

Program EVALPLOT  
(Version 2021-1)

by

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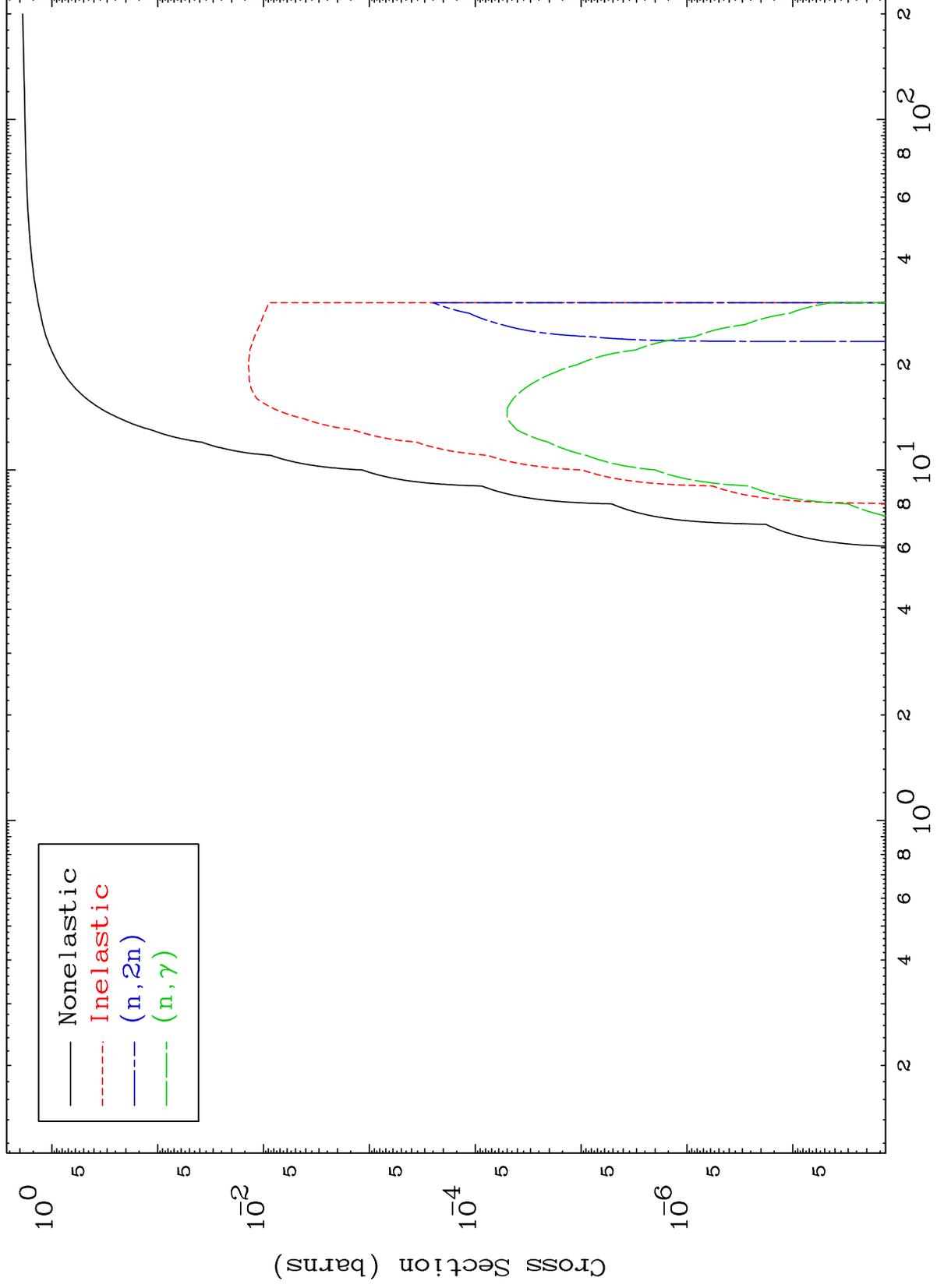
Press Mouse Button to Start

MAT 4210

$\alpha$  Major

42-Mo-87

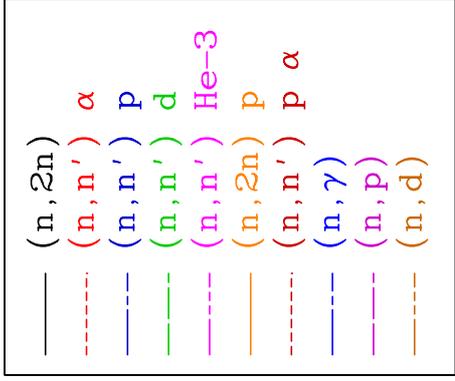
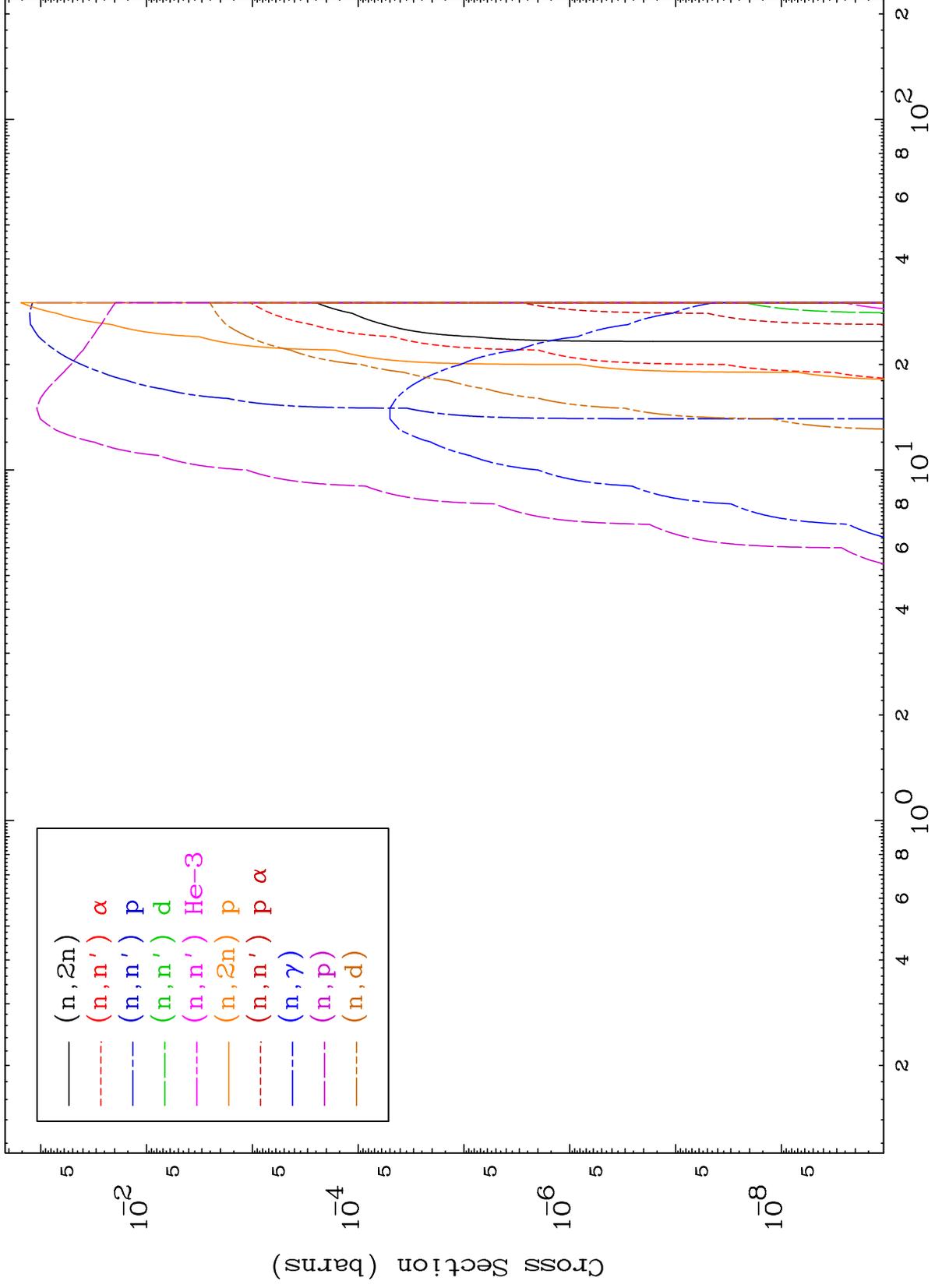
0 Kelvin Cross Sections



