

Program EVALPLOT  
(Version 2021-1)

by

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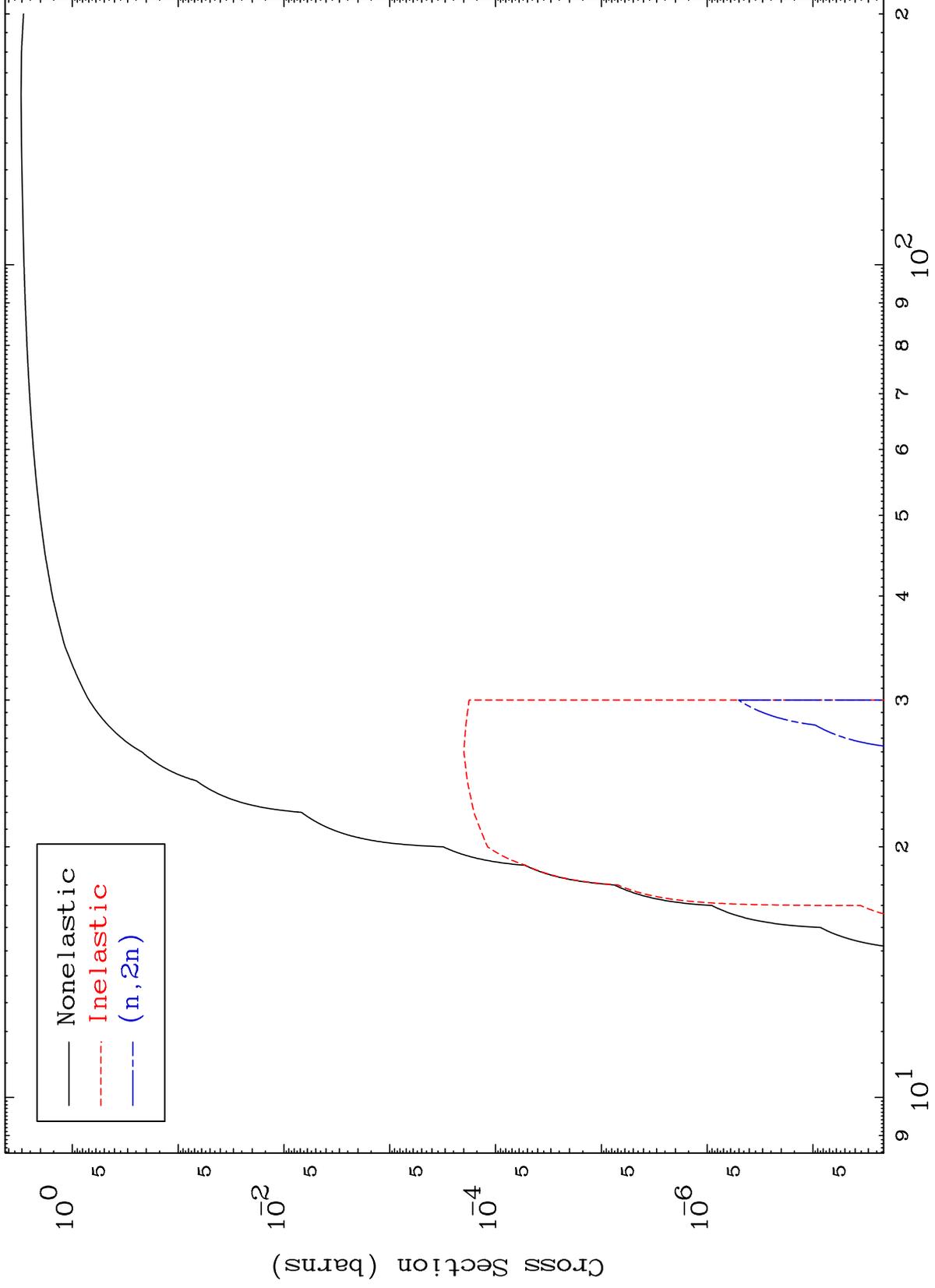
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9951

0 Kelvin Cross Sections

101-Md-250



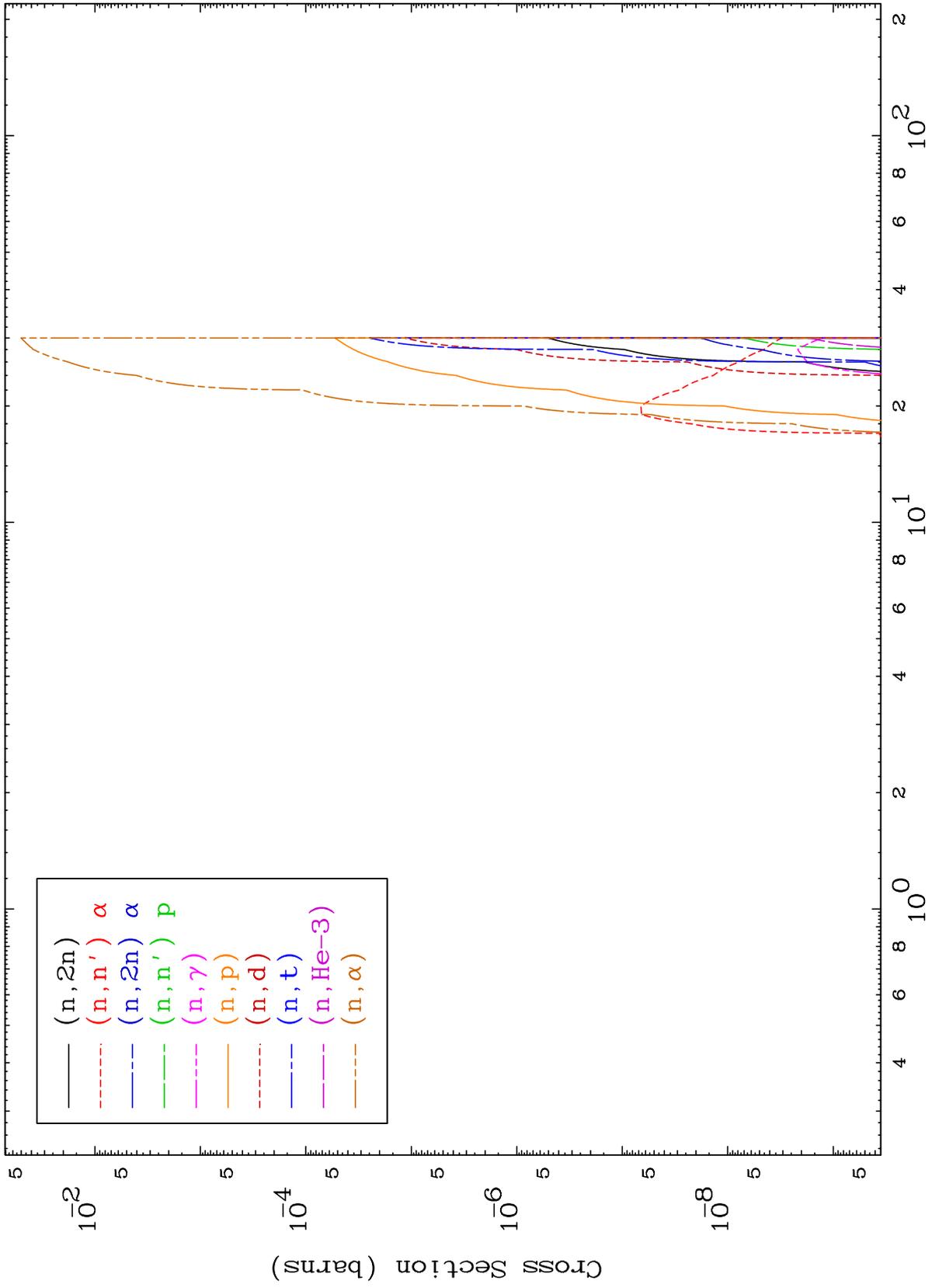
Incident Energy (MeV)

