

Program EVALPLOT  
(Version 2021-1)

by

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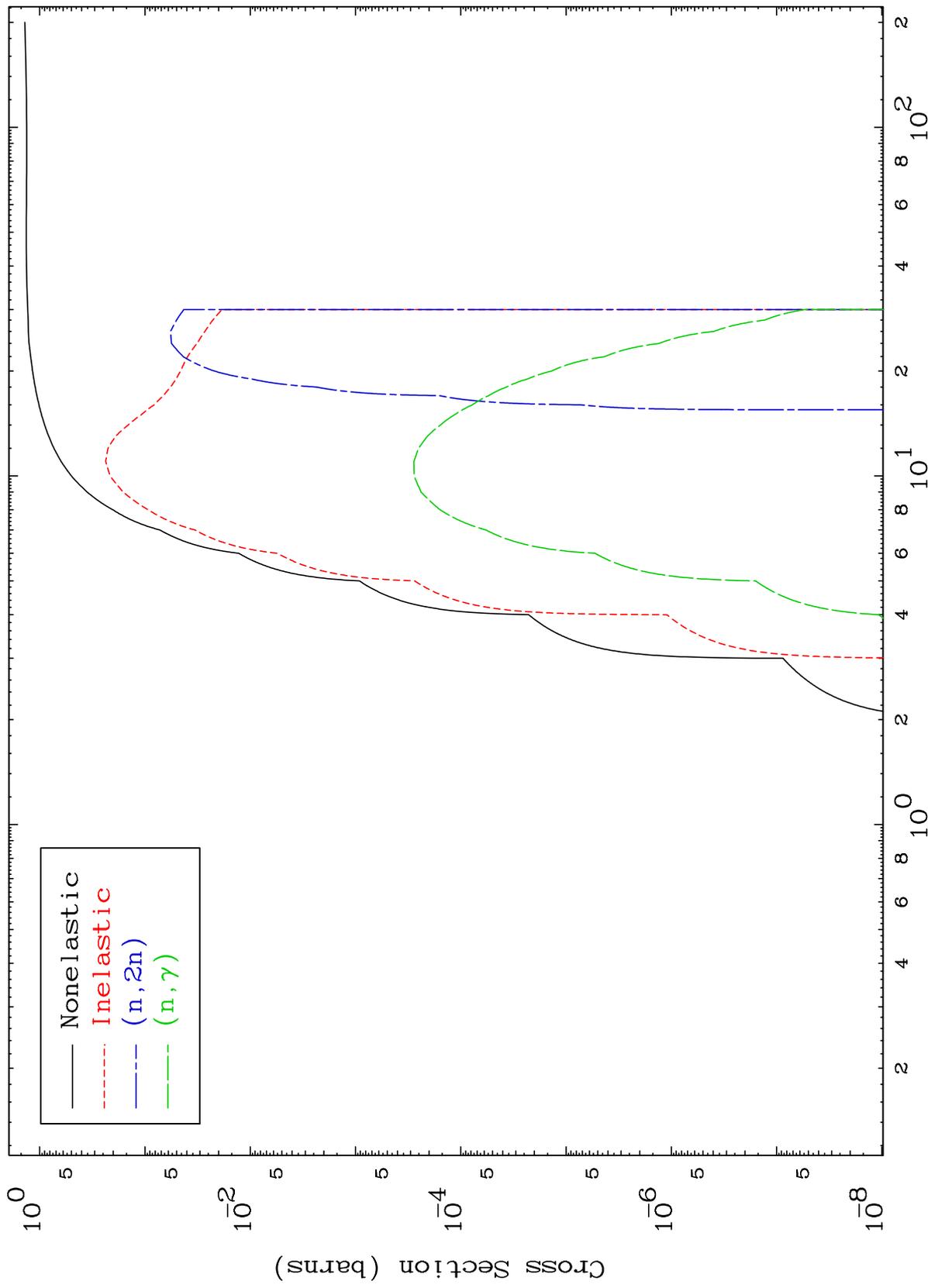
Press Mouse Button to Start

