

Program EVALPLOT  
(Version 2021-1)

by

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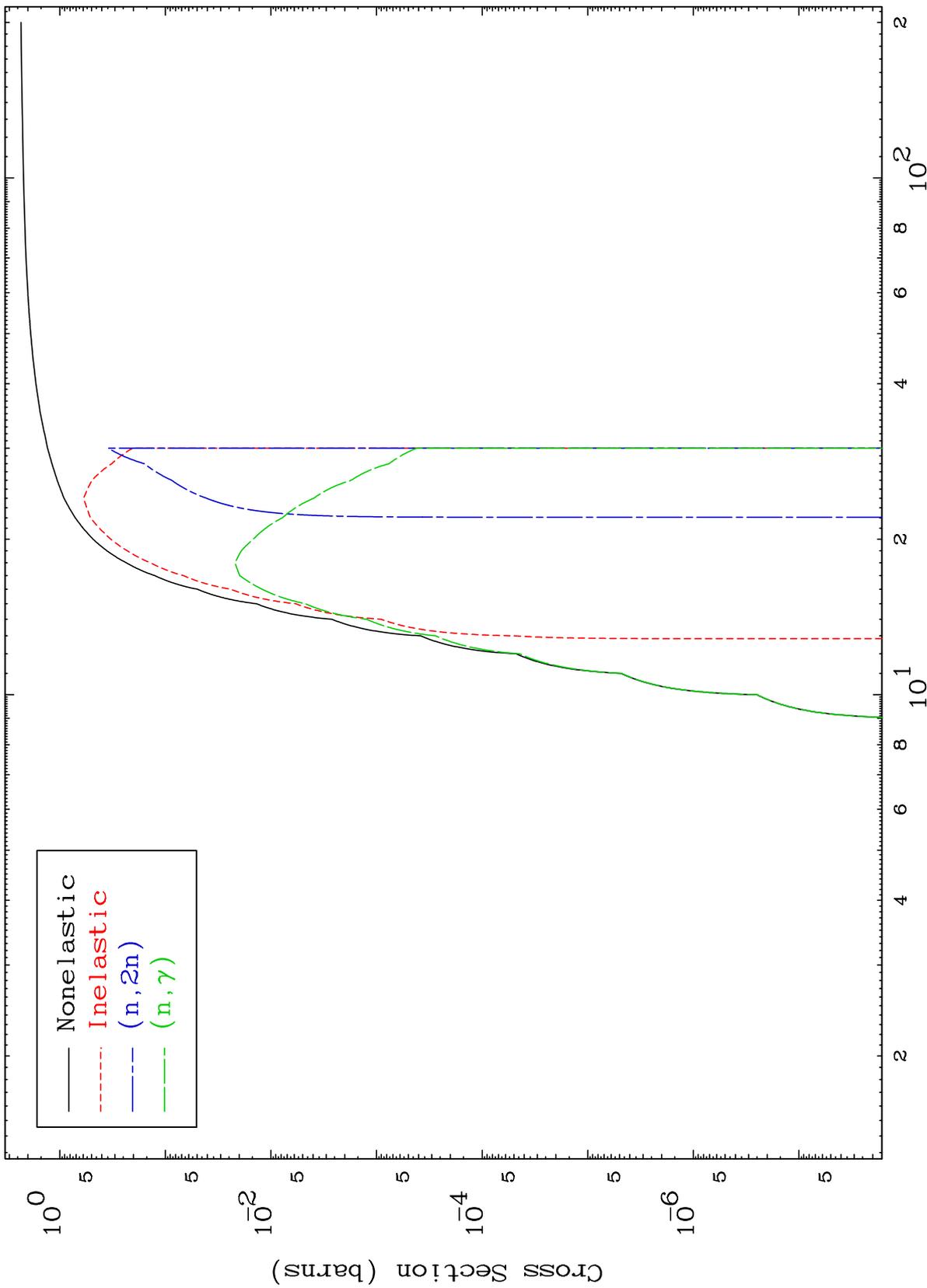
Press Mouse Button to Start

MAT 6007

$\alpha$  Major

60-Nd-136

0 Kelvin Cross Sections



1

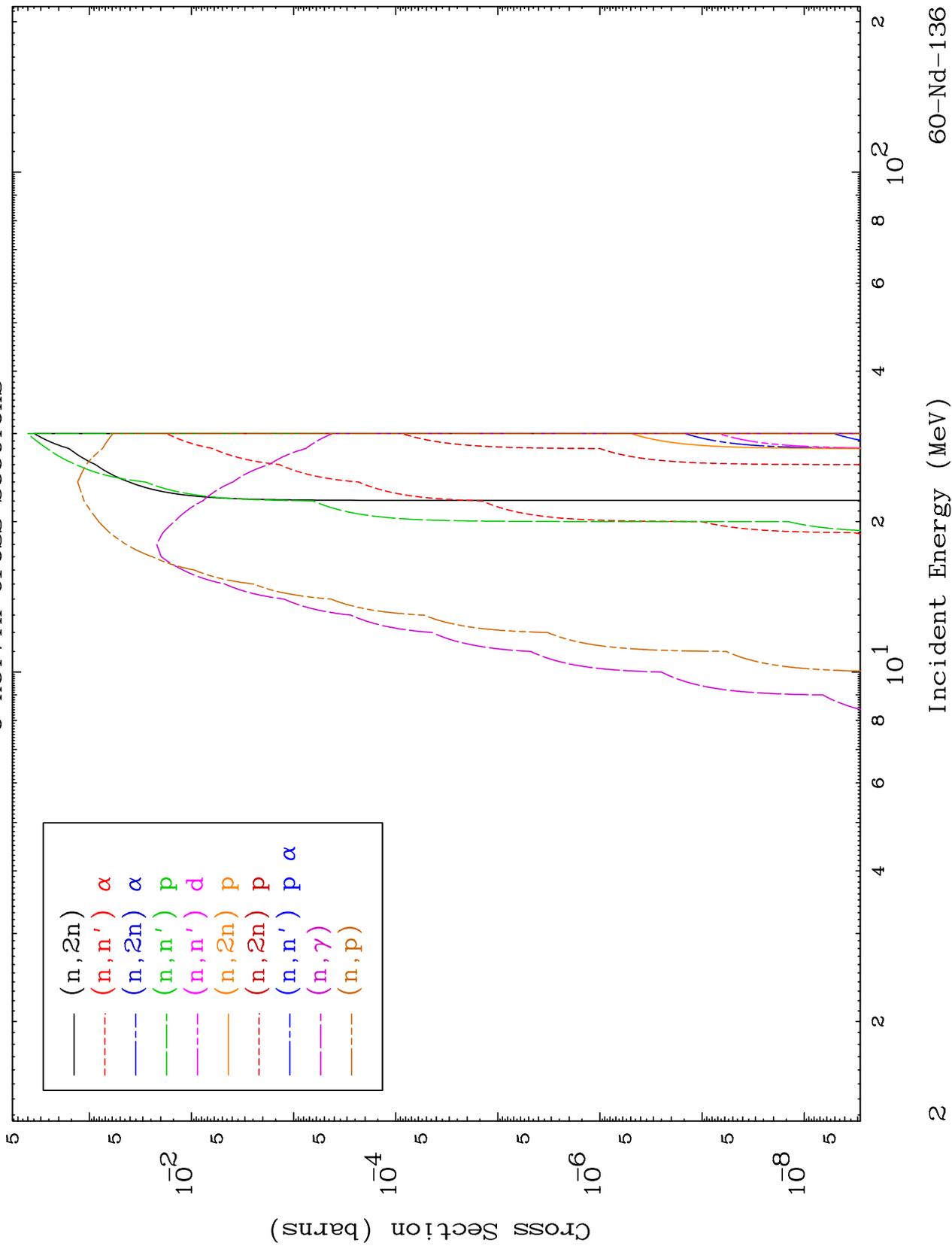
Incident Energy (MeV)