101-Md-250

MAT 9951

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

101-Md-250

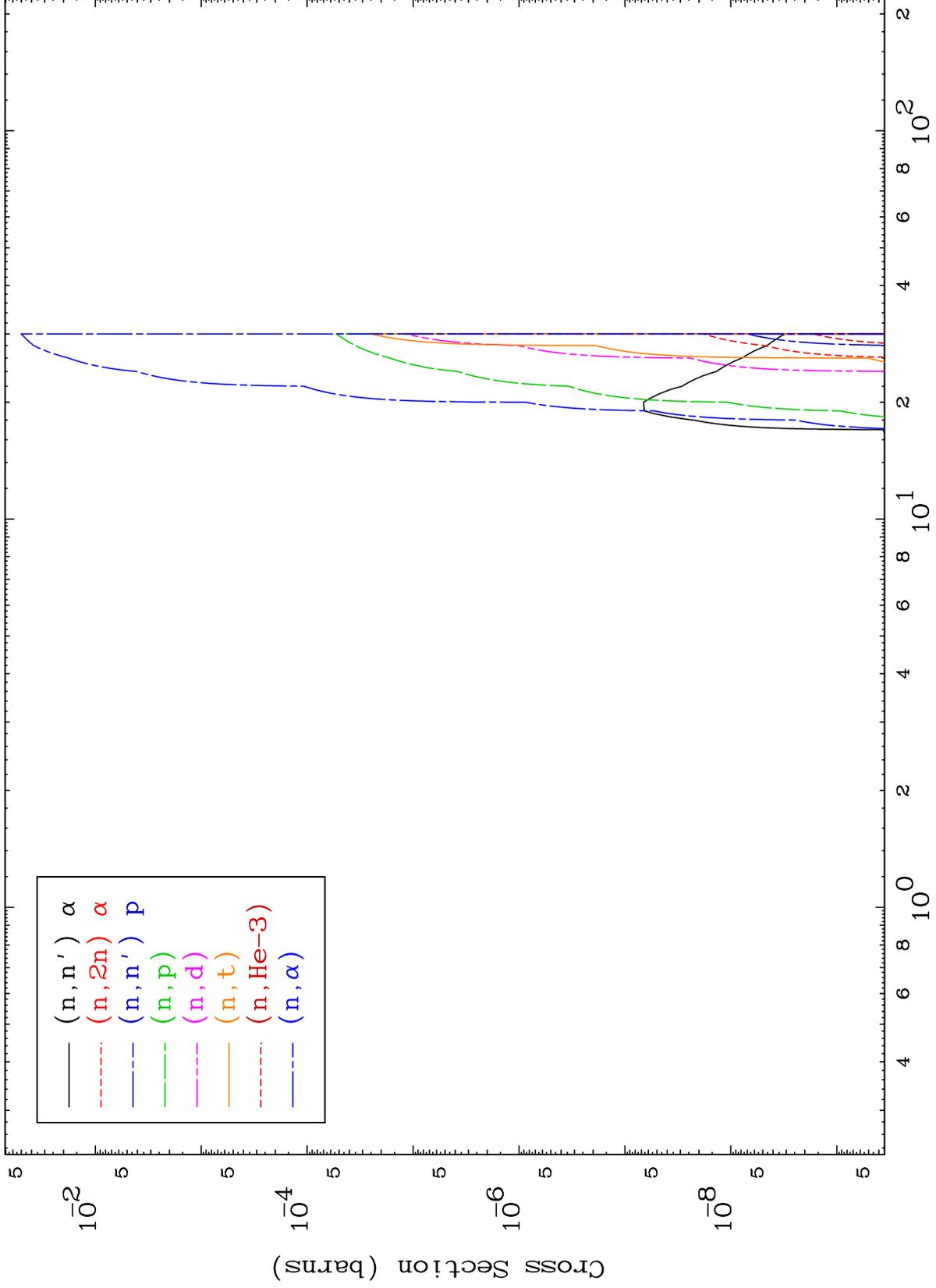


MAT 9951

$\alpha$  Charged Particle

101-Md-250