MAT 4210

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

42-Mo-87



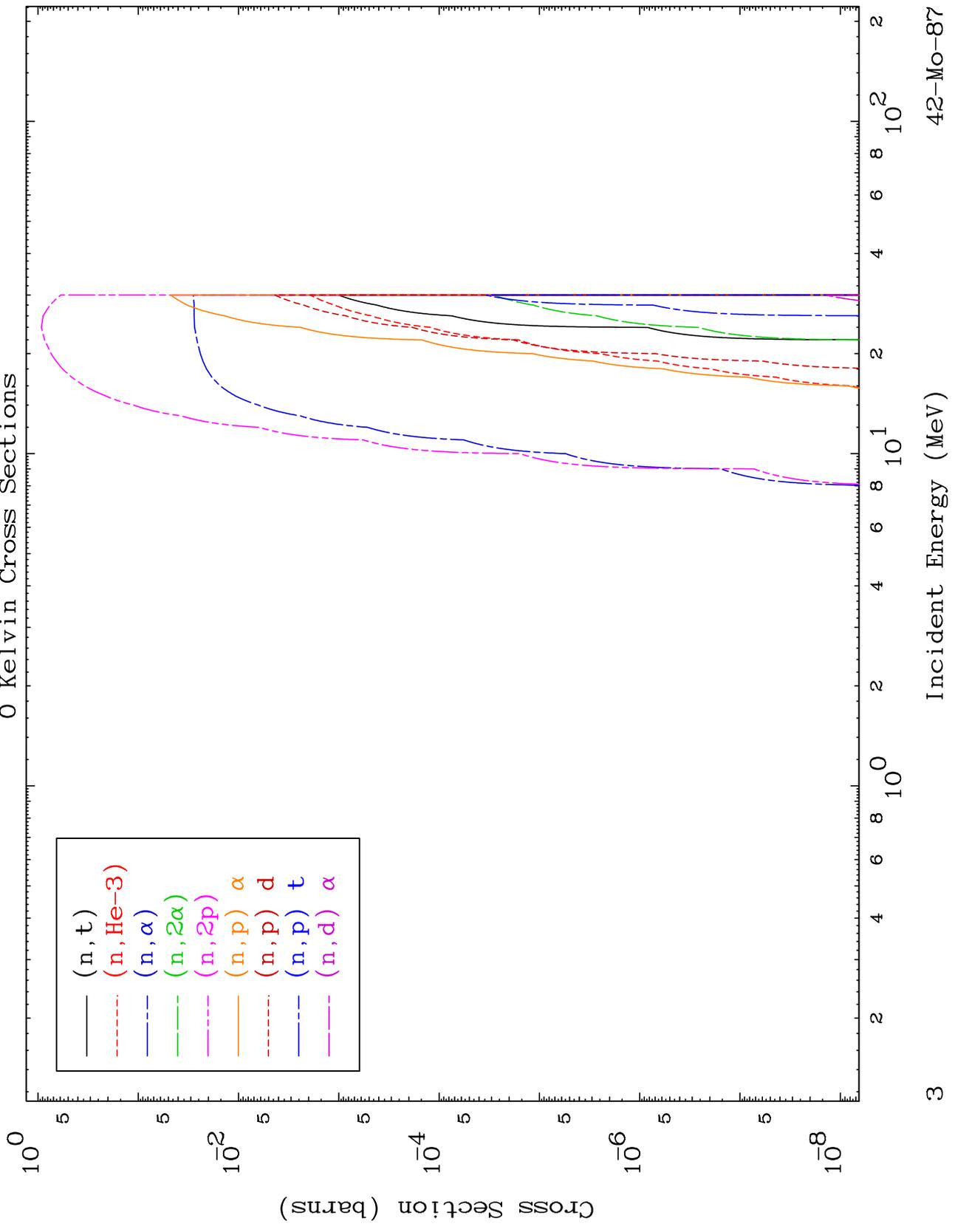
42-Mo-87

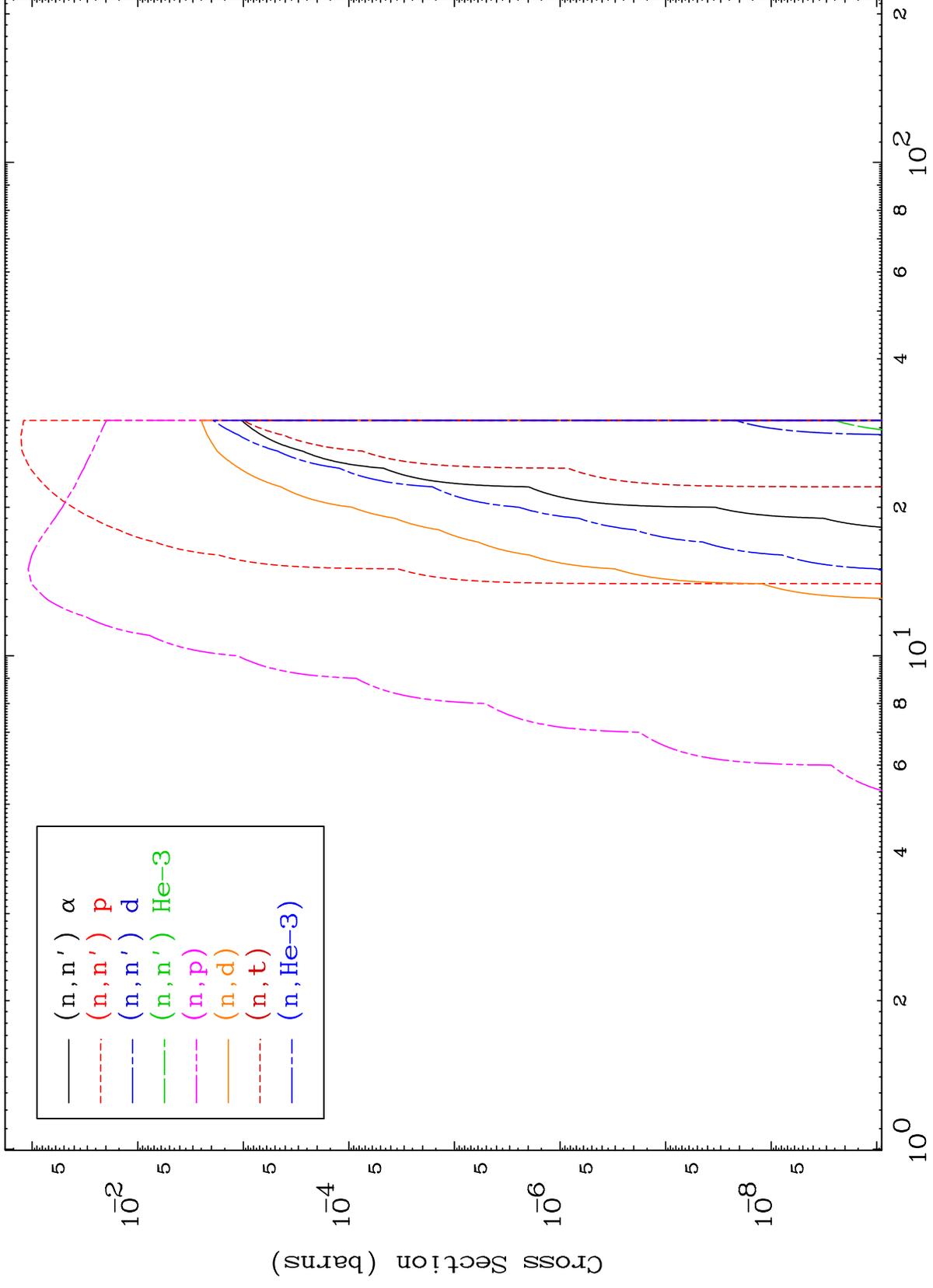
Incident Energy (MeV)

