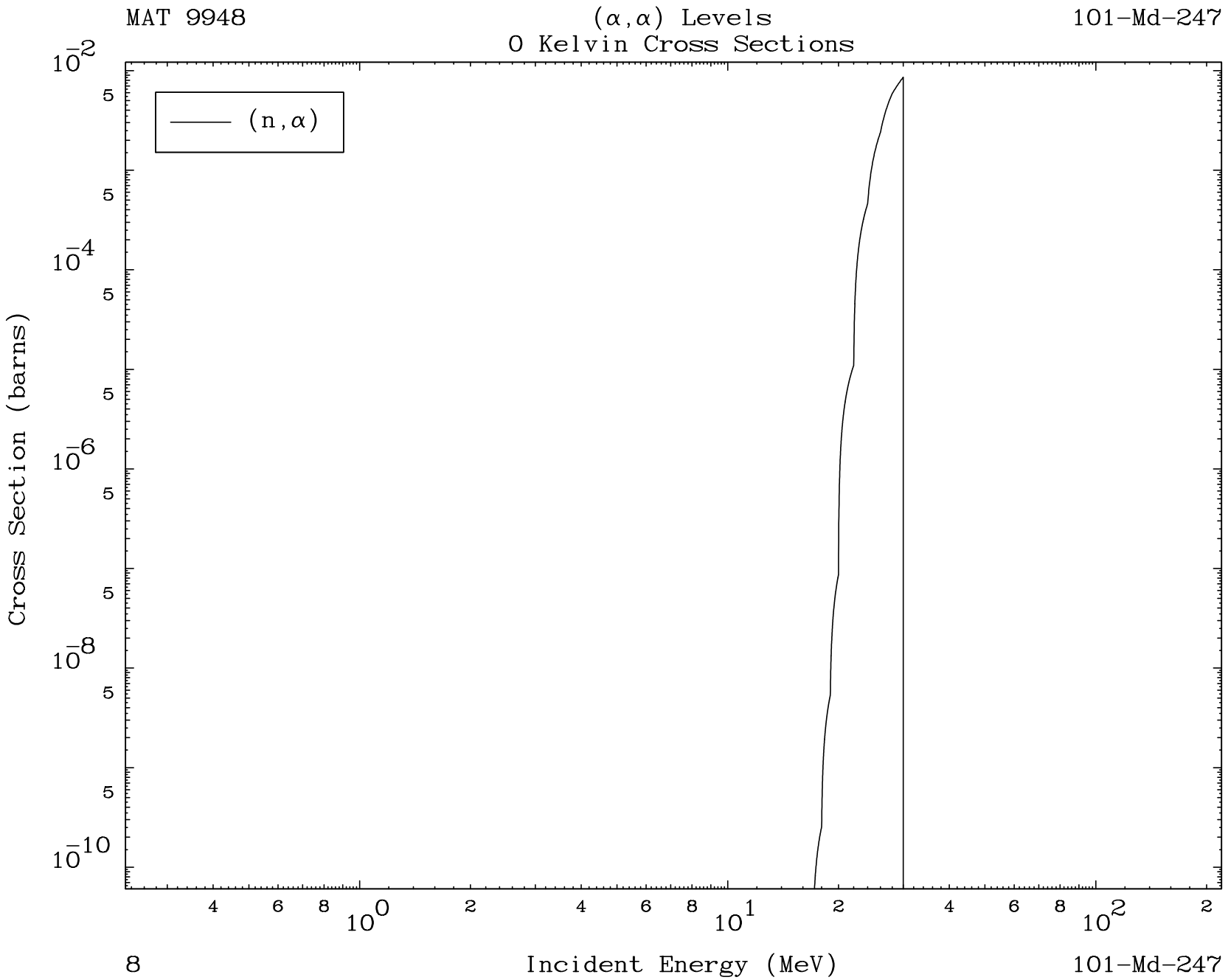

0 Kelvin Cross Sections
(n,p)
Cross Section (barns)
10^-6
10^-8
10^-10
10^-12
5
2
3
4
5
6
7
8
9
10^2
2
Incident Energy (MeV)