MAT 2123

$\alpha$  Major

21-Sc-44m

0 Kelvin Cross Sections



1

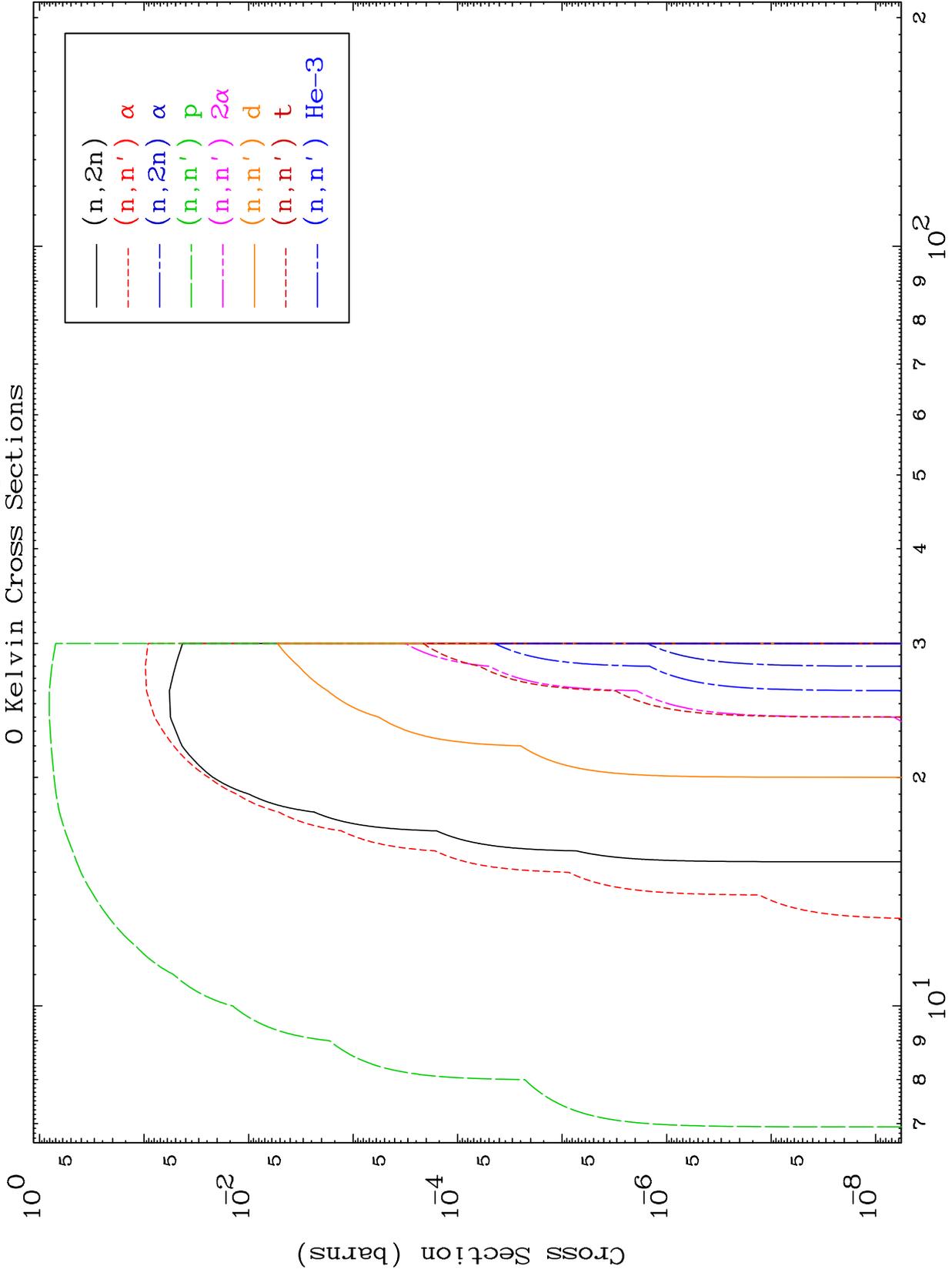
Incident Energy (MeV)