60-Nd-136

MAT 6007

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

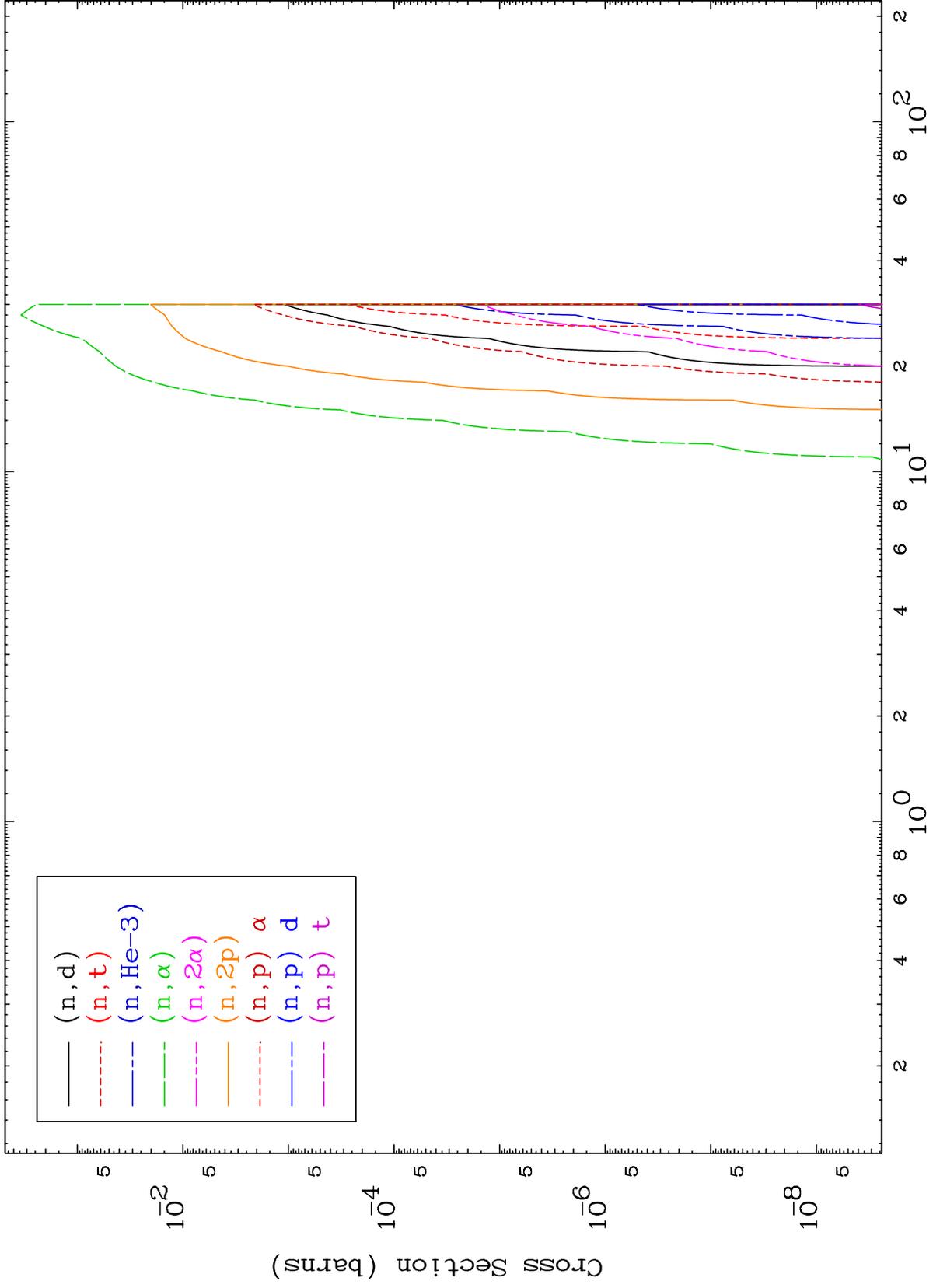
60-Nd-136



MAT 6007

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

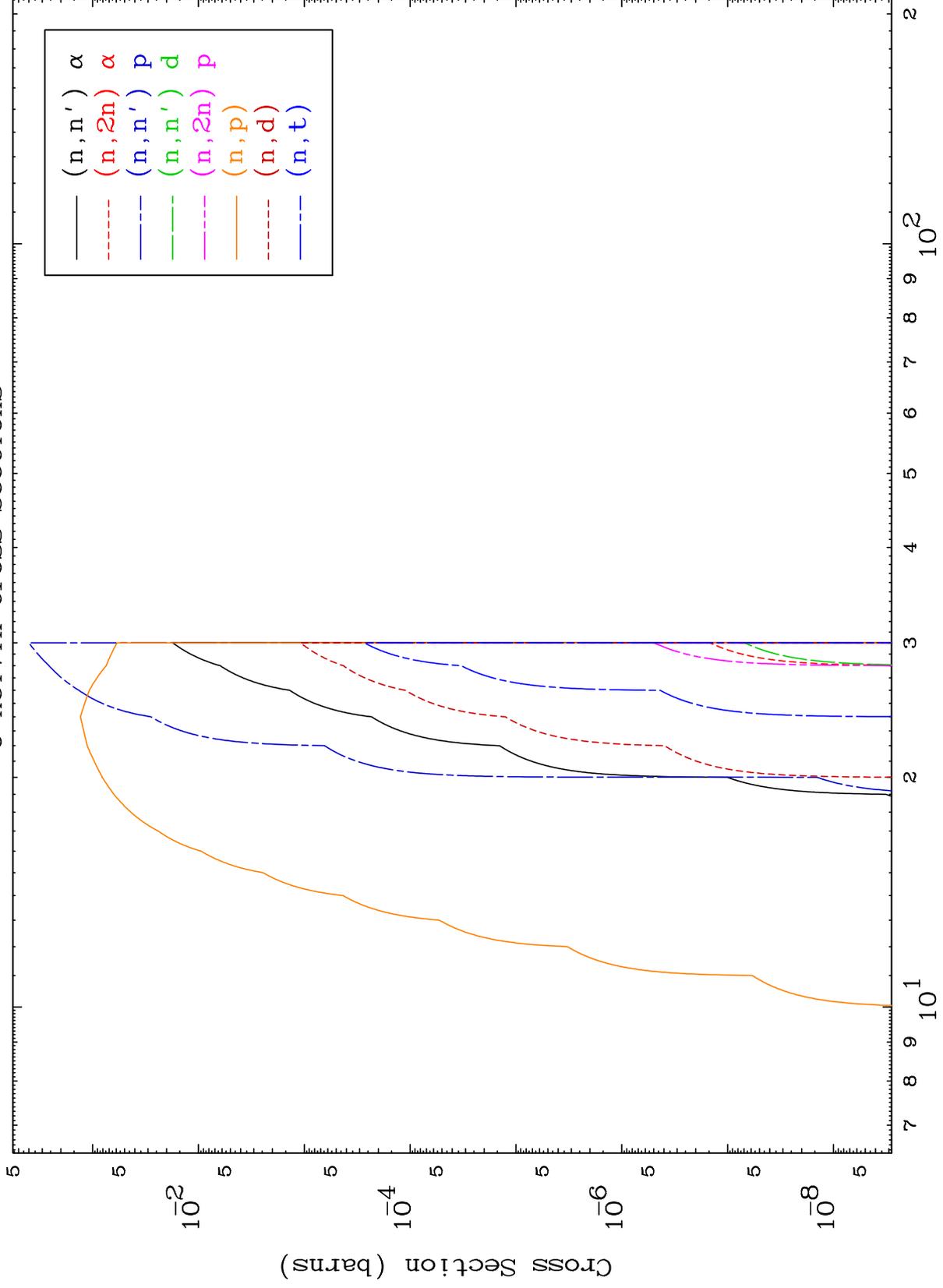
60-Nd-136



MAT 6007

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

60-Nd-136



4

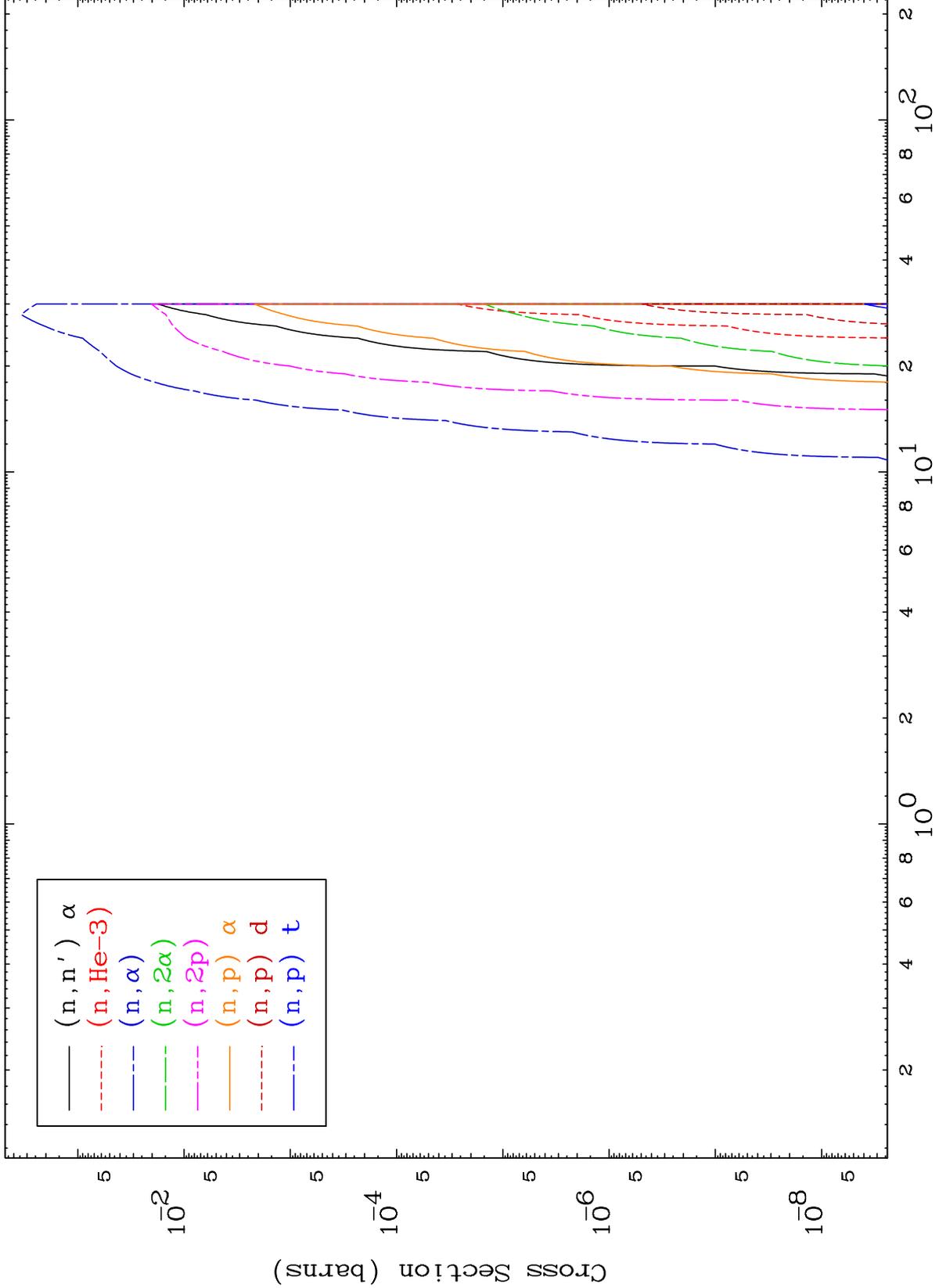
Incident Energy (MeV)

