

Program EVALPLOT
(Version 2021-1)

by

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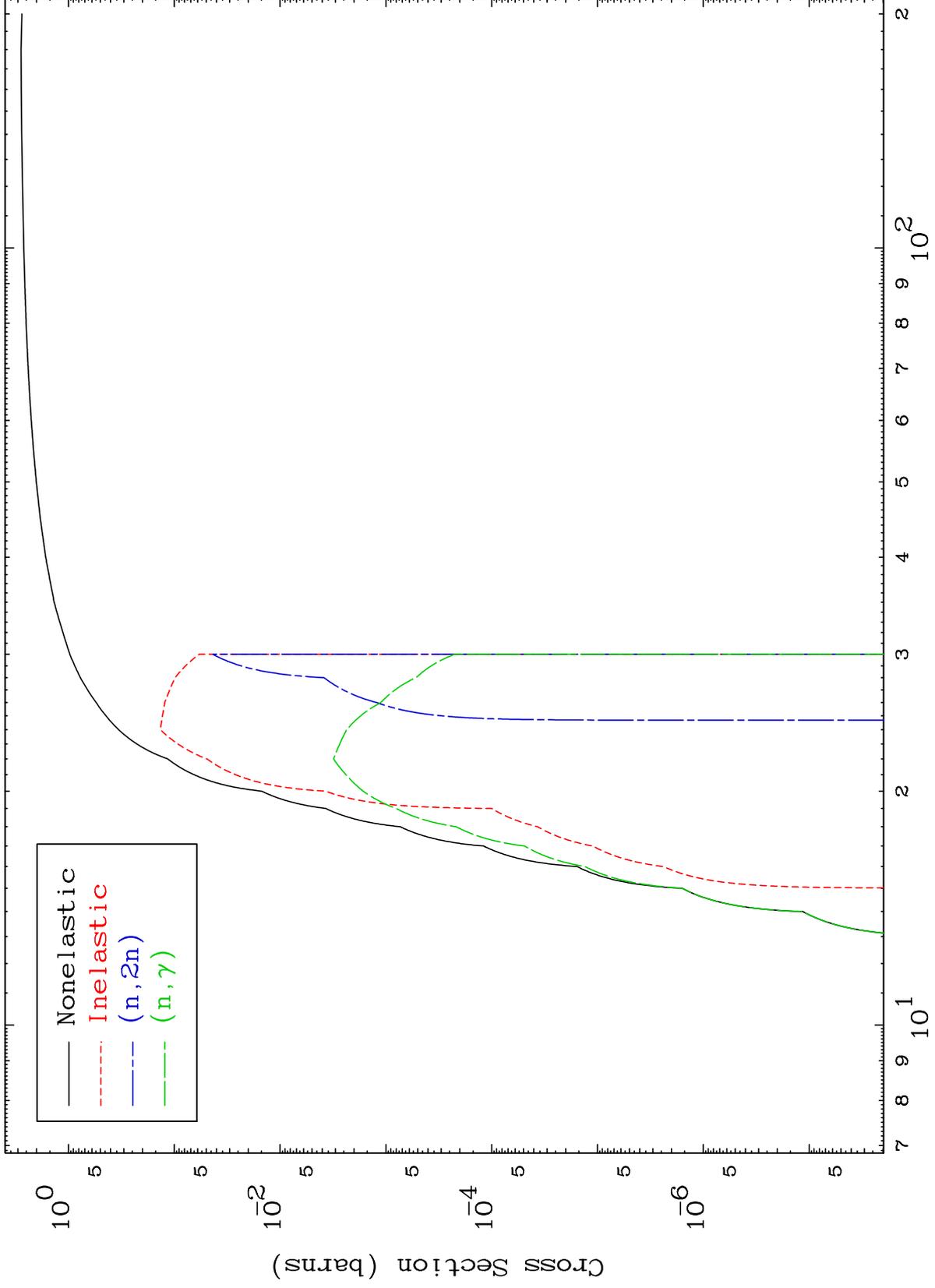
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Press Mouse Button to Start

