

Program EVALPLOT  
(Version 2021-1)

by

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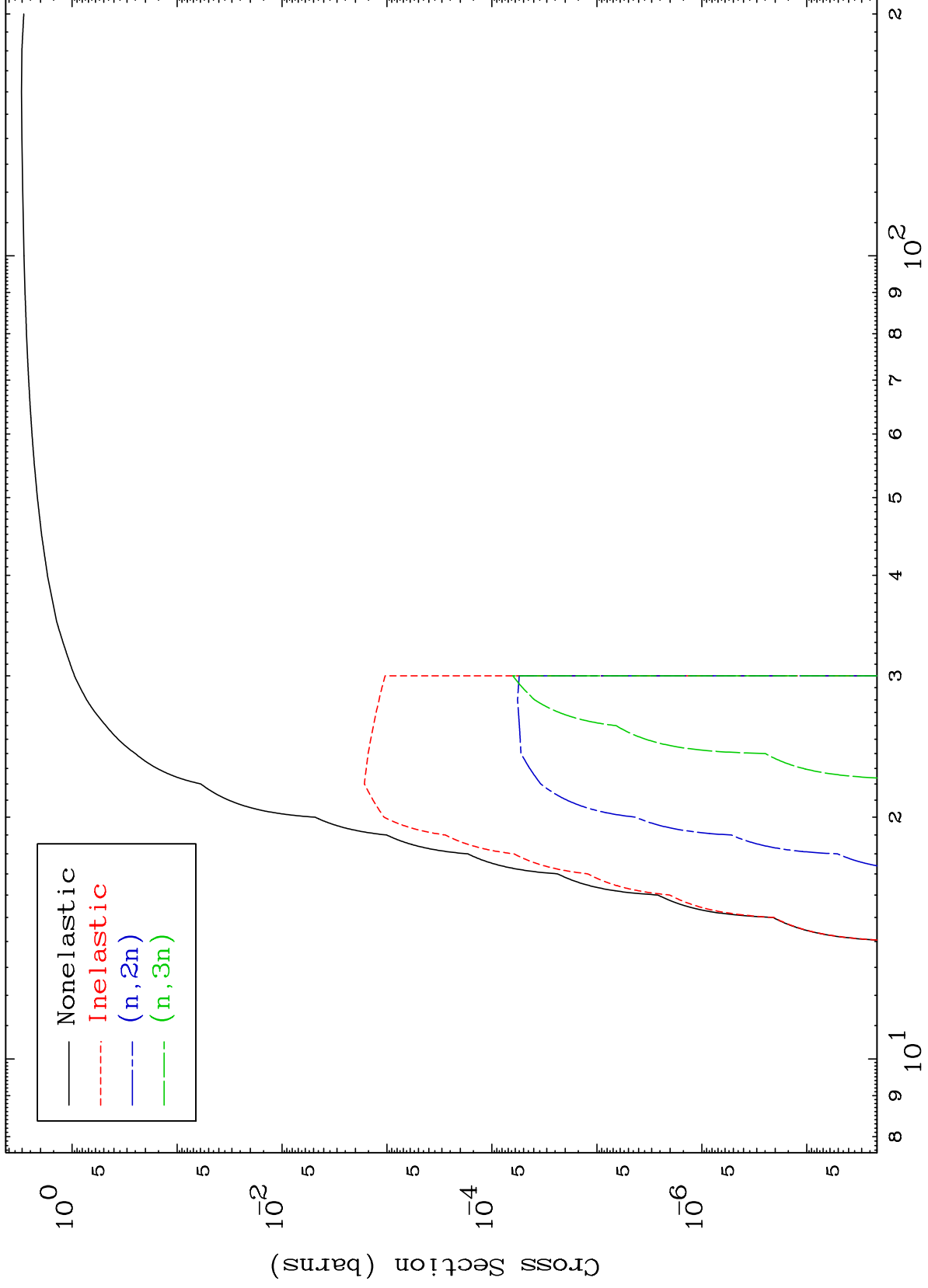
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9364

0 Kelvin Cross Sections

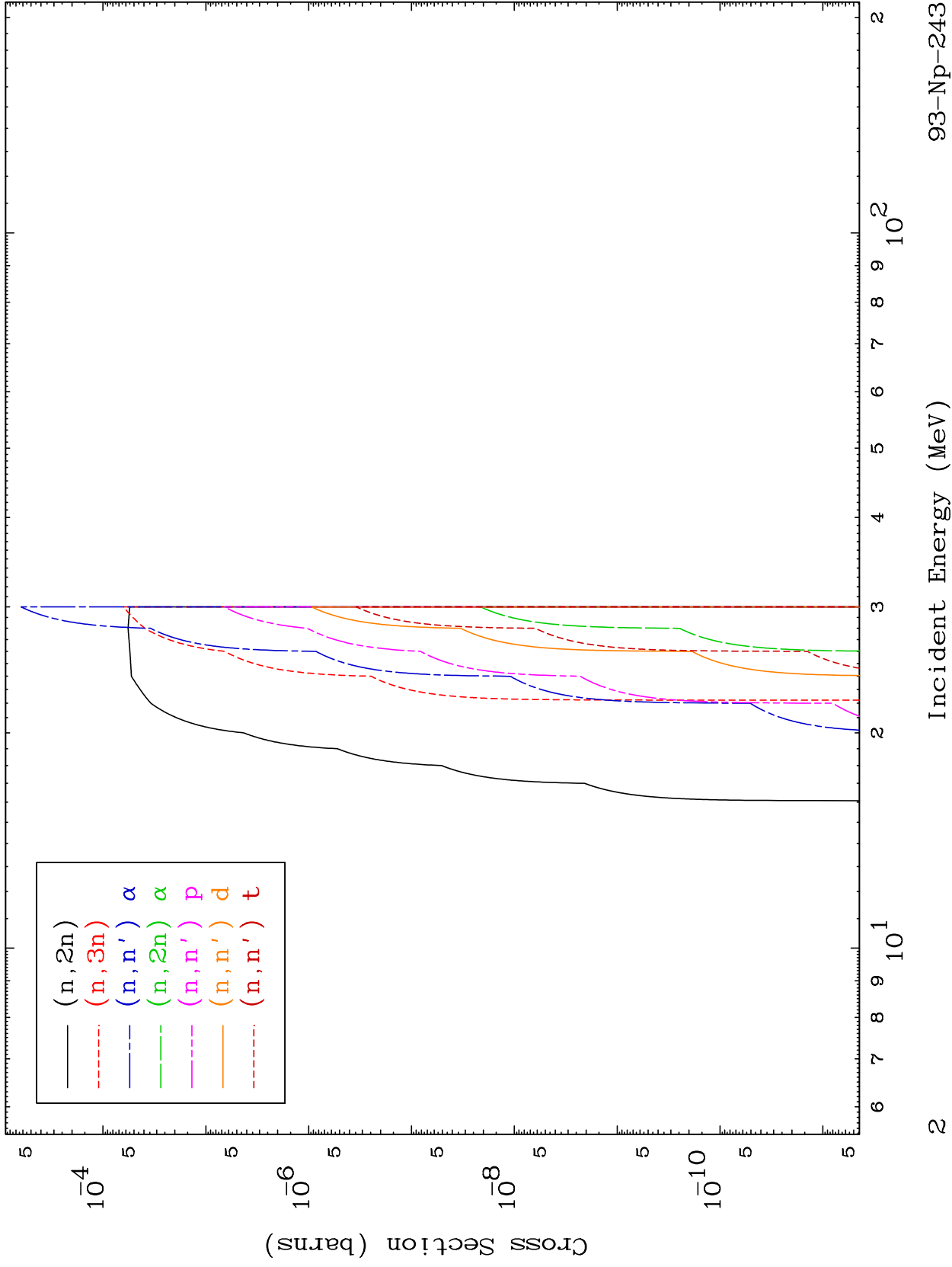
93-Np-243



Incident Energy (MeV)