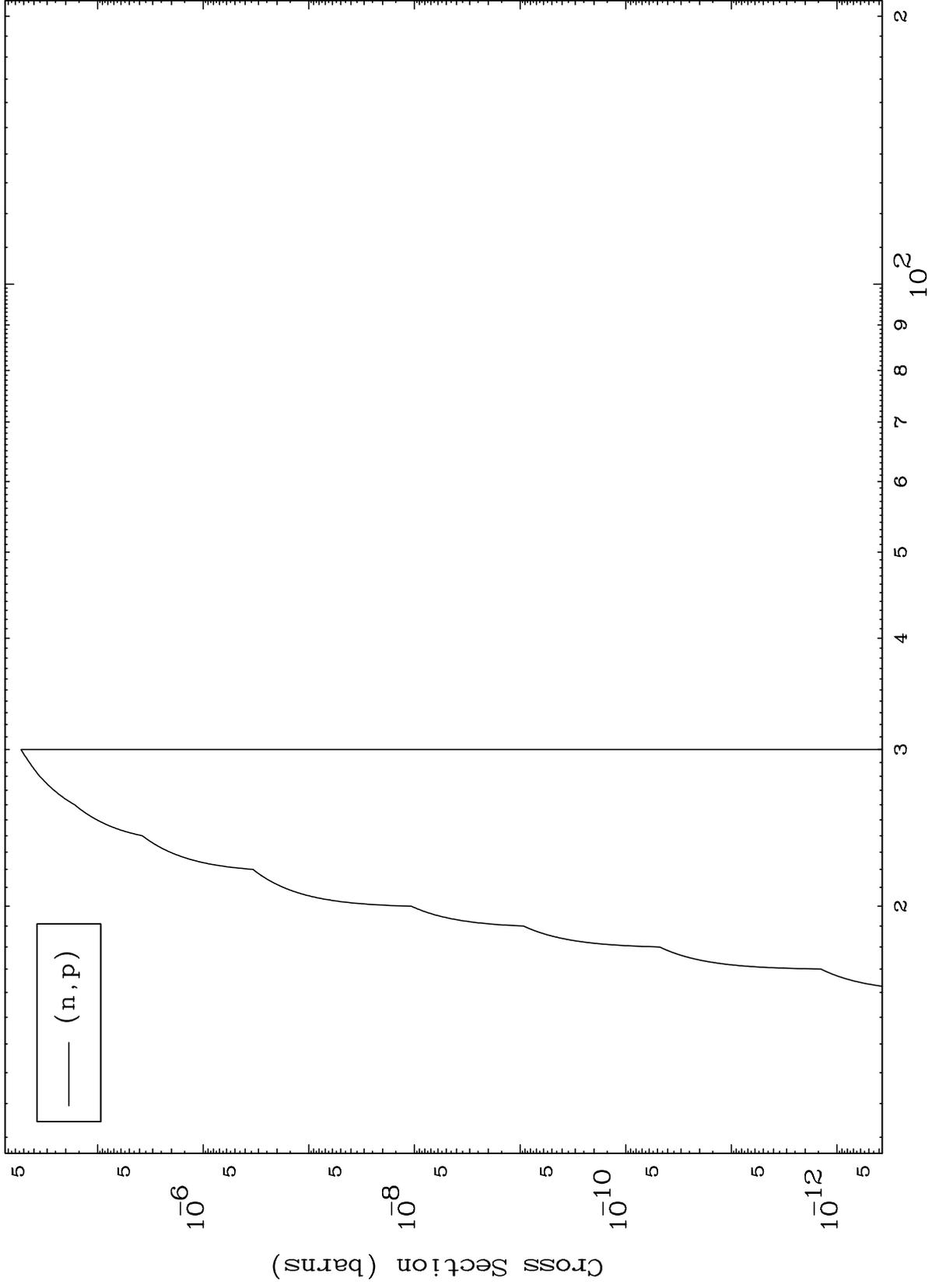
0 Kelvin Cross Sections



MAT 9951

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

101-Md-250



4

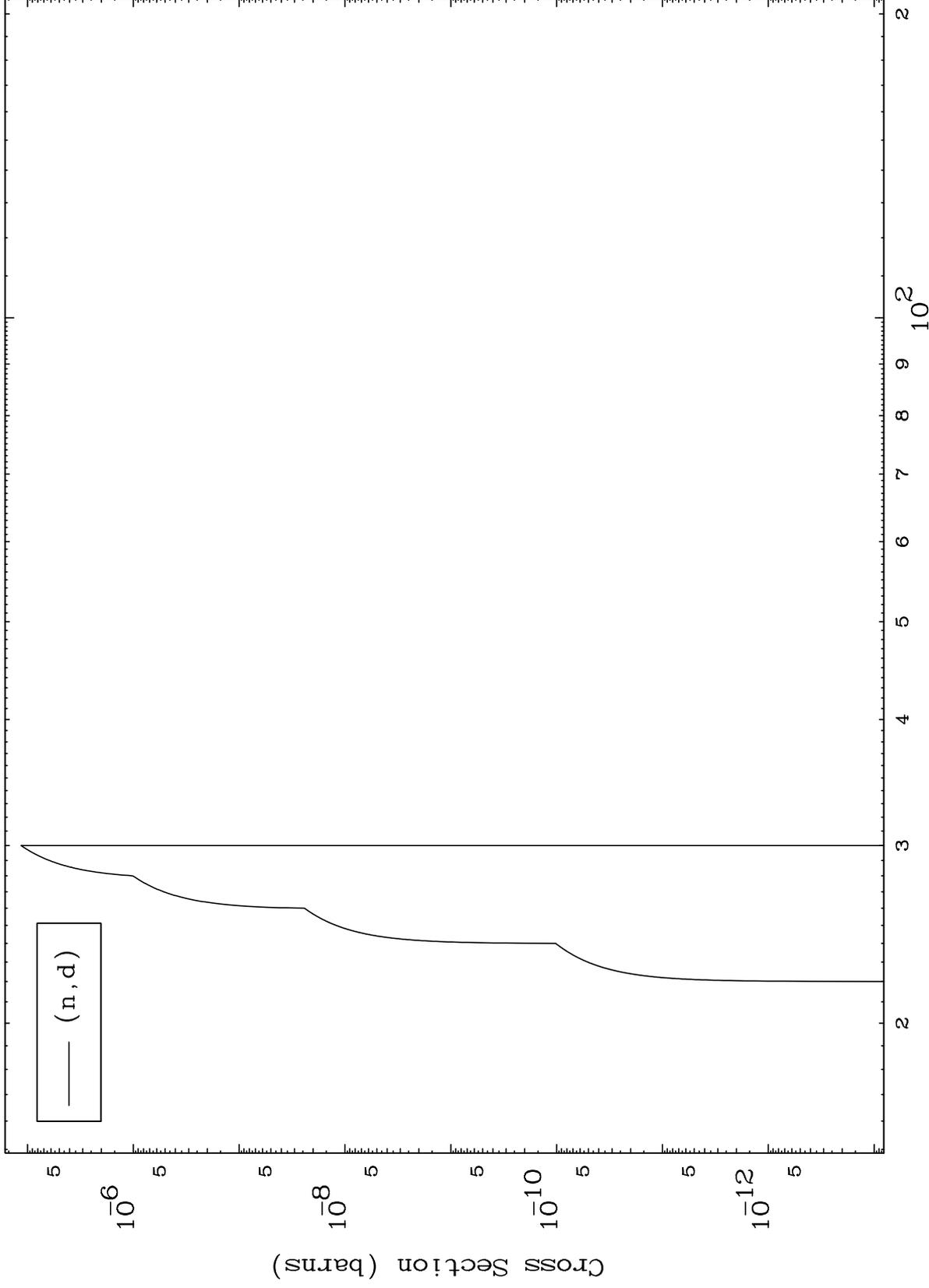
Incident Energy (MeV)