MAT 8613

0 Kelvin Cross Sections

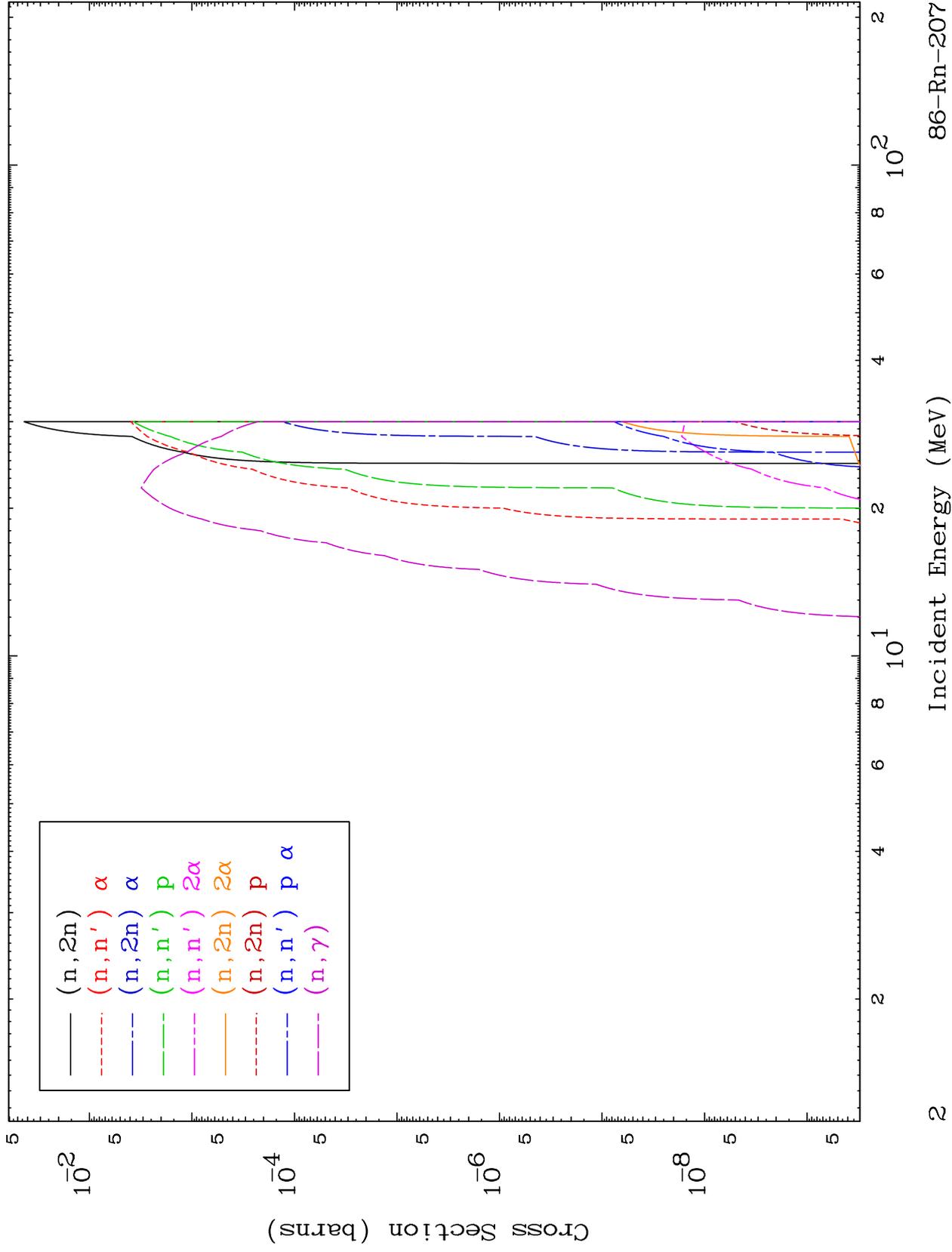
86-Rn-207

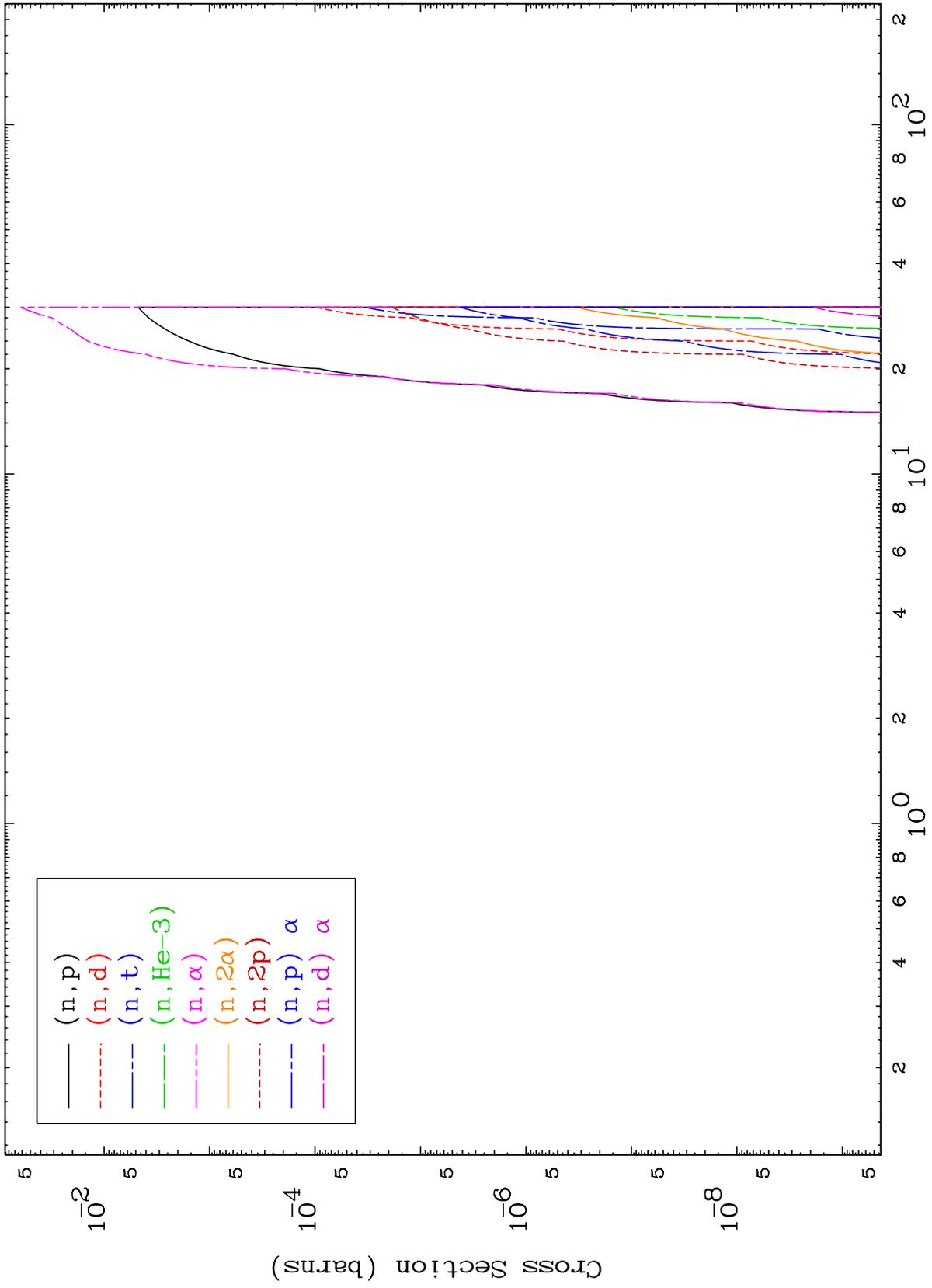


