

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

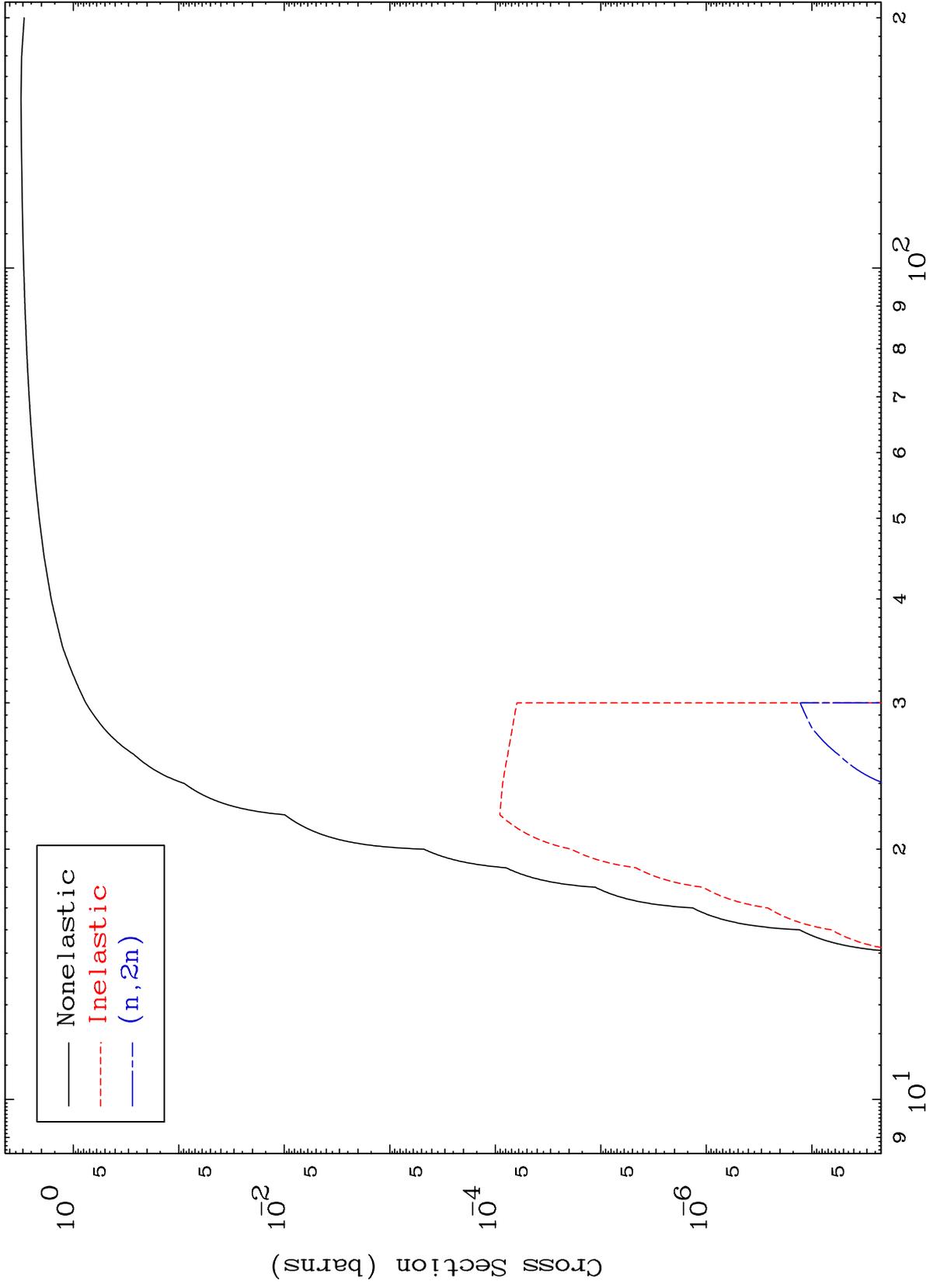
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Press Mouse Button to Start

