

Program EVALPLOT  
(Version 2021-1)

by

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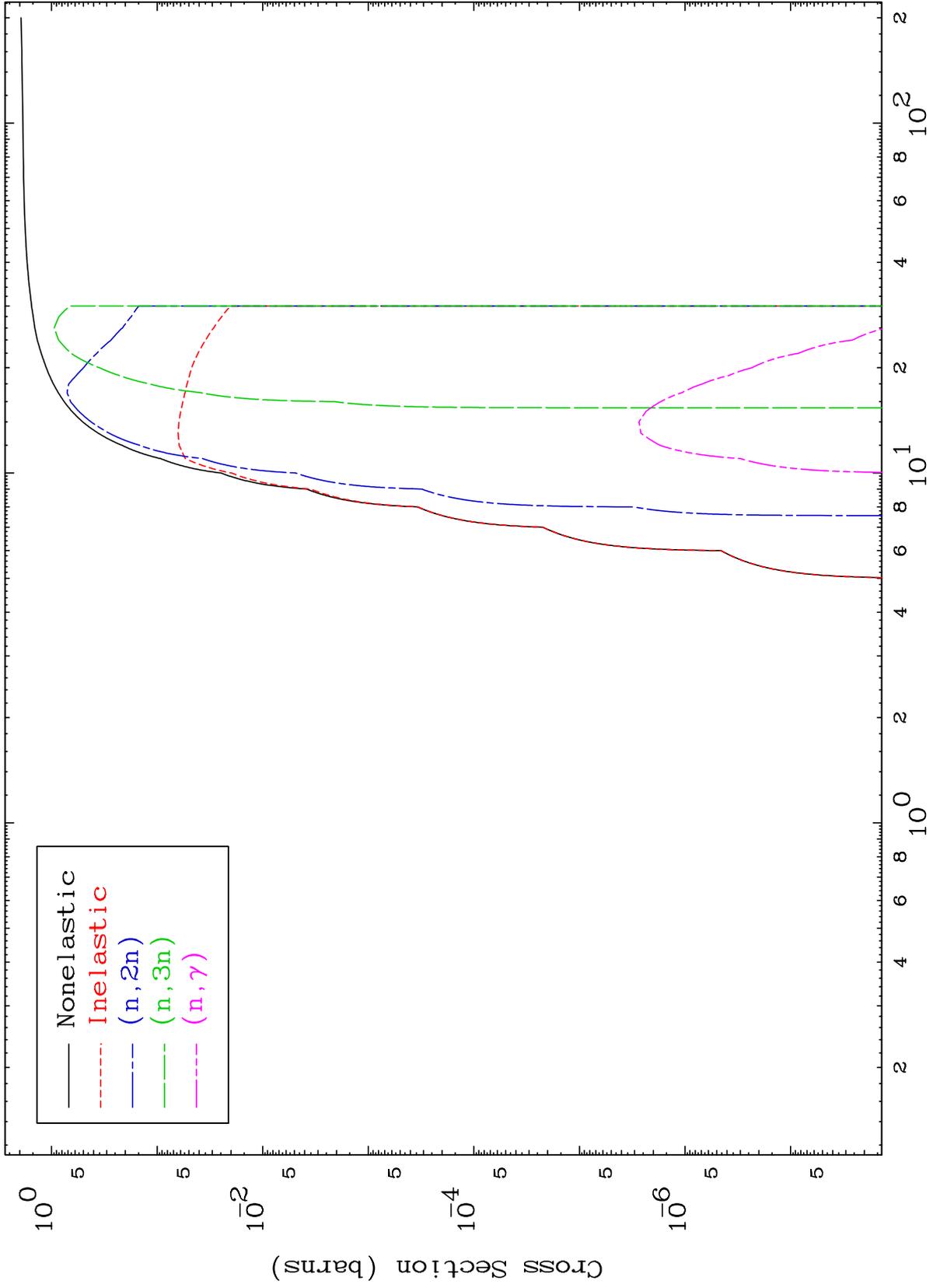
Press Mouse Button to Start