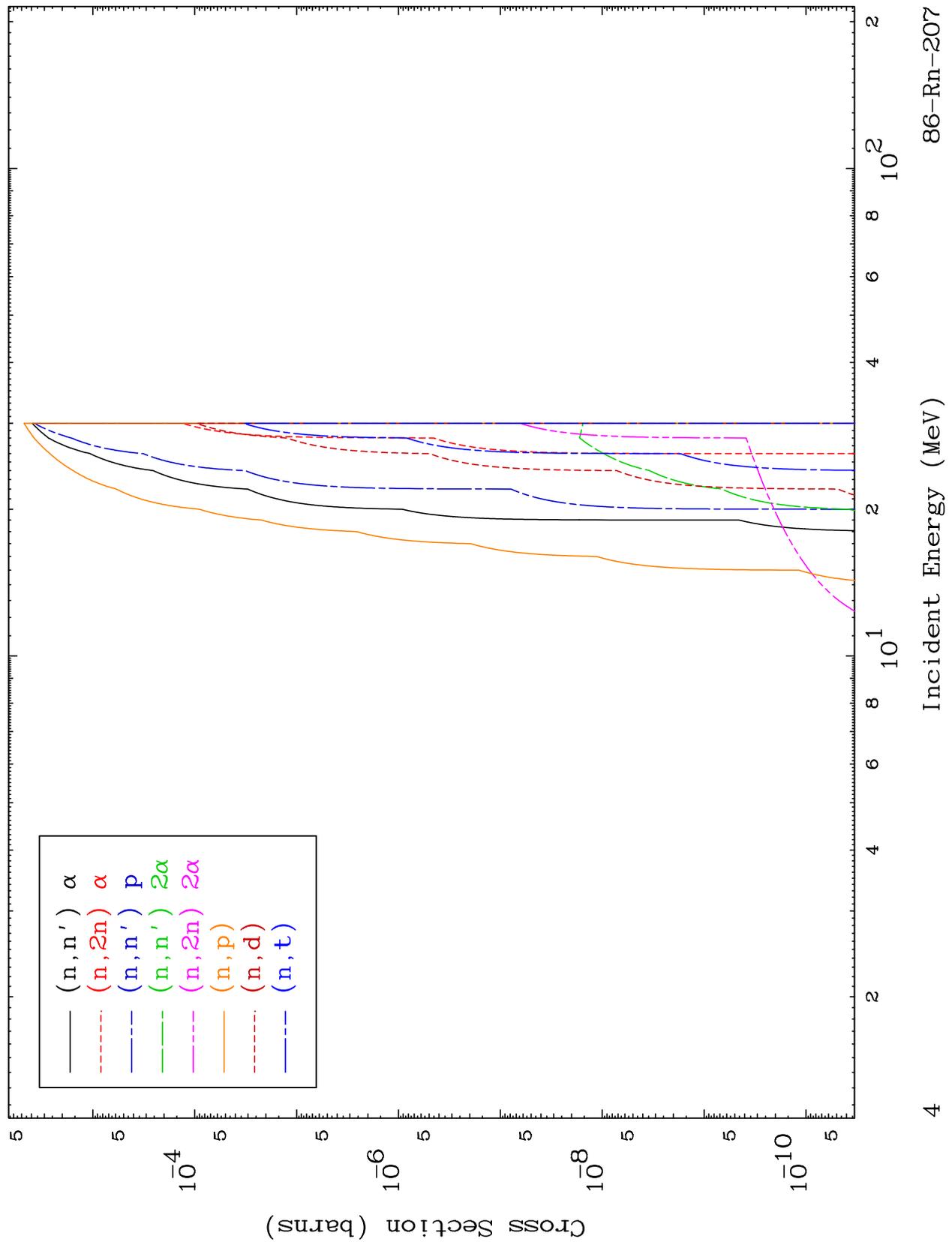
1

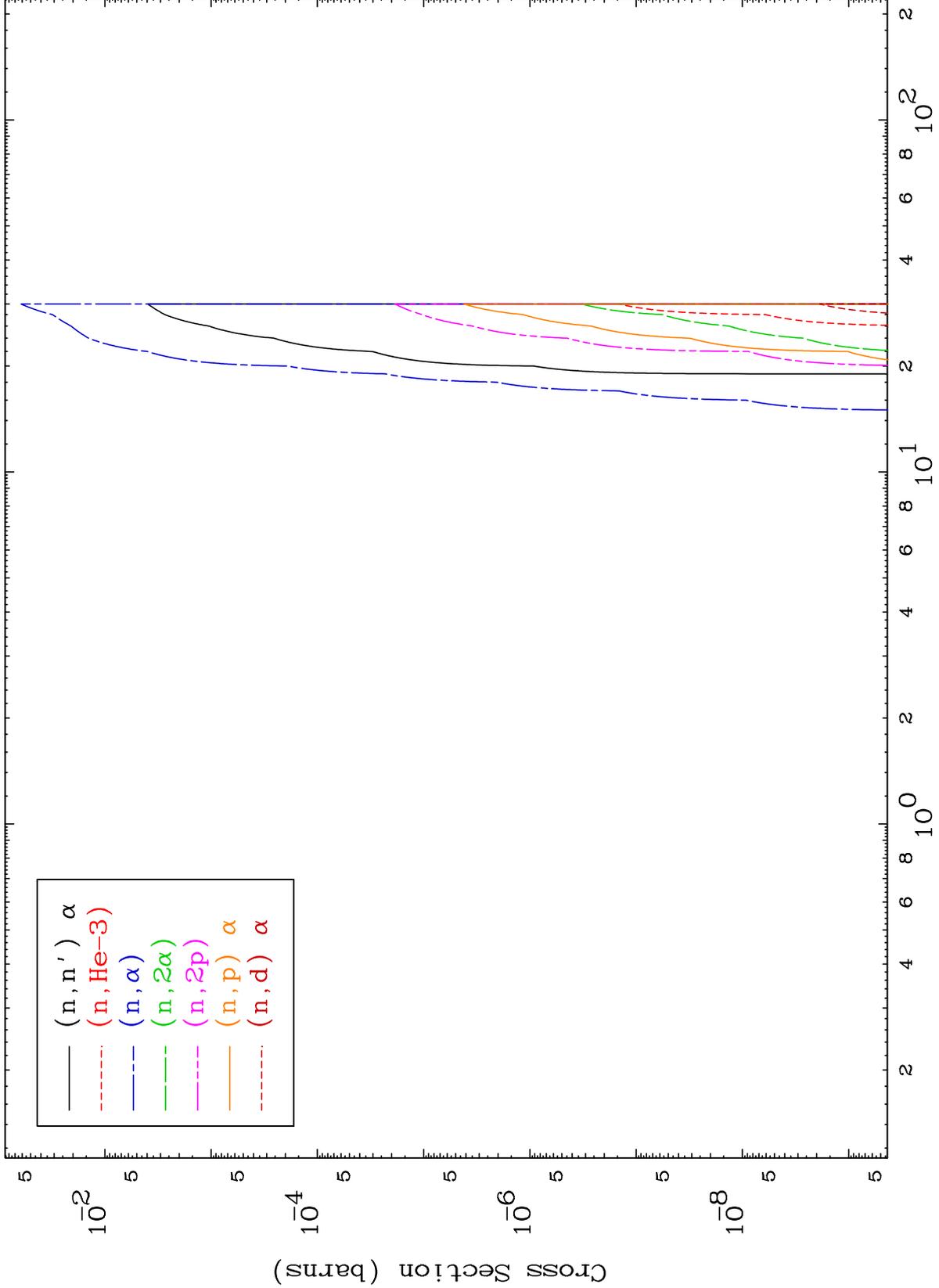
Incident Energy (MeV)