MAT 9962

0 Kelvin Cross Sections

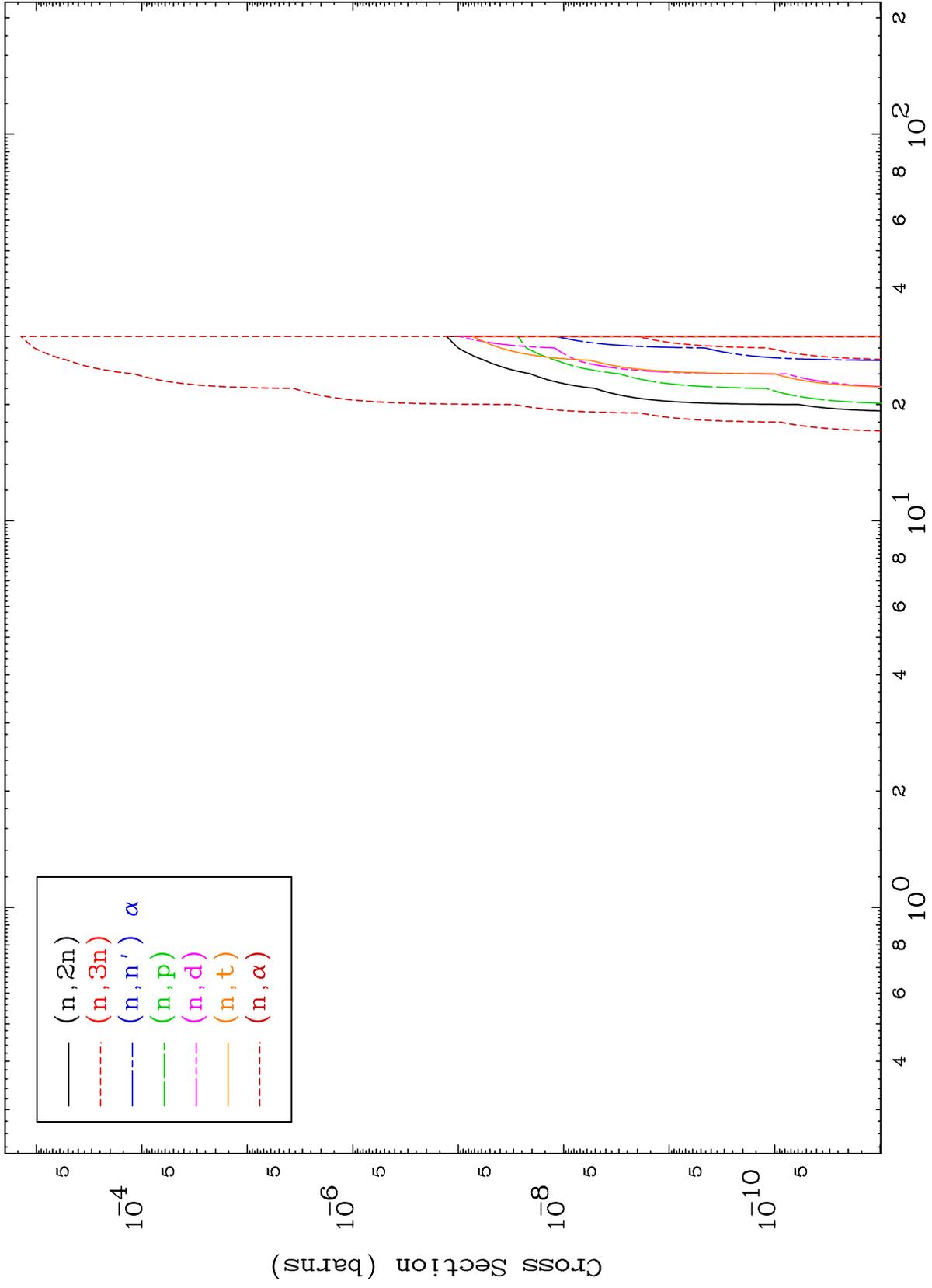