101-Md-250

MAT 9951

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

101-Md-250



5

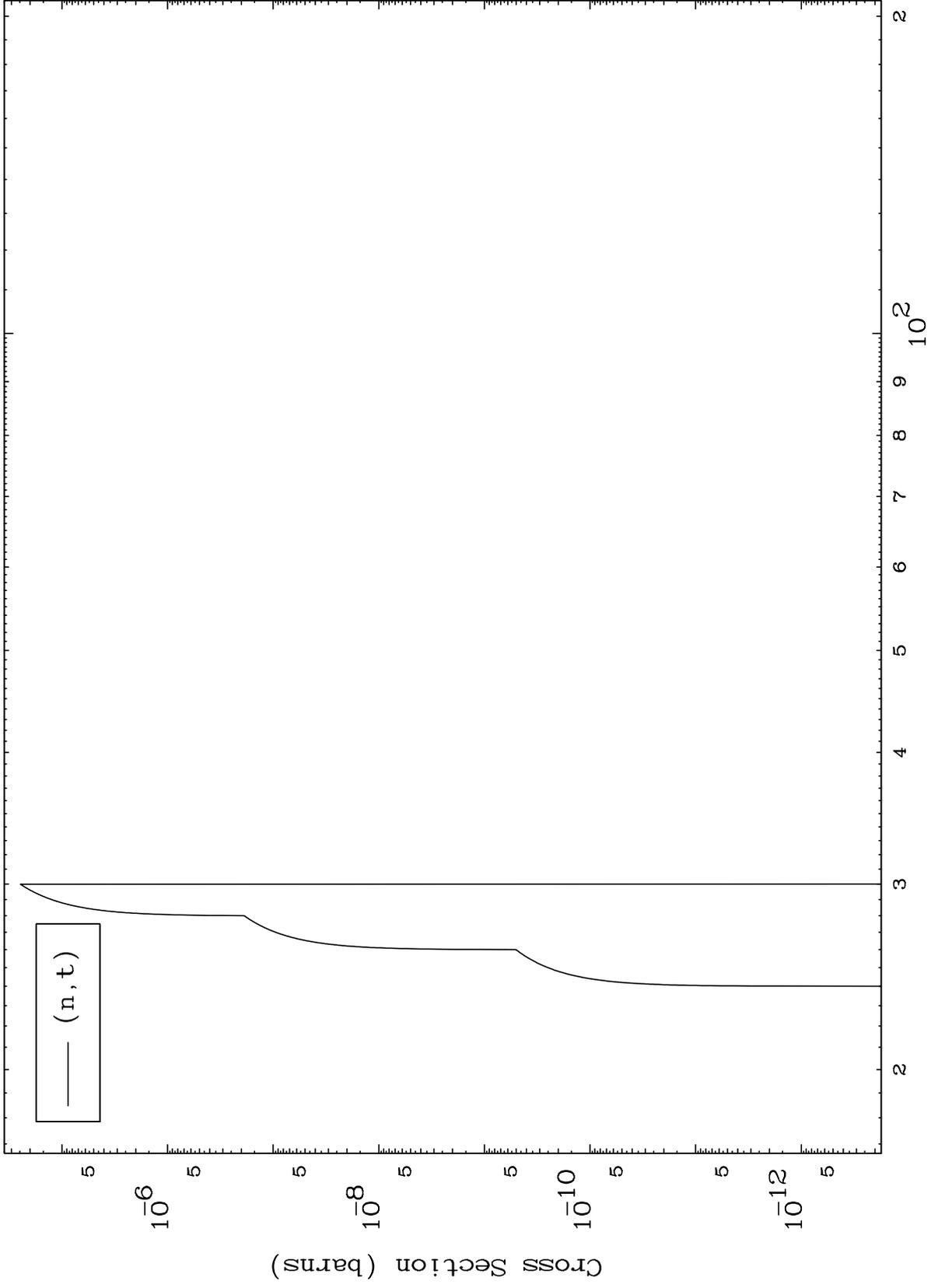
Incident Energy (MeV)

101-Md-250

MAT 9951

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

101-Md-250



6

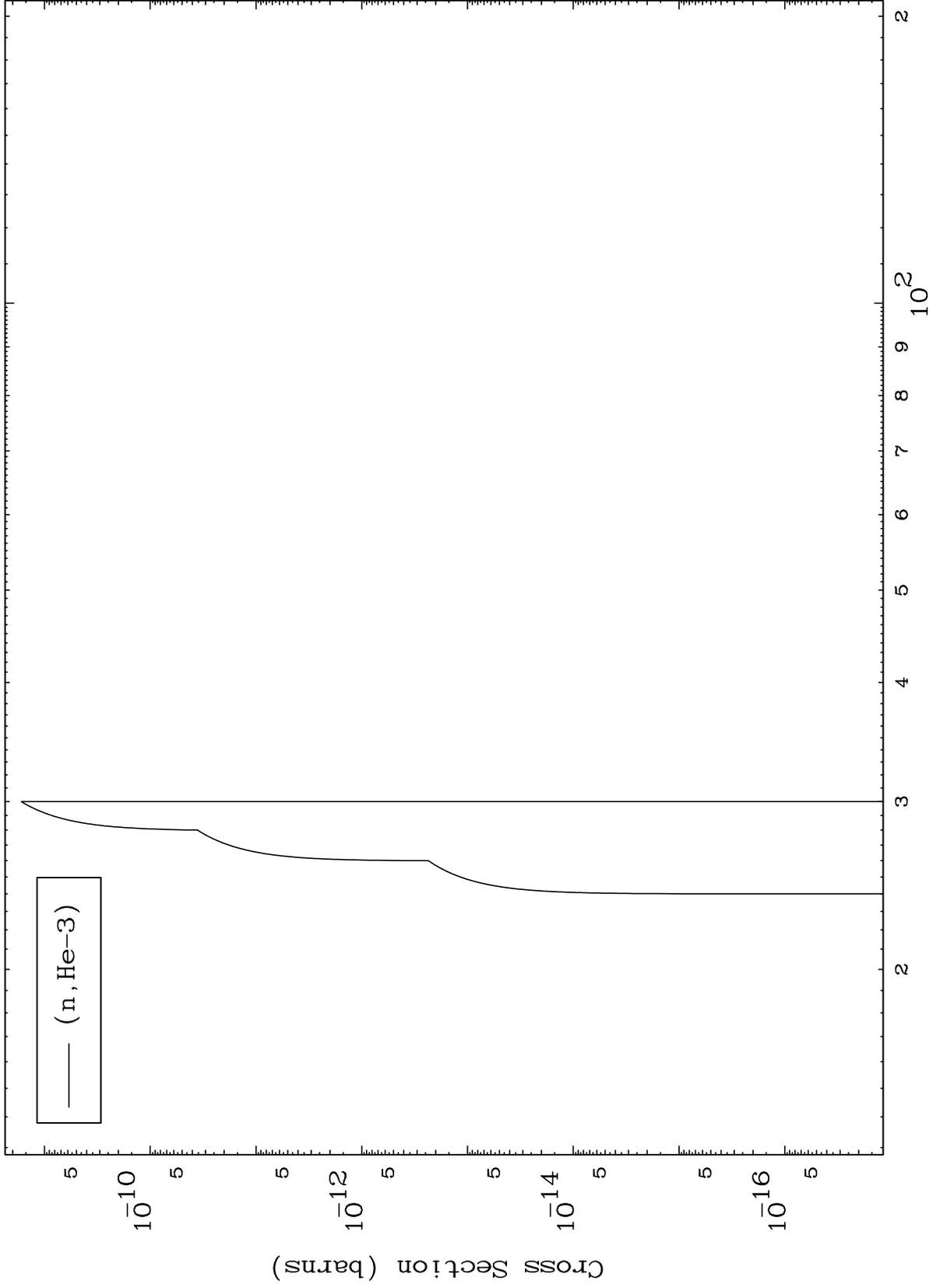
Incident Energy (MeV)