86-Rn-207





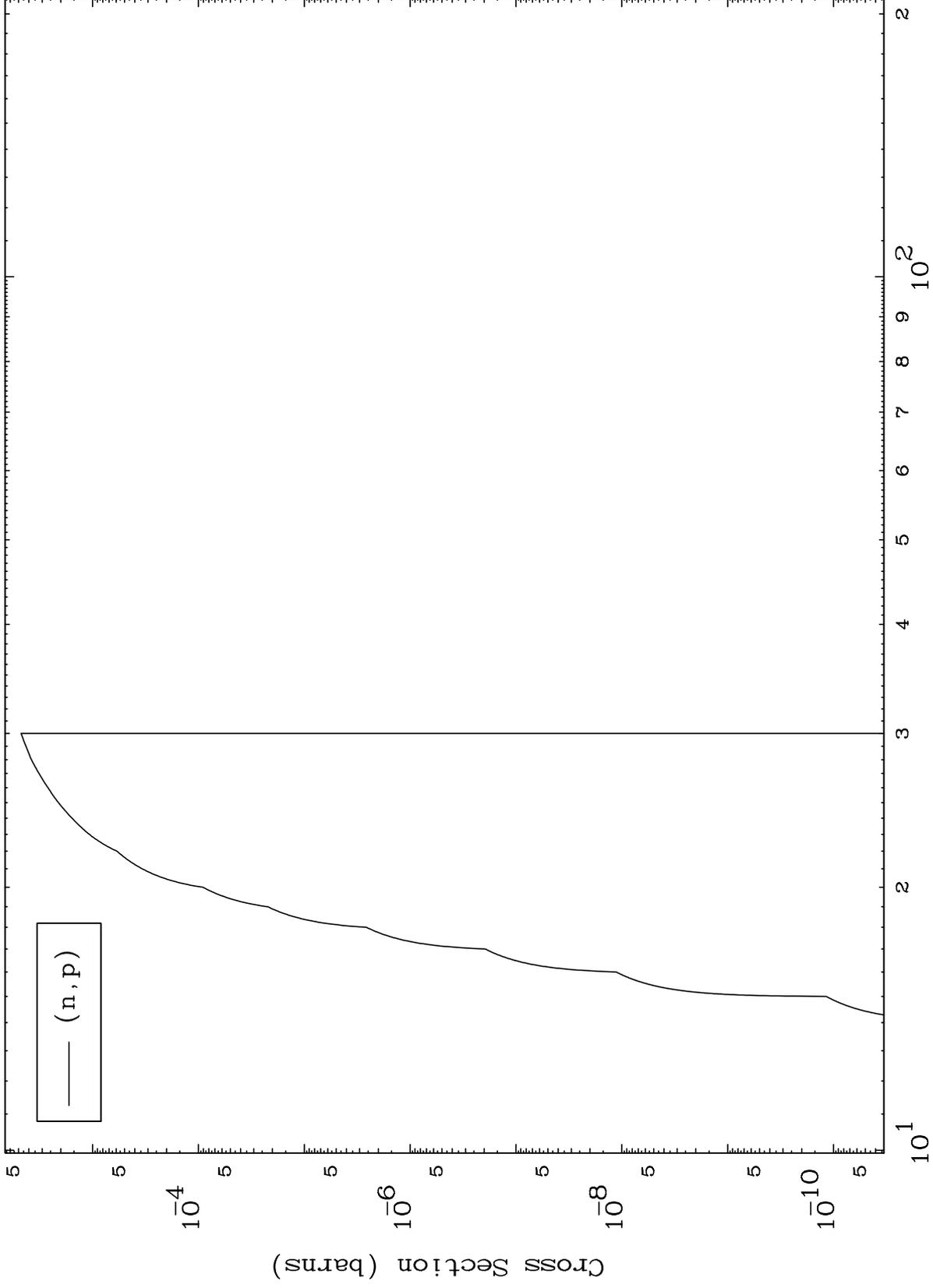




MAT 8613

(α, p) Levels
0 Kelvin Cross Sections

86-Rn-207



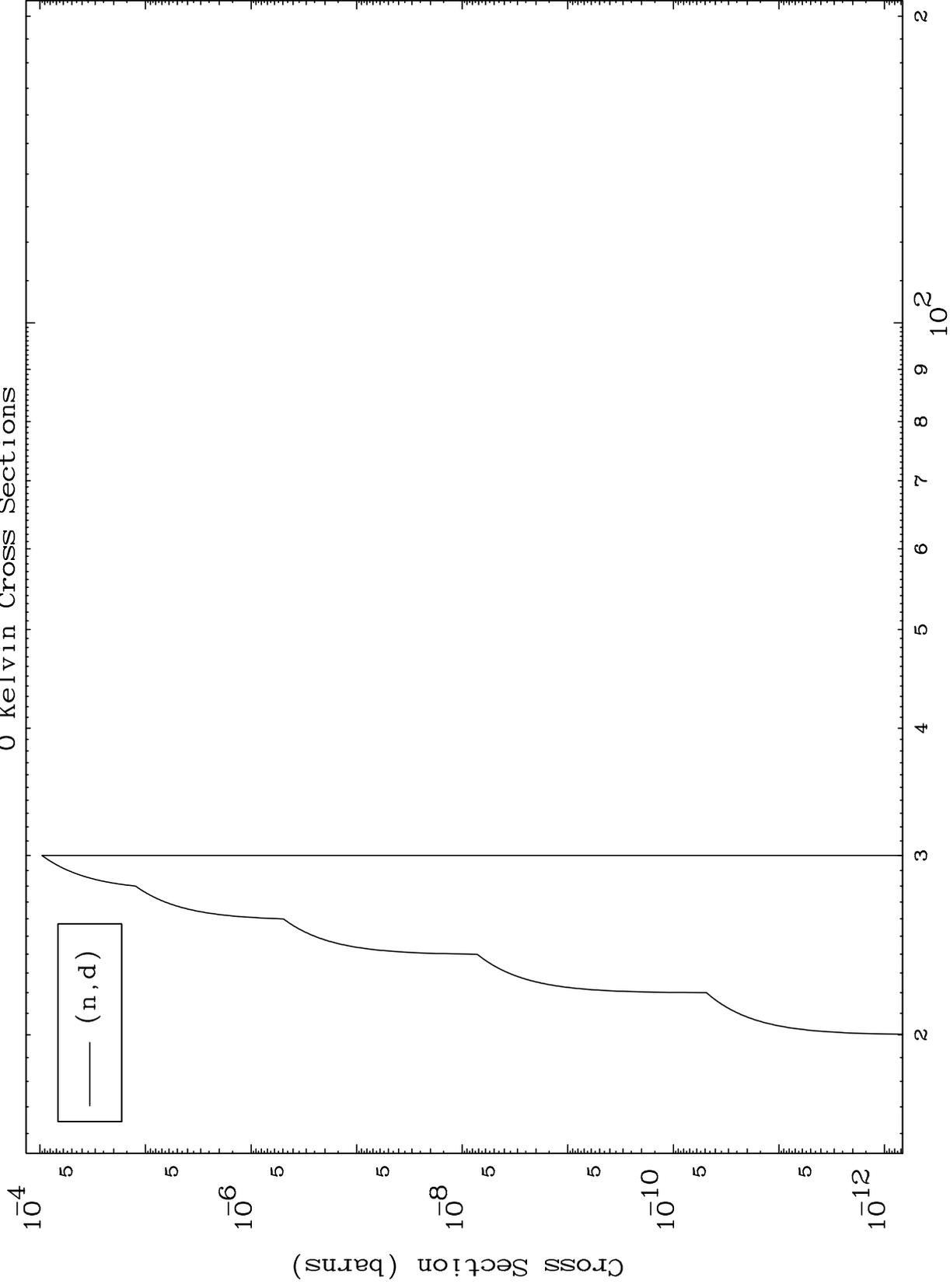
Incident Energy (MeV)