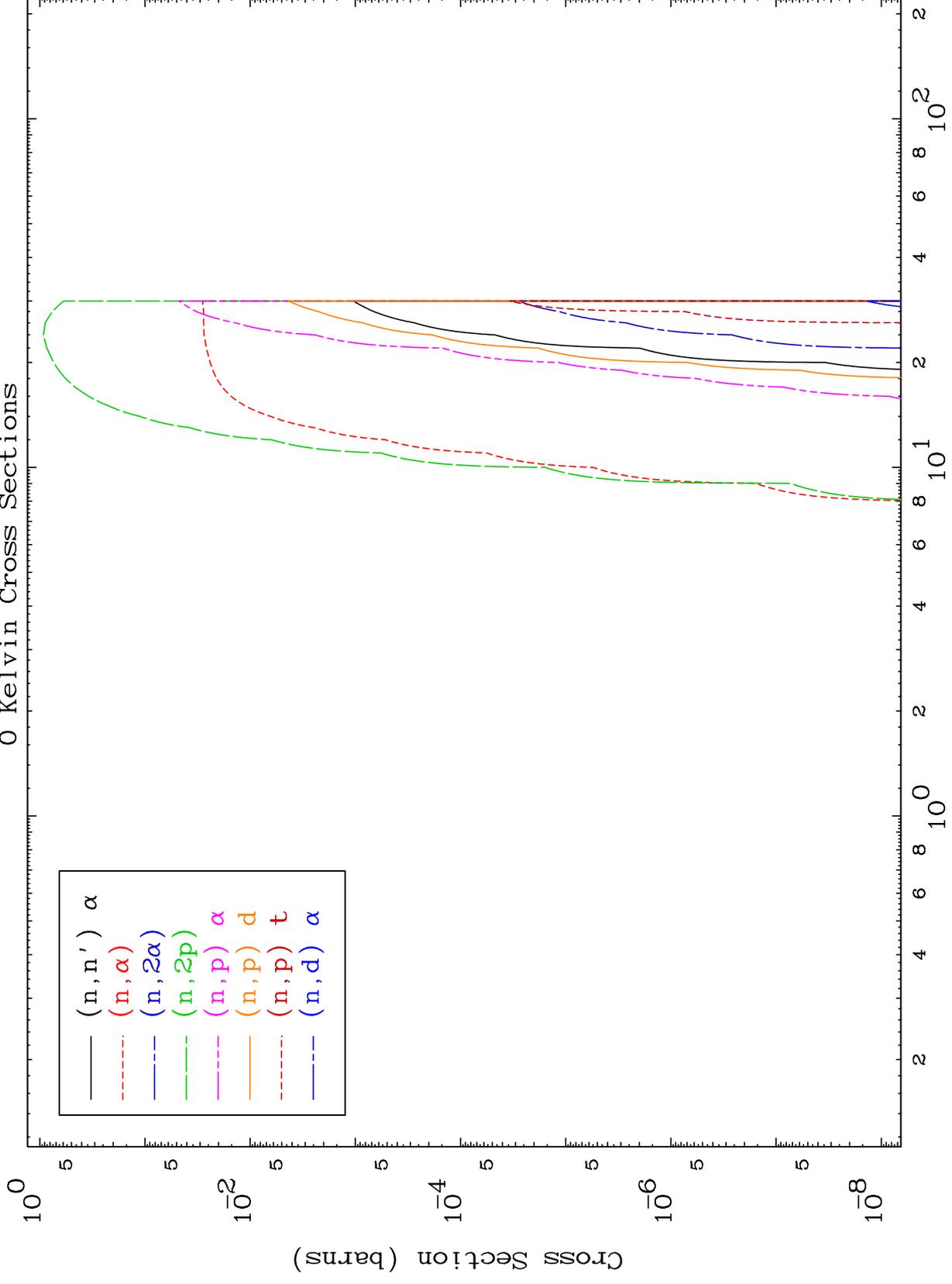
MAT 4210

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

42-Mo-87





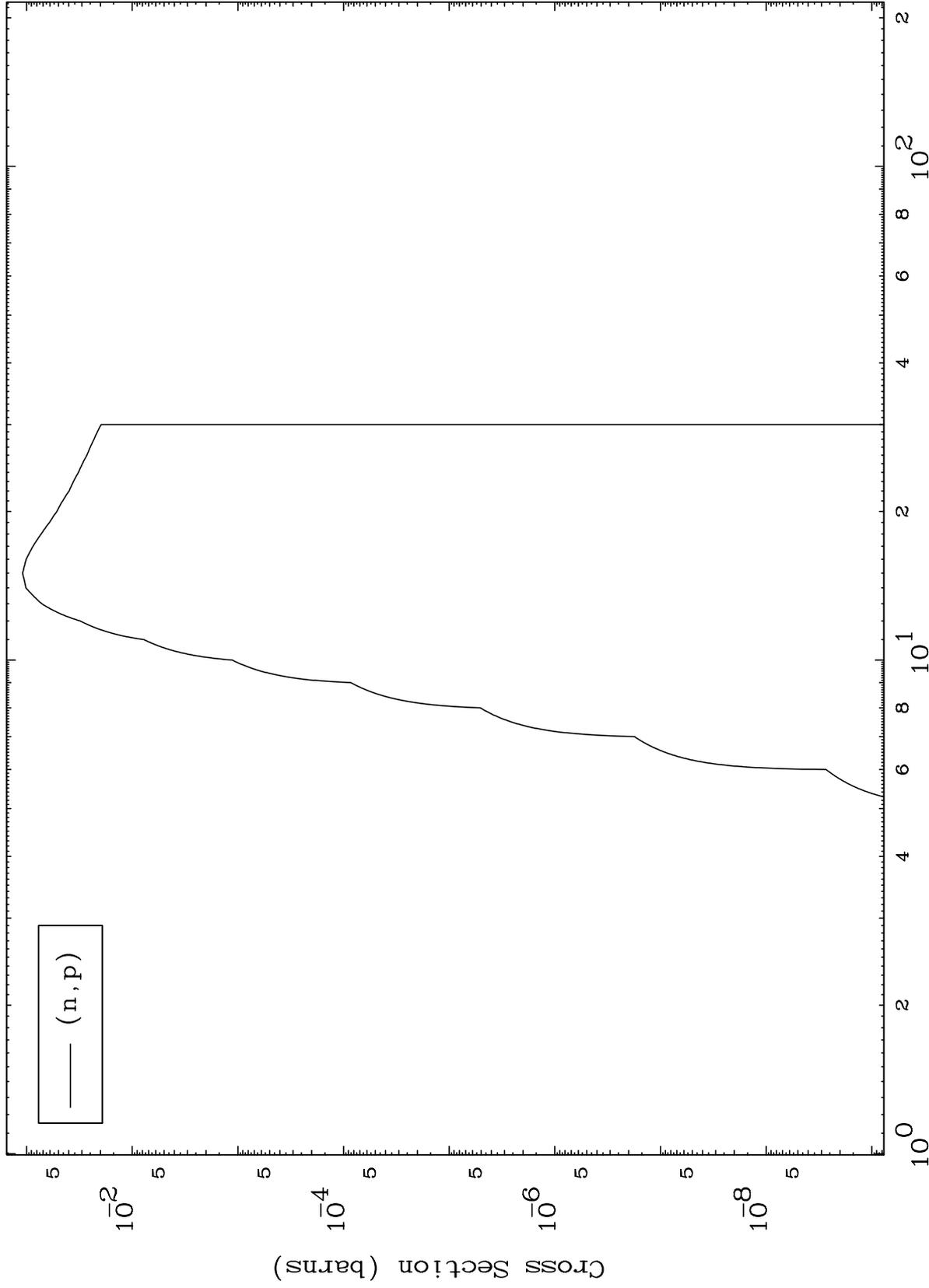