60-Nd-136

MAT 6007

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

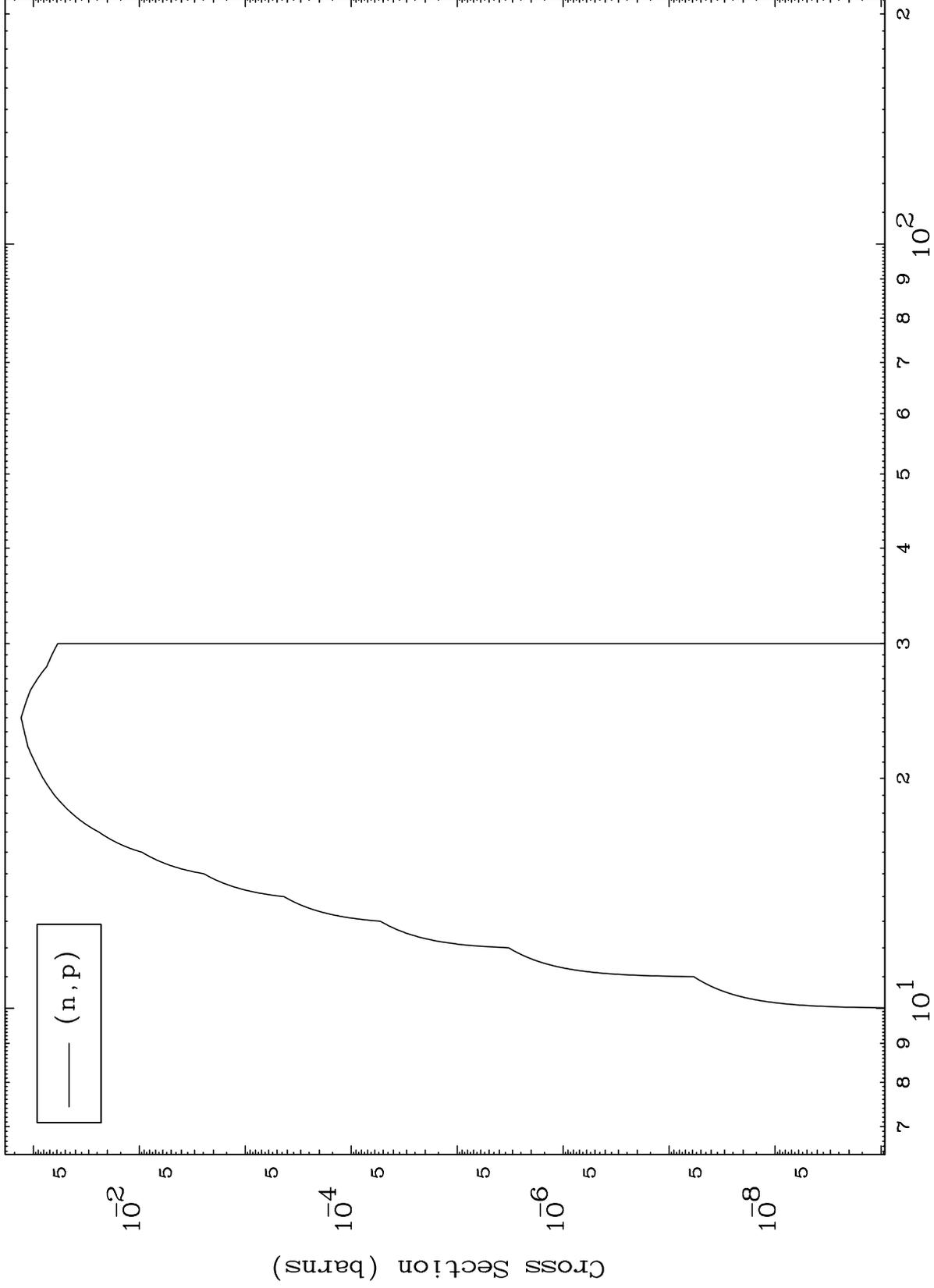
60-Nd-136



MAT 6007

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

60-Nd-136



6