MAT 3743

$\alpha$  Major

37-Rb-91

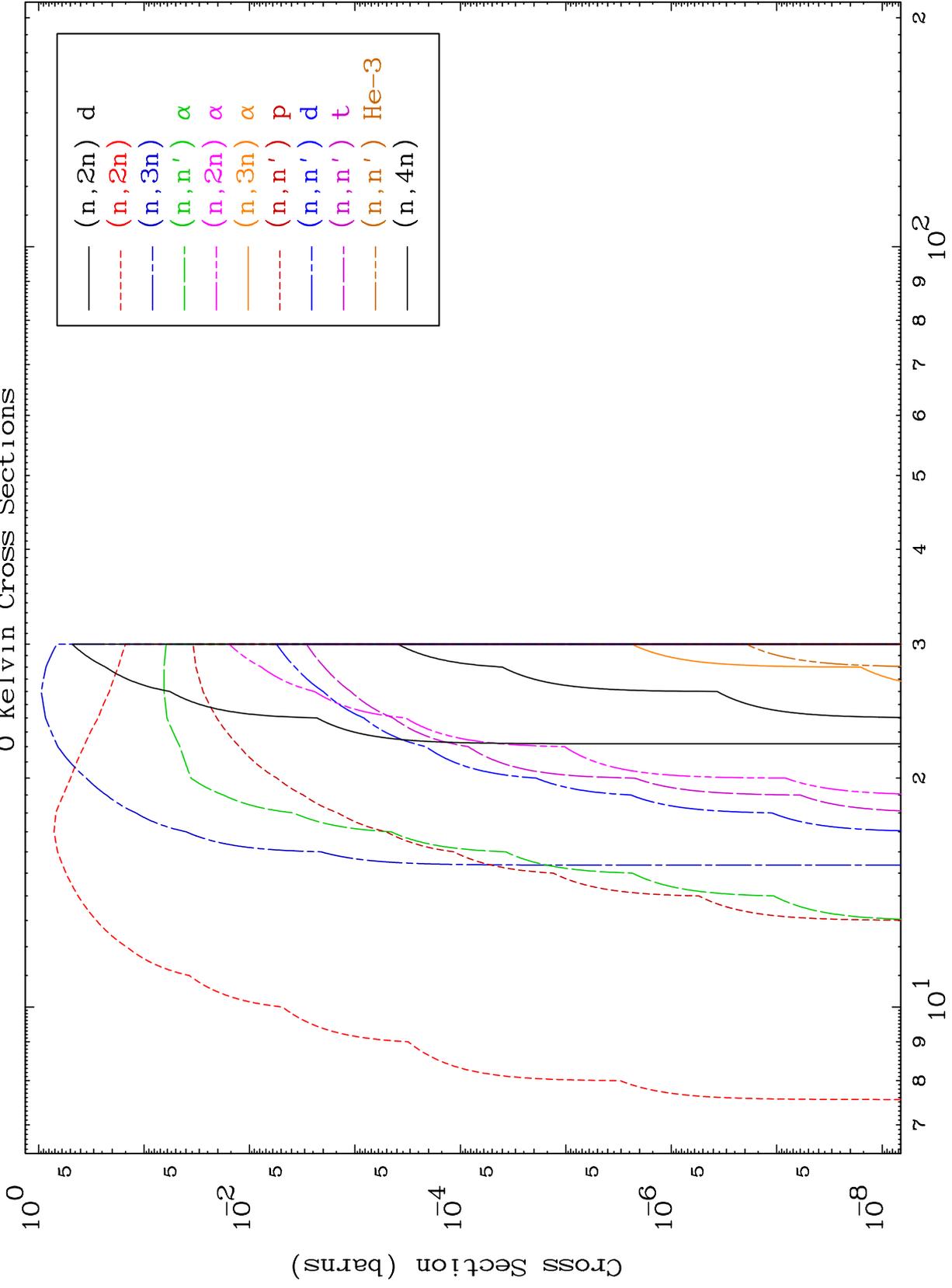
0 Kelvin Cross Sections



MAT 3743

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

37-Rb-91

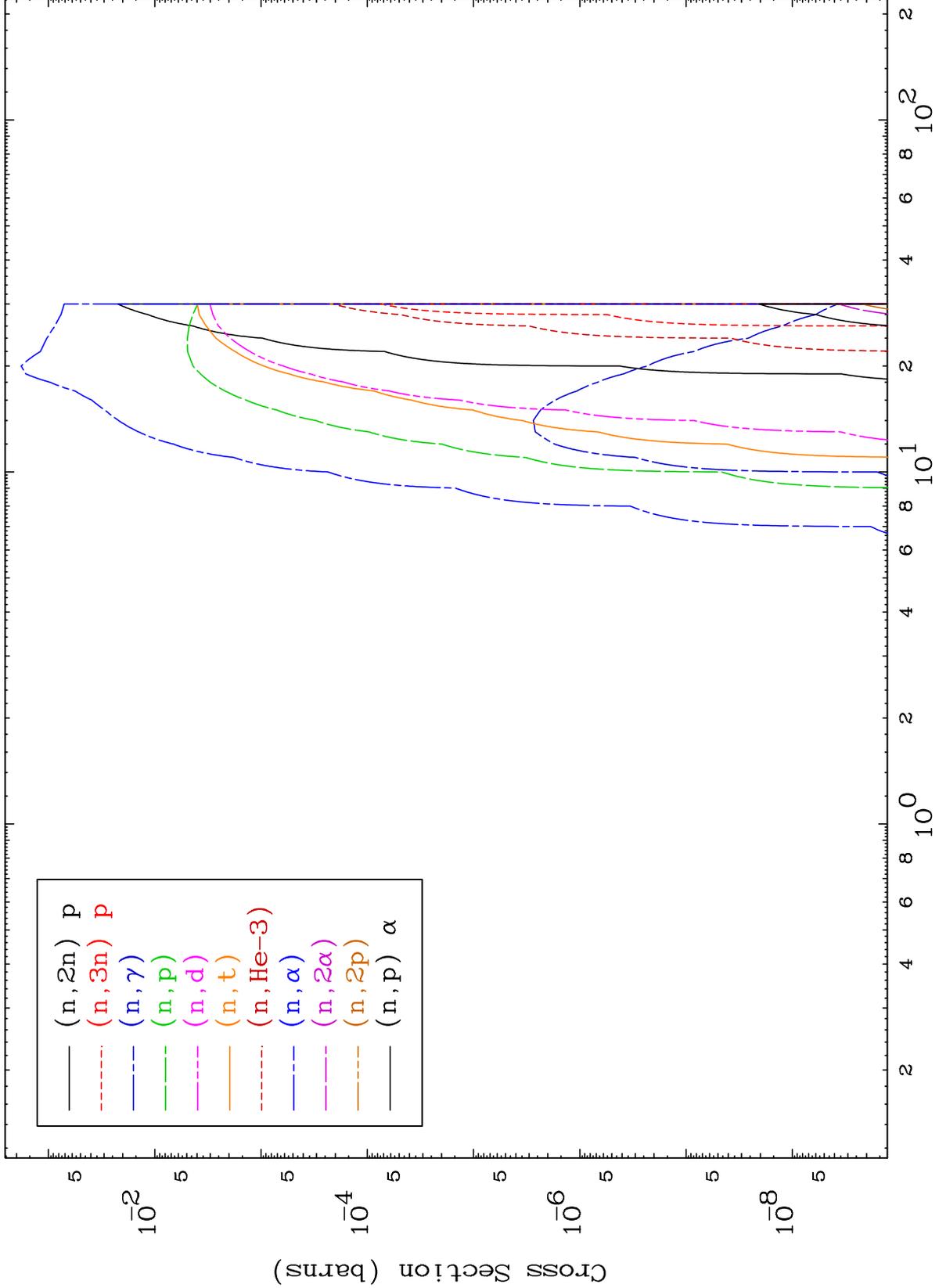


2

Incident Energy (MeV)