MAT 4210

( $\alpha, p$ ) Levels

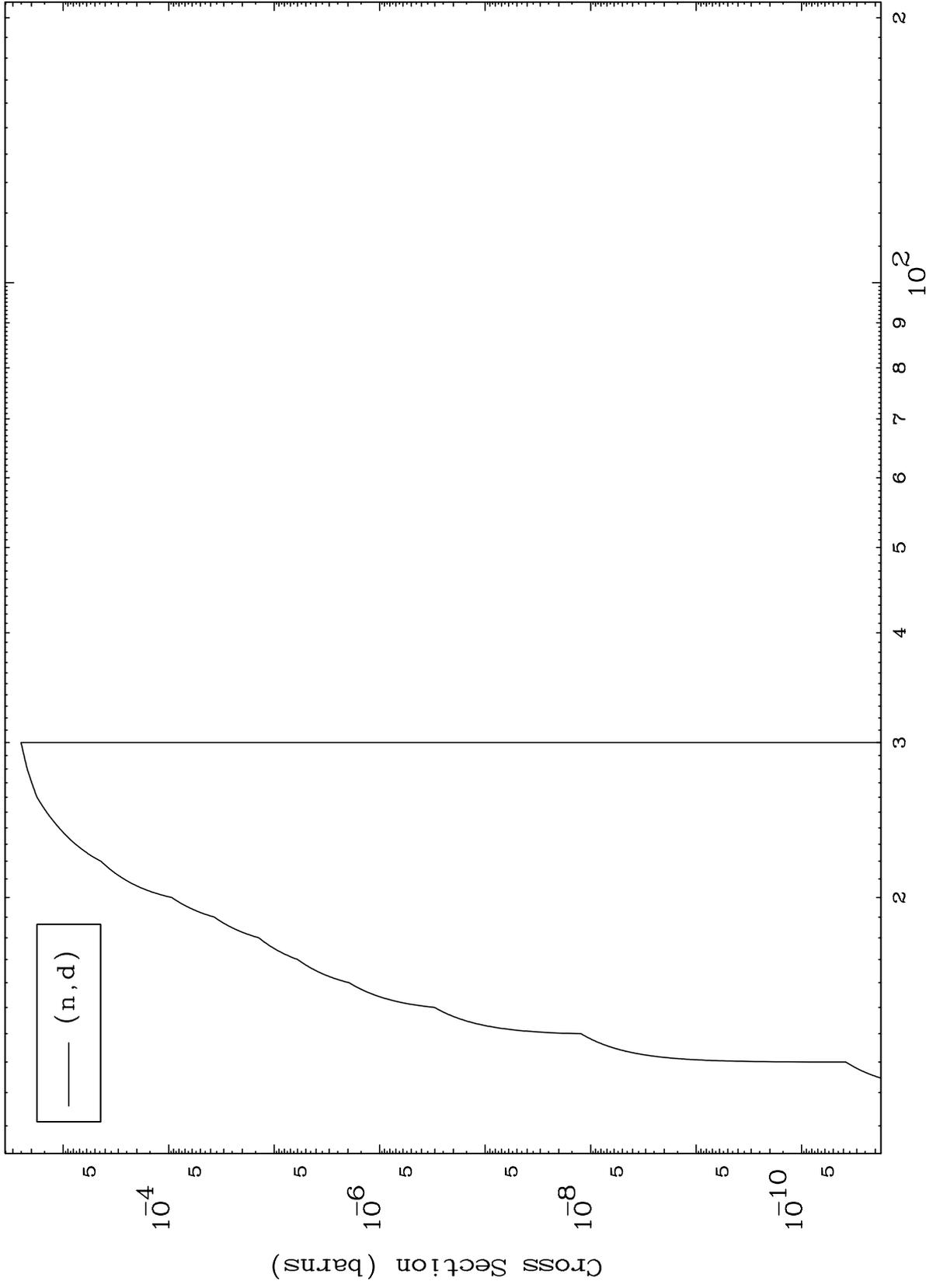
42-Mo-87

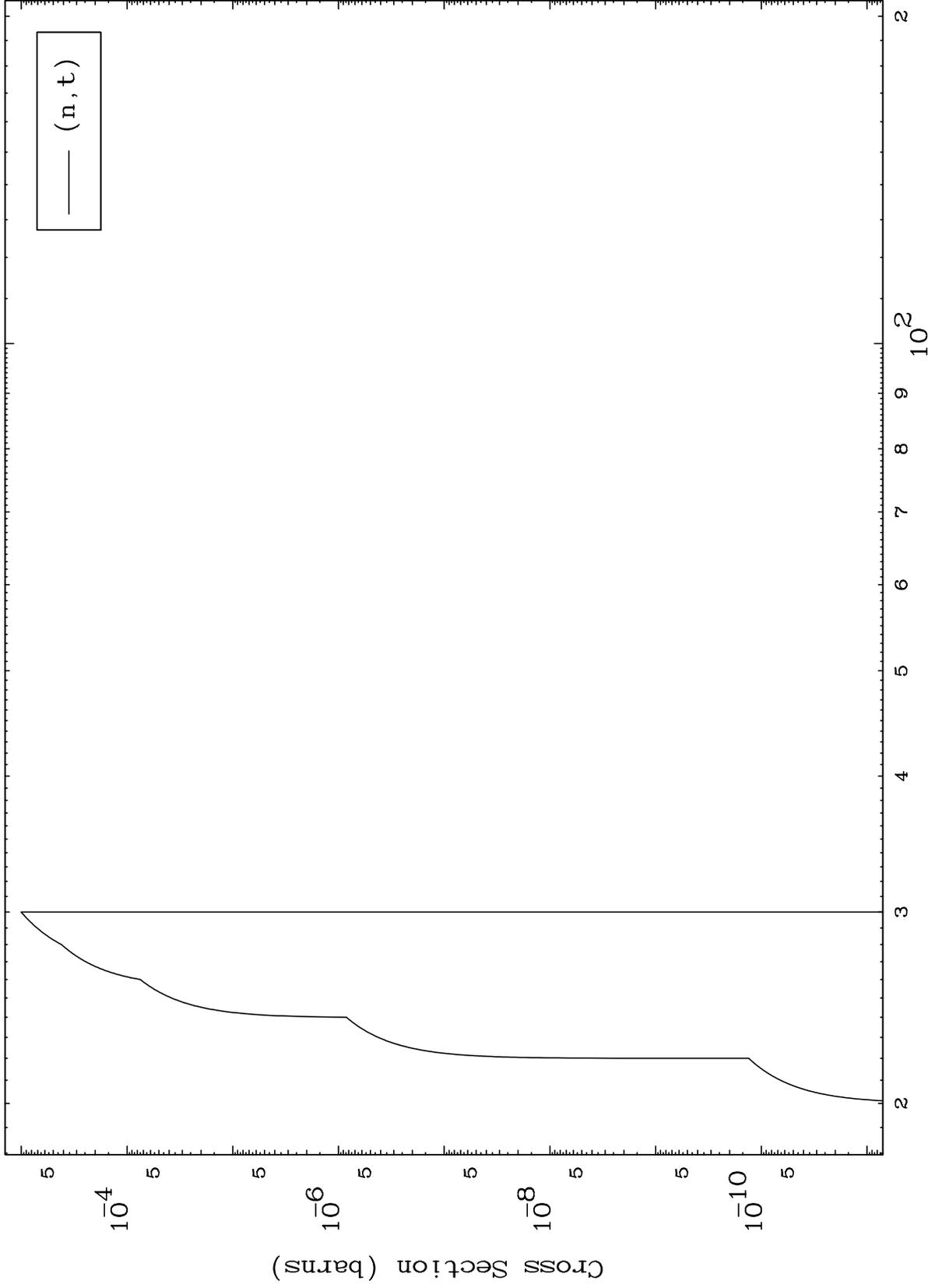
0 Kelvin Cross Sections