101-Md-250

MAT 9951

( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections

101-Md-250

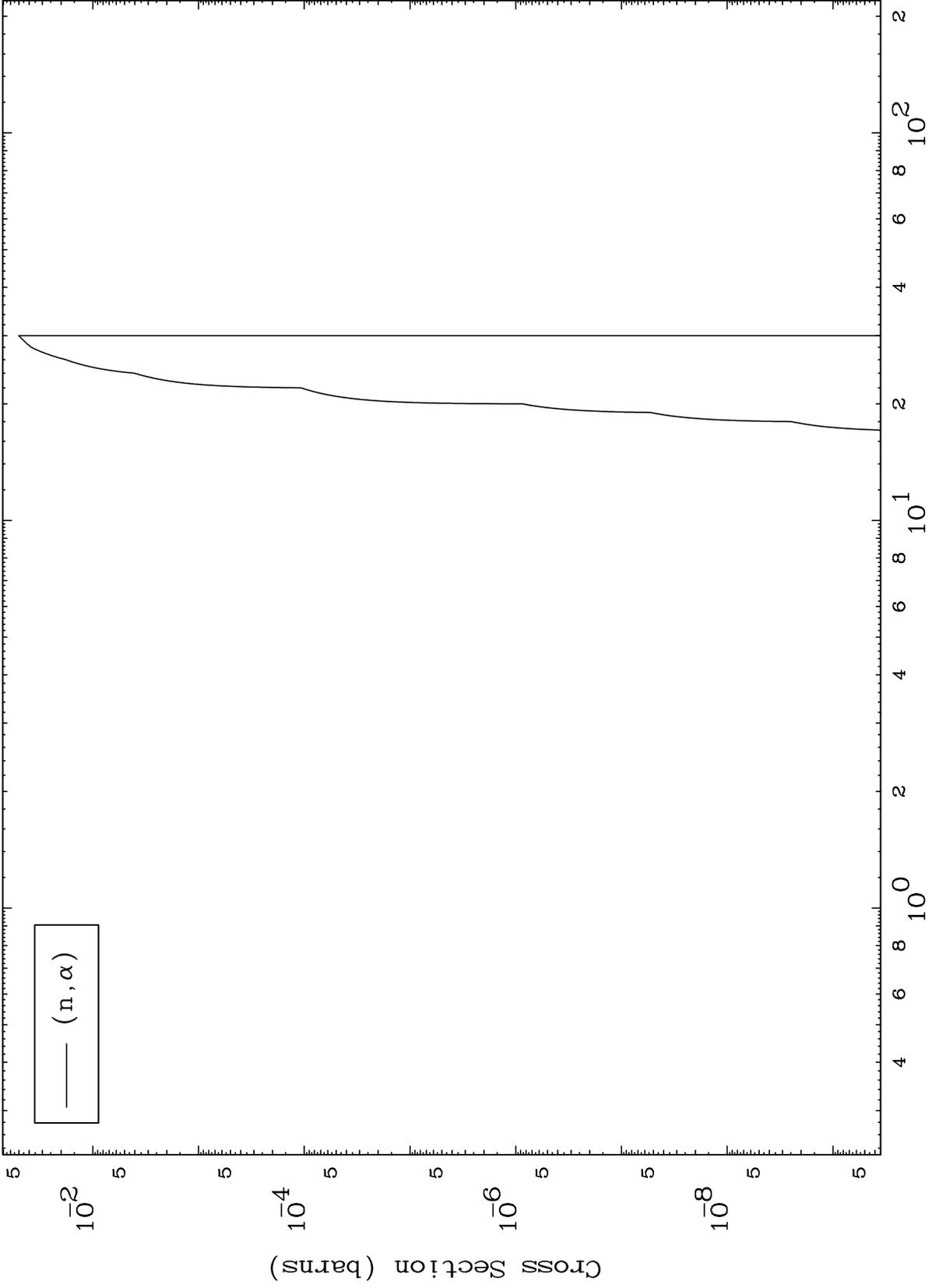


MAT 9951

( $\alpha, \alpha$ ) Levels

101-Md-250

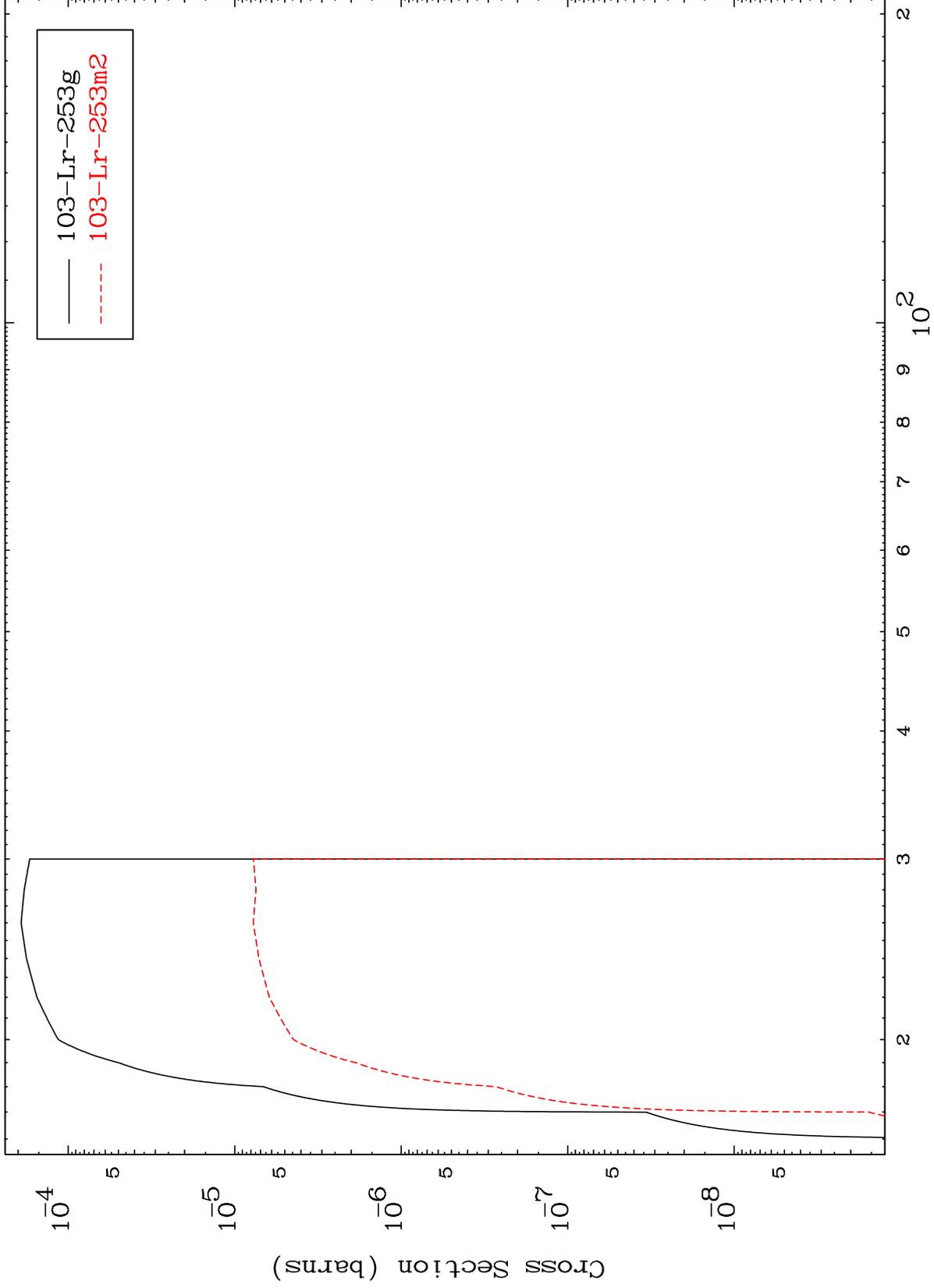
0 Kelvin Cross Sections



