

Program EVALPLOT  
(Version 2021-1)

by

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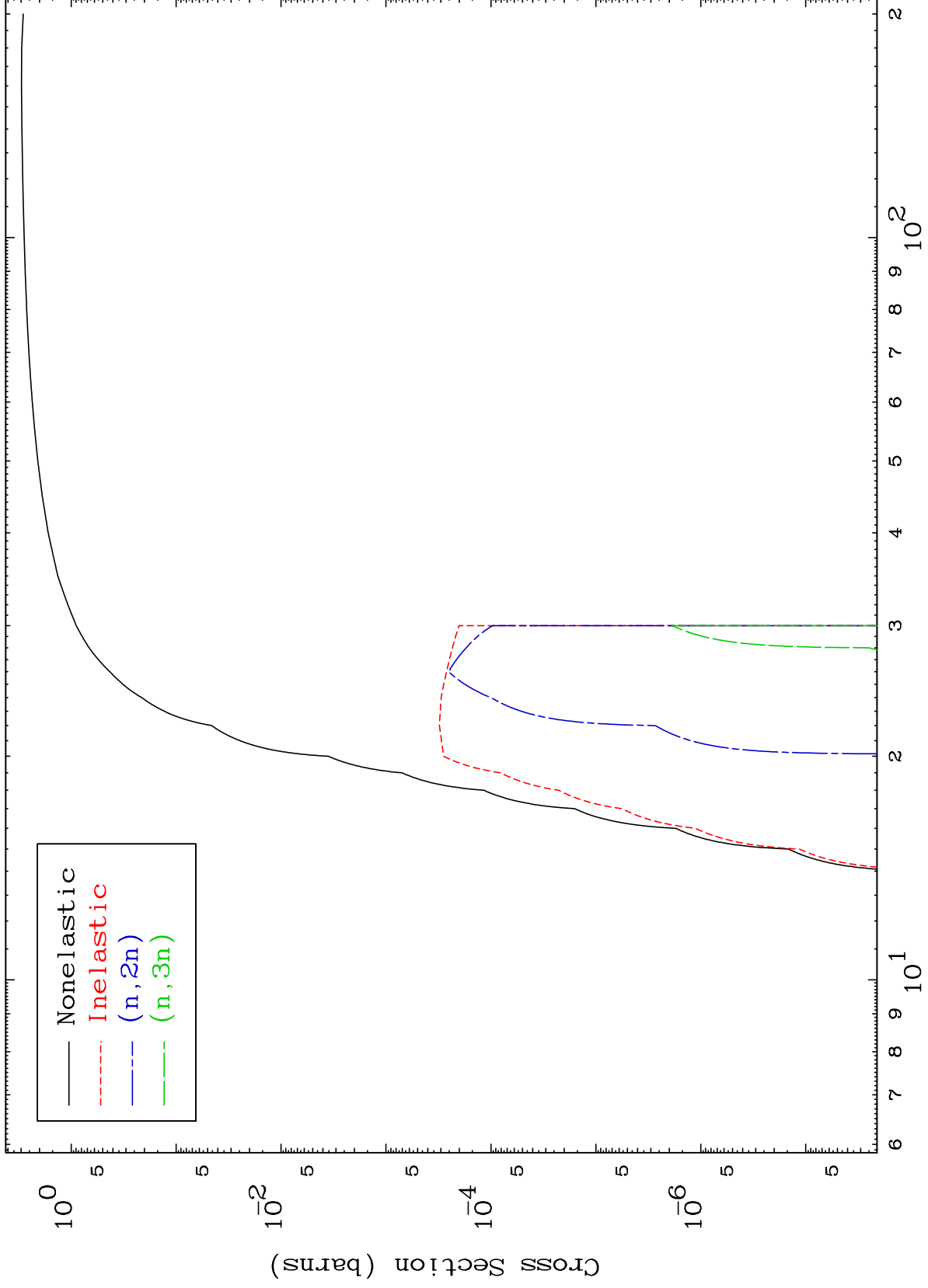
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9337