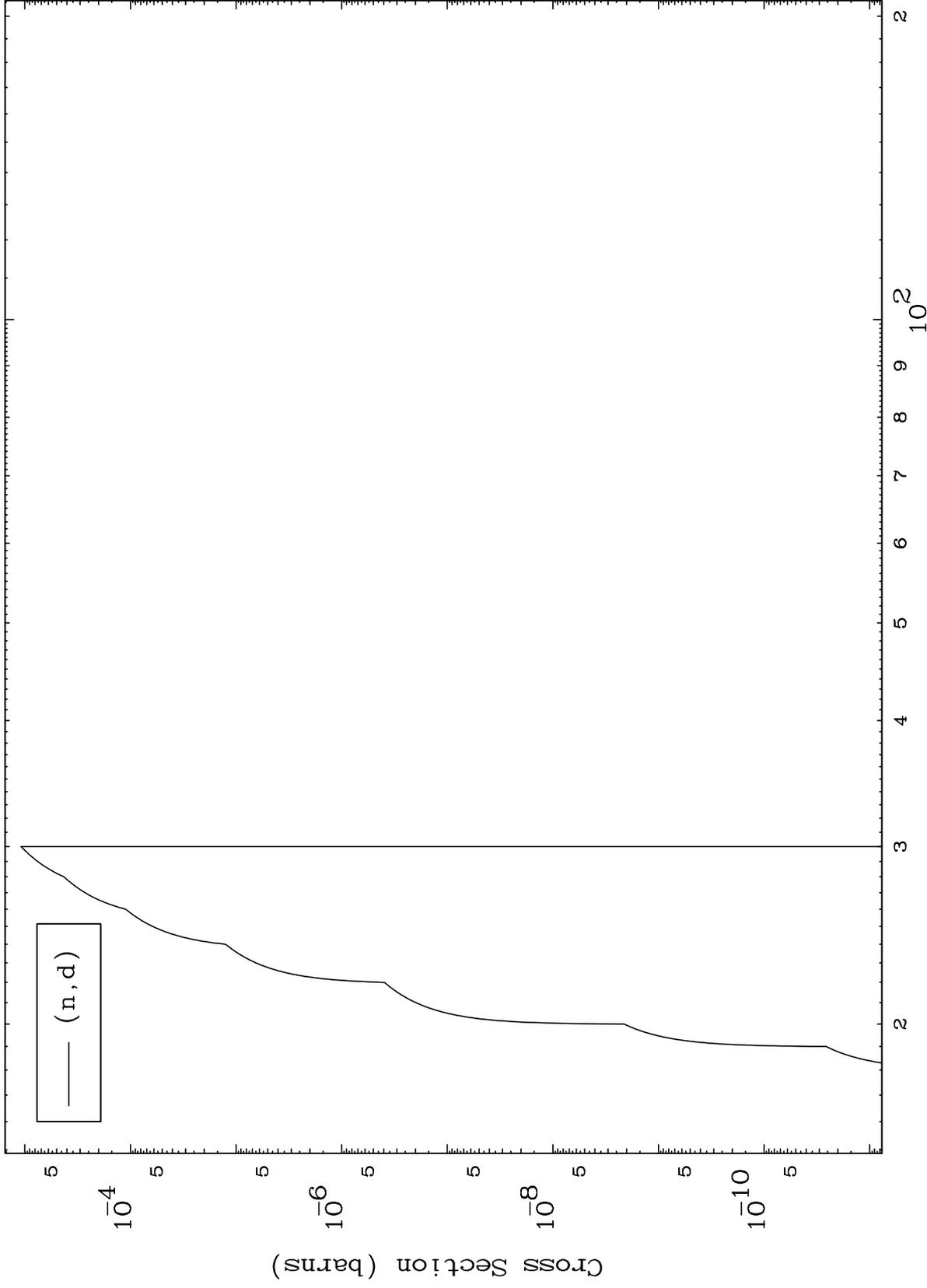
Incident Energy (MeV)

60-Nd-136

MAT 6007

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

60-Nd-136



7

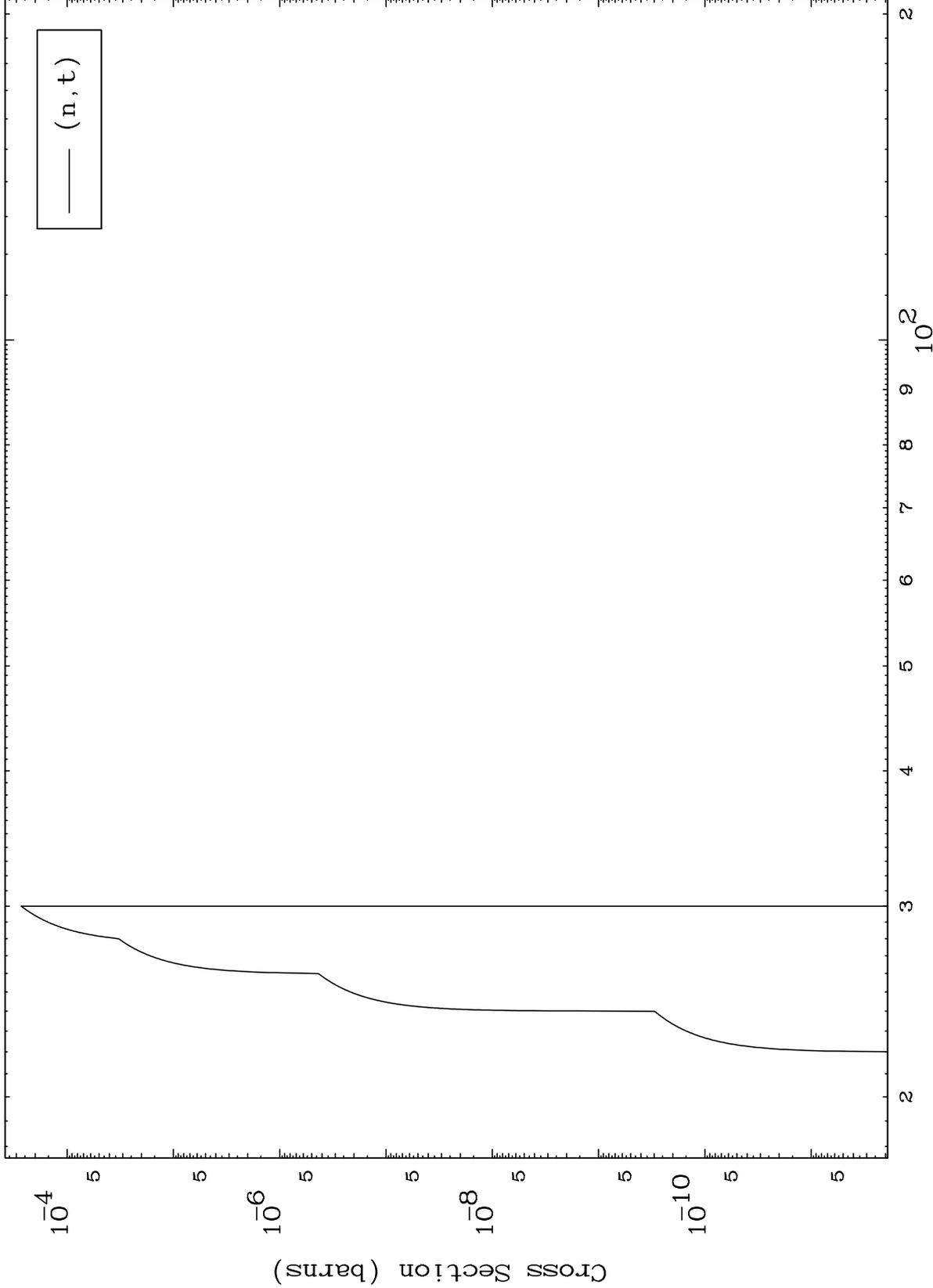
Incident Energy (MeV)