93-Np-243

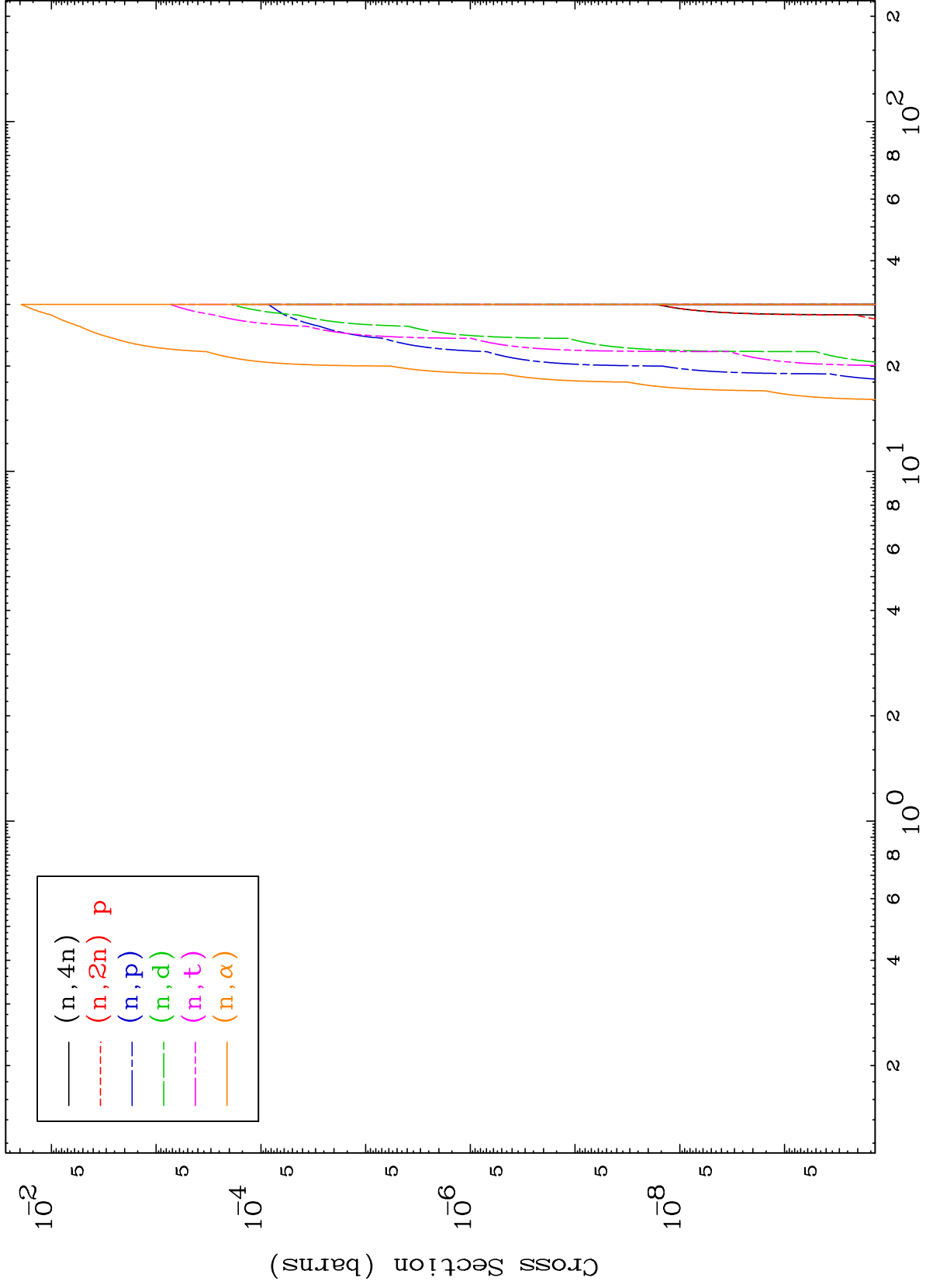
1



MAT 9364

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