0 Kelvin Cross Sections

93-Np-234



1

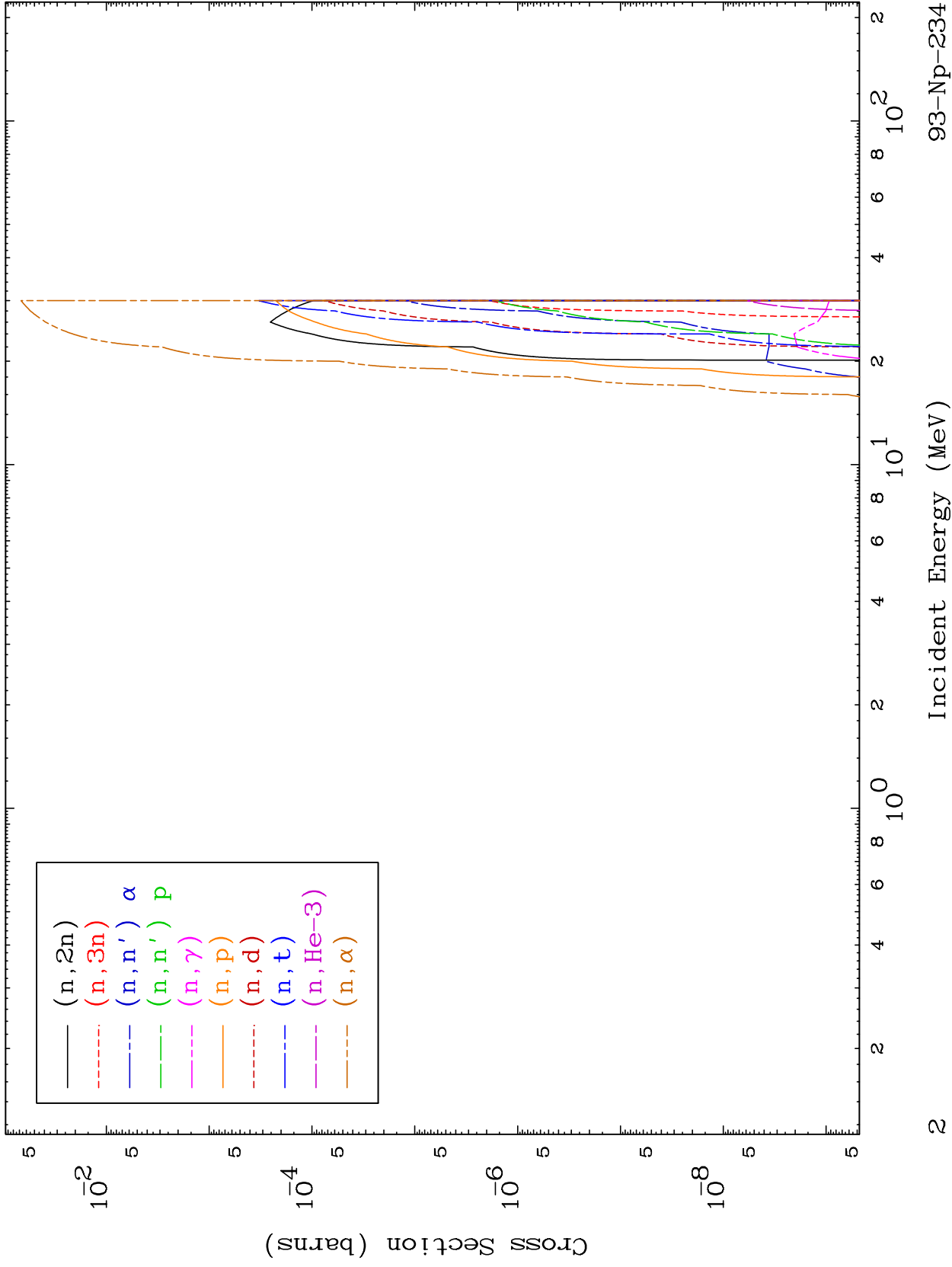
Incident Energy (MeV)