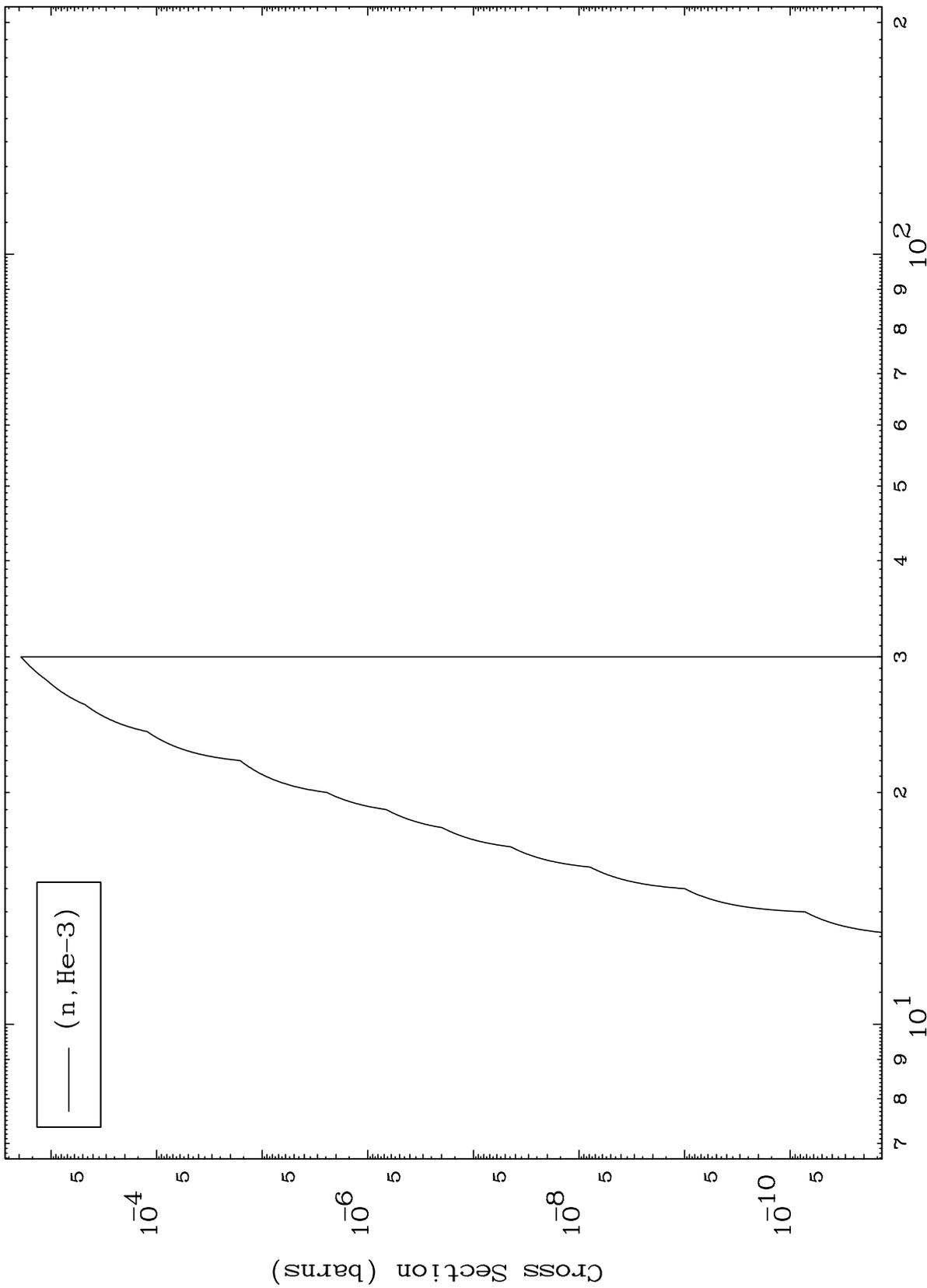
Incident Energy (MeV)

42-Mo-87





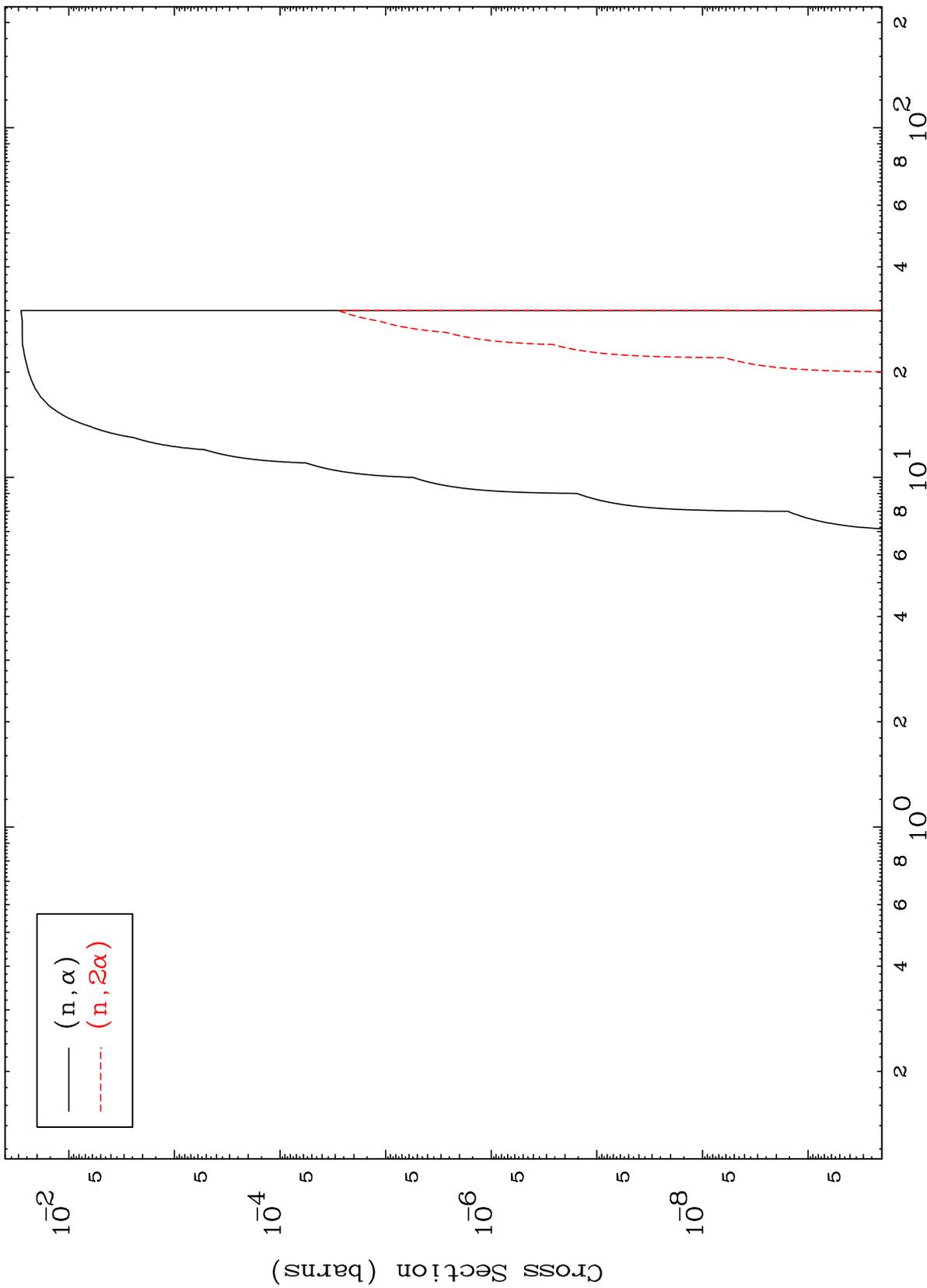
( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections