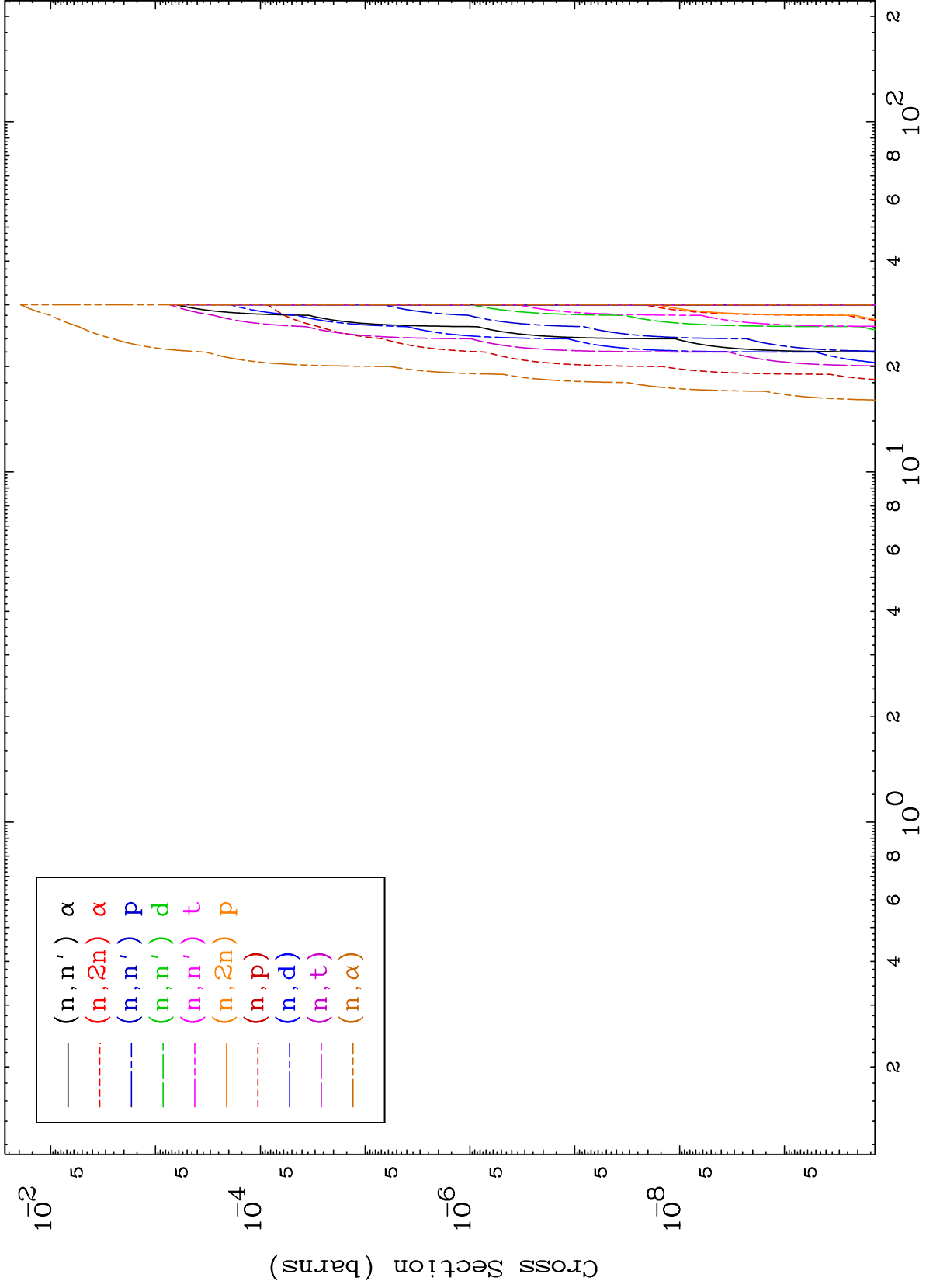
93-Np-243

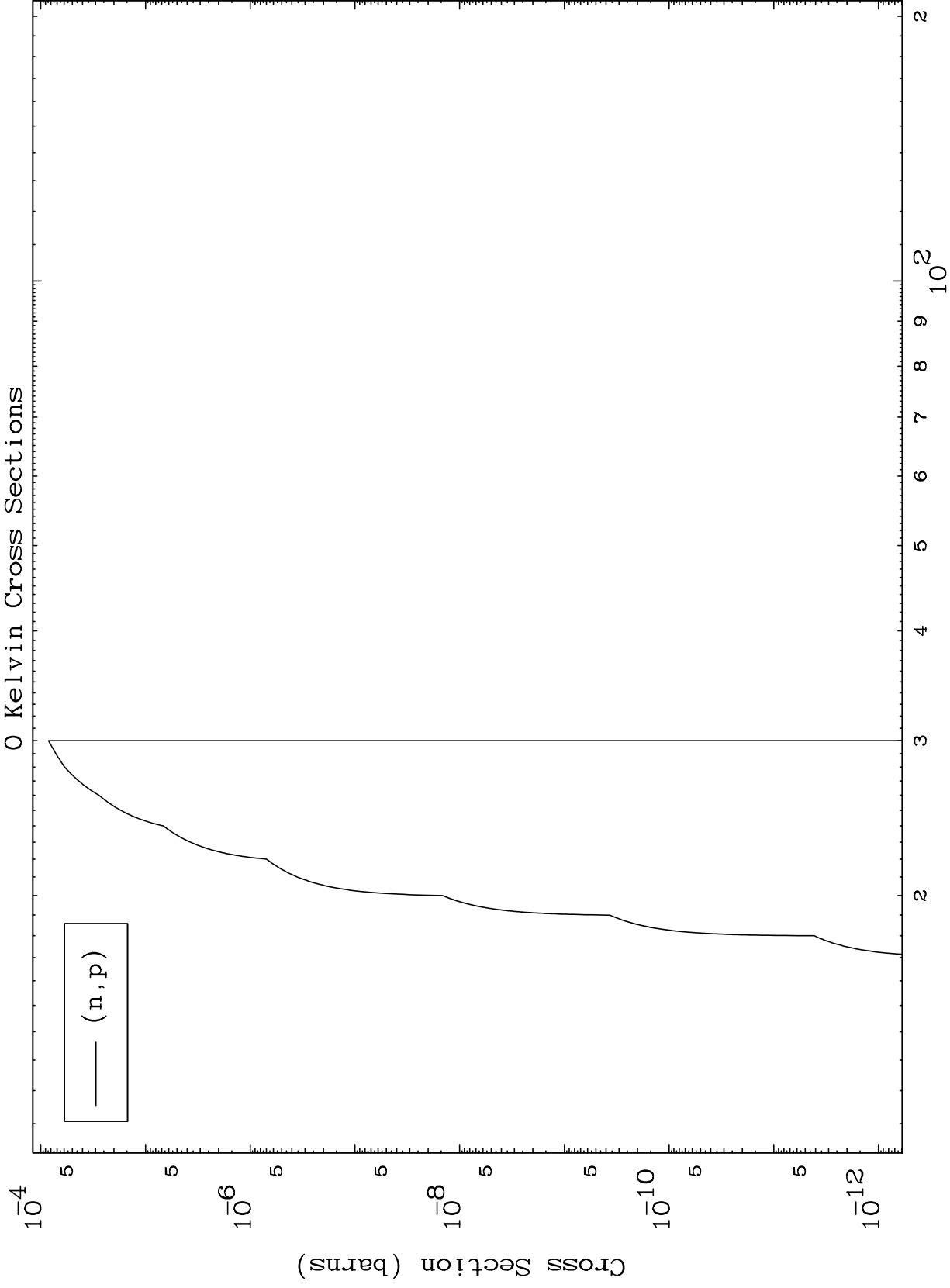


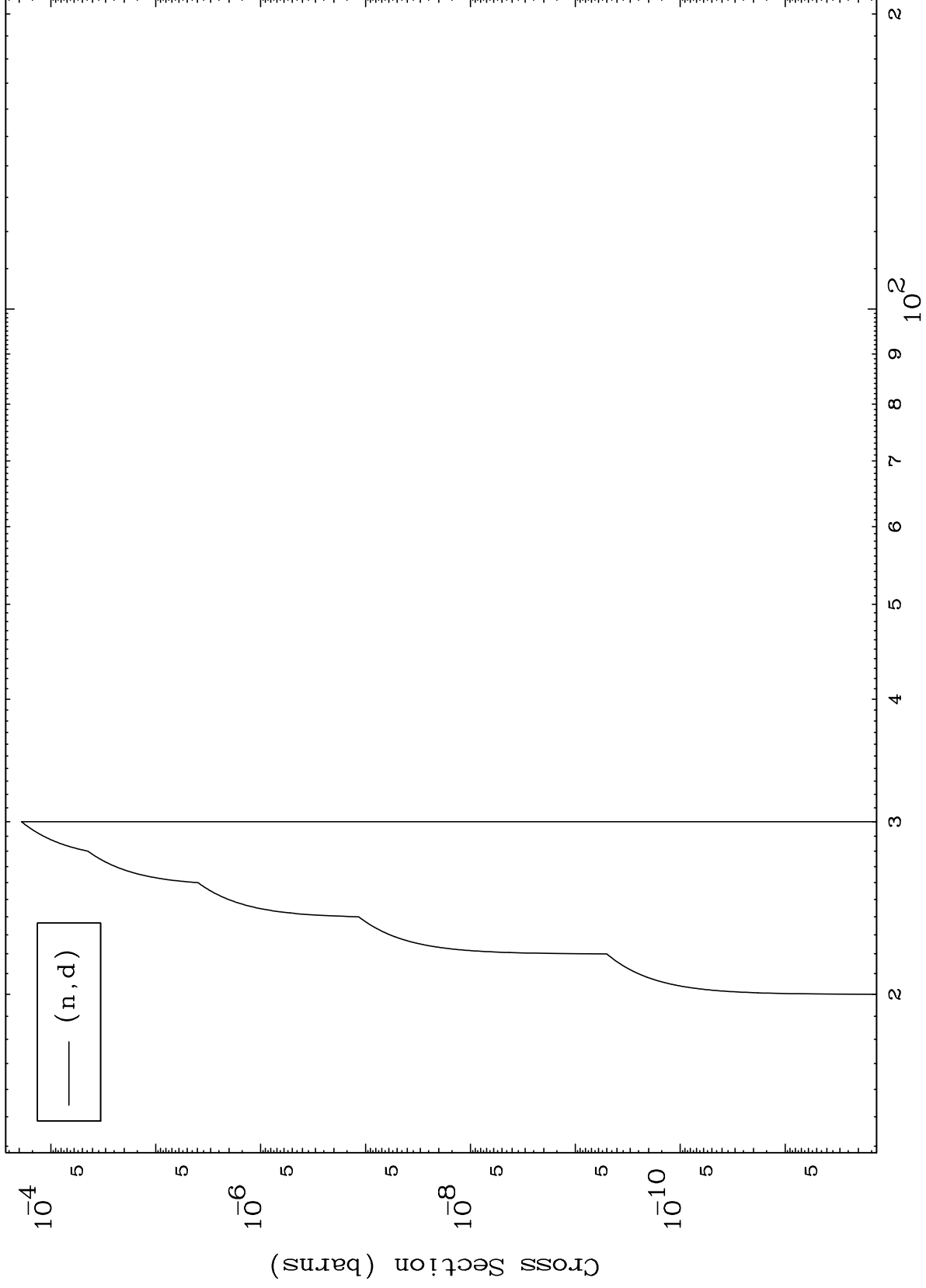