93-Np-234

MAT 9337

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

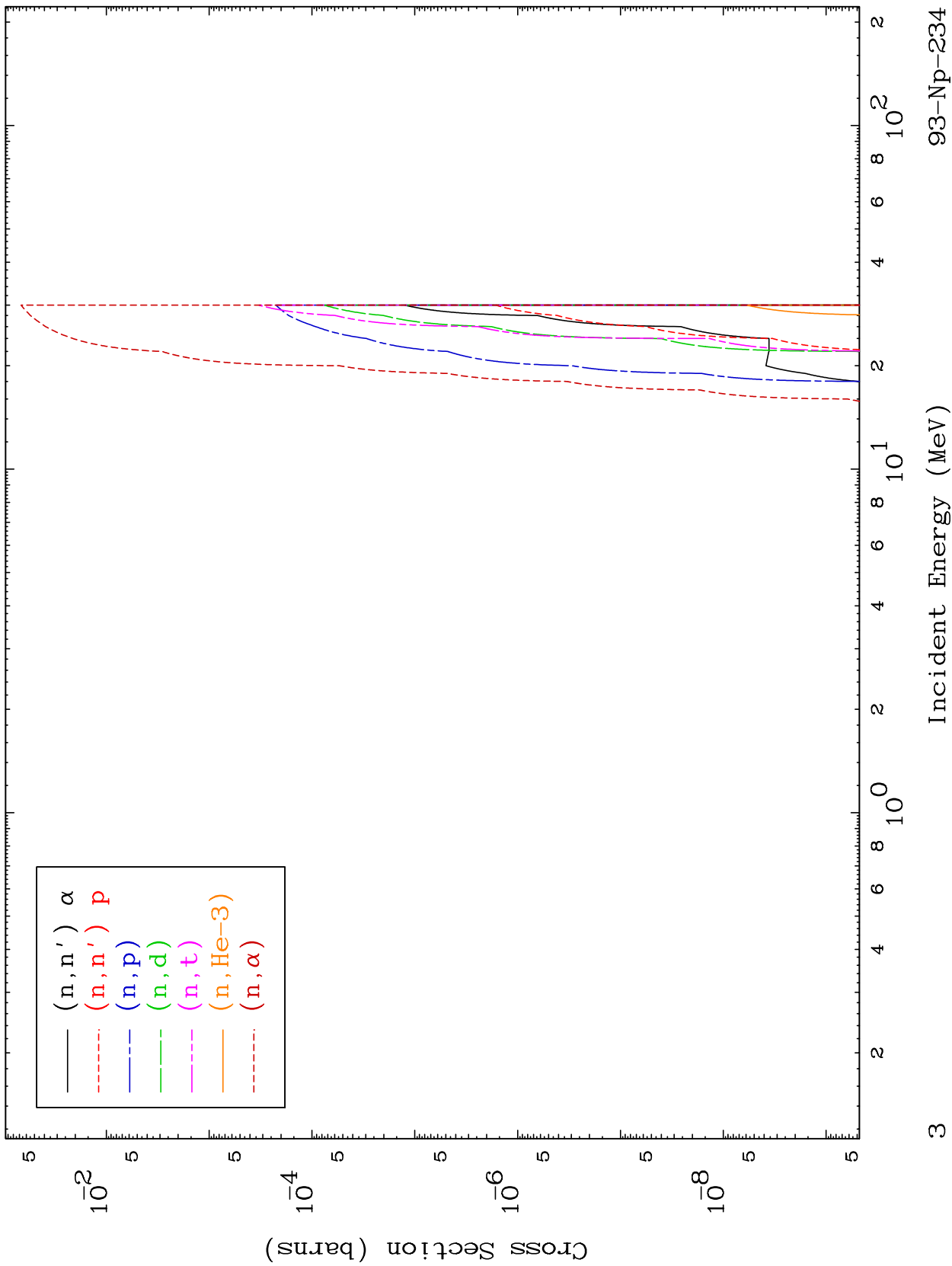
93-Np-234



MAT 9337

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