86-Rn-207

MAT 8613

(α, d) Levels
0 Kelvin Cross Sections

86-Rn-207



7

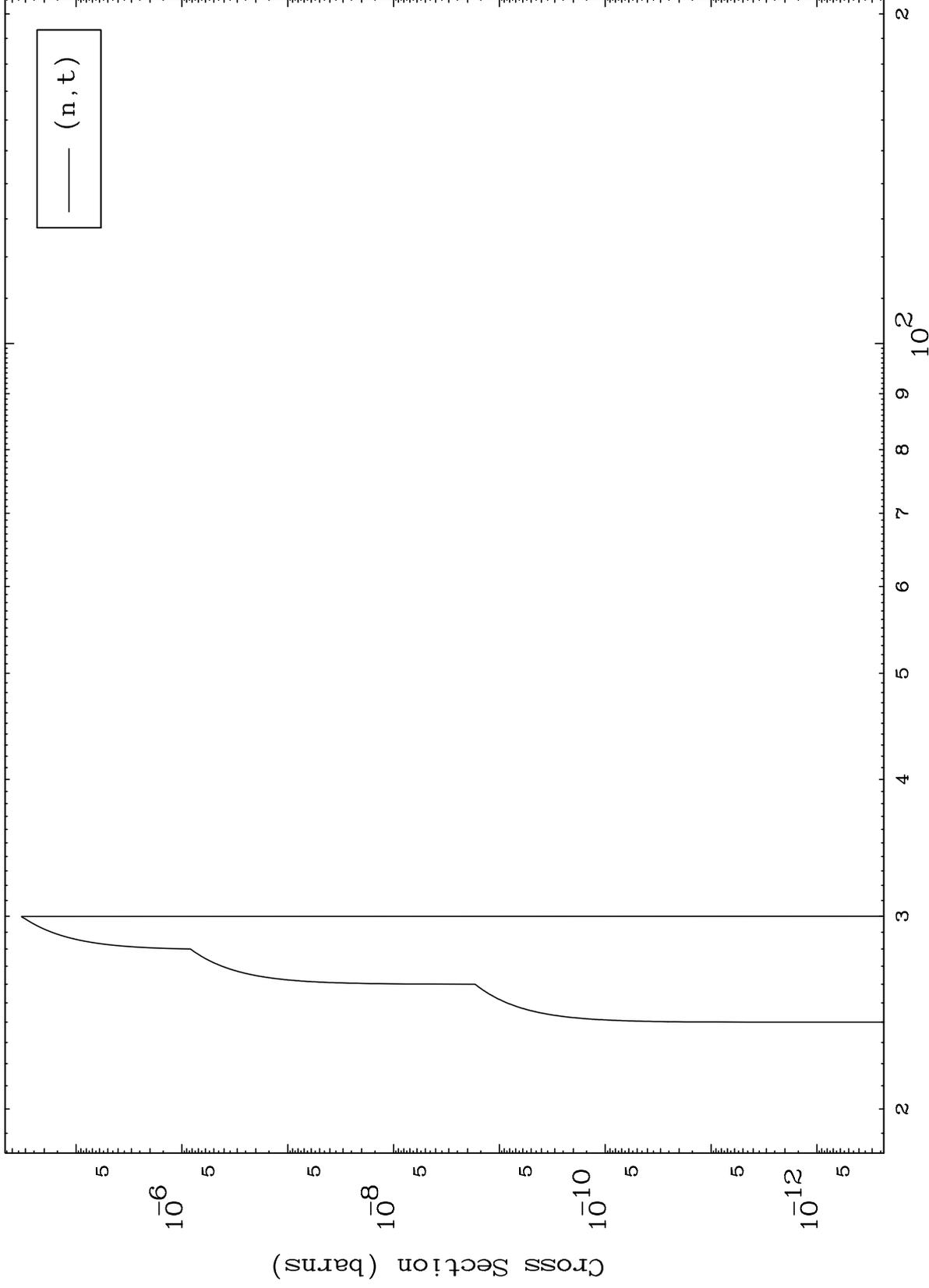
Incident Energy (MeV)

86-Rn-207

MAT 8613

(α, t) Levels
0 Kelvin Cross Sections

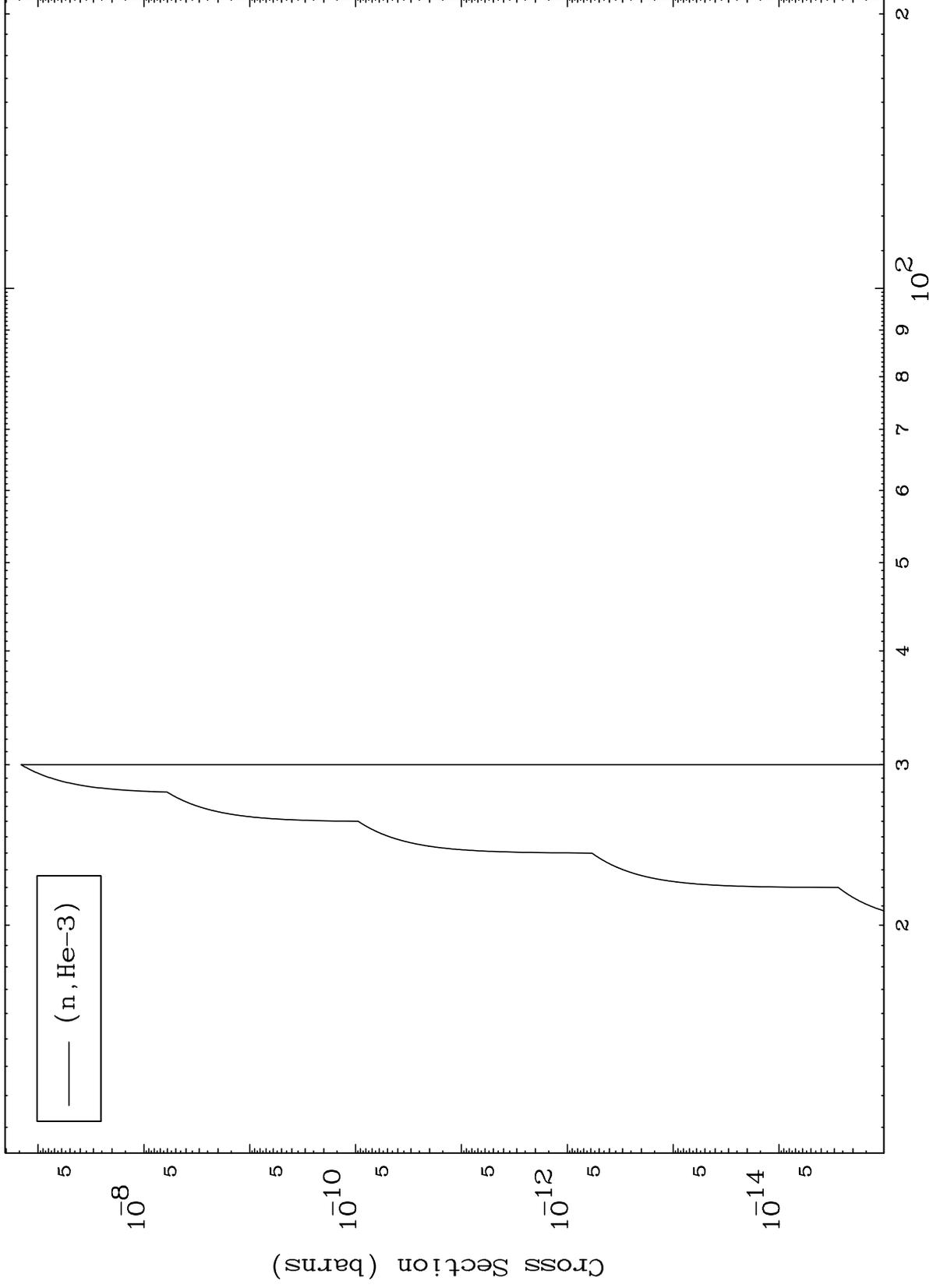
86-Rn-207



8

Incident Energy (MeV)