MAT 9364

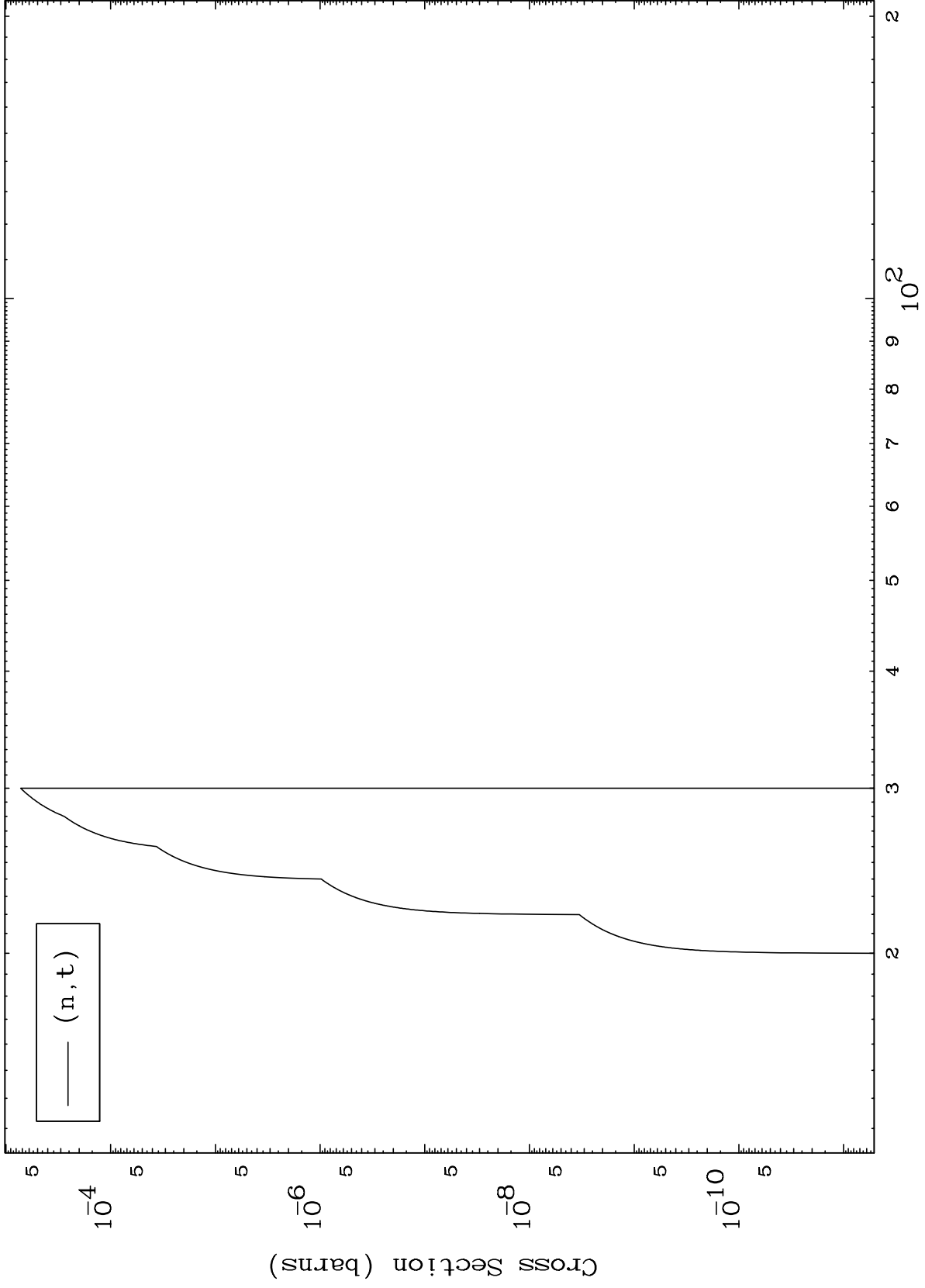
$\alpha$  Charged Particle  
0 Kelvin Cross Sections