60-Nd-136

MAT 6007

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

60-Nd-136

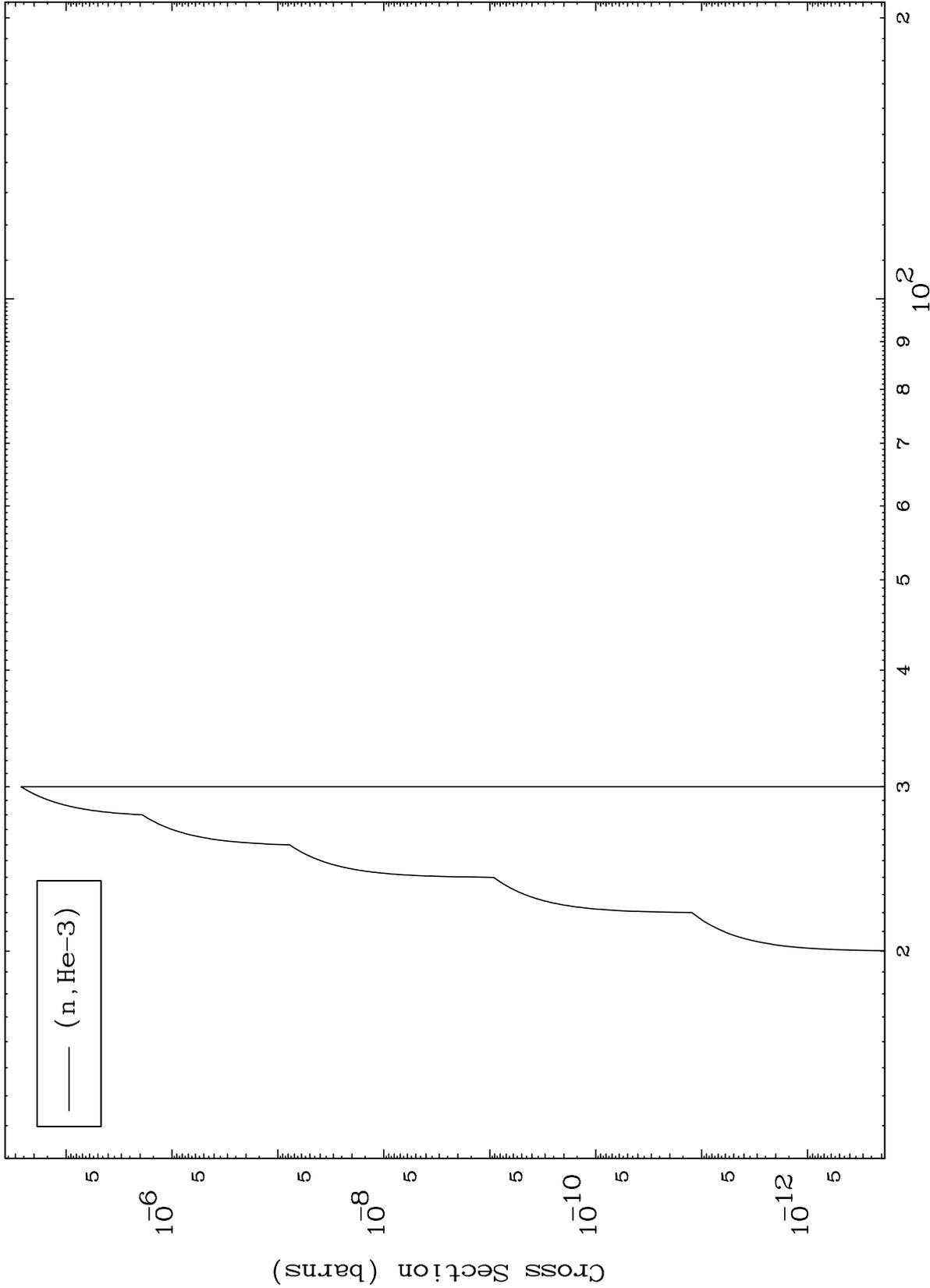


8

Incident Energy (MeV)

60-Nd-136

( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections



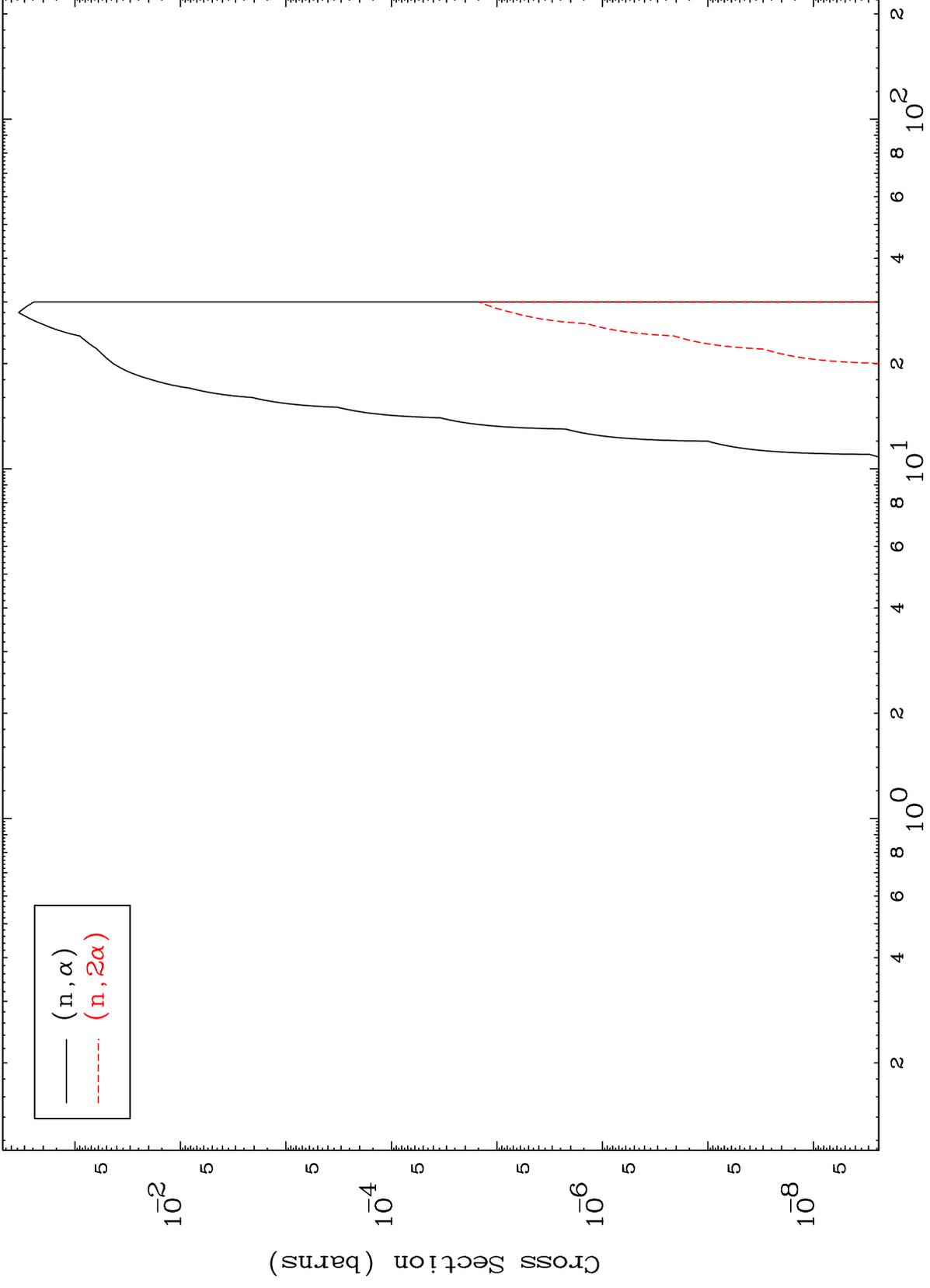
Incident Energy (MeV)