37-Rb-91

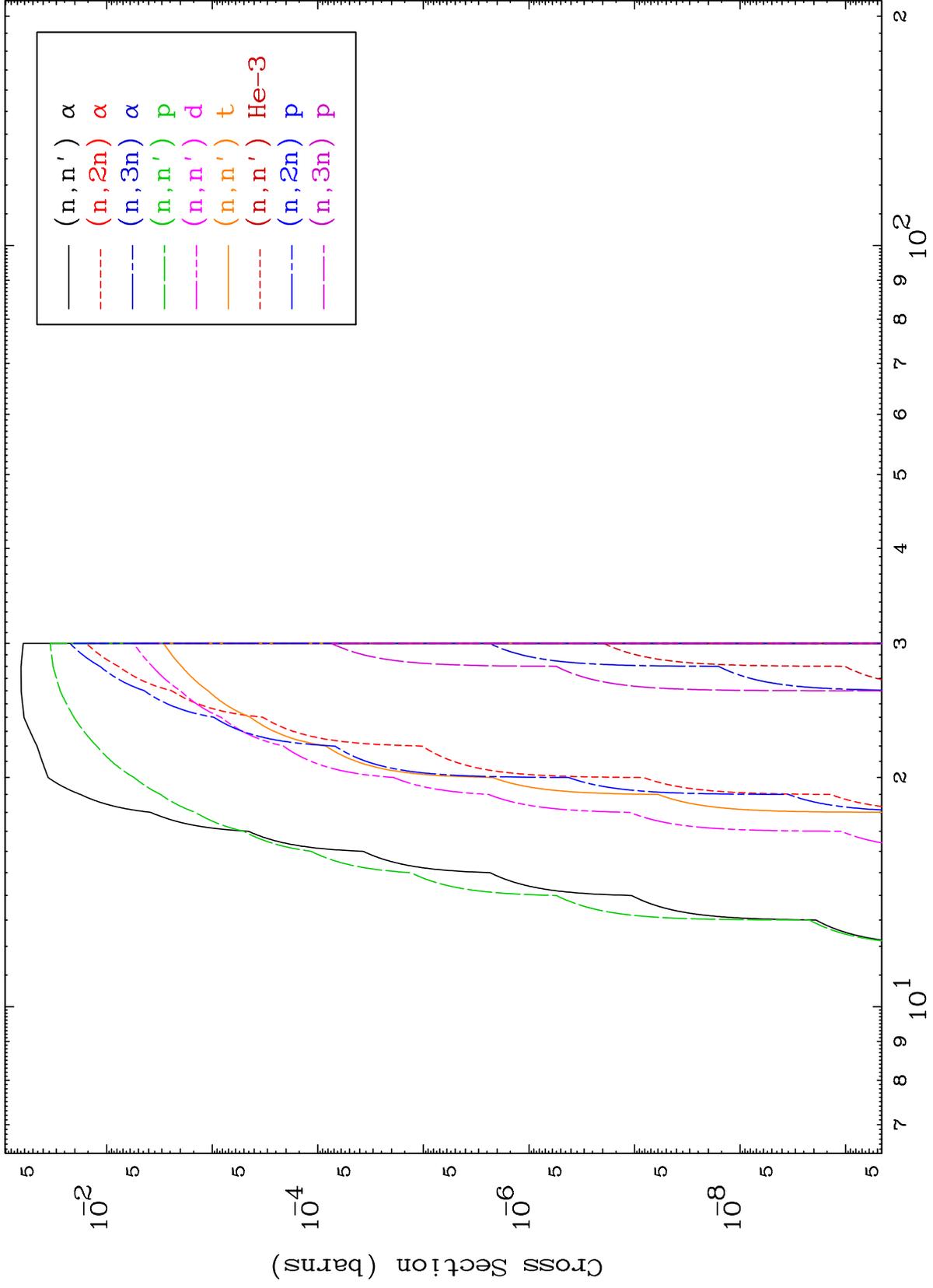
0 Kelvin Cross Sections



MAT 3743

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

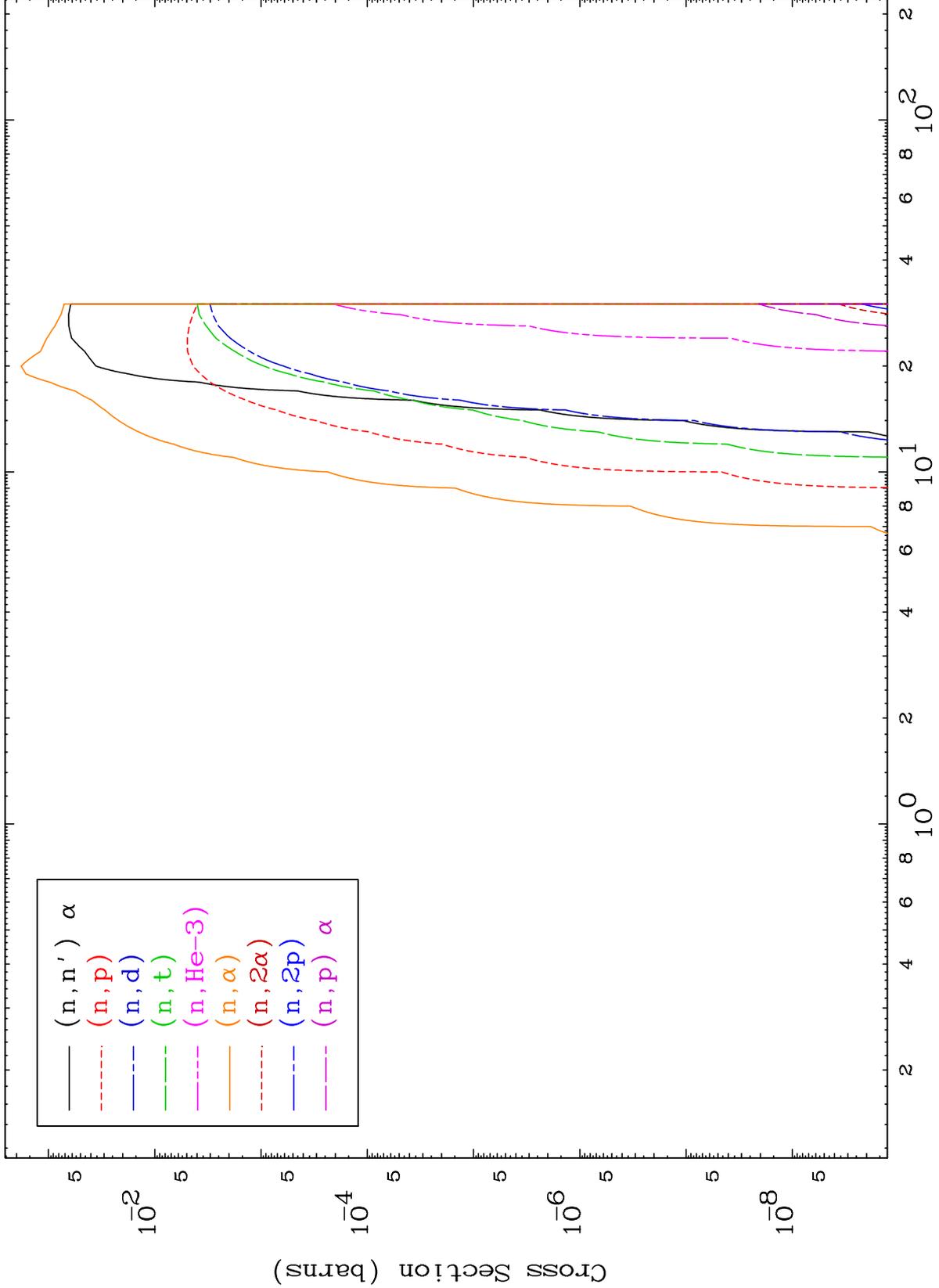
37-Rb-91



4

Incident Energy (MeV)