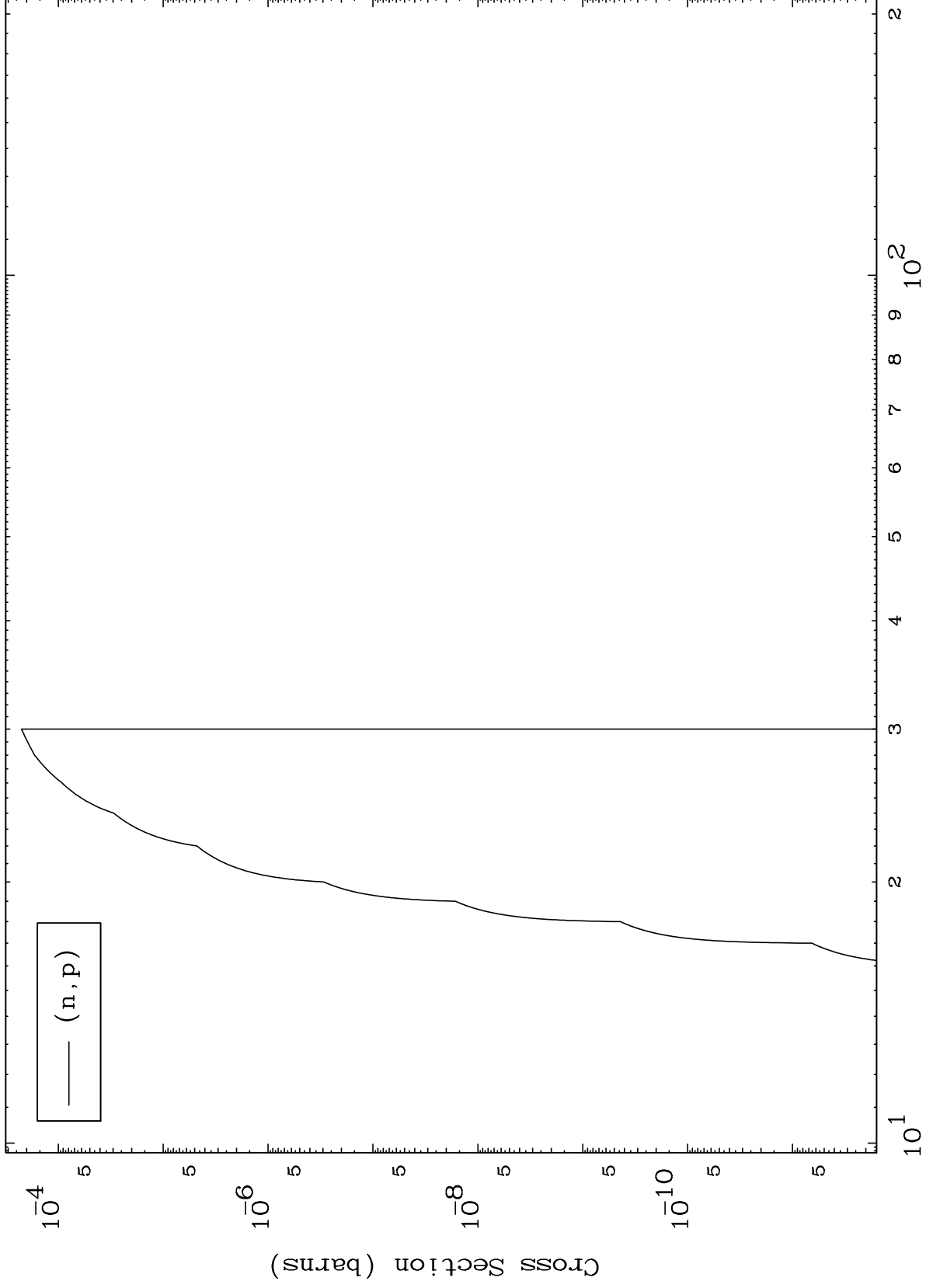
93-Np-234



MAT 9337

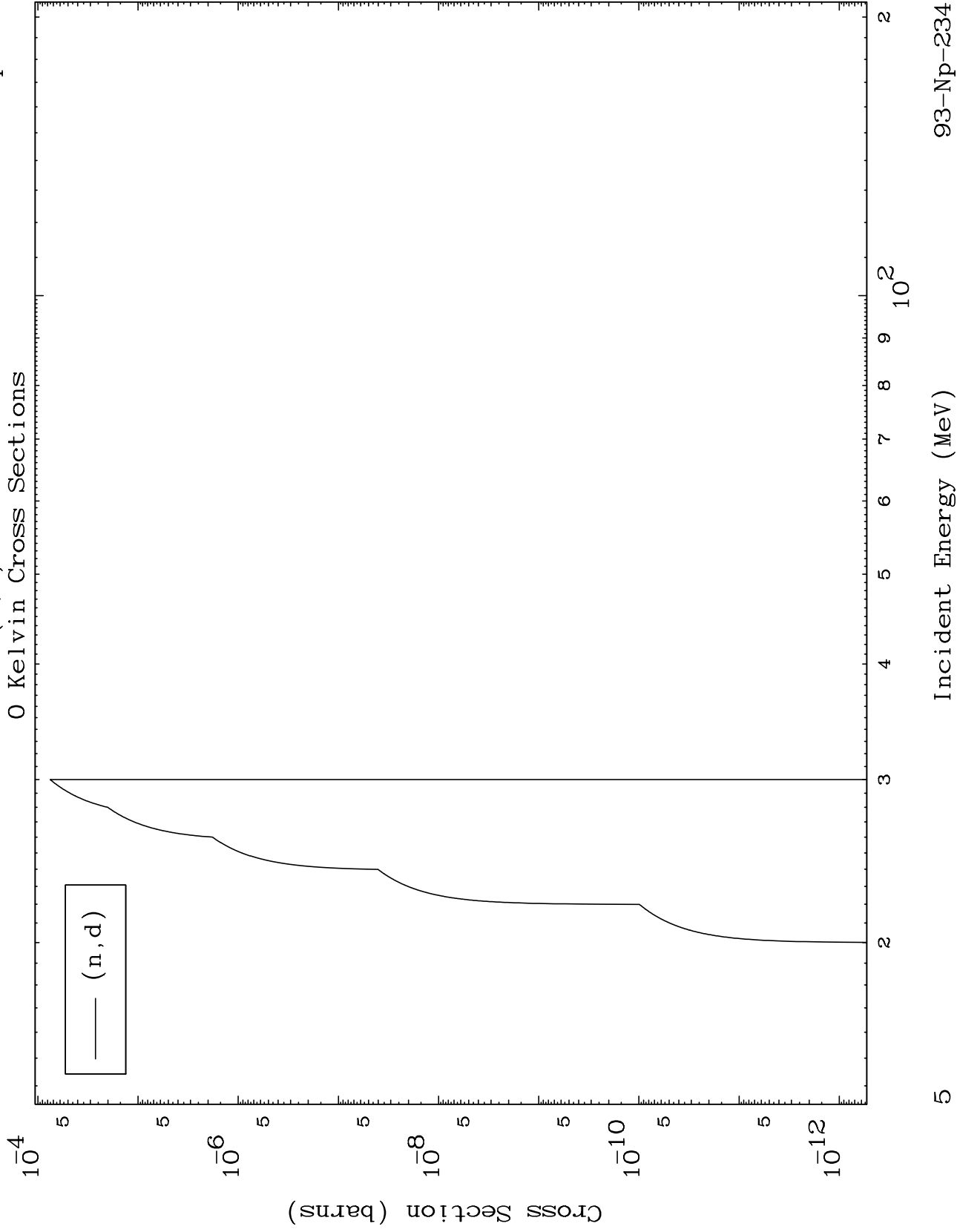
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