93-Np-243





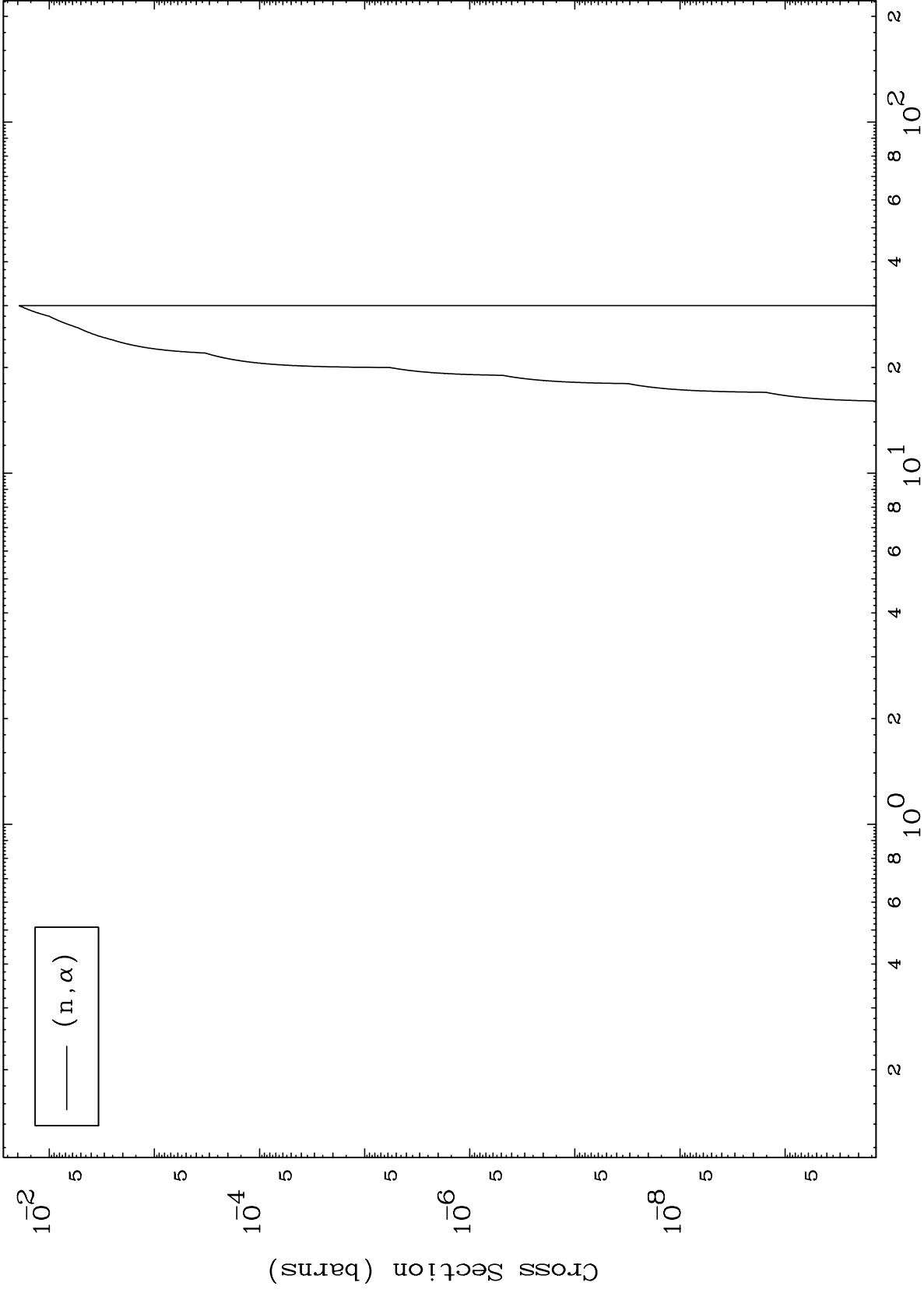




MAT 9364

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

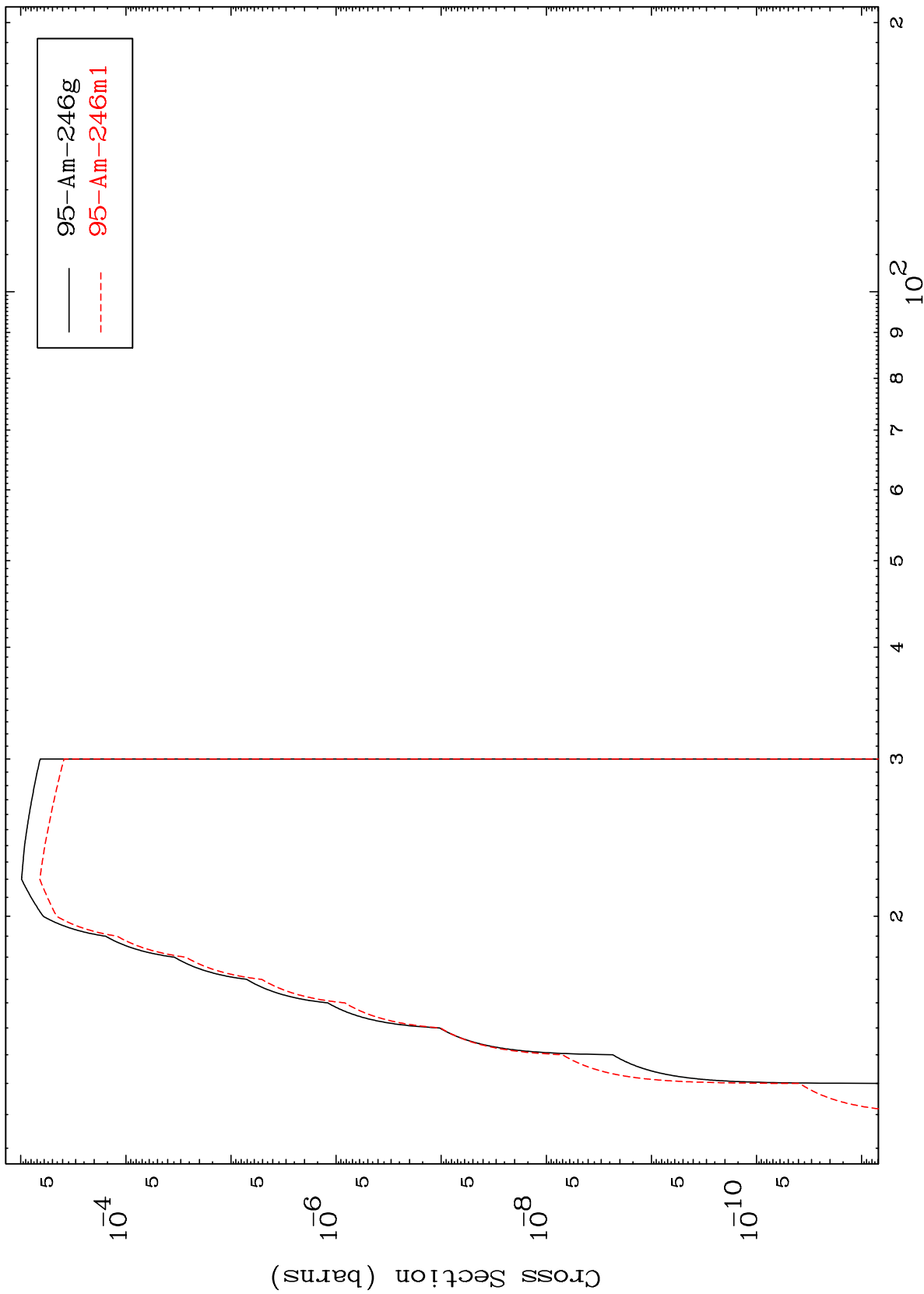
93-Np-243



MAT 9364

Inelastic  
Radionuclide Production Cross Section

93-Np-243



9

Incident Energy (MeV)