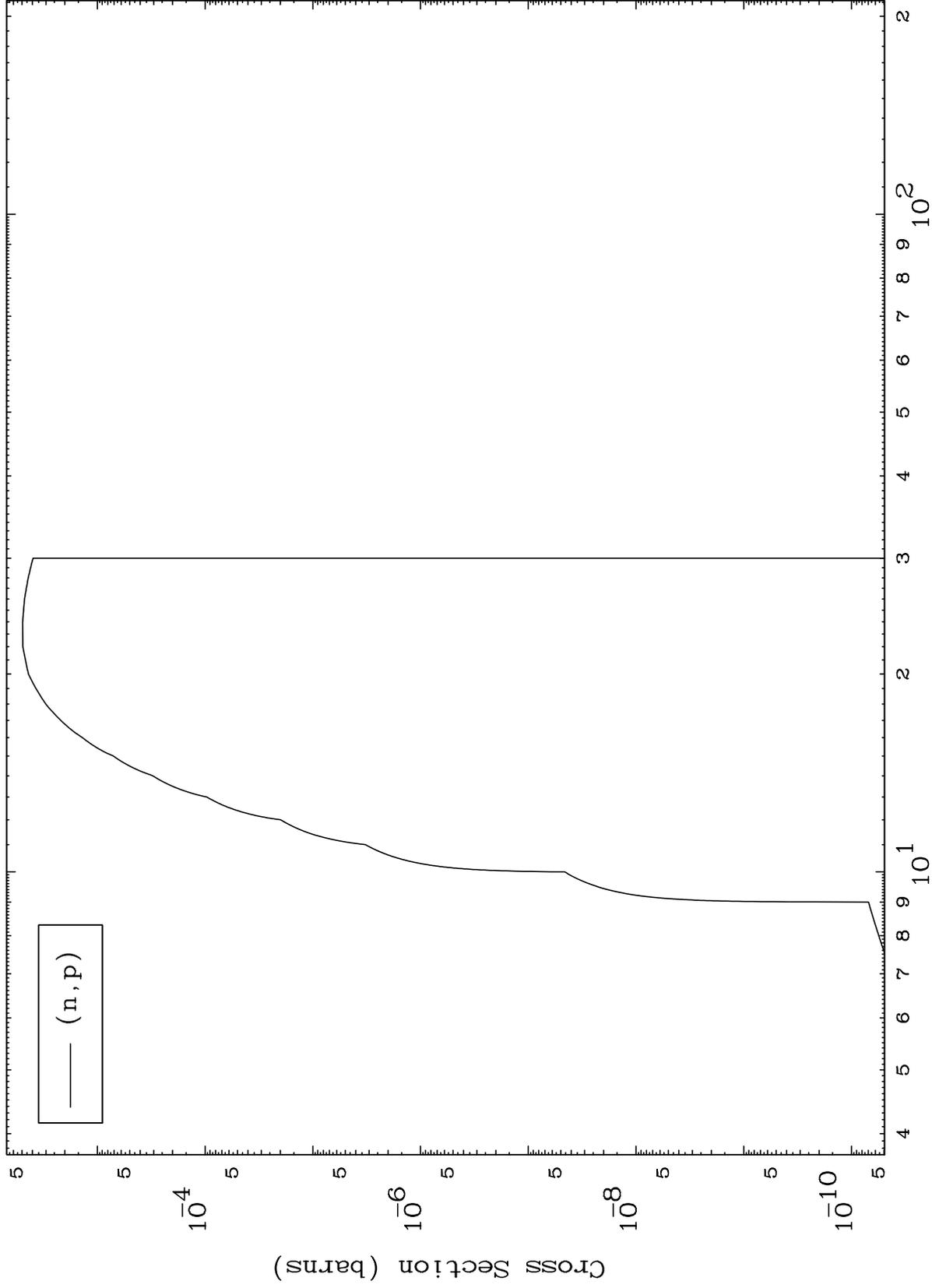
37-Rb-91



MAT 3743

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

37-Rb-91



6

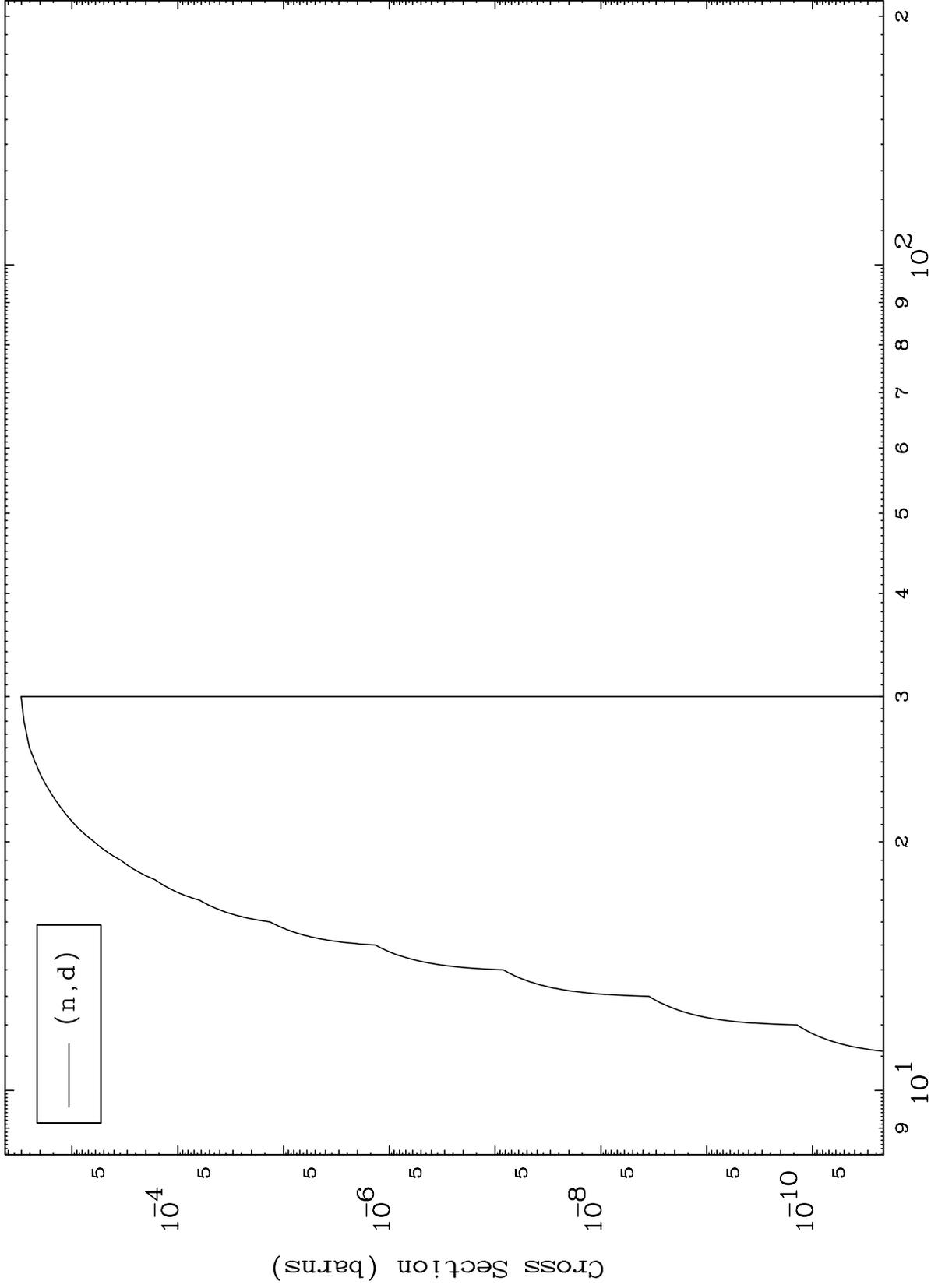
Incident Energy (MeV)