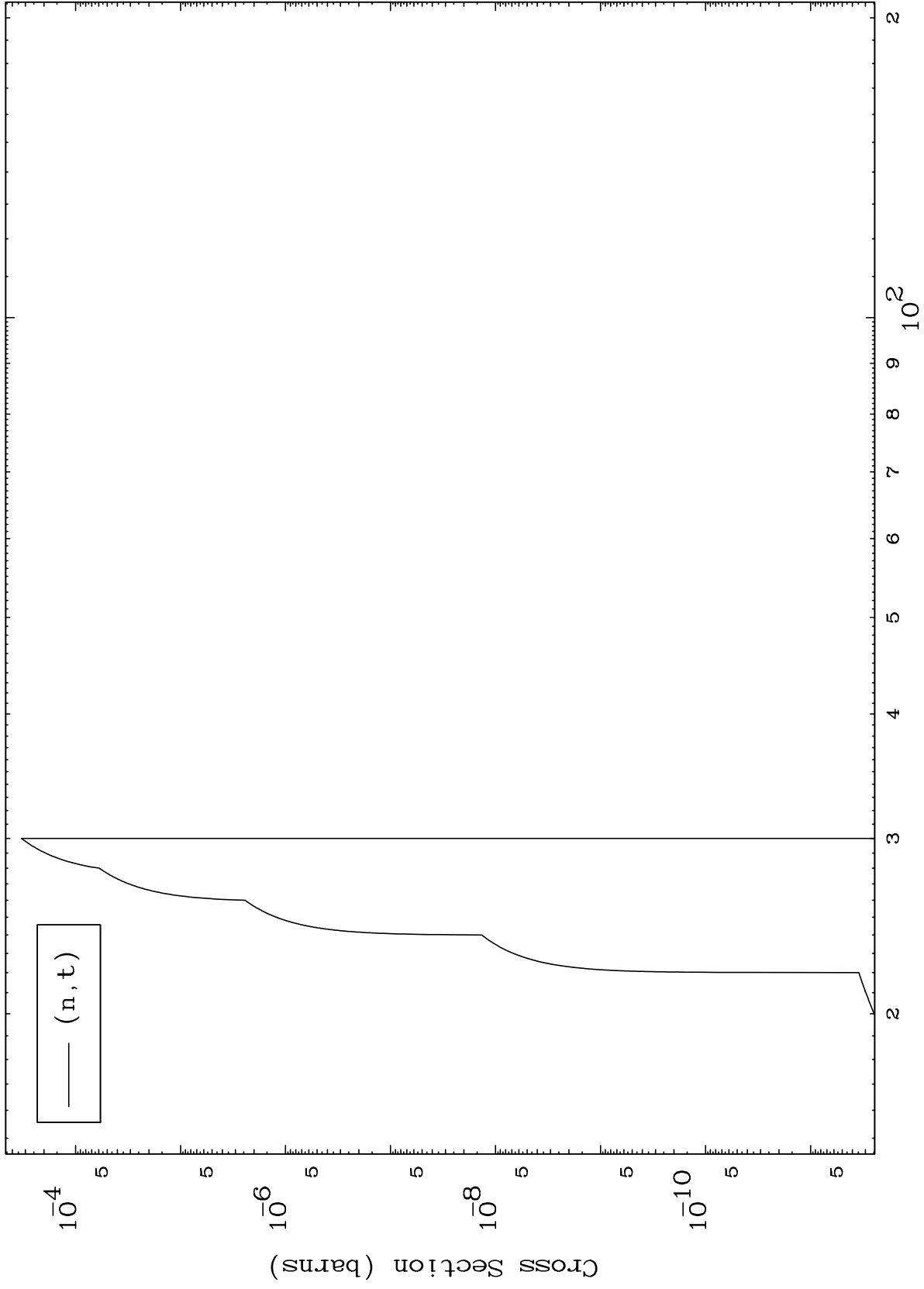
93-Np-234



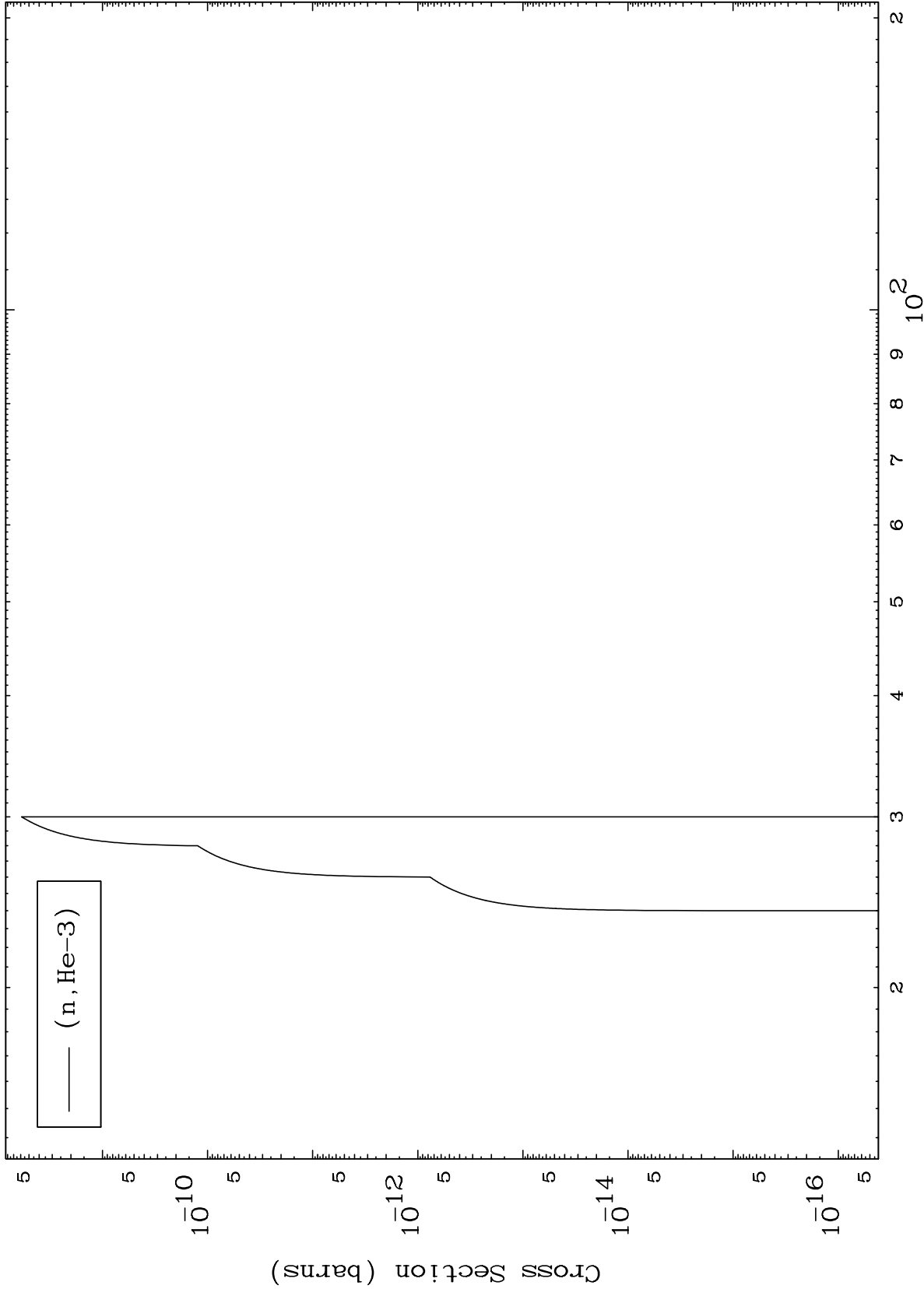
Incident Energy (MeV)