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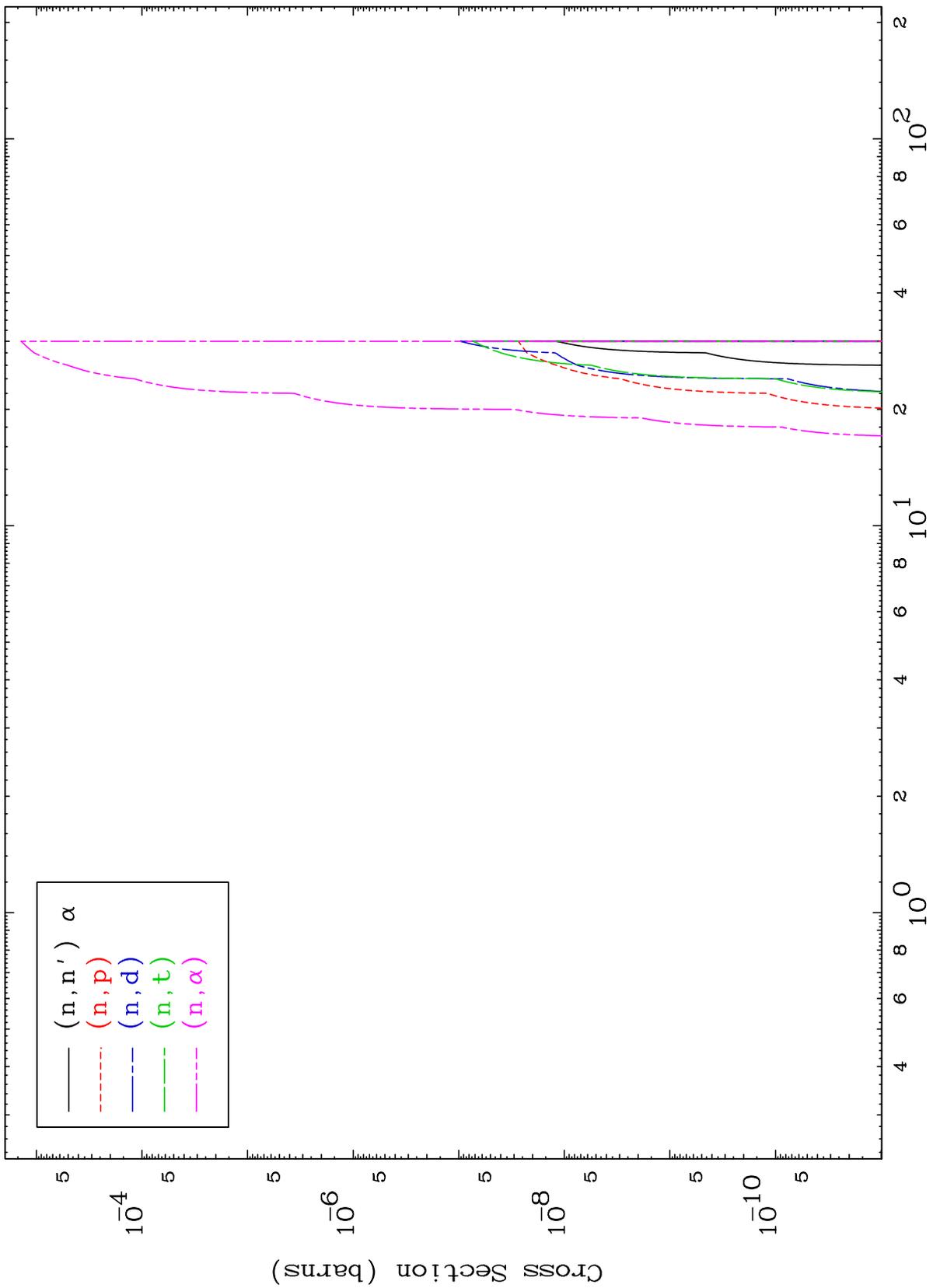