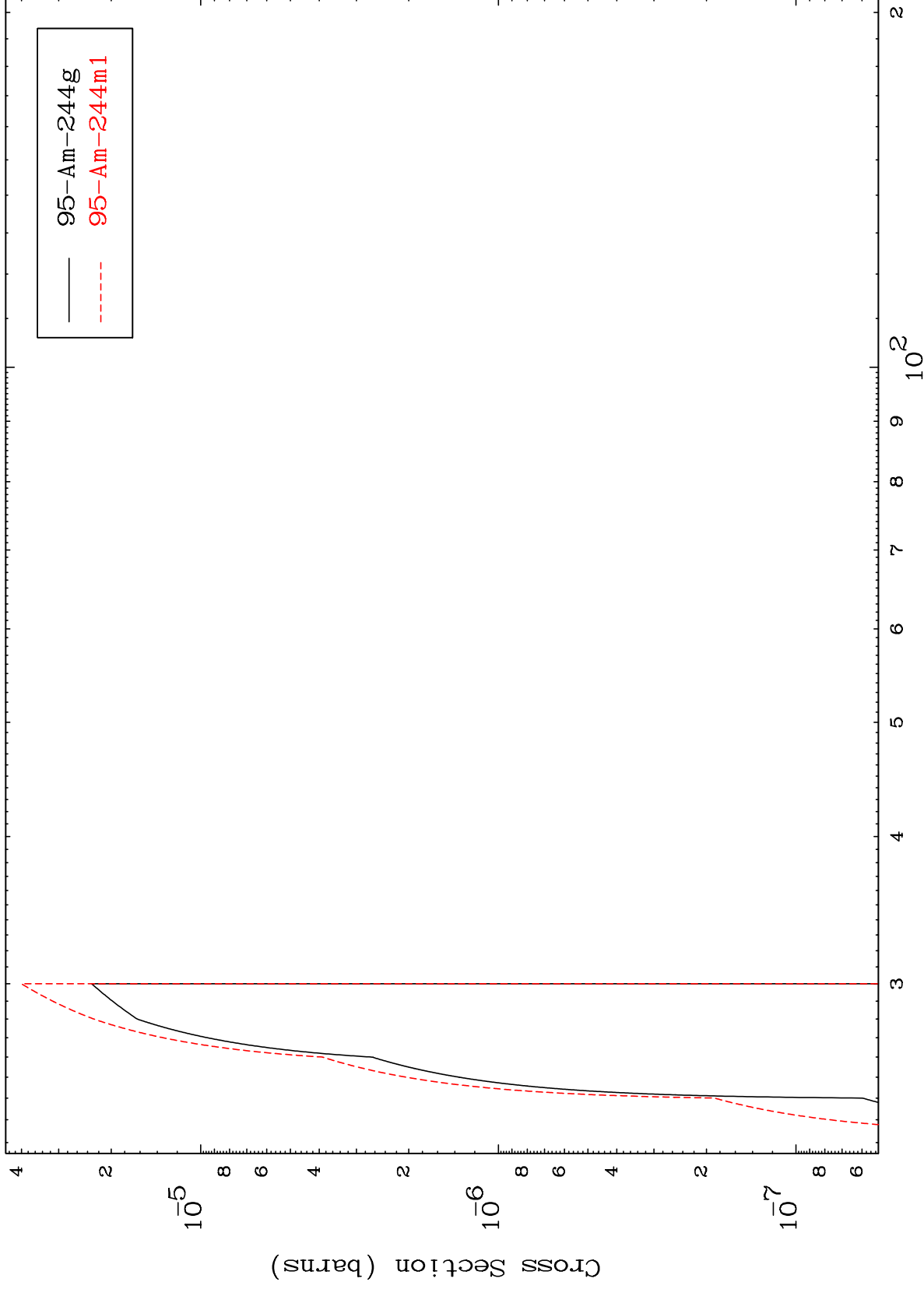
93-Np-243

MAT 9364

(n,3n)

93-Np-243

Radionuclide Production Cross Section



10

Incident Energy (MeV)