MAT 6007

( $\alpha, \alpha$ ) Levels

60-Nd-136

0 Kelvin Cross Sections



10

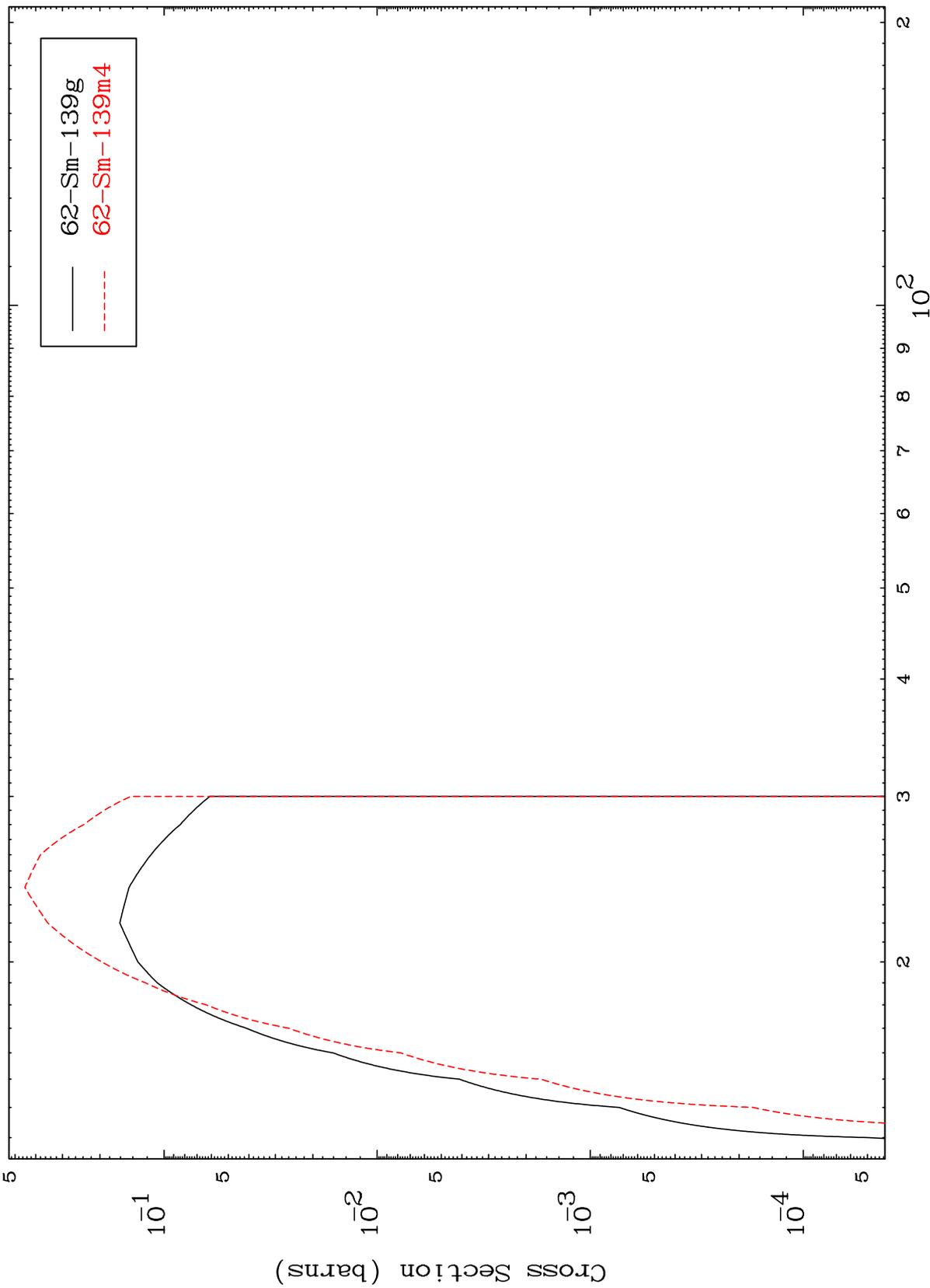
Incident Energy (MeV)

60-Nd-136

MAT 6007

60-Nd-136

Inelastic  
Radionuclide Production Cross Section

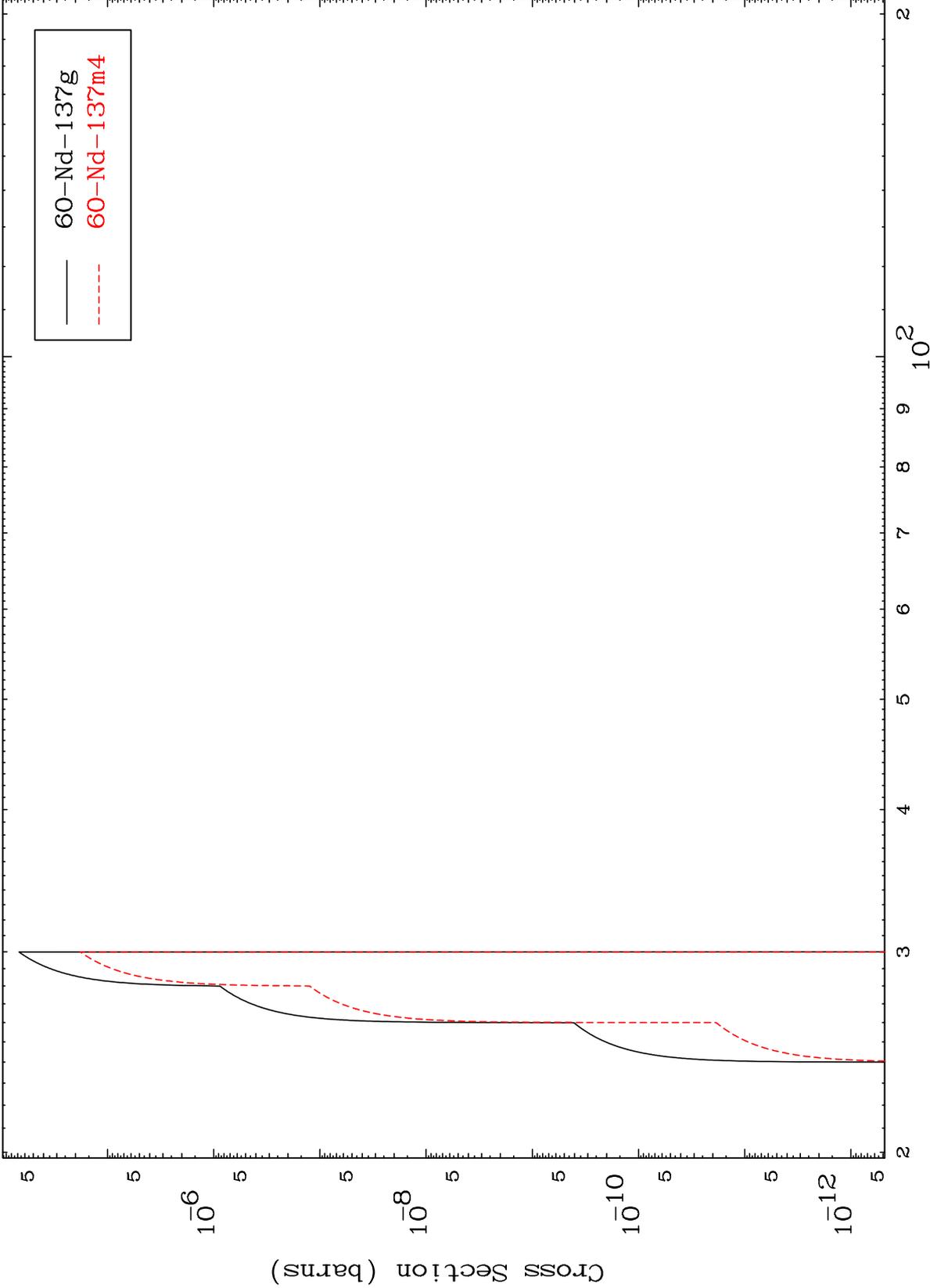


MAT 6007

(n,2n) p

60-Nd-136

Radionuclide Production Cross Section



12

Incident Energy (MeV)