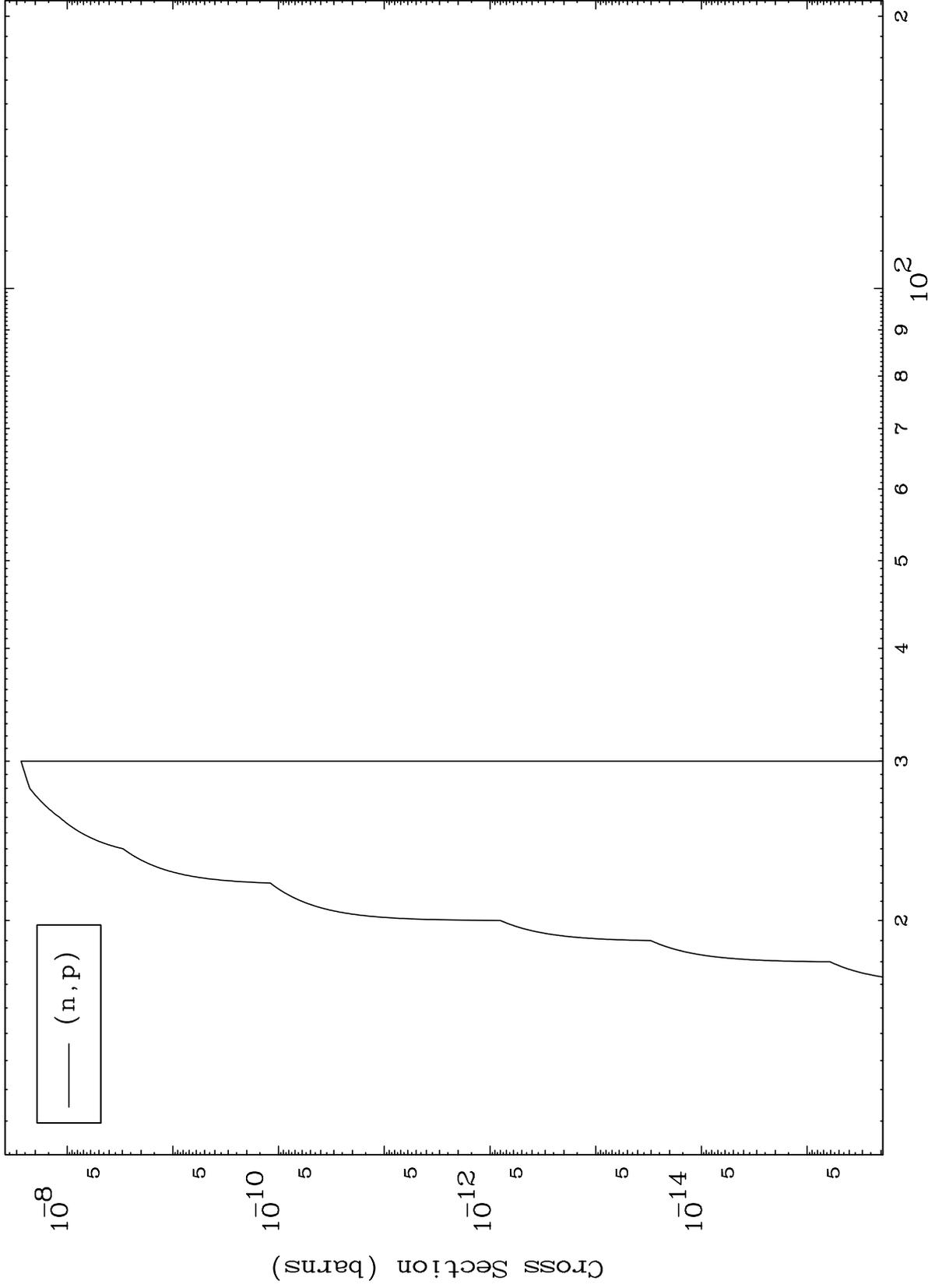
101-Md-261

Incident Energy (MeV)