86-Rn-207

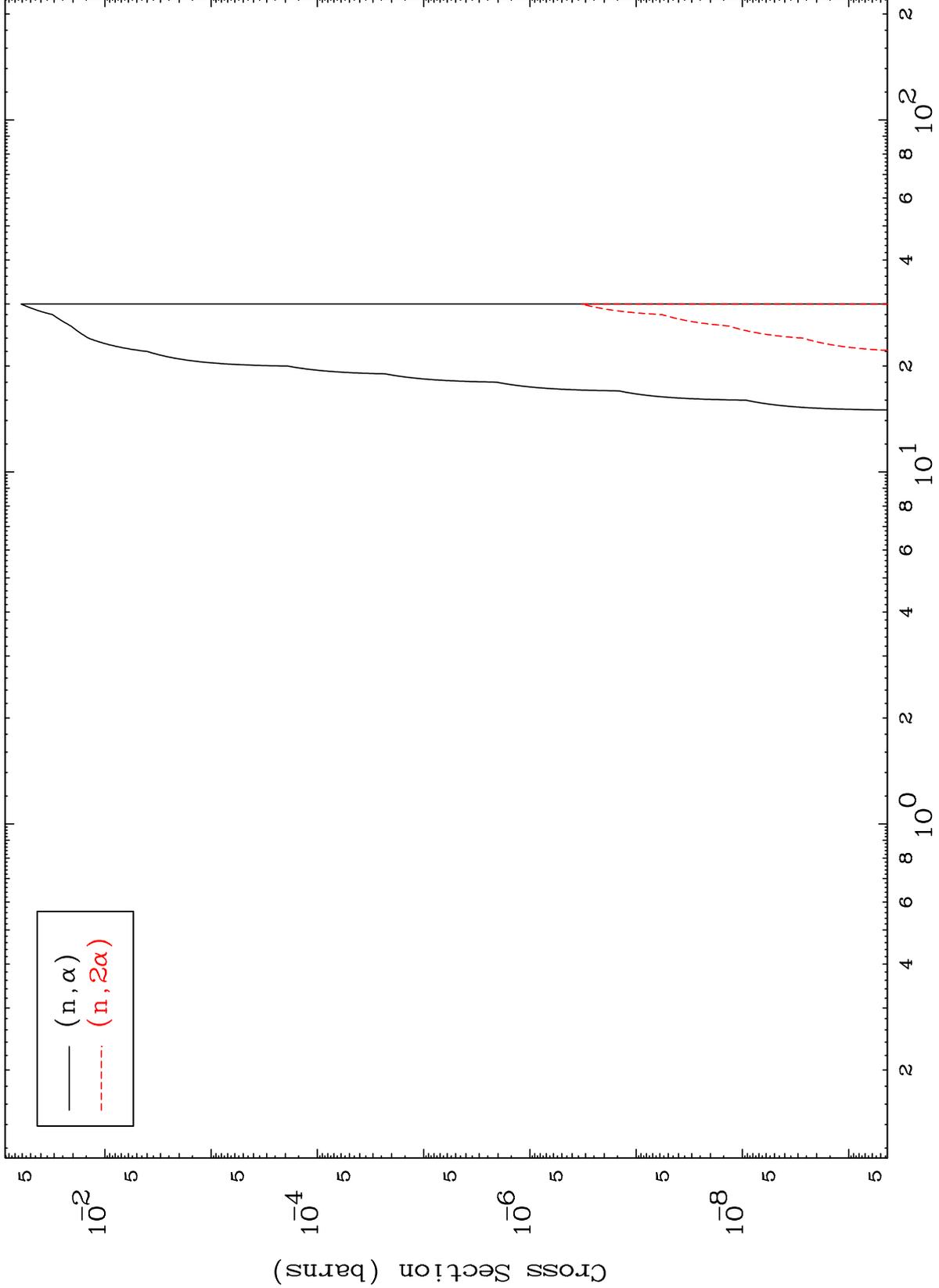


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(α, α) Levels

86-Rn-207

0 Kelvin Cross Sections



— (n, α)
- - - ($n, 2\alpha$)

10

Incident Energy (MeV)