MAT 9951

101-Md-250

Inelastic  
Radionuclide Production Cross Section



9

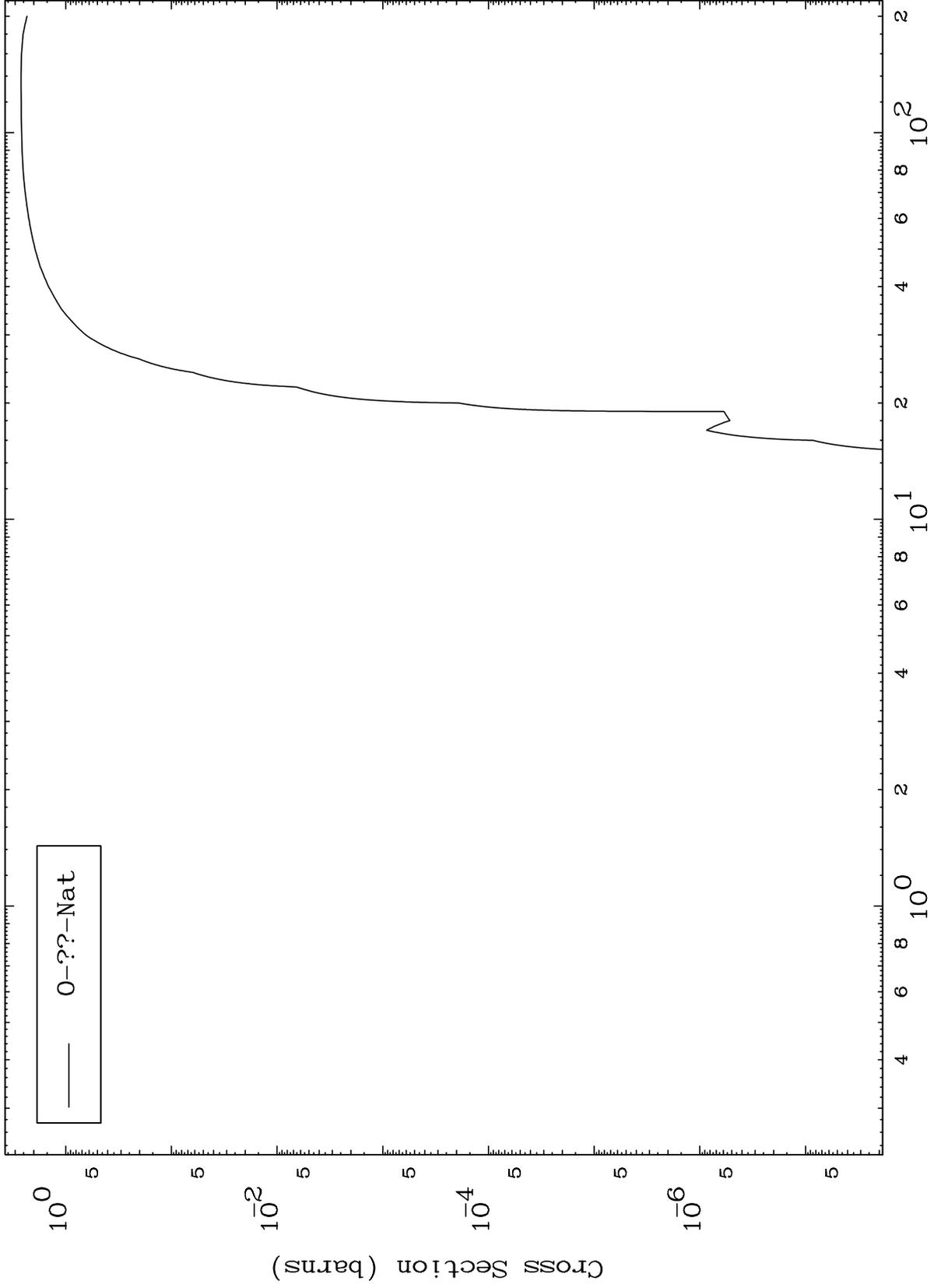
Incident Energy (MeV)

101-Md-250

MAT 9951

101-Md-250

Fission  
Radionuclide Production Cross Section



10

Incident Energy (MeV)