1



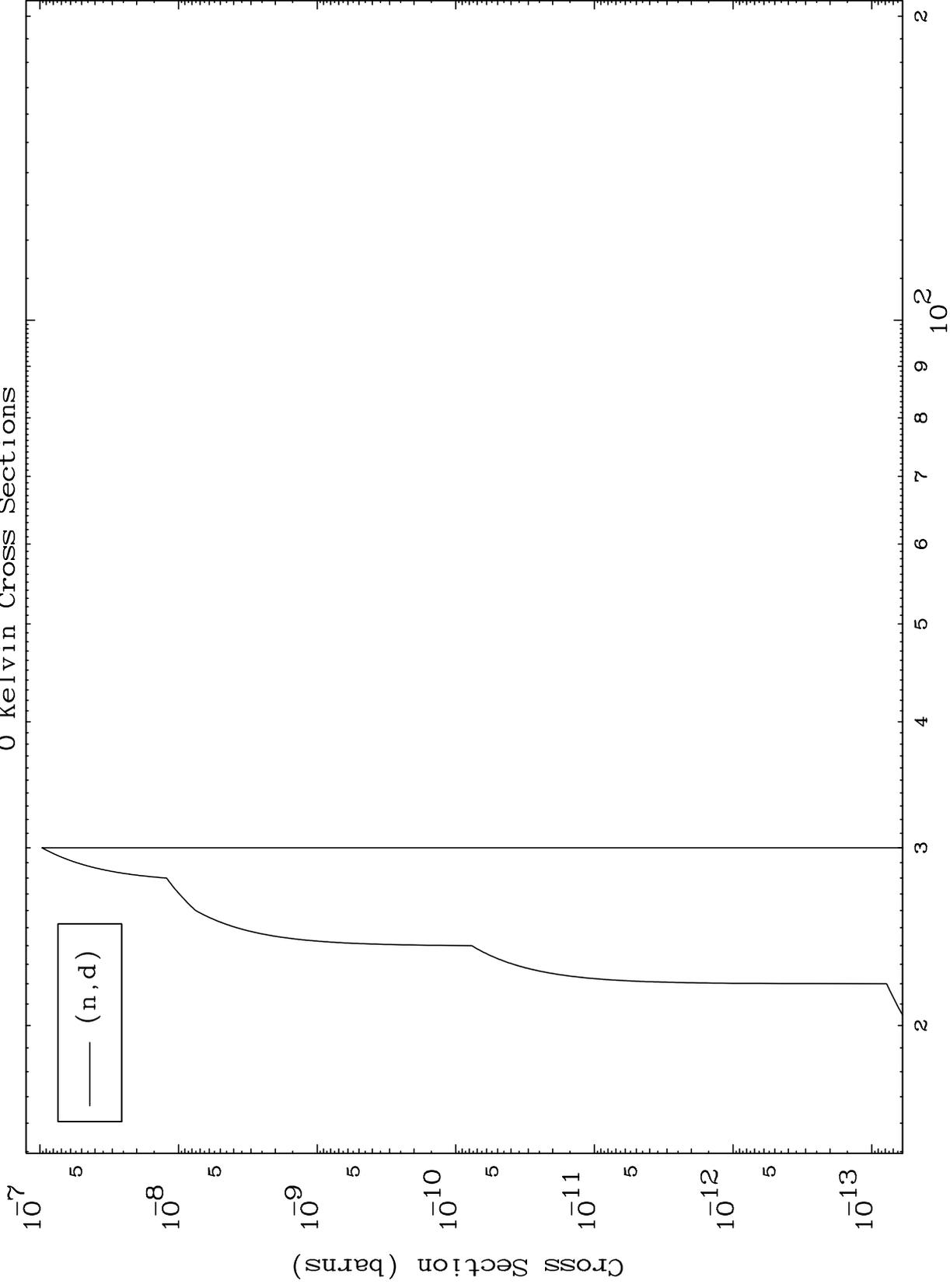




MAT 9962

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