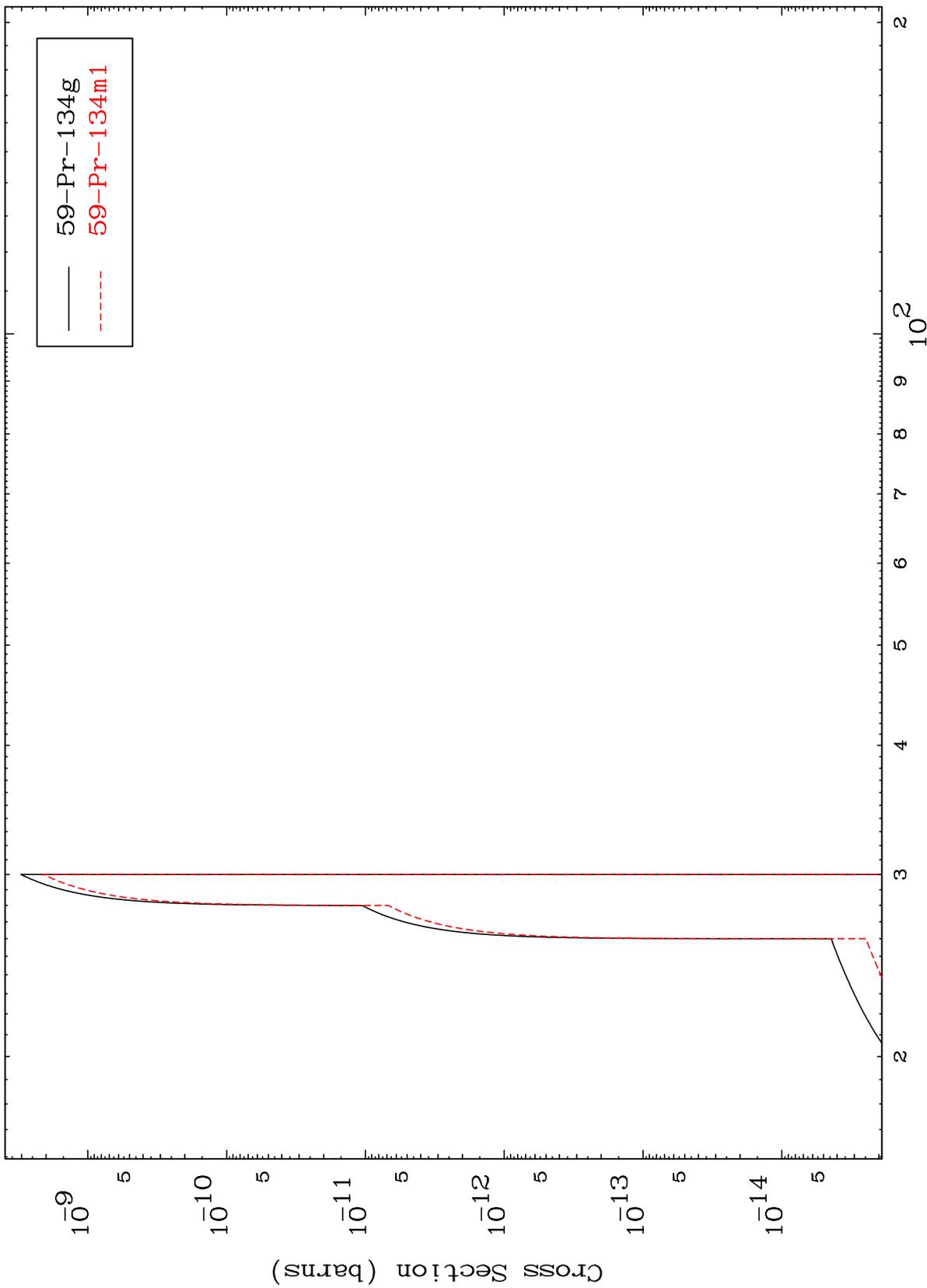
60-Nd-136

MAT 6007

(n,n') p  $\alpha$

60-Nd-136

Radionuclide Production Cross Section



13

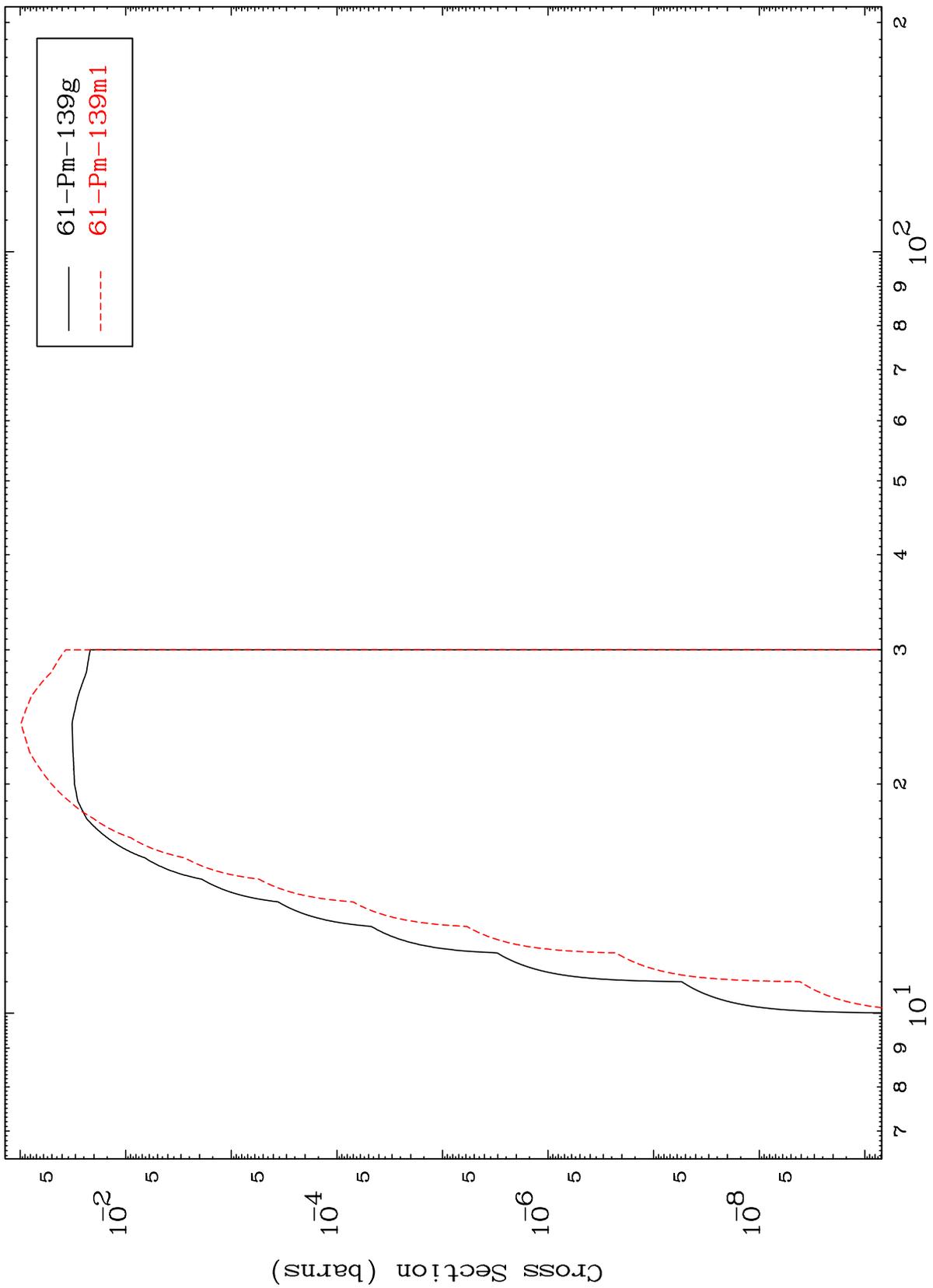
Incident Energy (MeV)

60-Nd-136

MAT 6007

60-Nd-136

(n,p)  
Radionuclide Production Cross Section



14

Incident Energy (MeV)