MAT 4210

42-Mo-87

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

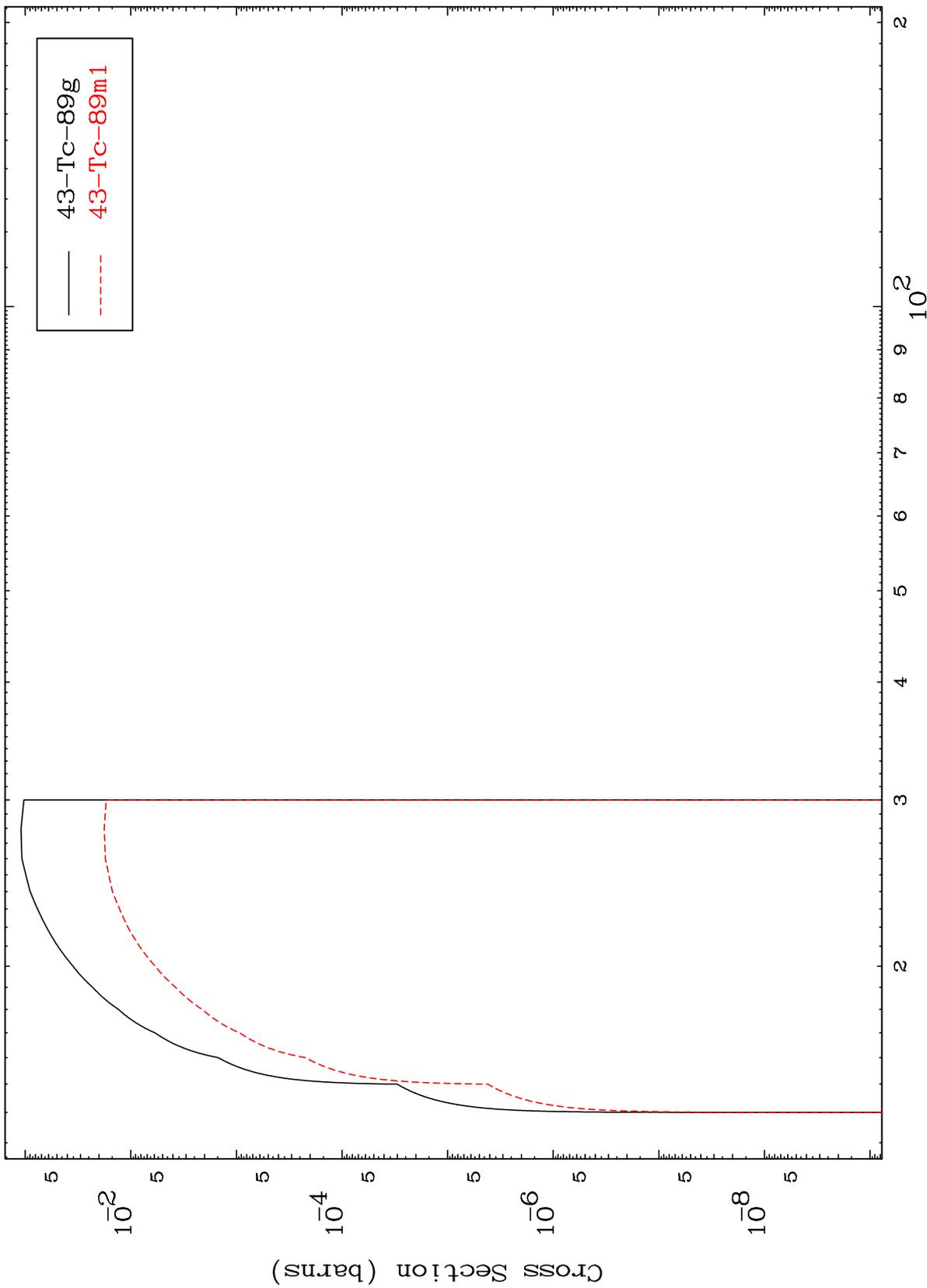


42-Mo-87

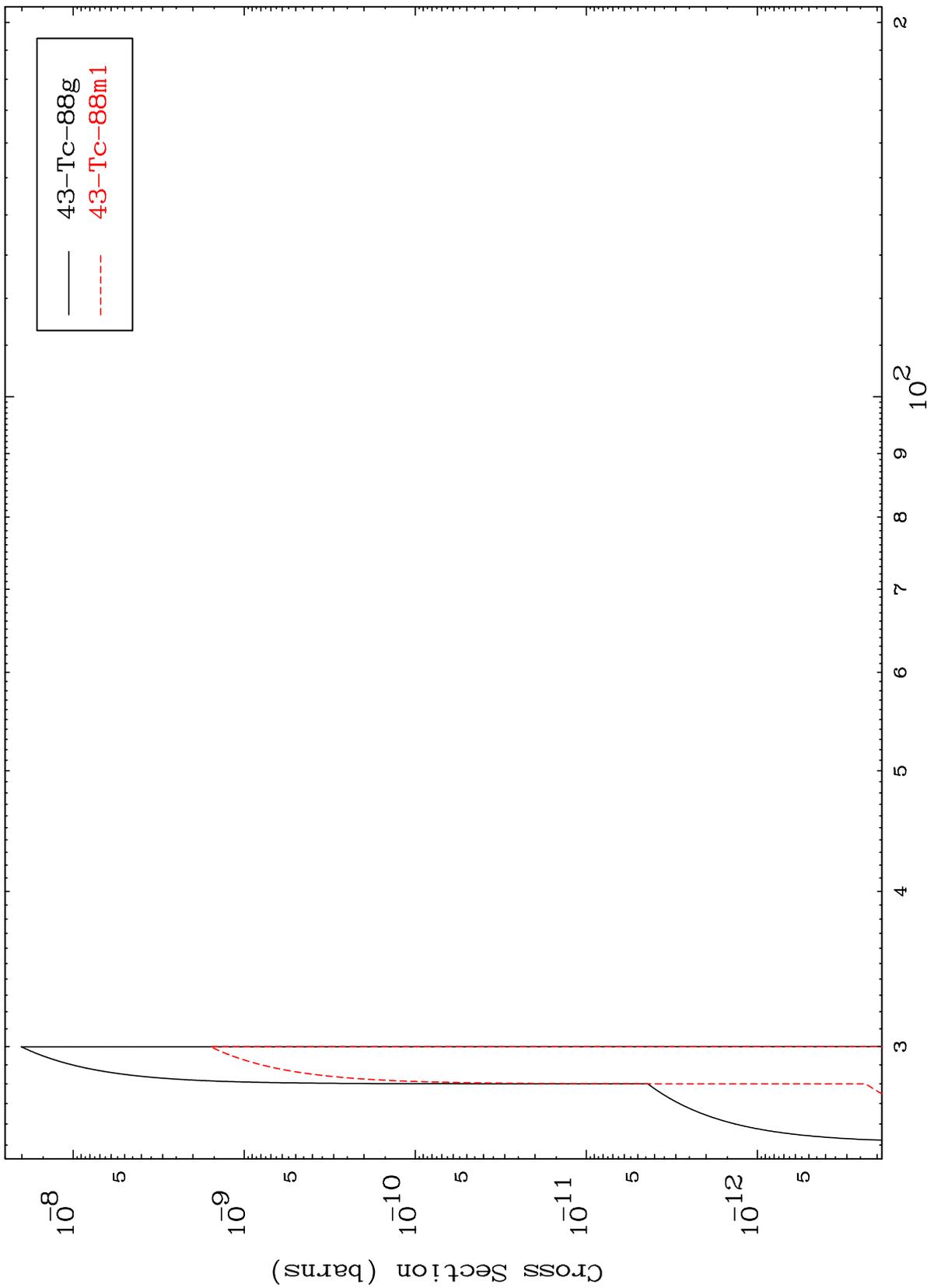
Incident Energy (MeV)