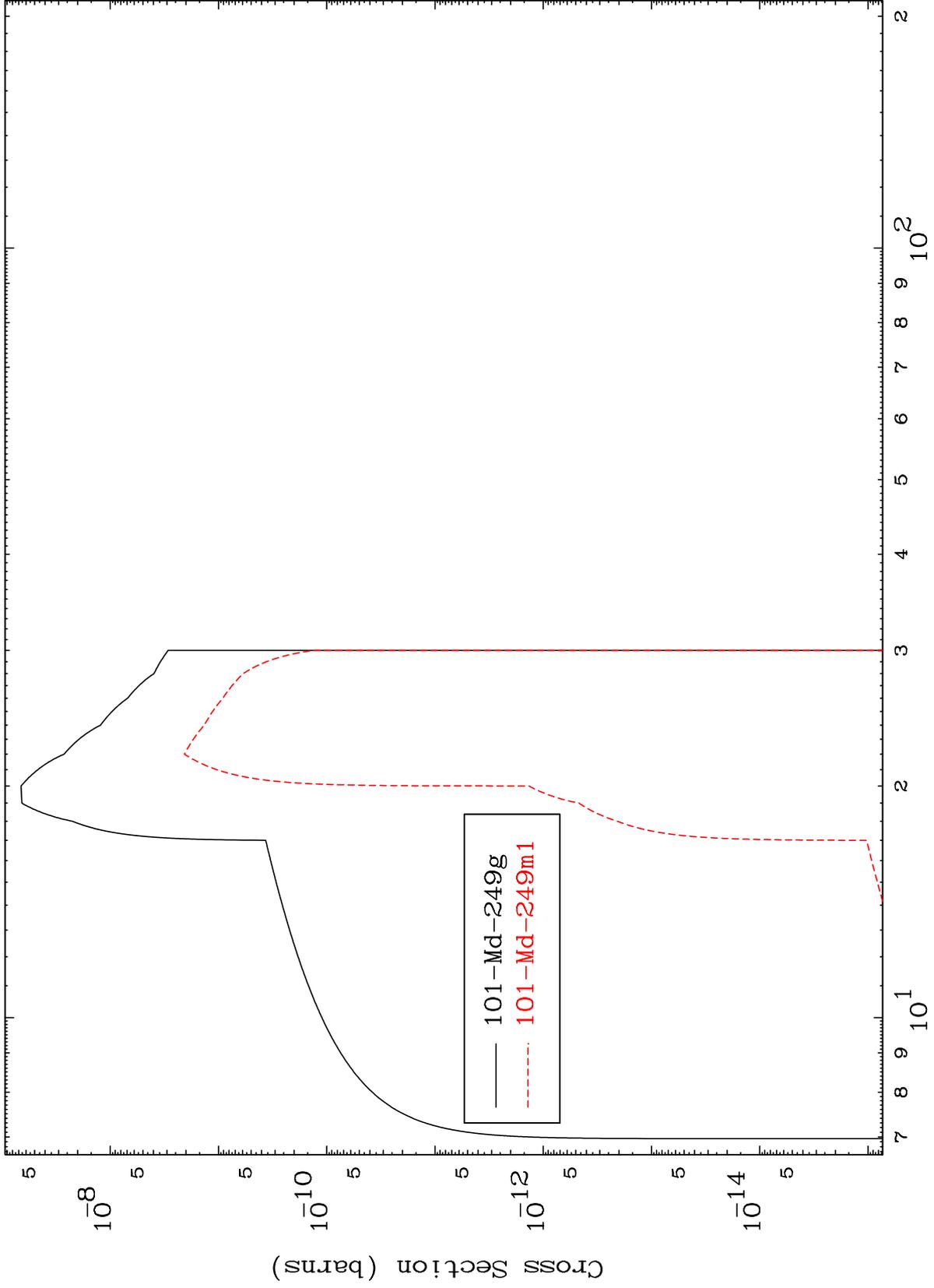
101-Md-250

MAT 9951

$(n, n') \alpha$

101-Md-250

Radionuclide Production Cross Section

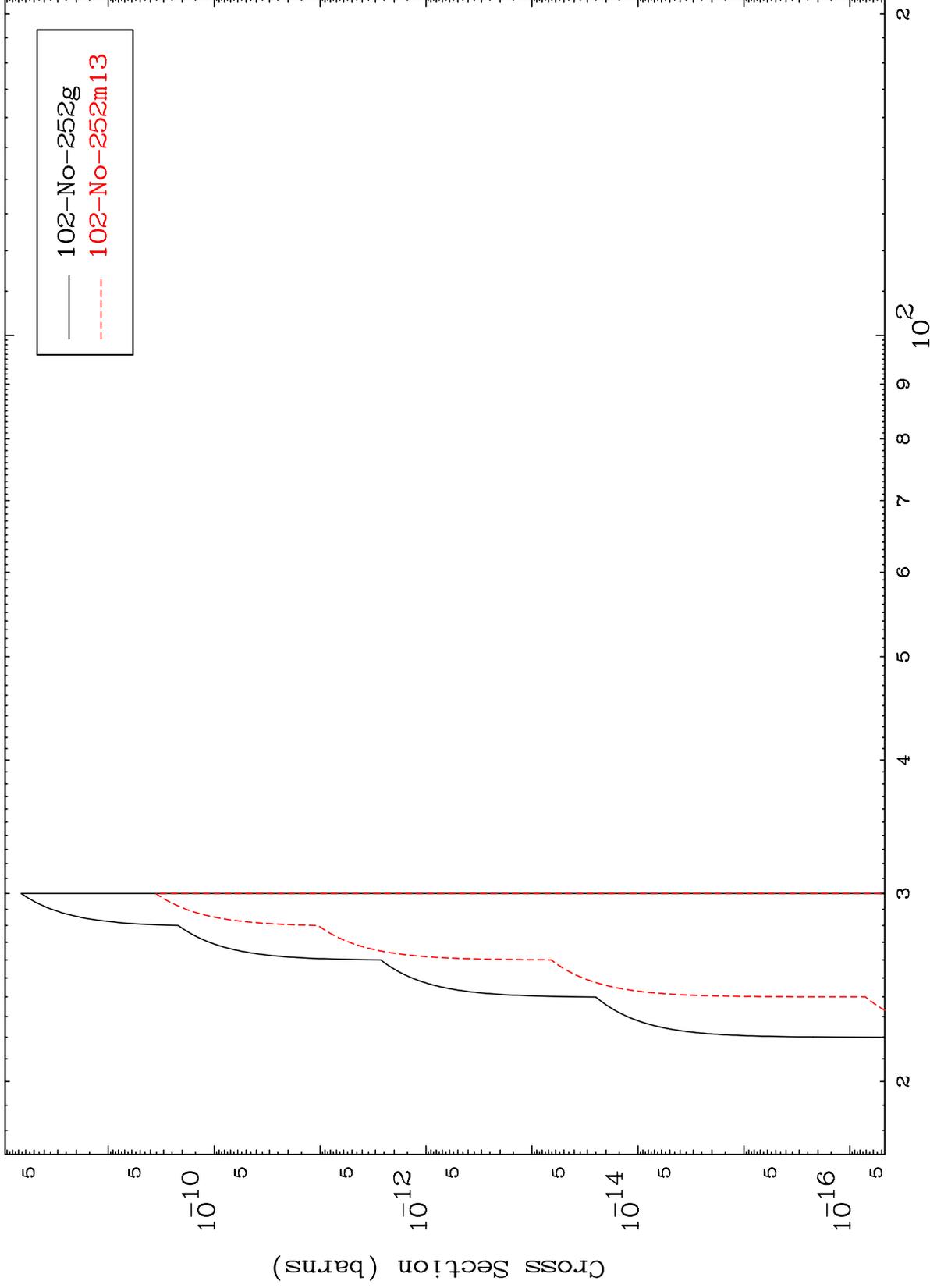


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(n,n') p

101-Md-250

Radionuclide Production Cross Section



12