37-Rb-91

MAT 3743

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

$^{37}\text{Rb-91}$



7

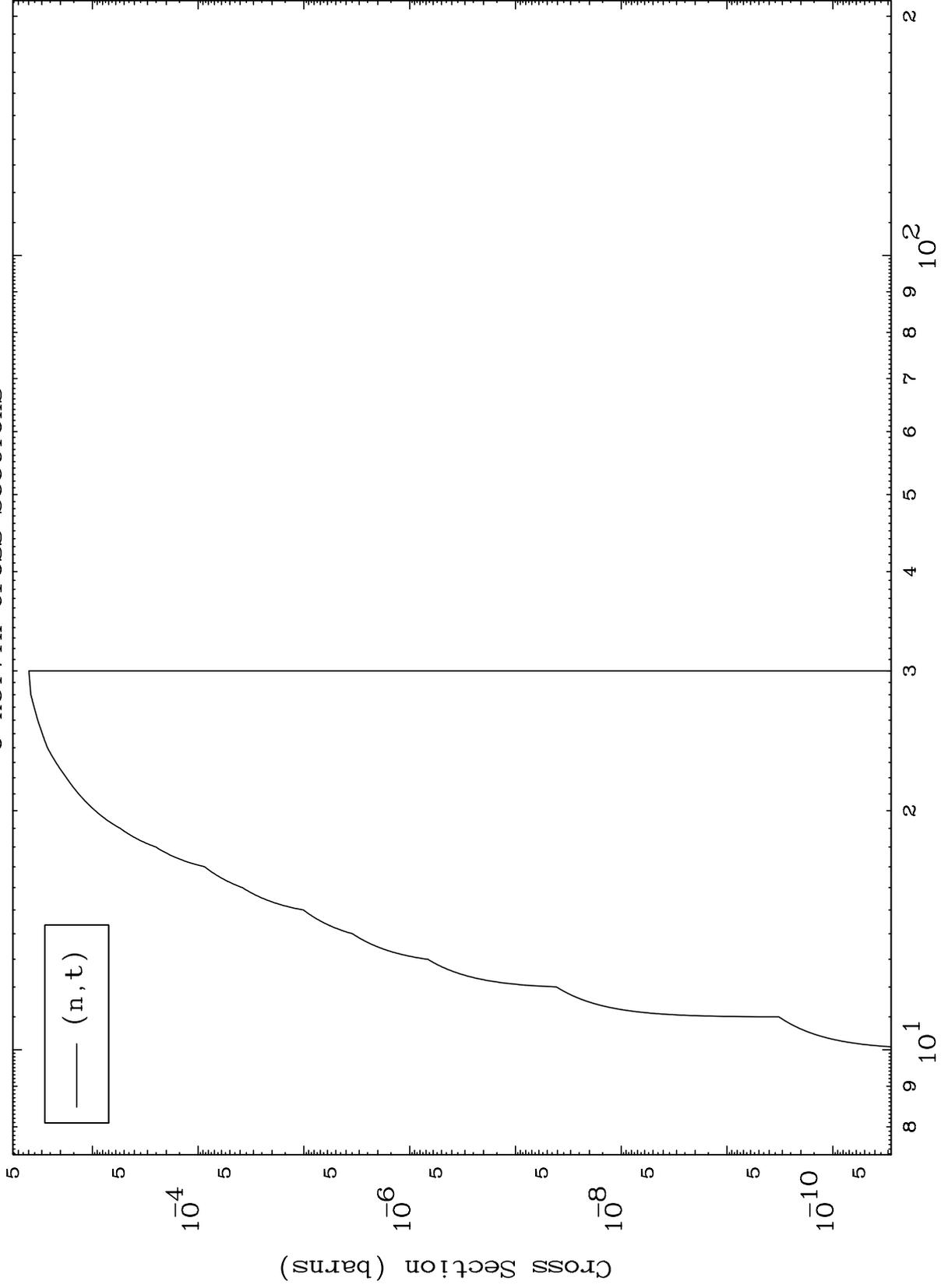
Incident Energy (MeV)