10

Radionuclide Production Cross Section

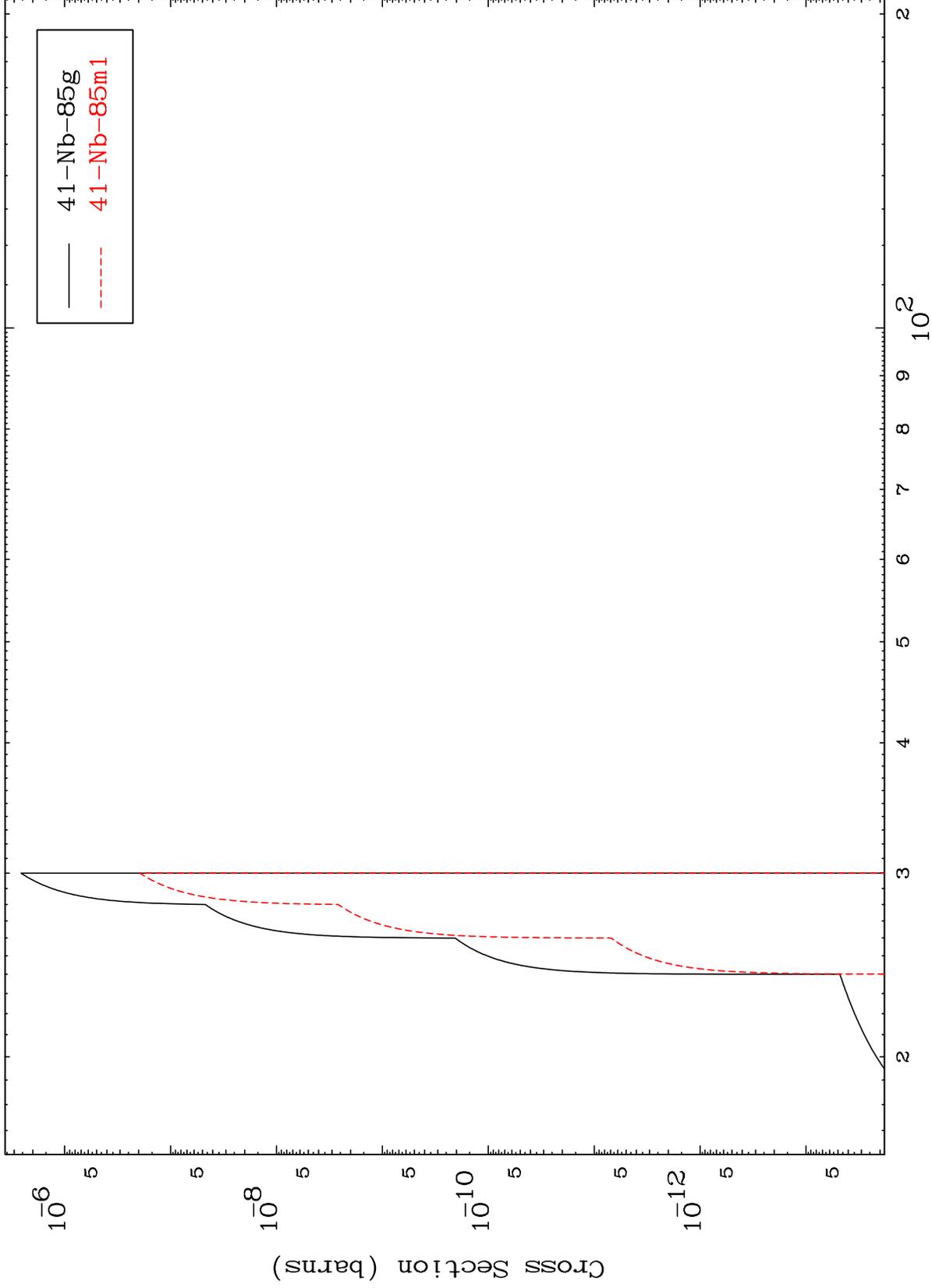


Radionuclide Production Cross Section



43-Tc-88g  
43-Tc-88m1

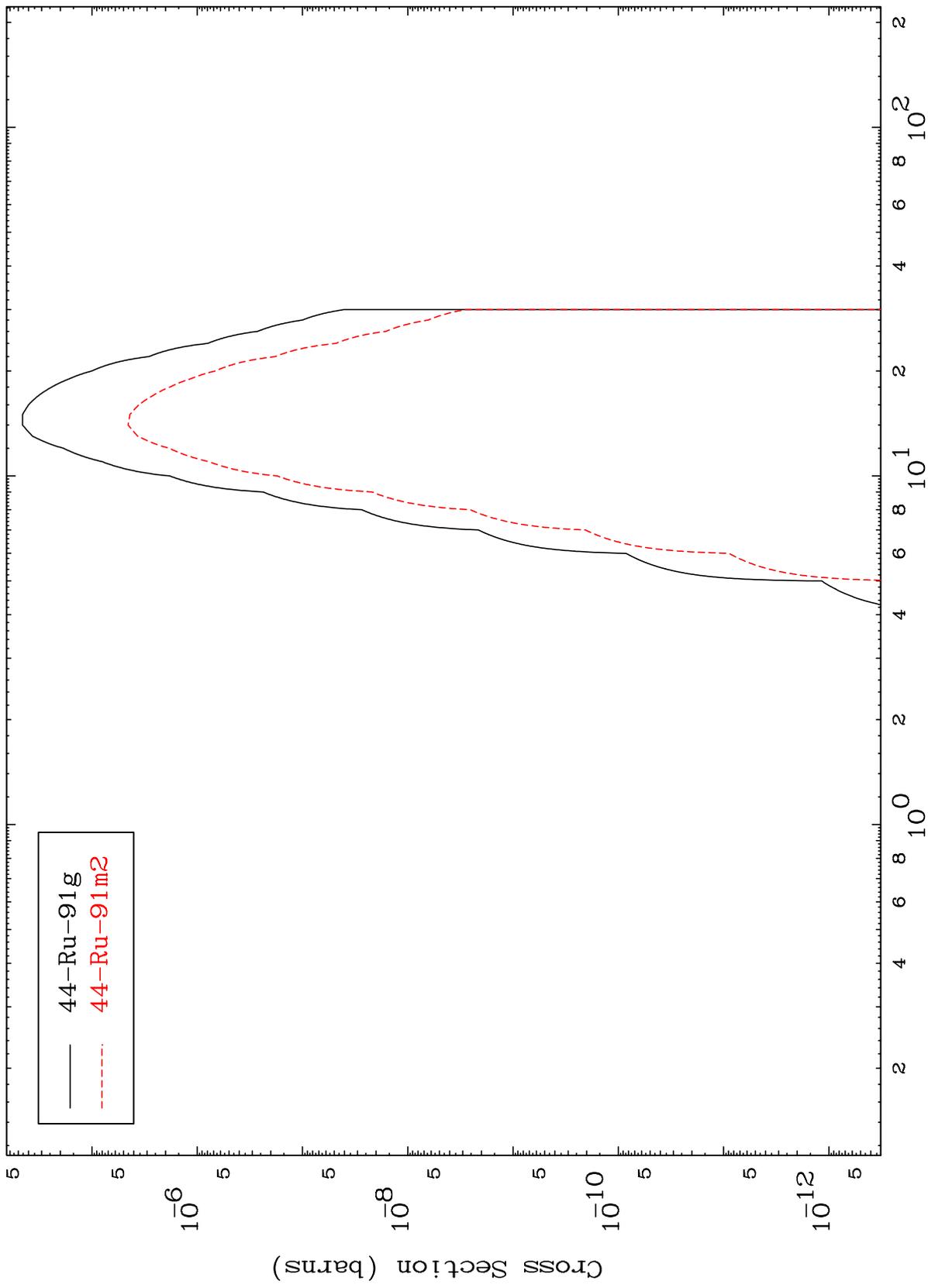
Radionuclide Production Cross Section



MAT 4210

42-Mo-87

Radionuclide Production Cross Section



42-Mo-87