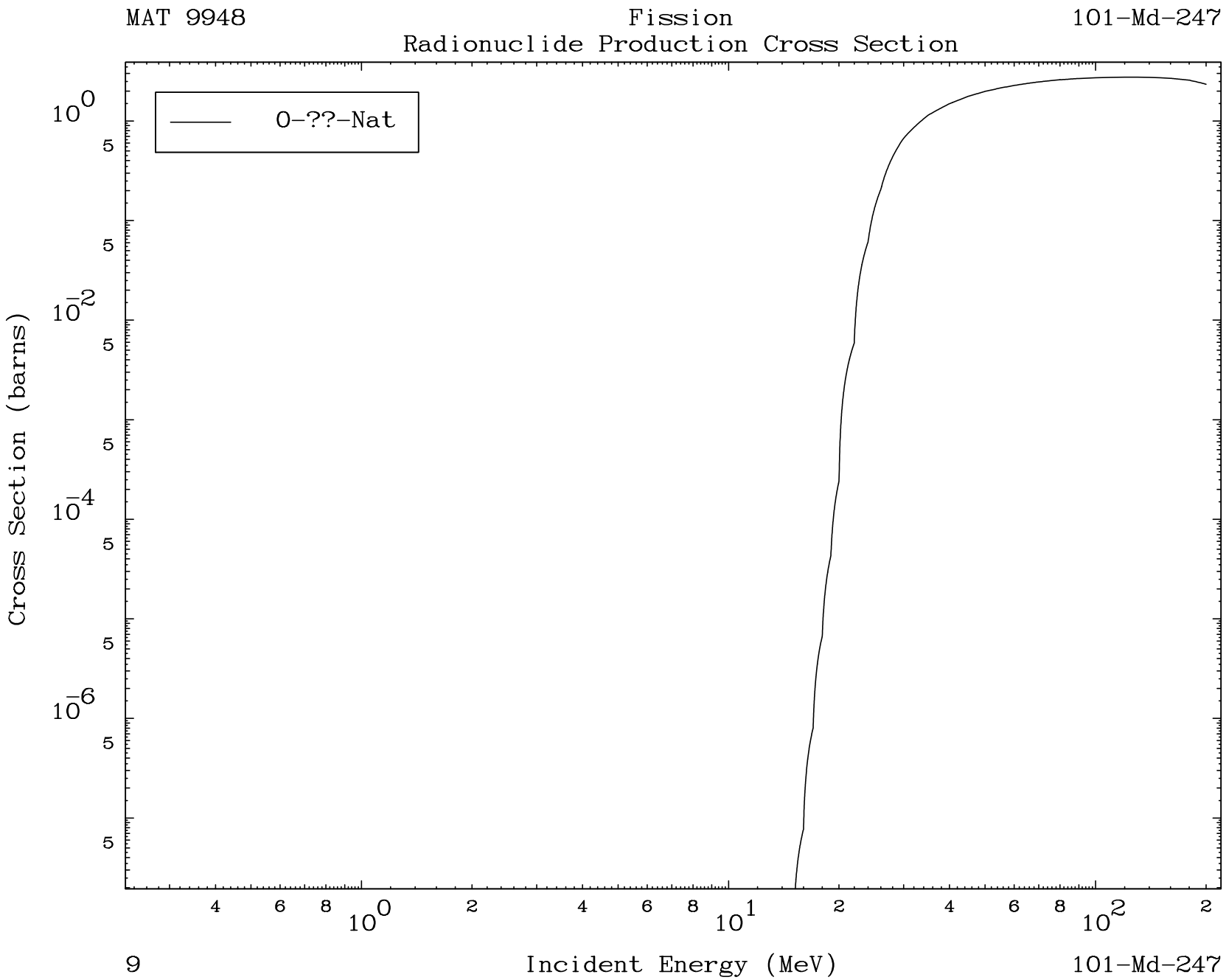

MAT 9948
(α,d) Levels
101-Md-247
0 Kelvin Cross Sections
(n,d)
Cross Section (barns)
10^-6
10^-8
10^-10
10^-12
Incident Energy (MeV)
10^2
101-Md-247