$^{37}\text{Rb-91}$

MAT 3743

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

37-Rb-91

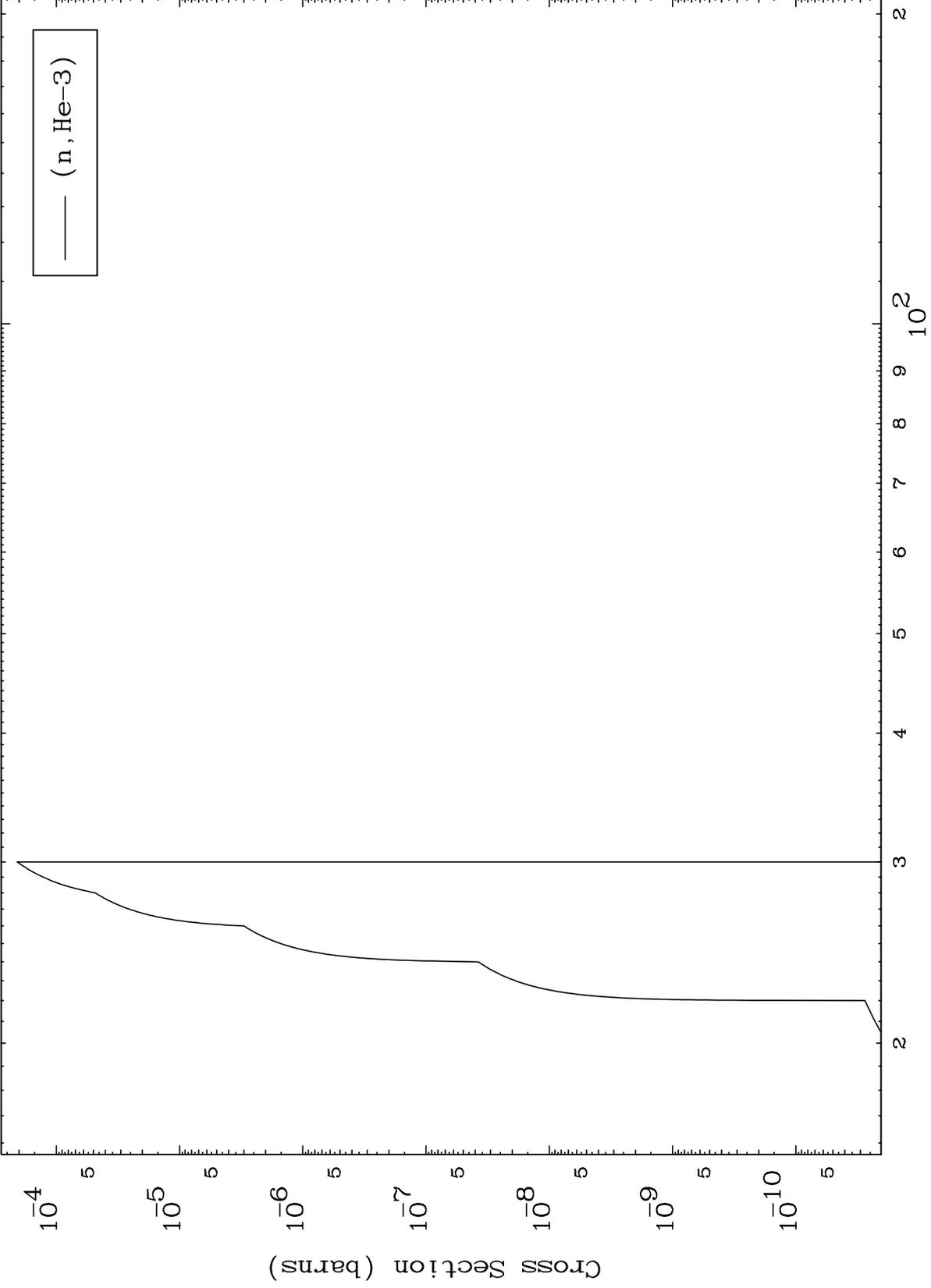


8

Incident Energy (MeV)