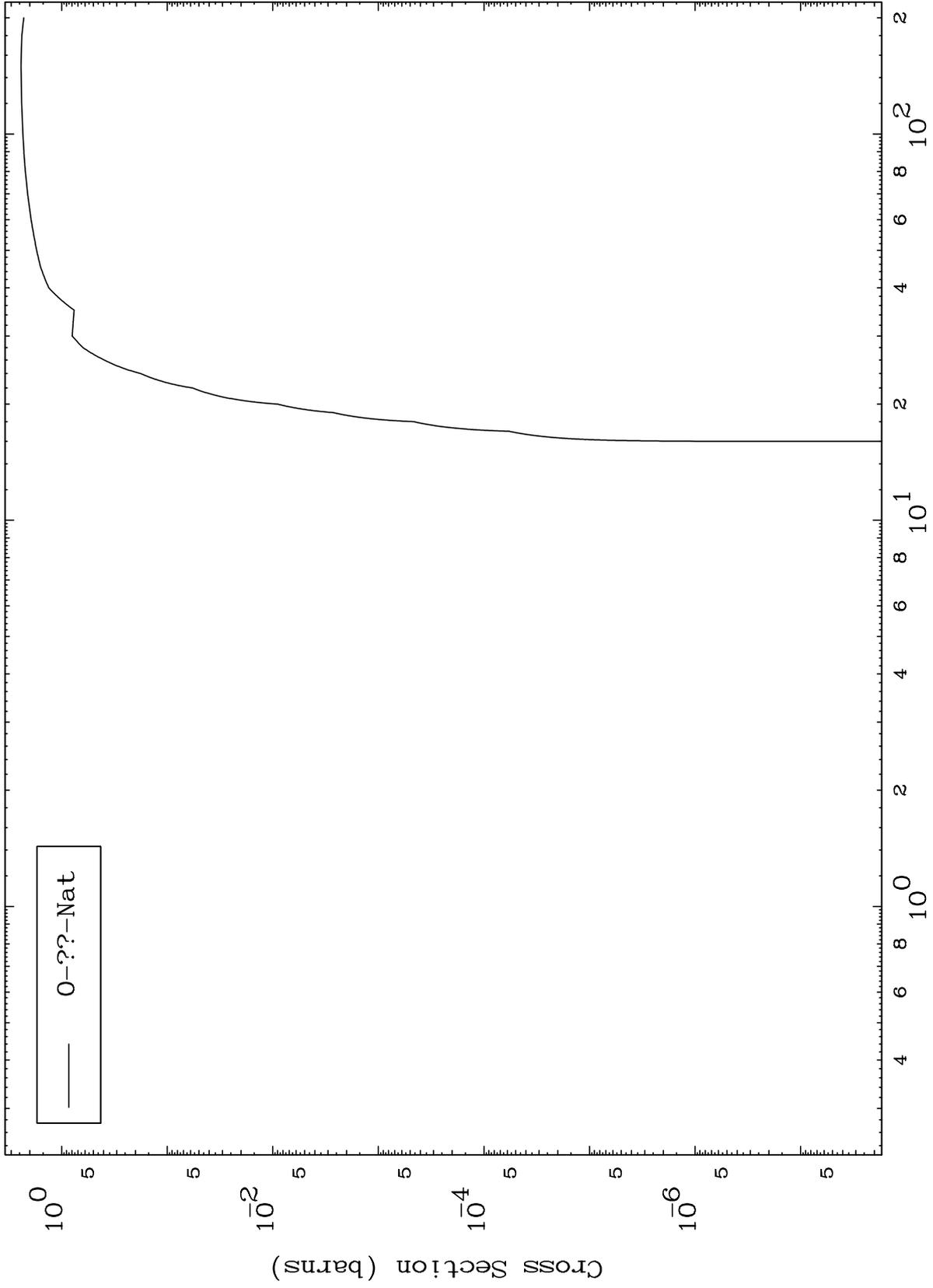
86-Rn-207

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Fission

86-Rn-207

Radionuclide Production Cross Section

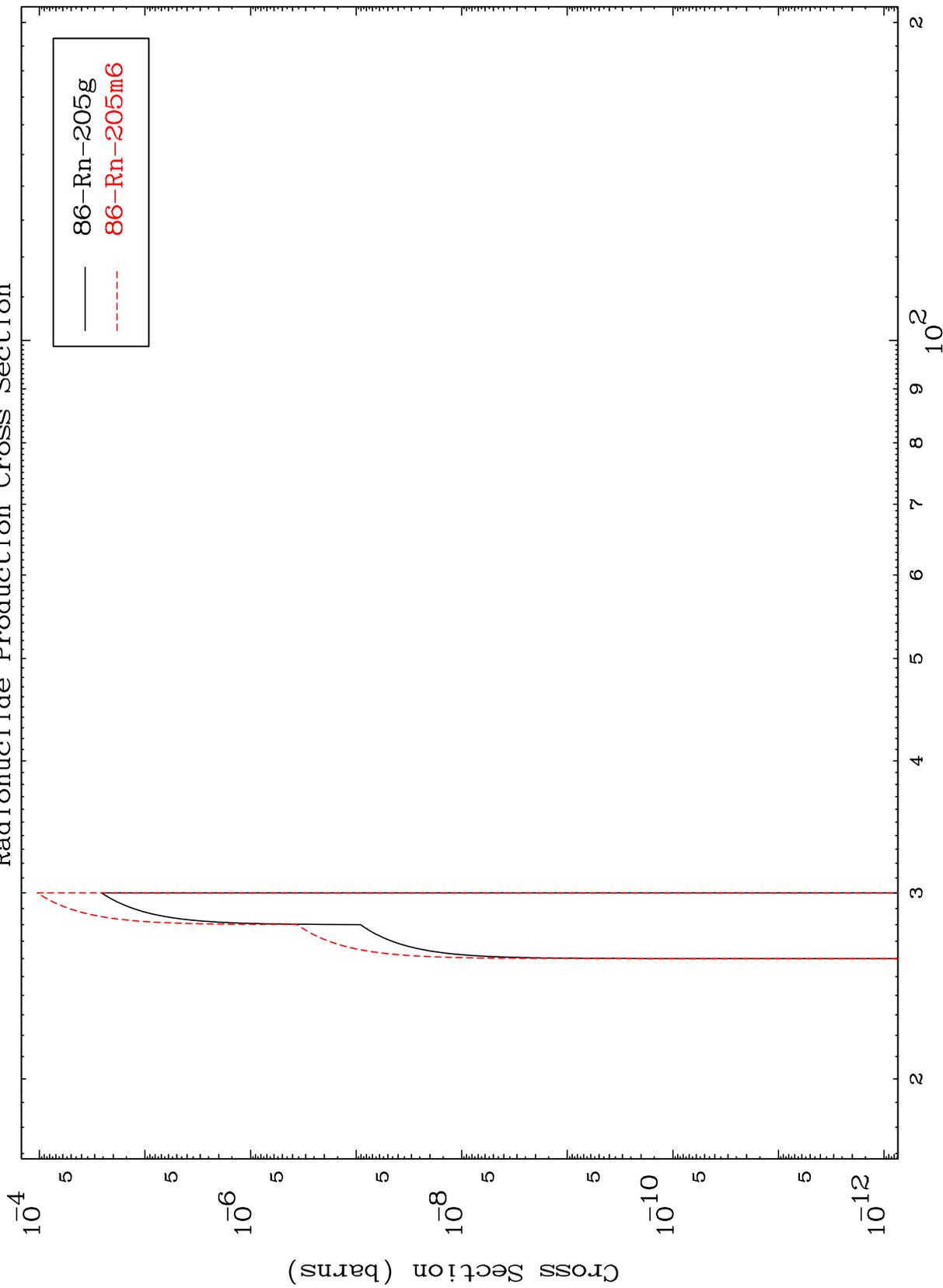


MAT 8613

(n,2n) α

86-Rn-207

Radionuclide Production Cross Section



12

Incident Energy (MeV)