93-Np-234





( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

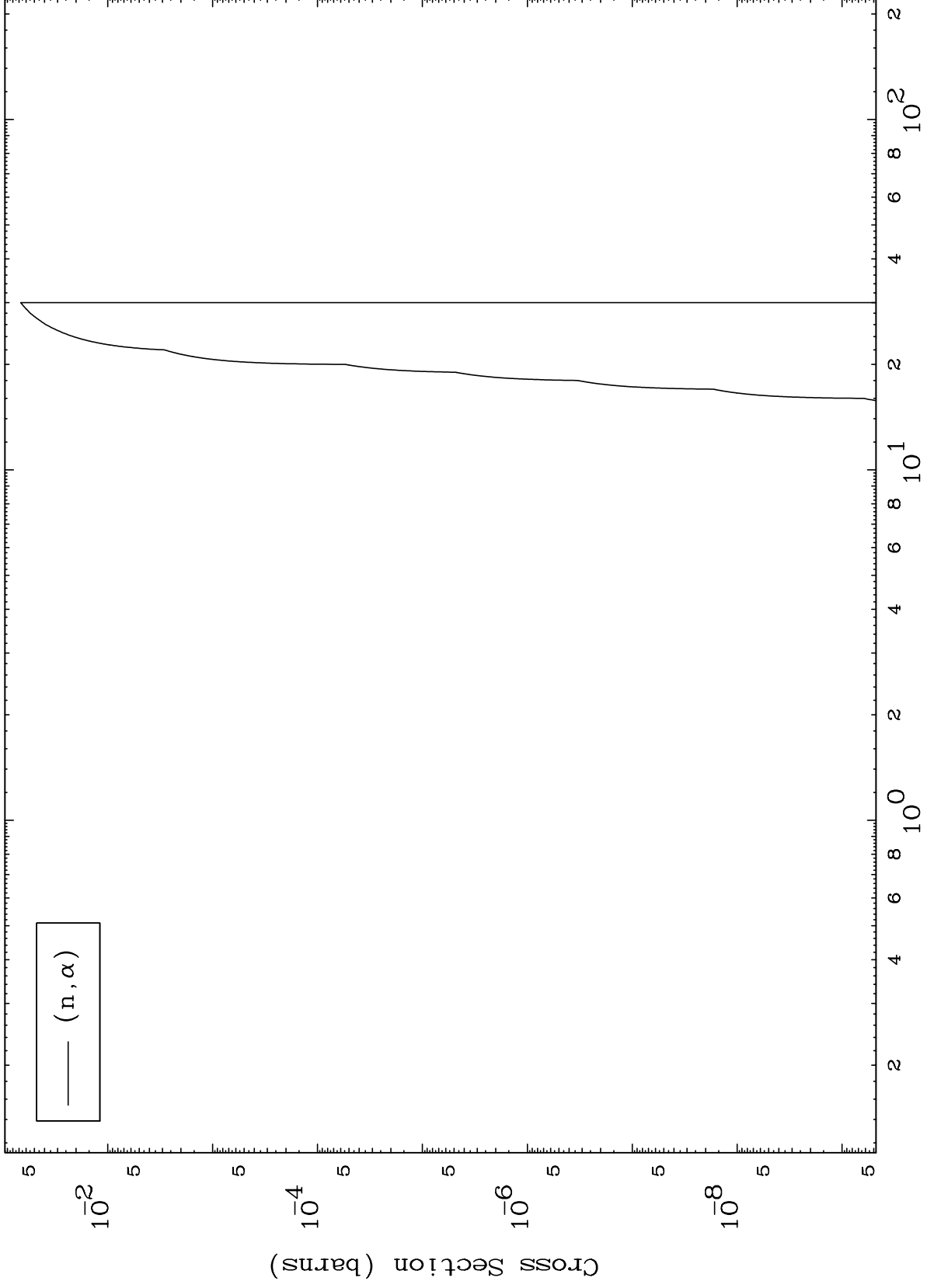


MAT 9337

( $\alpha, \alpha$ ) Levels

93-Np-234

0 Kelvin Cross Sections

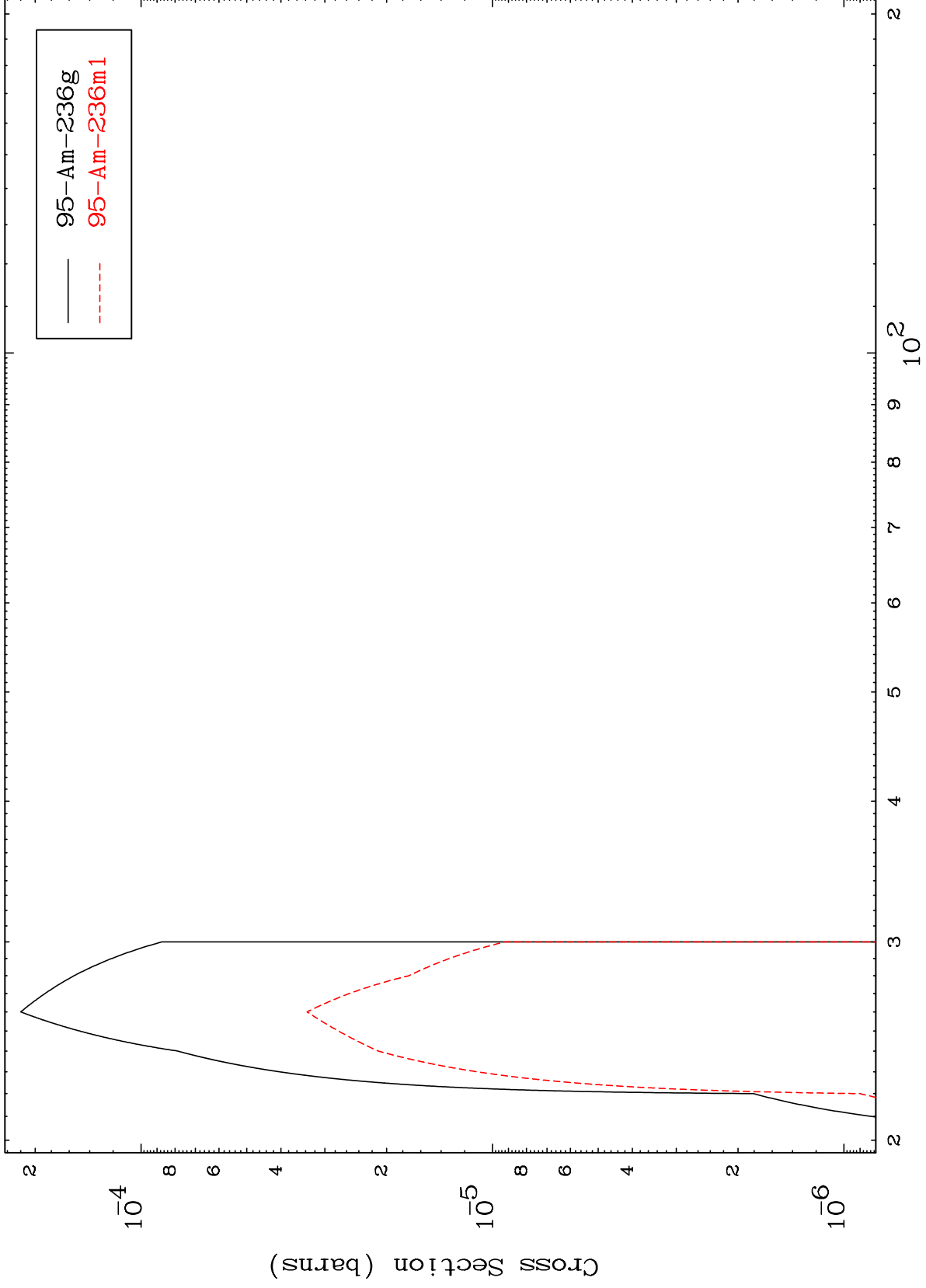


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(n,2n)

93-Np-234

Radionuclide Production Cross Section



9

Incident Energy (MeV)