37-Rb-91

( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections

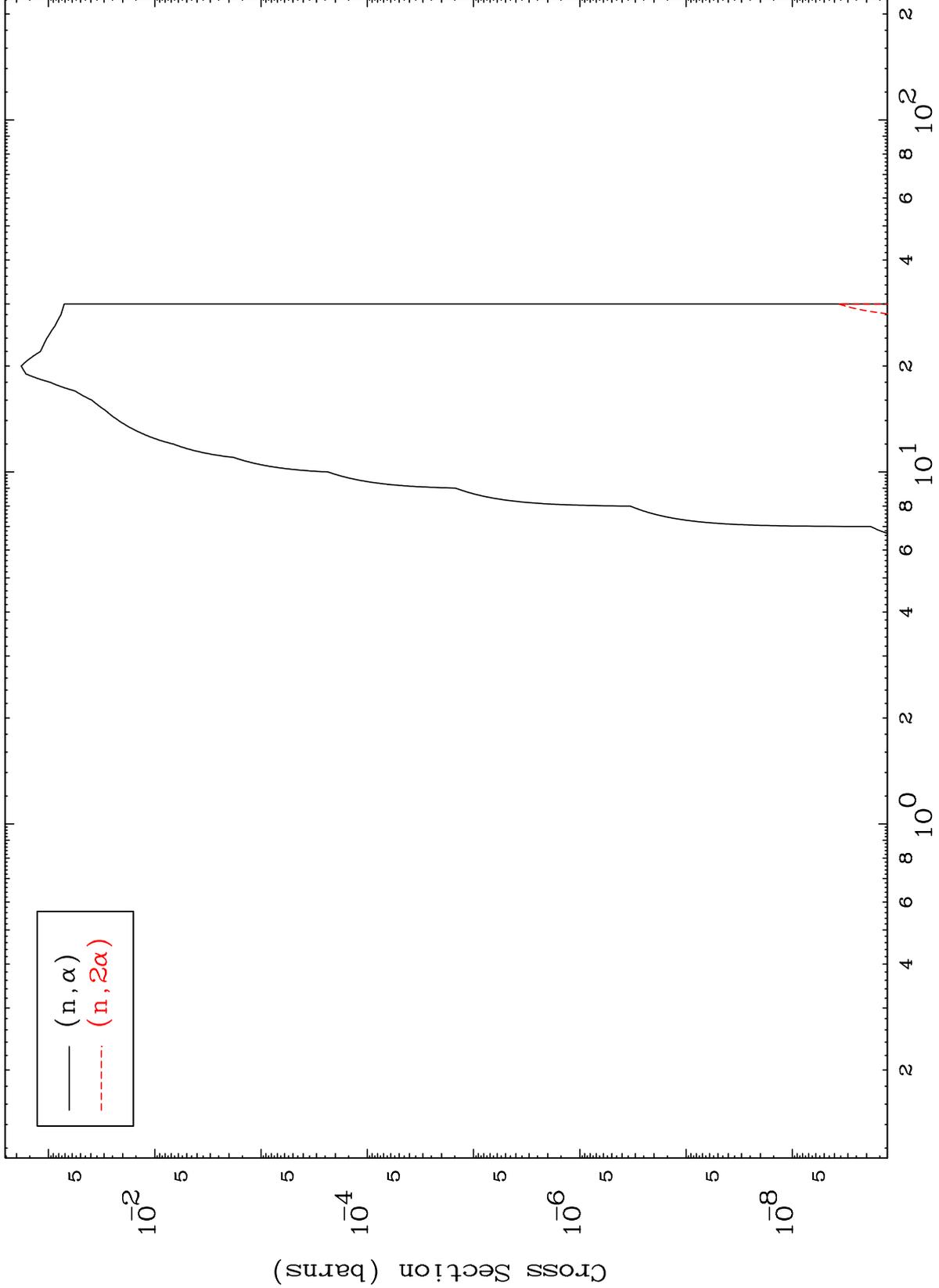


MAT 3743

( $\alpha, \alpha$ ) Levels

37-Rb-91

0 Kelvin Cross Sections



10

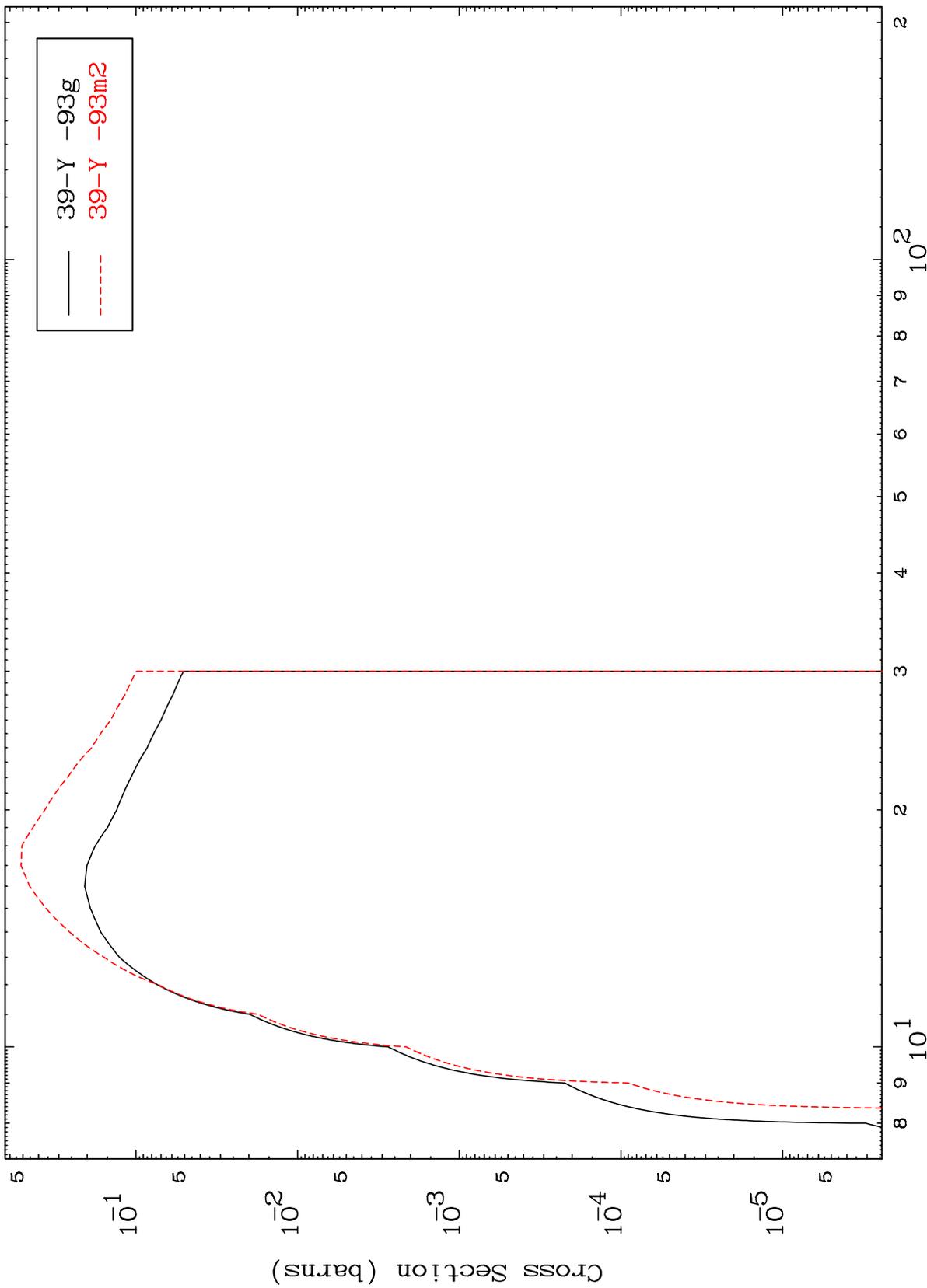
Incident Energy (MeV)

37-Rb-91

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37-Rb-91

(n,2n)  
Radionuclide Production Cross Section



39-Y -93g  
39-Y -93m2

37-Rb-91

Incident Energy (MeV)