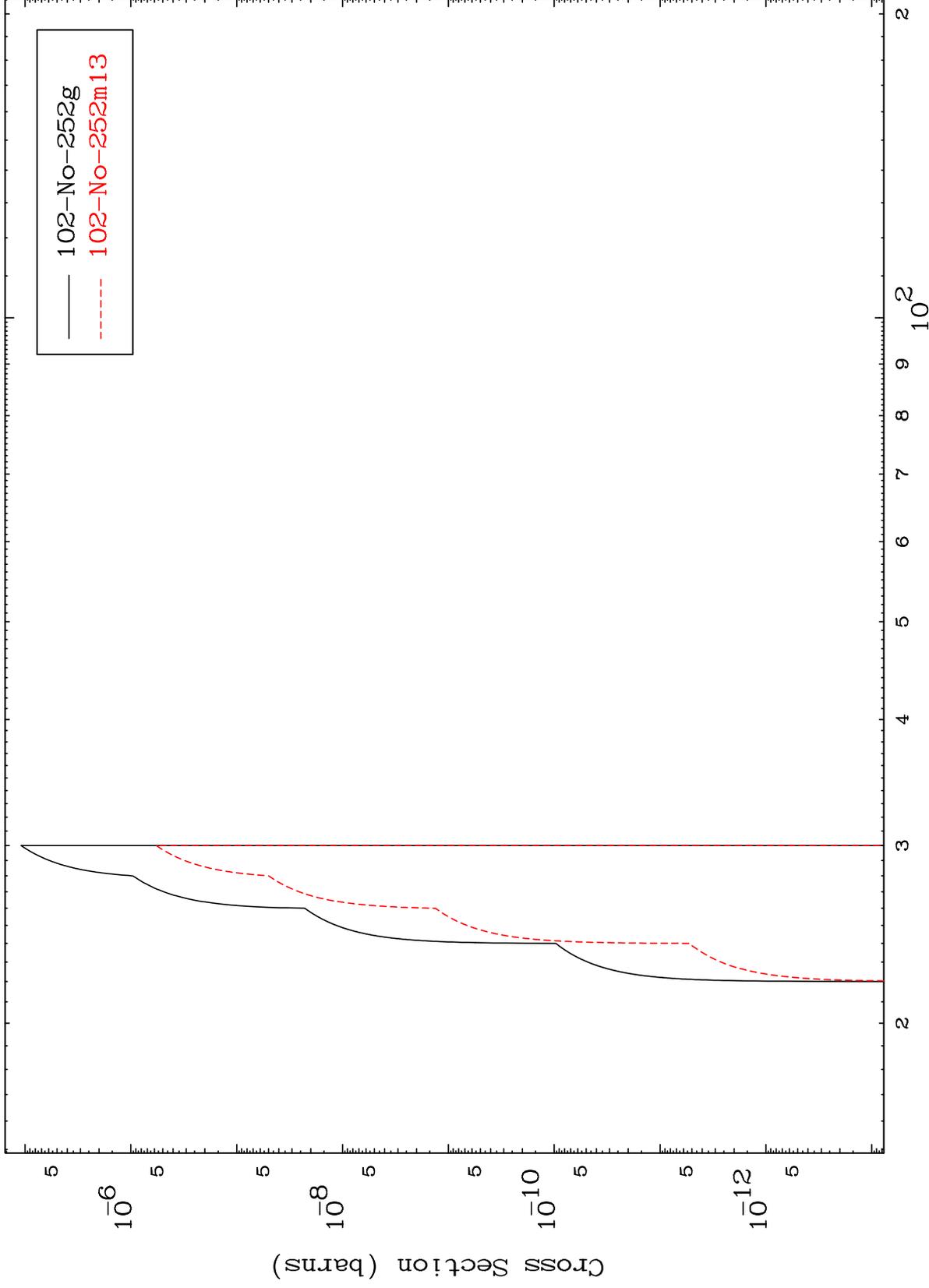
Incident Energy (MeV)

101-Md-250

MAT 9951

101-Md-250

(n,d)  
Radionuclide Production Cross Section



13

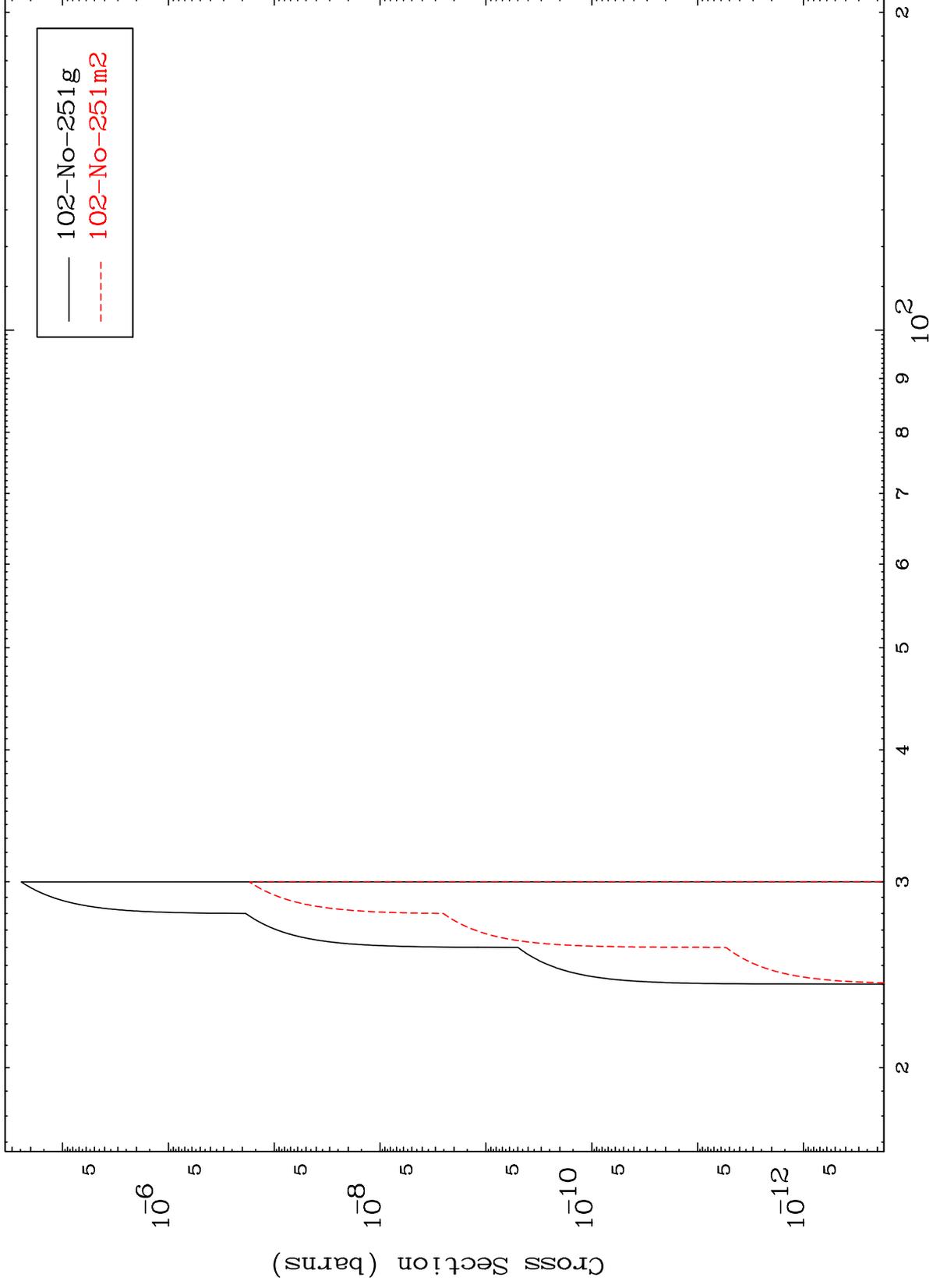
101-Md-250

Incident Energy (MeV)

MAT 9951

101-Md-250

(n, t)  
Radionuclide Production Cross Section



14

Incident Energy (MeV)

101-Md-250