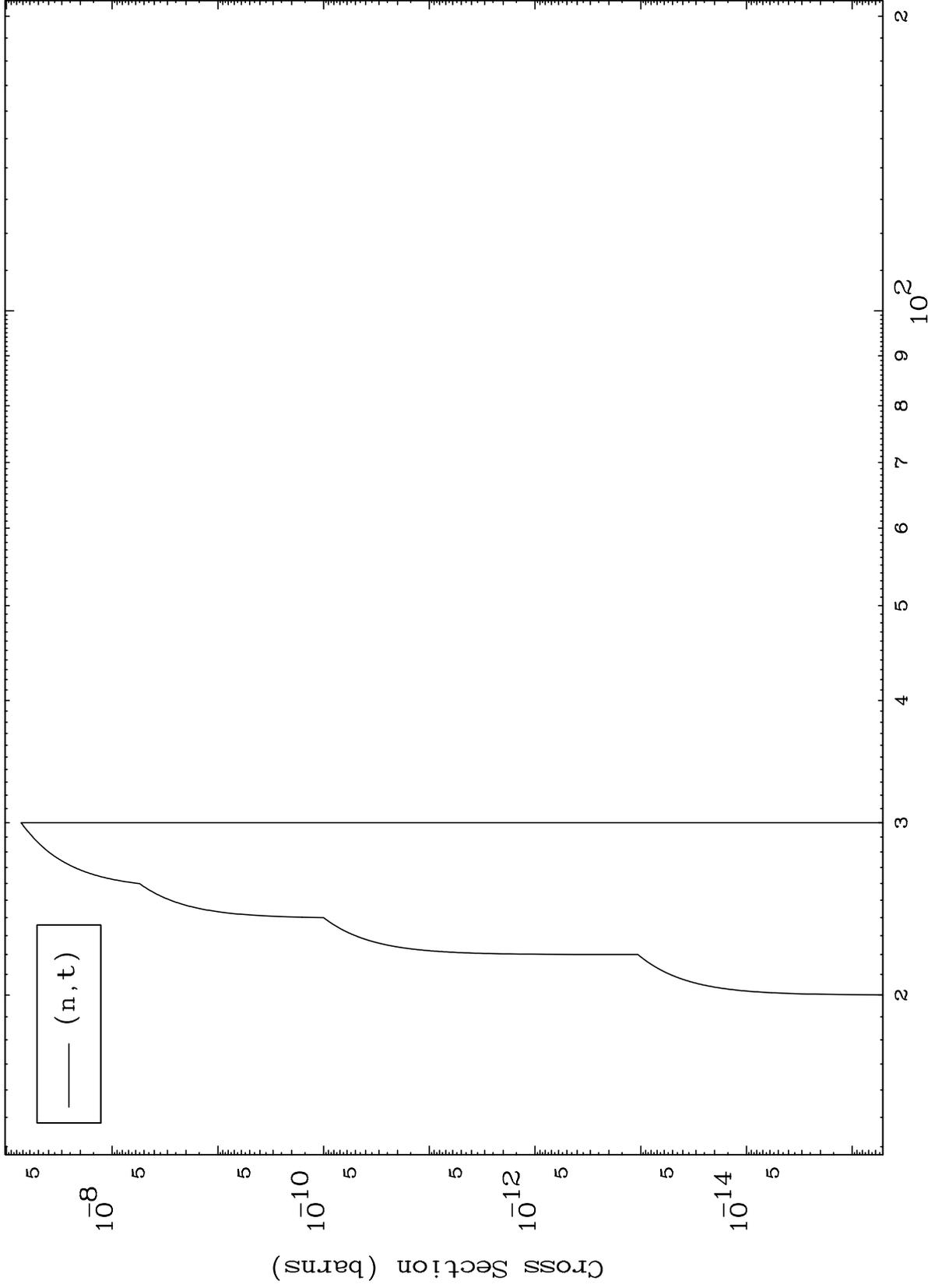
101-Md-261



5

Incident Energy (MeV)