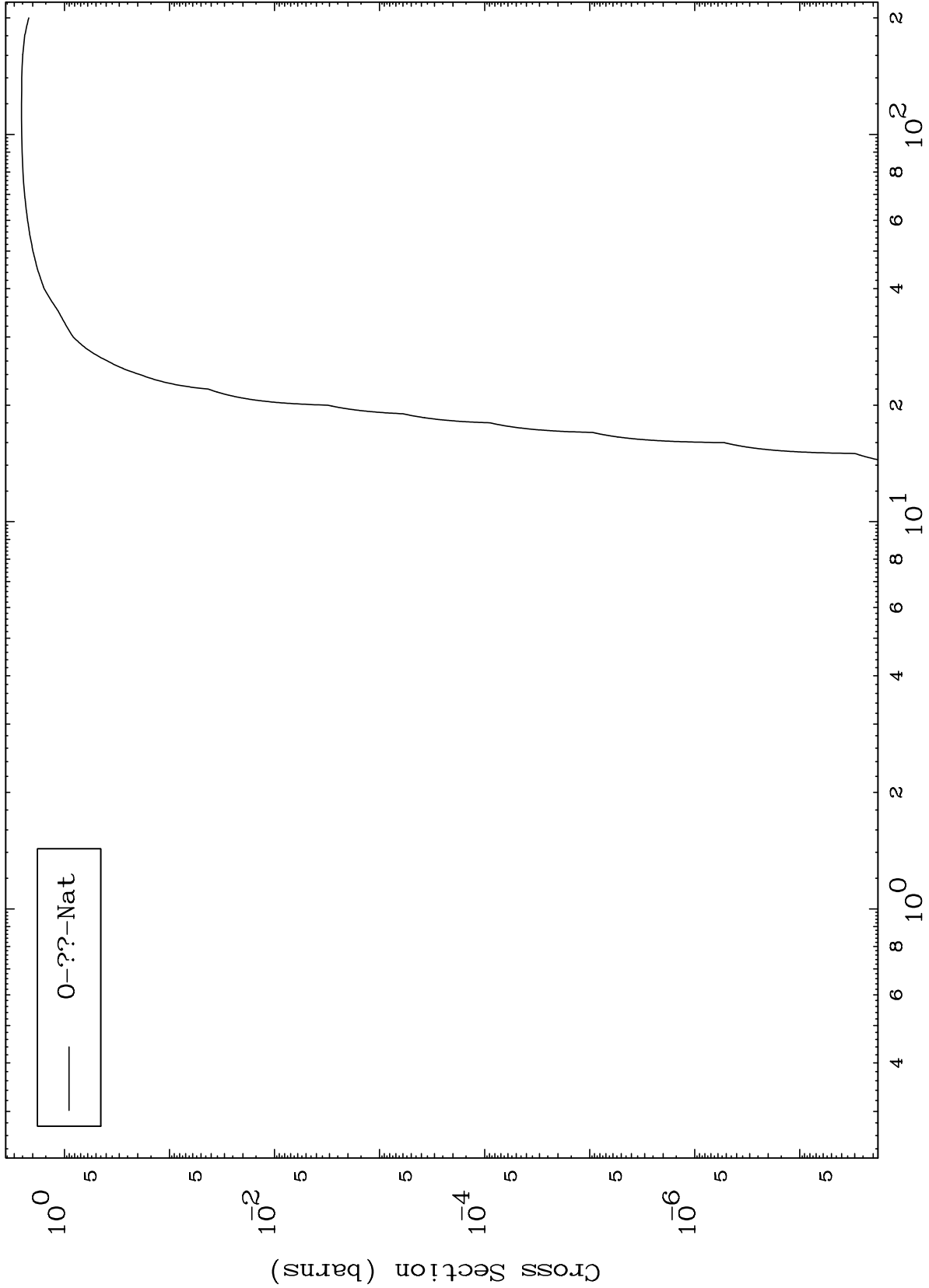
93-Np-234

MAT 9337

Fission

93-Np-234

Radionuclide Production Cross Section



10

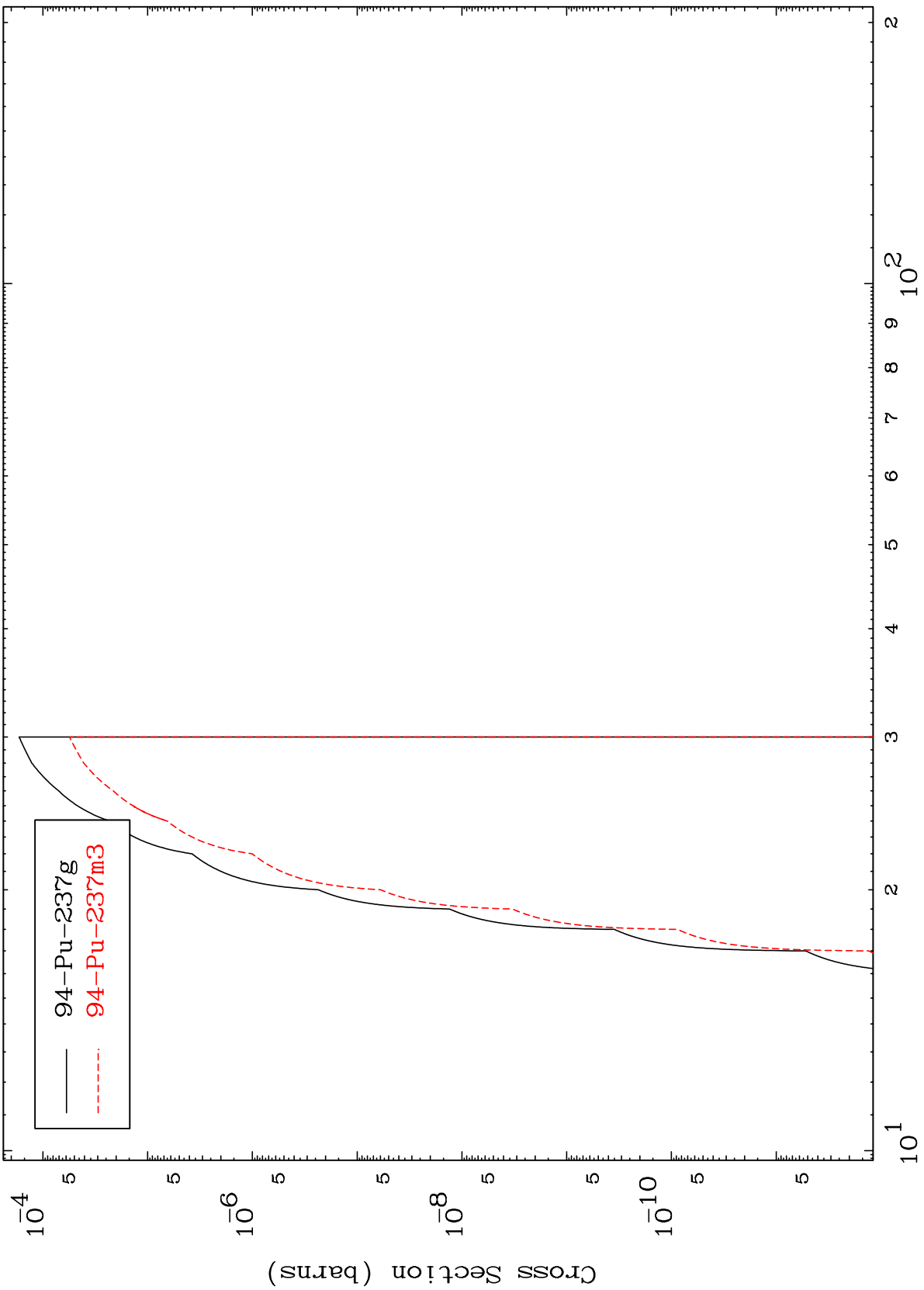
Incident Energy (MeV)

93-Np-234

MAT 9337

93-Np-234

(n,p)  
Radionuclide Production Cross Section



94-Pu-237g  
94-Pu-237m3

93-Np-234

Incident Energy (MeV)

11