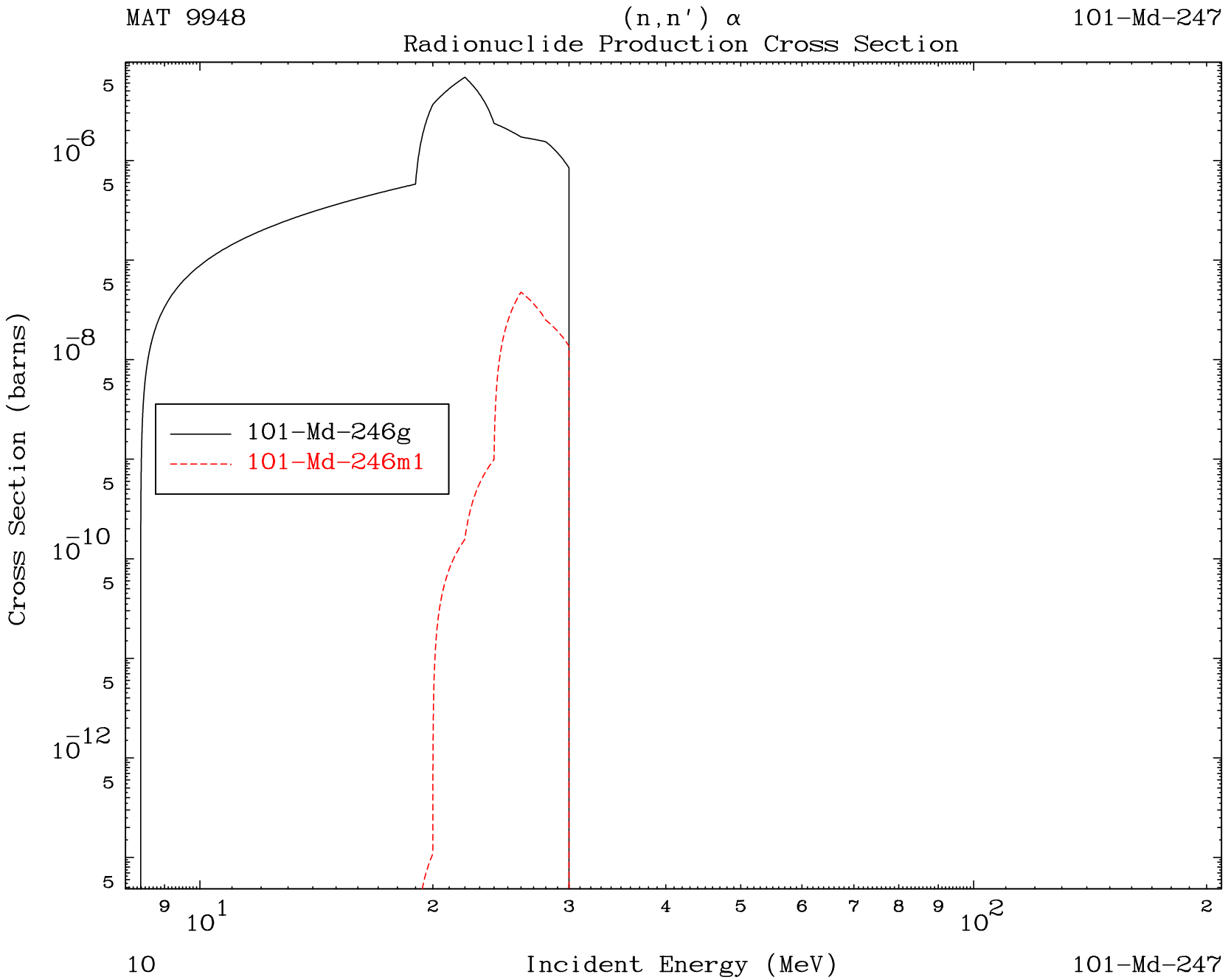
0 Kelvin Cross Sections




0 Kelvin Cross Sections
(n,He-3)
Cross Section (barns)
10-9
5
10-10
5
10-11
5
10-12
5
2
3
4
5
6
7
8
9
10 2
2
7
Incident Energy (MeV)
101-Md-247


MAT 9948
(α,α) Levels
101-Md-247
0 Kelvin Cross Sections
(n,α)
Cross Section (barns)
Incident Energy (MeV)
8
101-Md-247


MAT 9948
Fission
101-Md-247
Radionuclide Production Cross Section
0-??-Nat
Cross Section (barns)
Incident Energy (MeV)
9
101-Md-247


MAT 9948
(n,n') α
101-Md-247
Radionuclide Production Cross Section
Cross Section (barns)
10^-6
10^-8
10^-10
10^-12
101-Md-246g
101-Md-246m1
10^1
10^2
10
Incident Energy (MeV)
101-Md-247