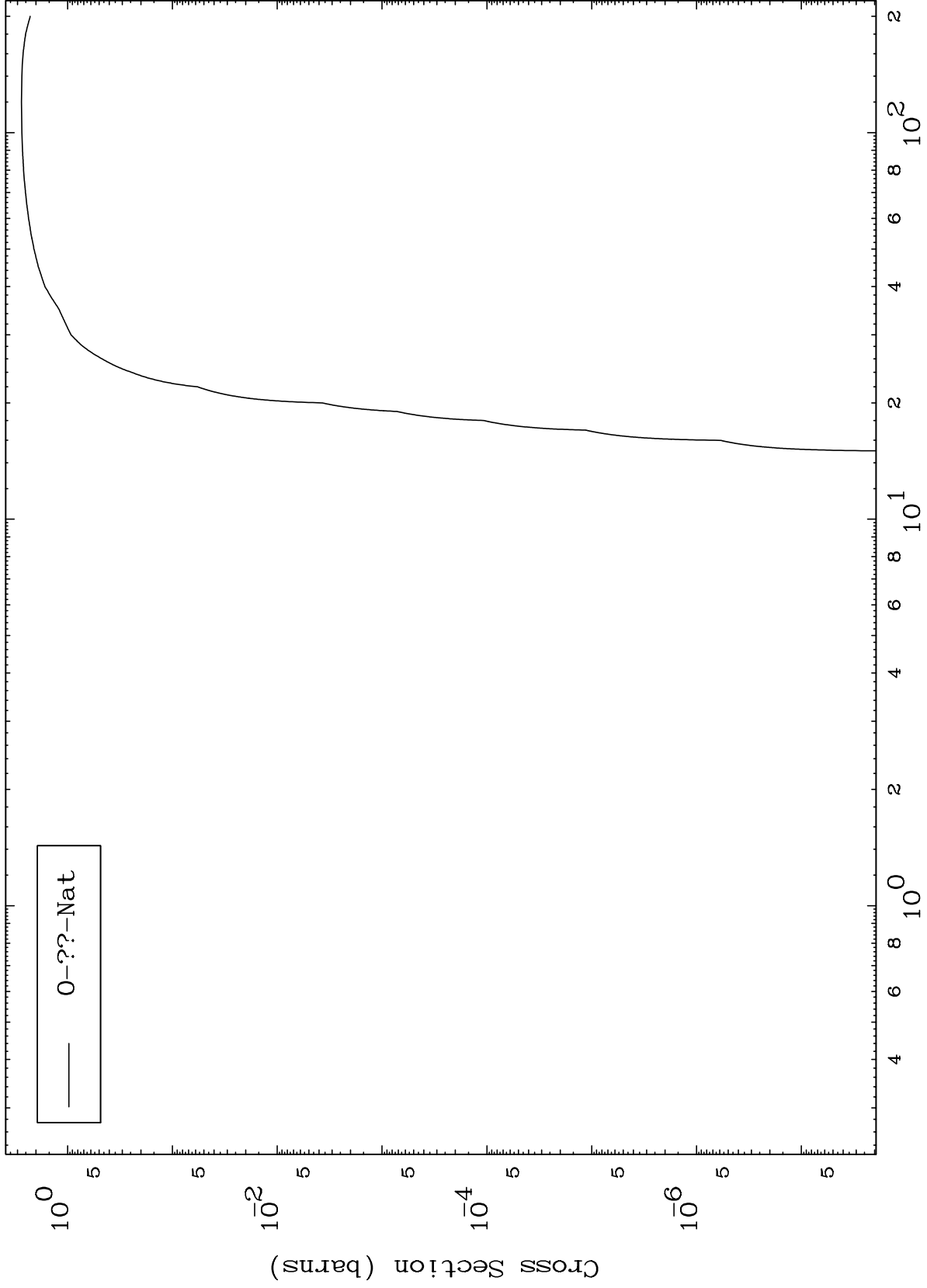
93-Np-243

MAT 9364

Fission

93-Np-243

Radionuclide Production Cross Section

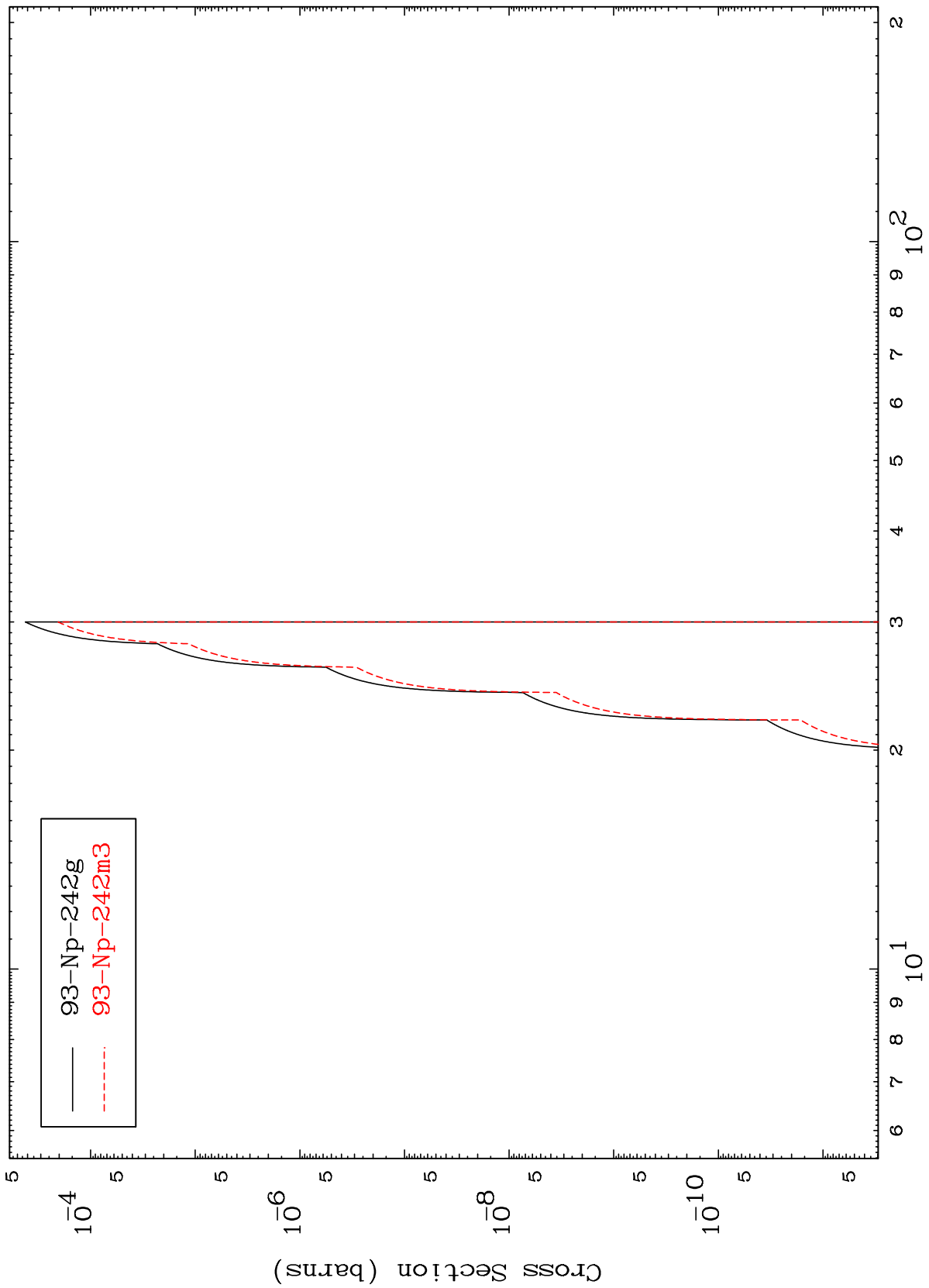


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93-Np-243

(n,n')  $\alpha$

Radionuclide Production Cross Section



93-Np-242g  
93-Np-242m3

93-Np-243

Incident Energy (MeV)

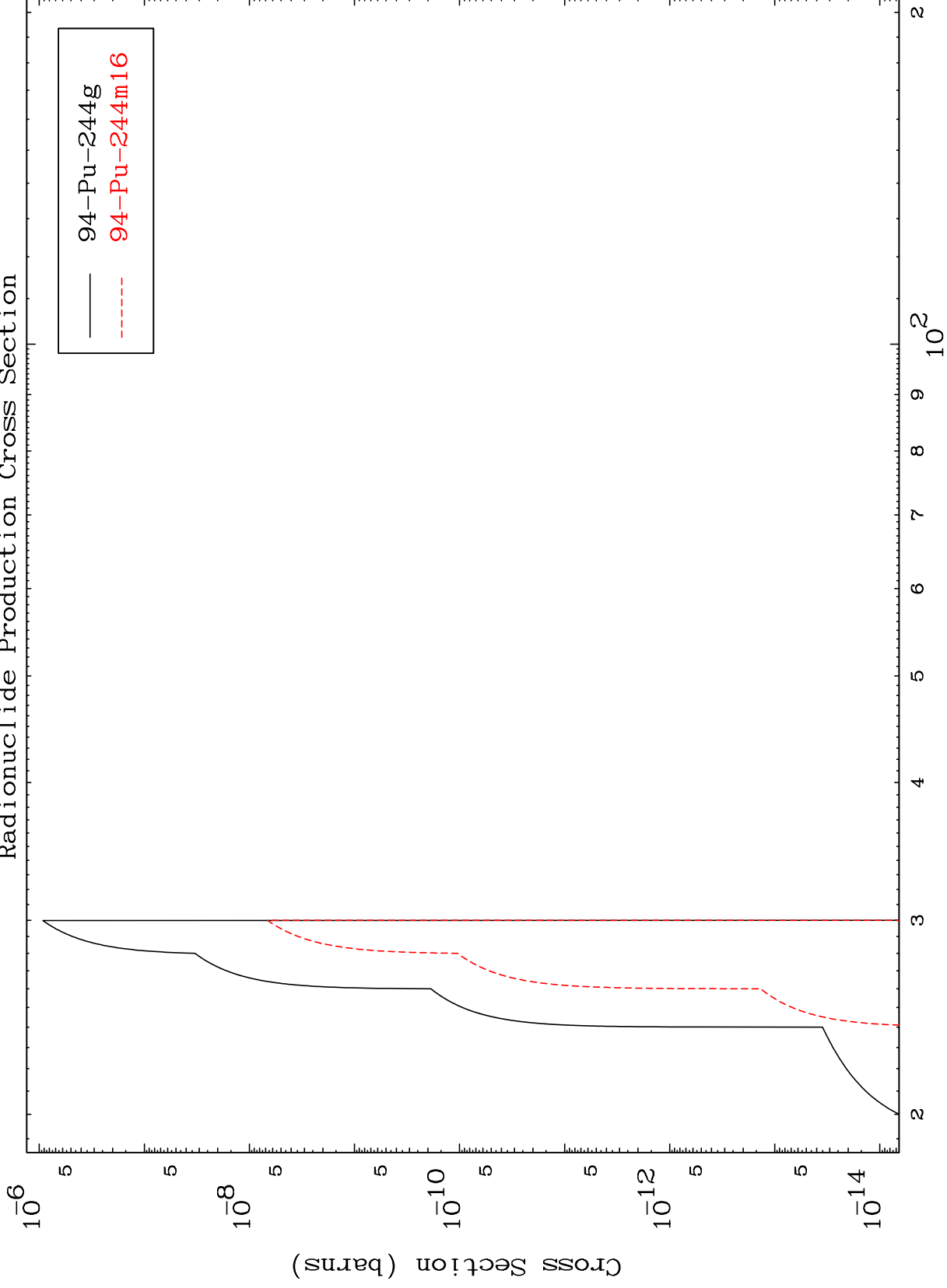
12

MAT 9364

(n,n') d

93-Np-243

Radionuclide Production Cross Section



13

Incident Energy (MeV)