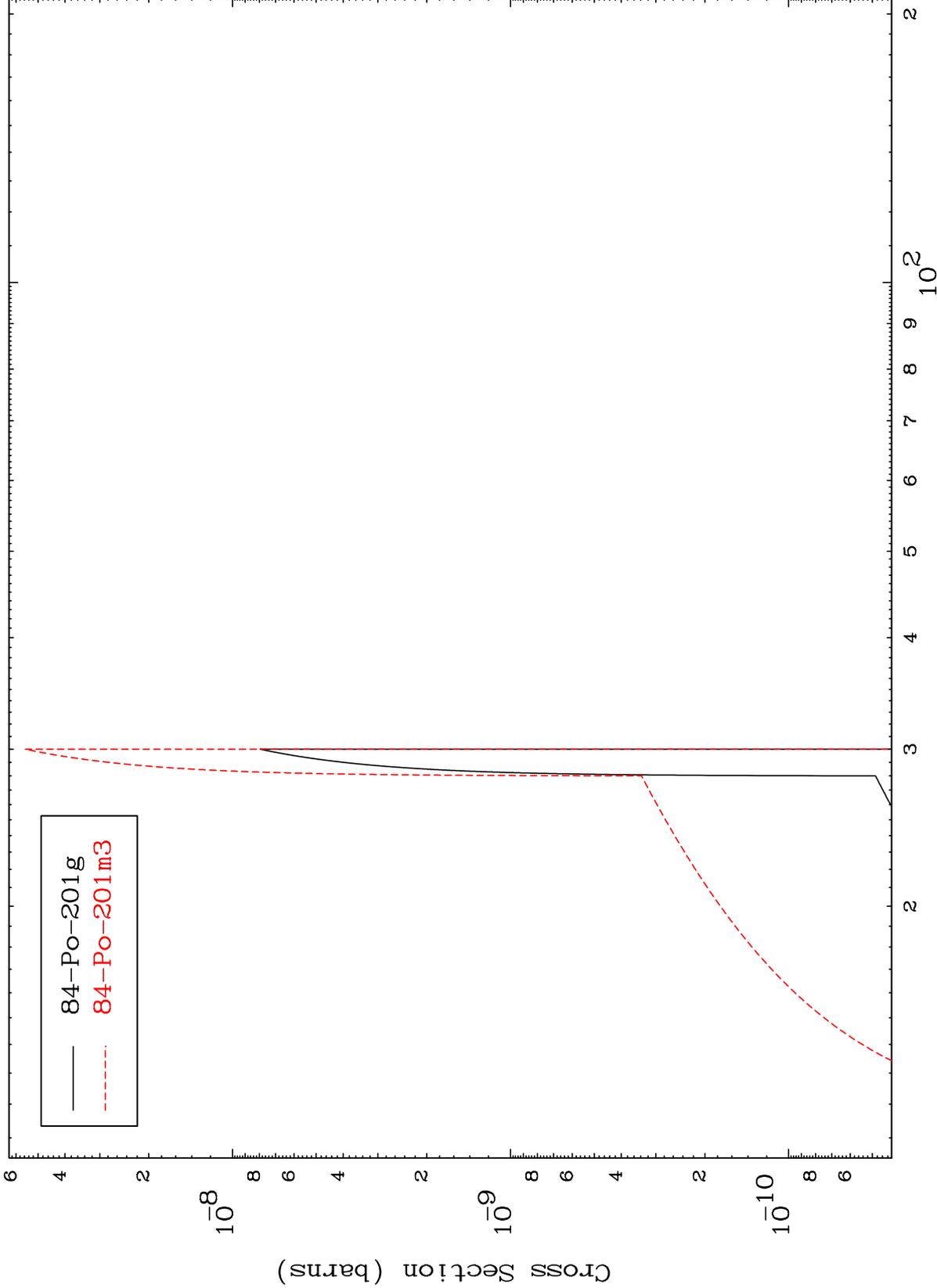
86-Rn-207

MAT 8613

(n,2n) 2α

86-Rn-207

Radionuclide Production Cross Section

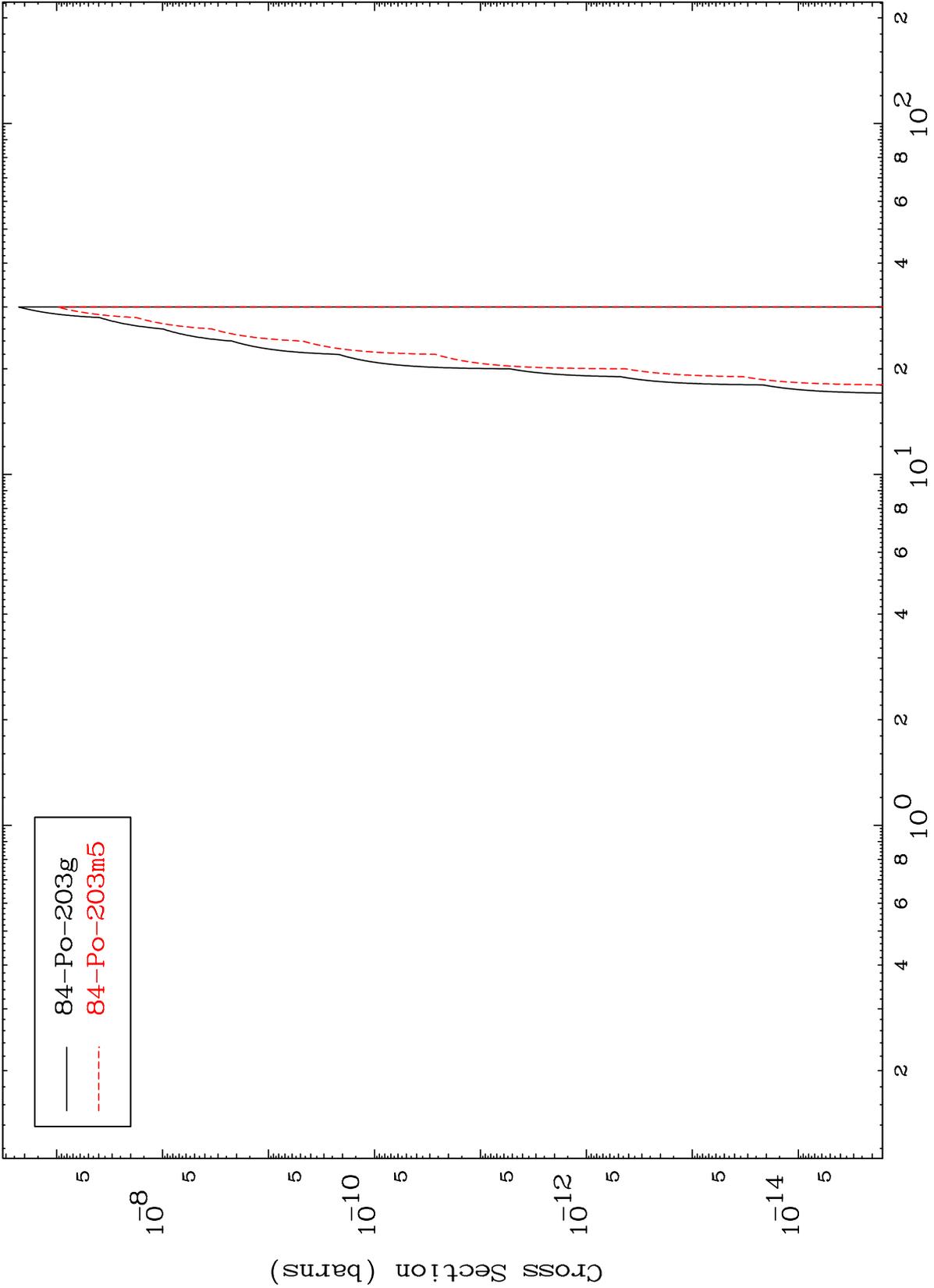


MAT 8613

(n,2α)

86-Rn-207

Radionuclide Production Cross Section



14

Incident Energy (MeV)

86-Rn-207