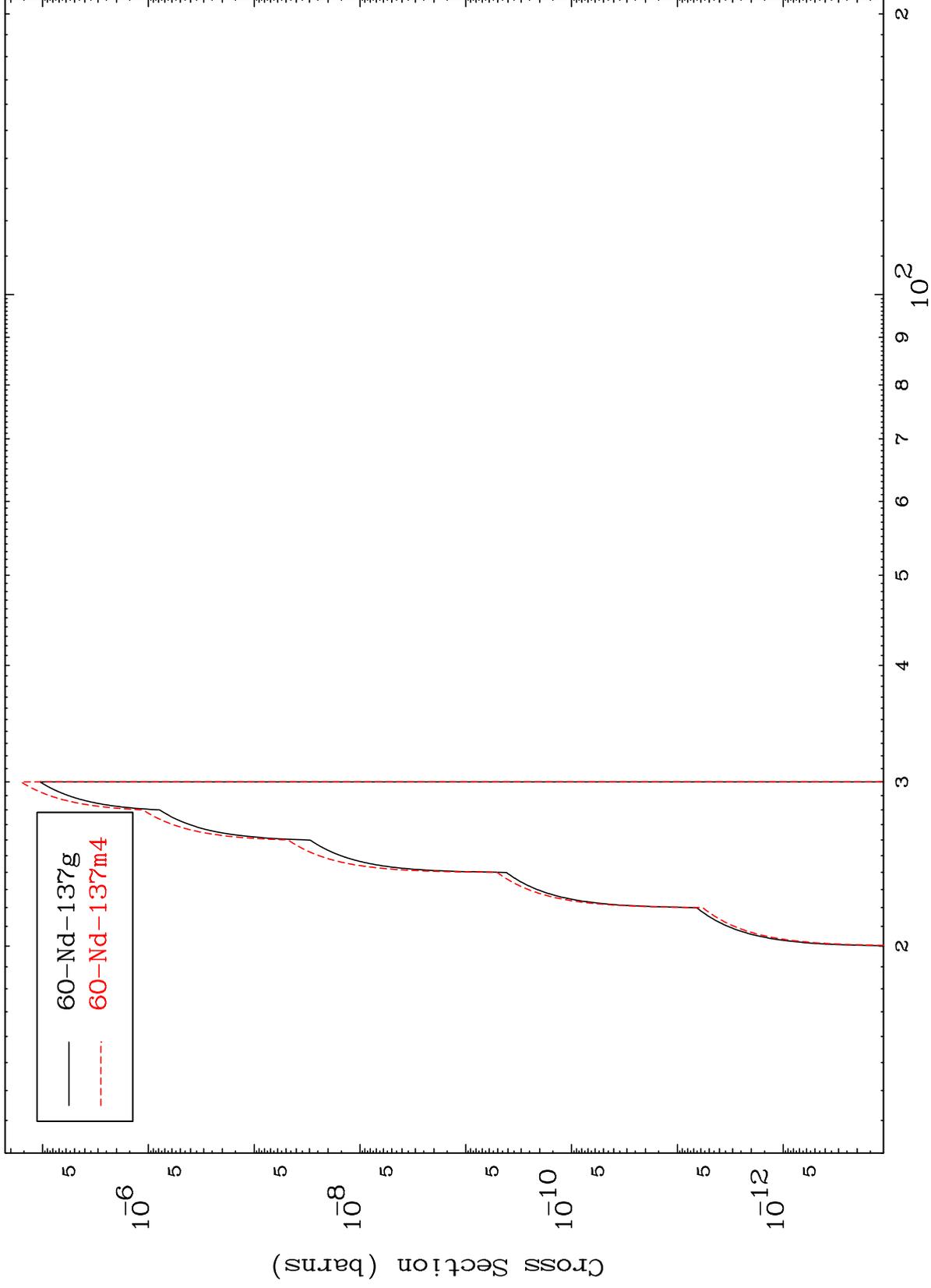
60-Nd-136

MAT 6007

(n,He-3)

60-Nd-136

Radionuclide Production Cross Section



15

Incident Energy (MeV)

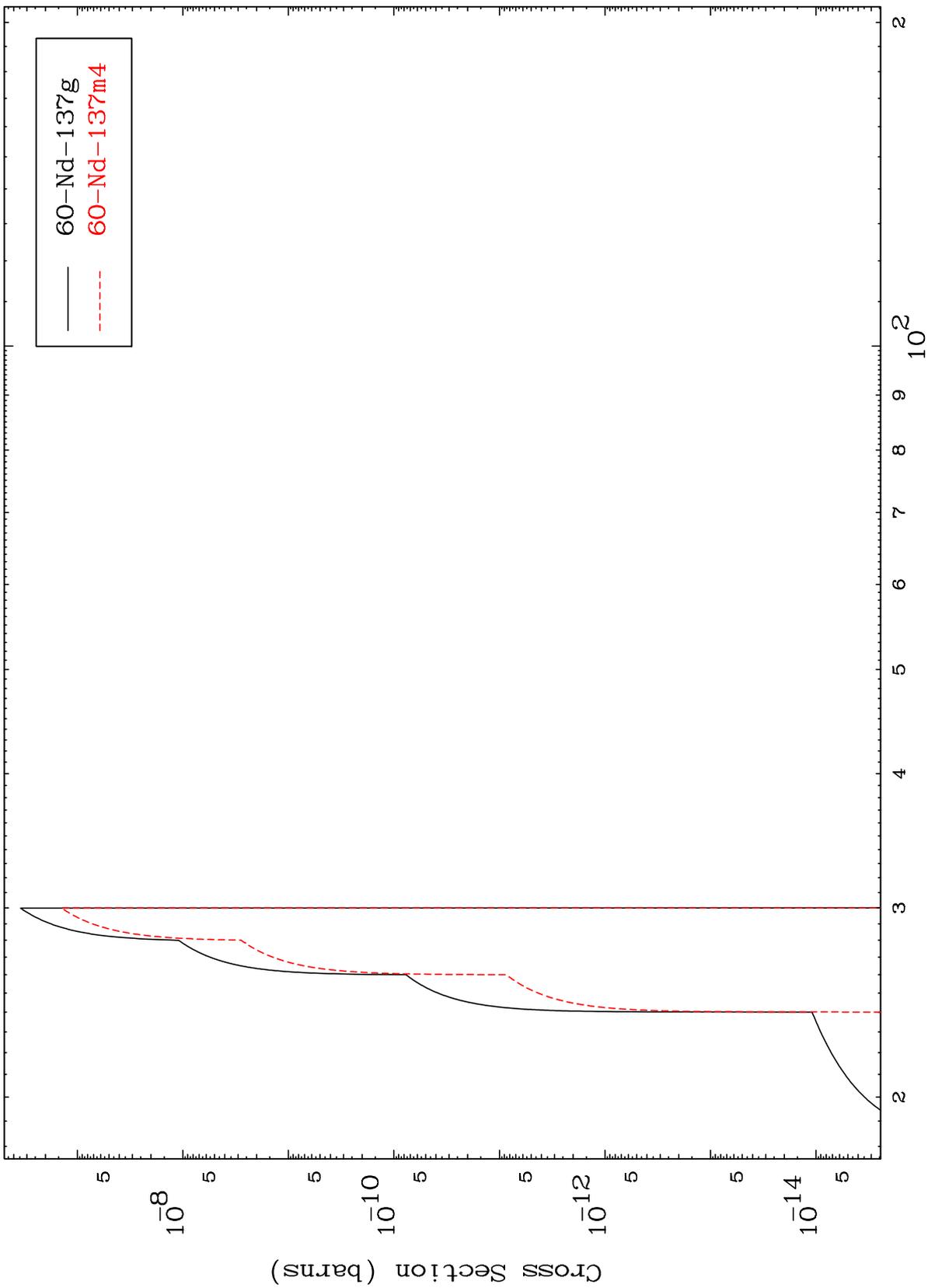
60-Nd-136

MAT 6007

(n,p) d

<sup>60</sup>Nd-136

Radionuclide Production Cross Section



16

Incident Energy (MeV)

<sup>60</sup>Nd-136