101-Md-261

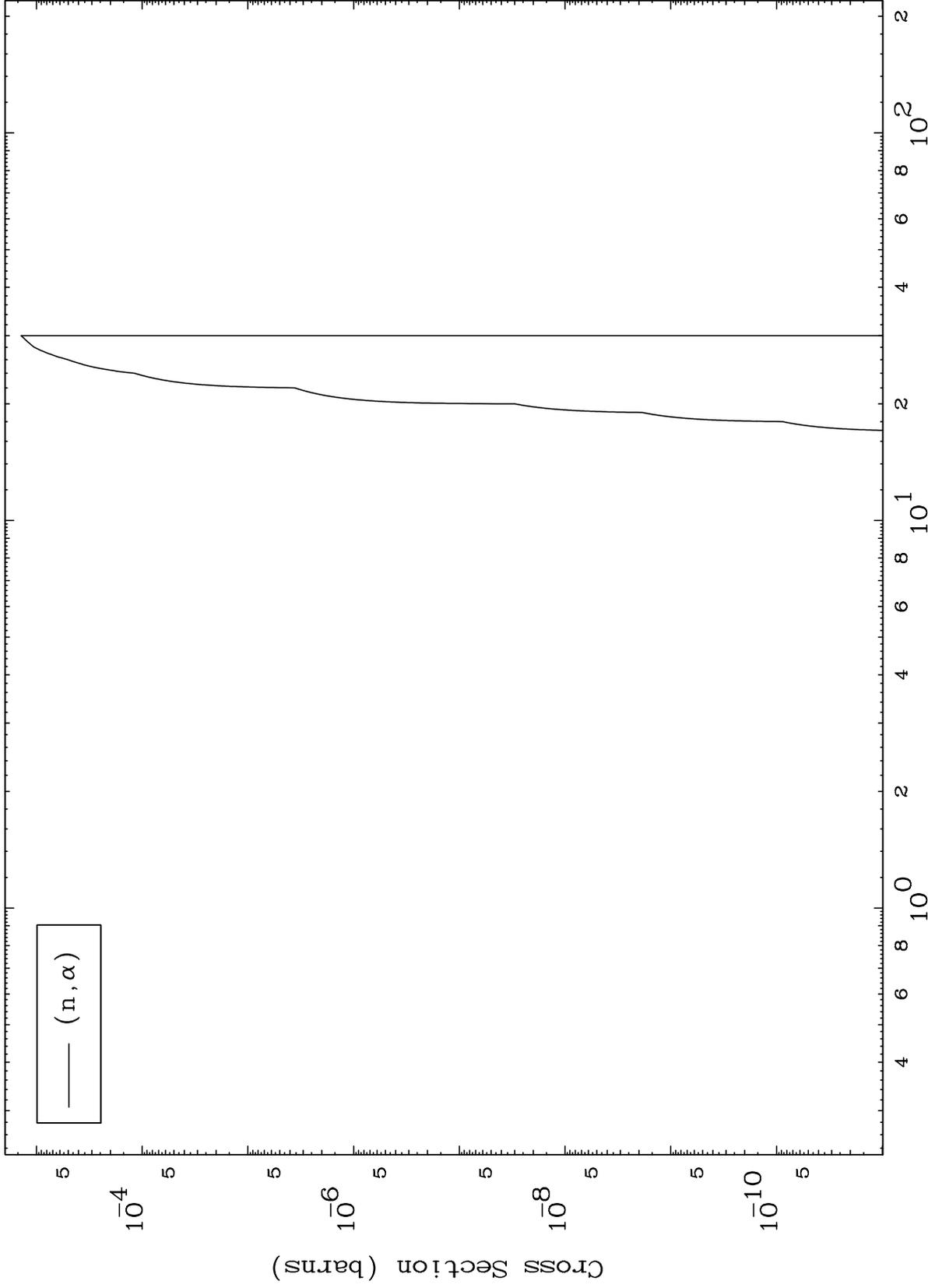


MAT 9962

( $\alpha, \alpha$ ) Levels

101-Md-261

0 Kelvin Cross Sections



7

Incident Energy (MeV)

101-Md-261

MAT 9962

101-Md-261

Fission  
Radionuclide Production Cross Section