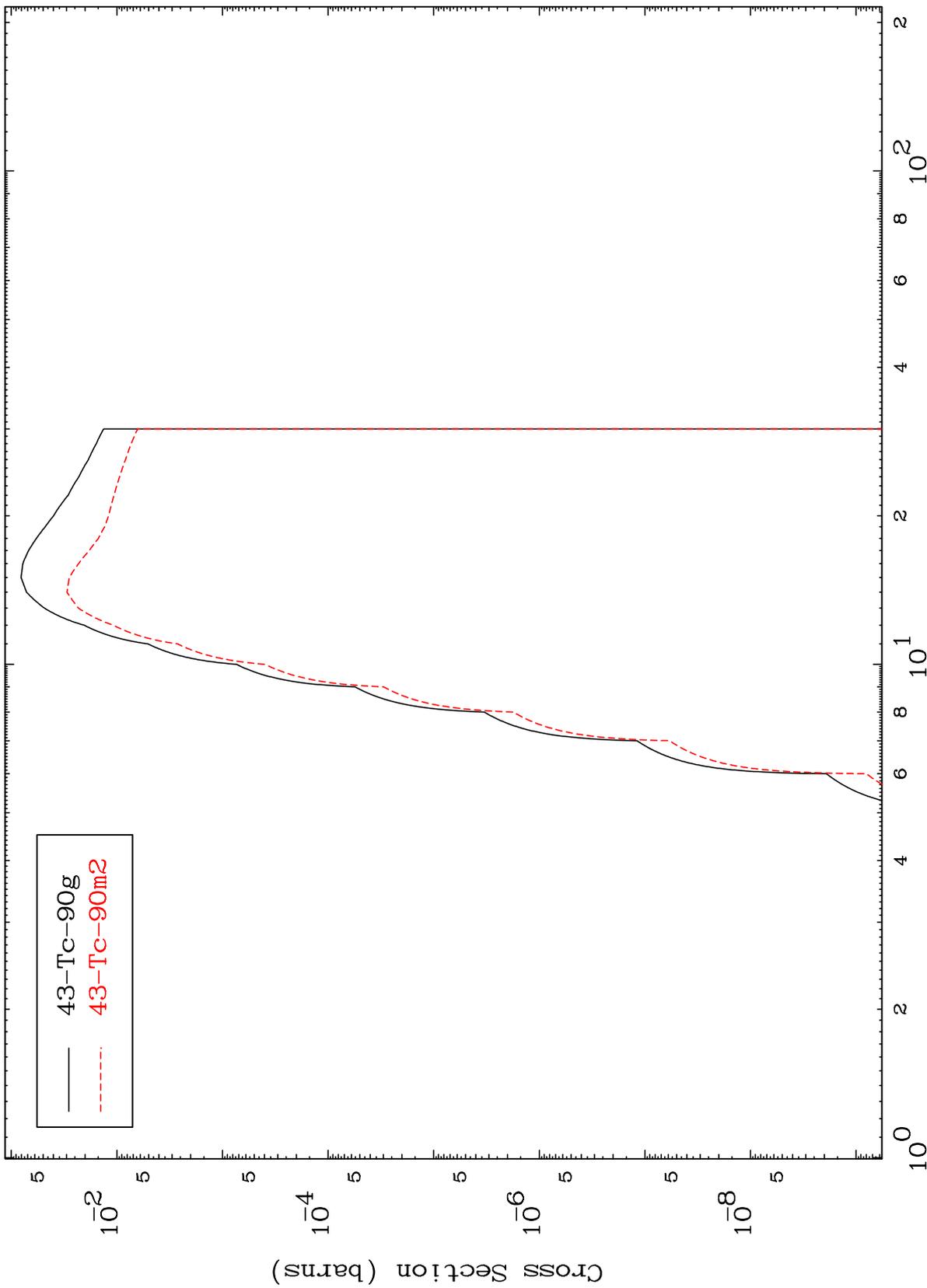
Incident Energy (MeV)

14

MAT 4210

42-Mo-87

(n,p)  
Radionuclide Production Cross Section



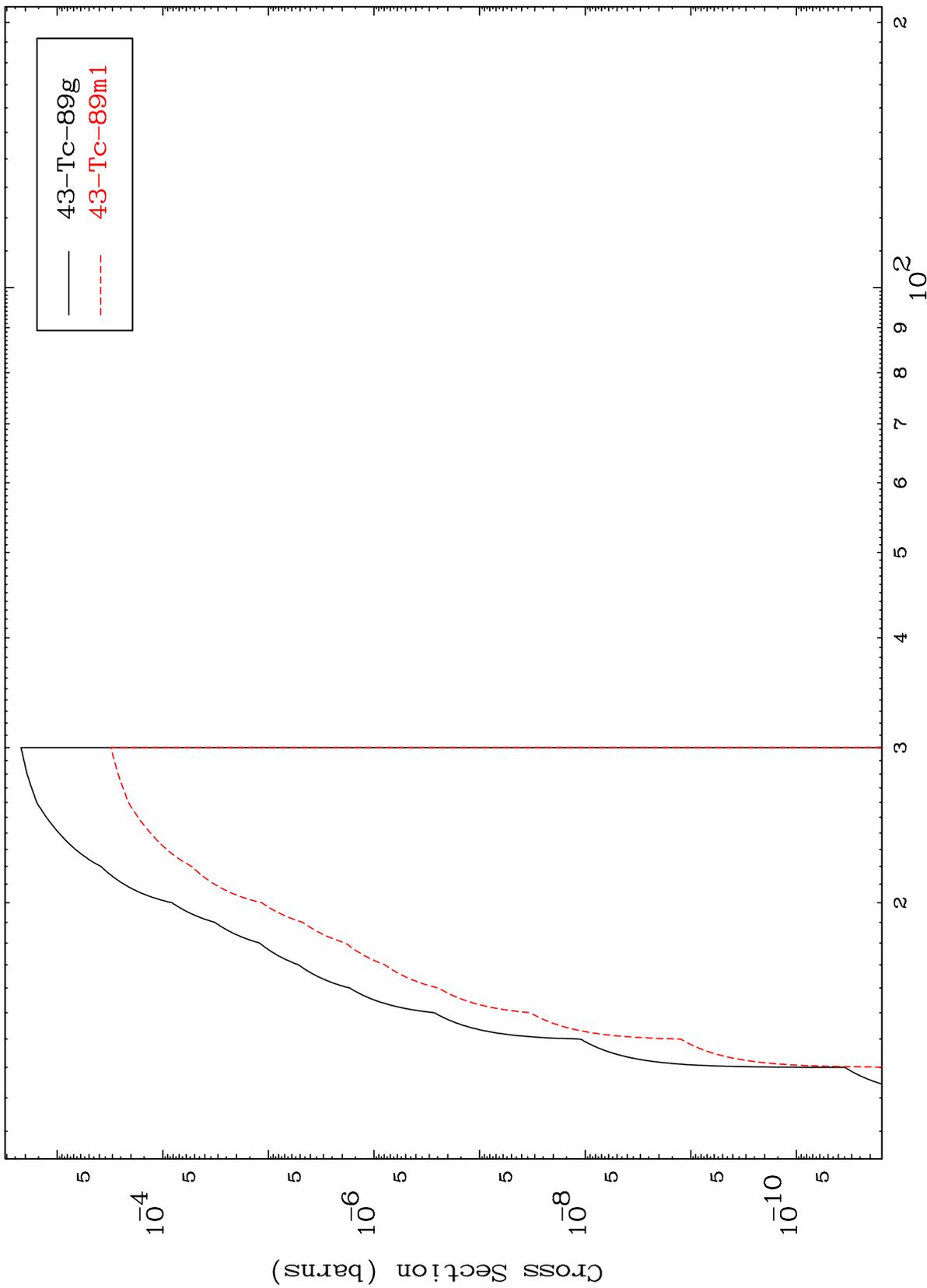
— 43-Tc-90g  
- - - 43-Tc-90m2

42-Mo-87

Incident Energy (MeV)

15

(n,d)  
Radionuclide Production Cross Section



(n, t)  
Radionuclide Production Cross Section