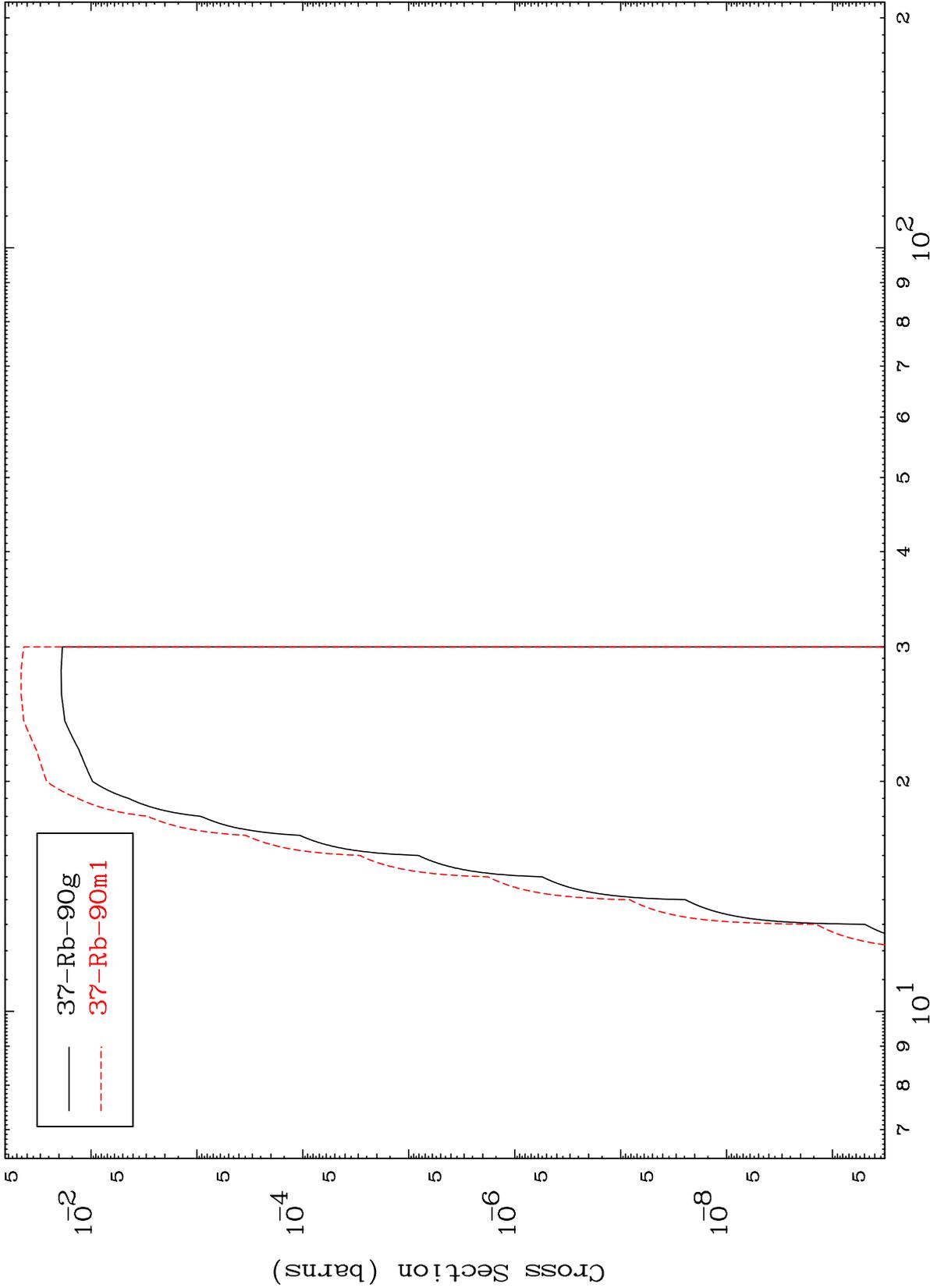
11

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37-Rb-91

(n,n')  $\alpha$

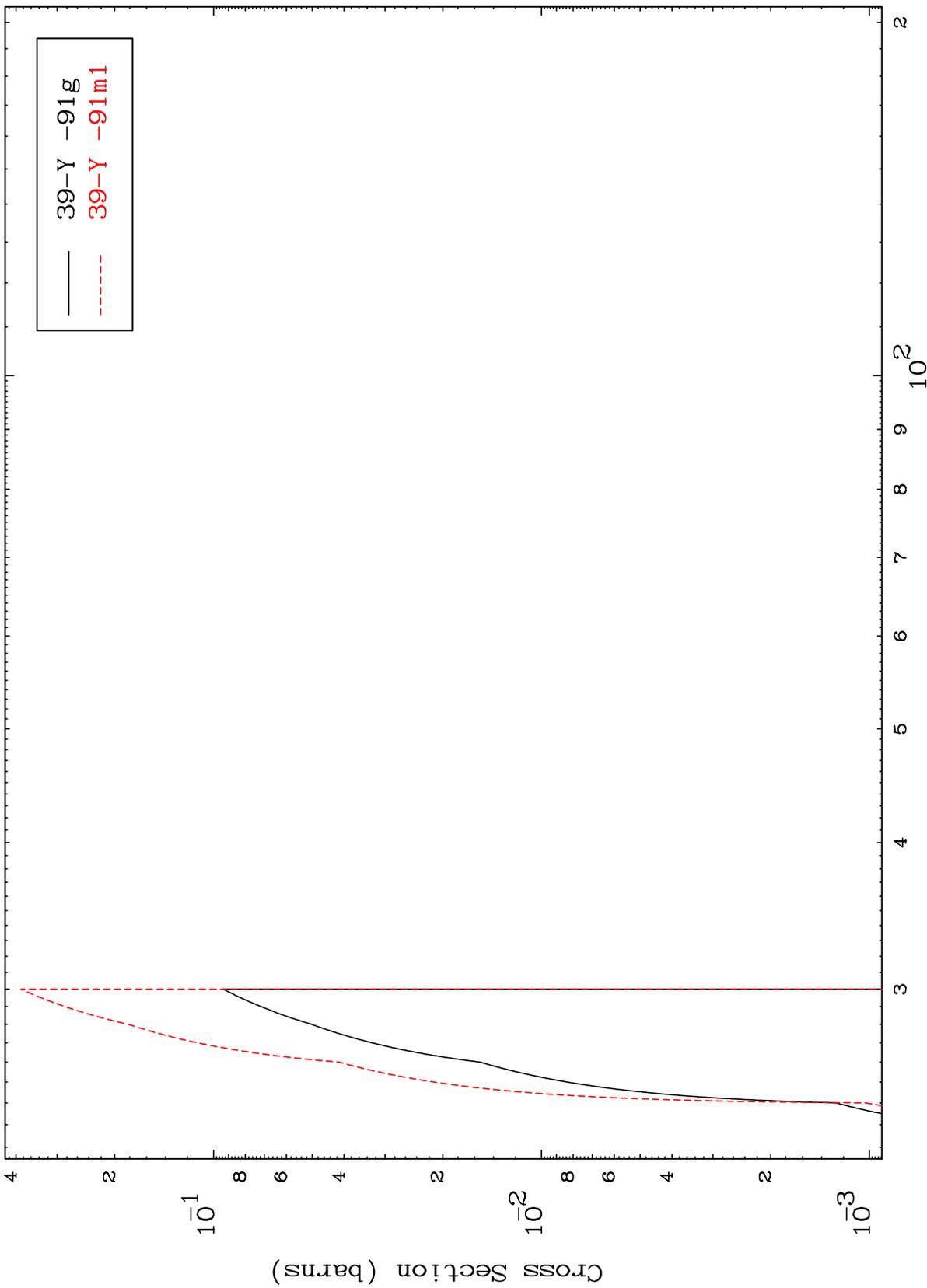
Radionuclide Production Cross Section



MAT 3743

37-Rb-91

(n,4n)  
Radionuclide Production Cross Section



37-Rb-91

Incident Energy (MeV)

13