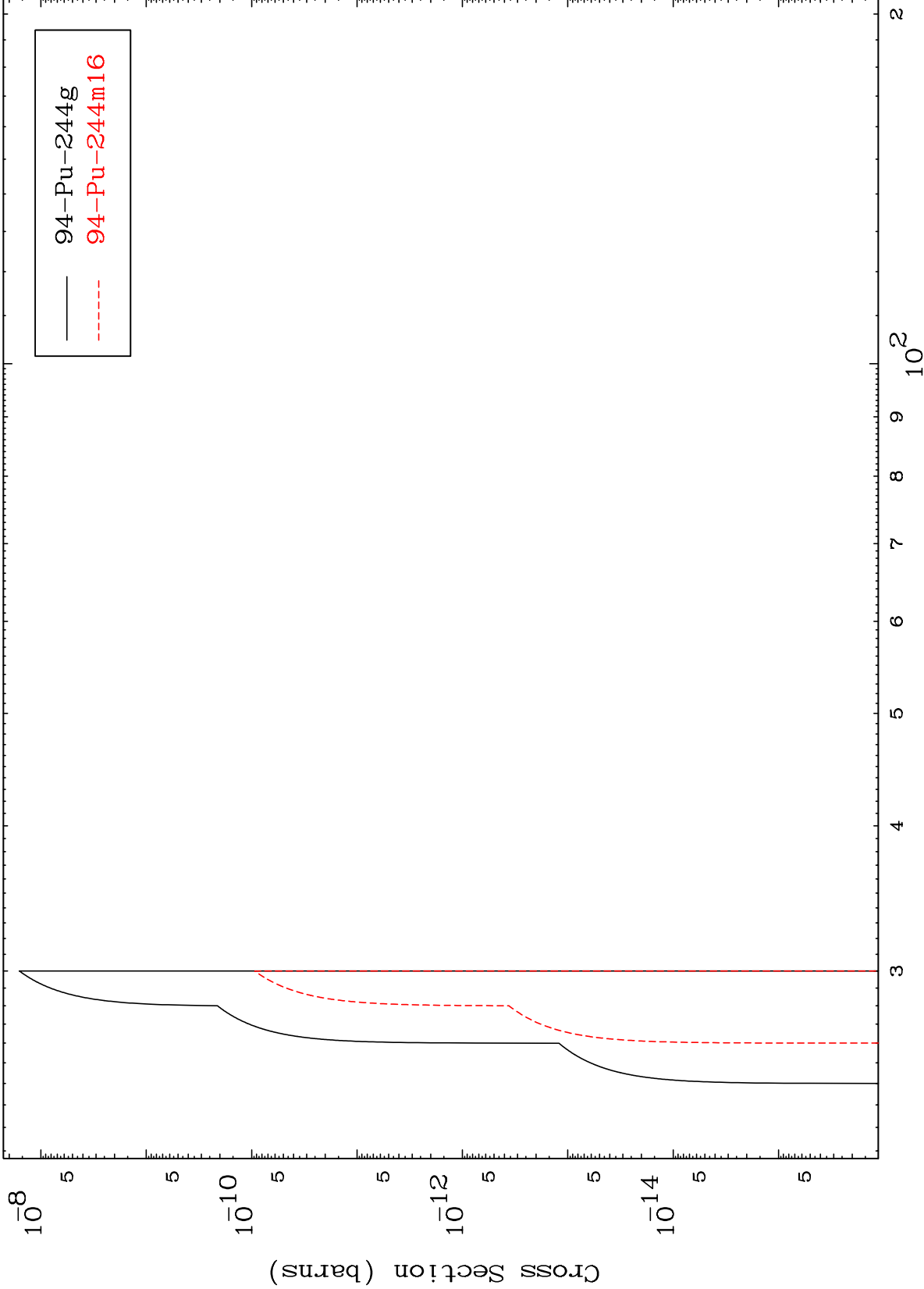
93-Np-243

MAT 9364

(n,2n) p

93-Np-243

Radionuclide Production Cross Section



14

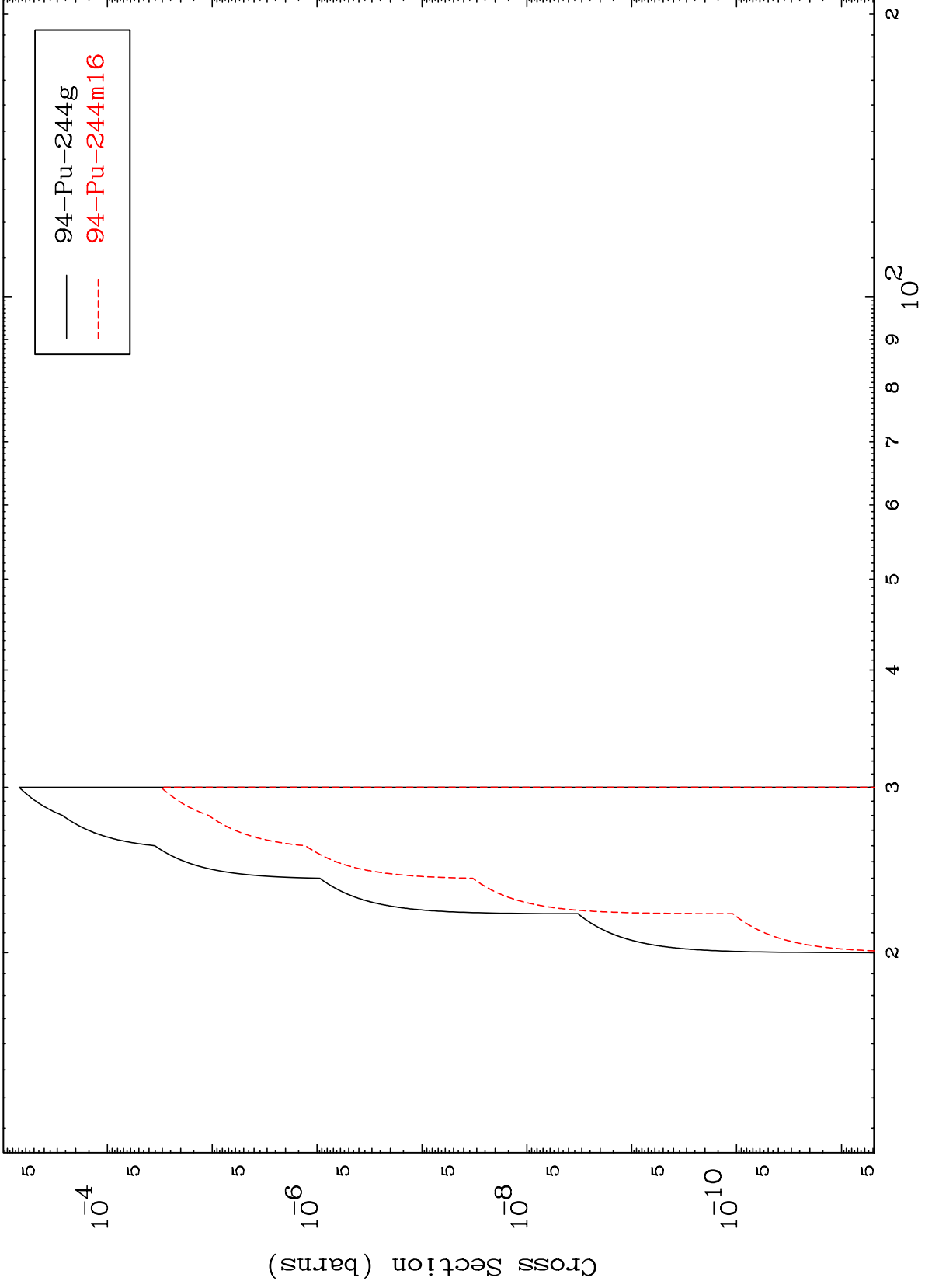
Incident Energy (MeV)

93-Np-243

MAT 9364

93-Np-243

(n, t)  
Radionuclide Production Cross Section



94-Pu-244g  
94-Pu-244m16

15

Incident Energy (MeV)

93-Np-243