21-Sc-44m

MAT 2123

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

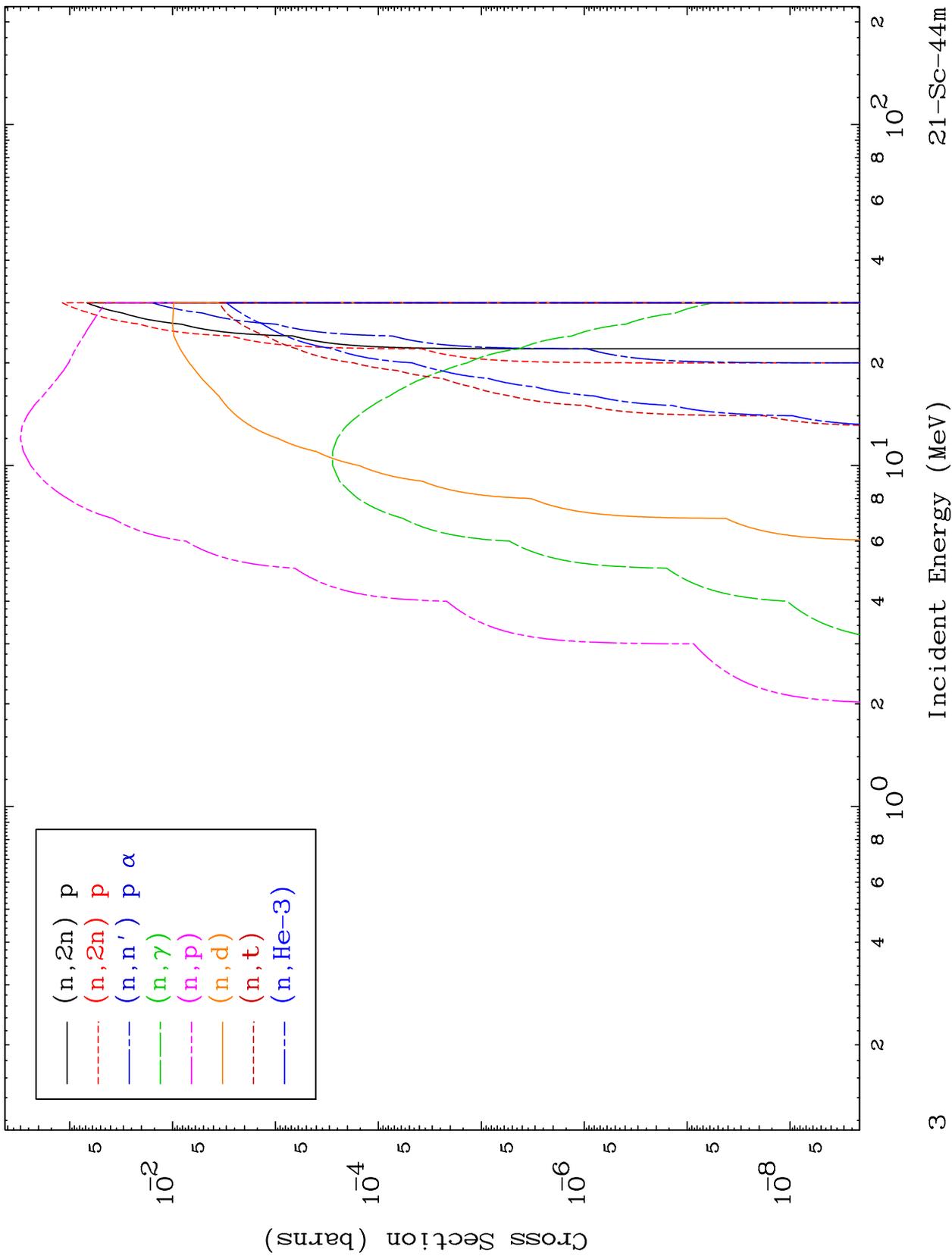
21-Sc-44m

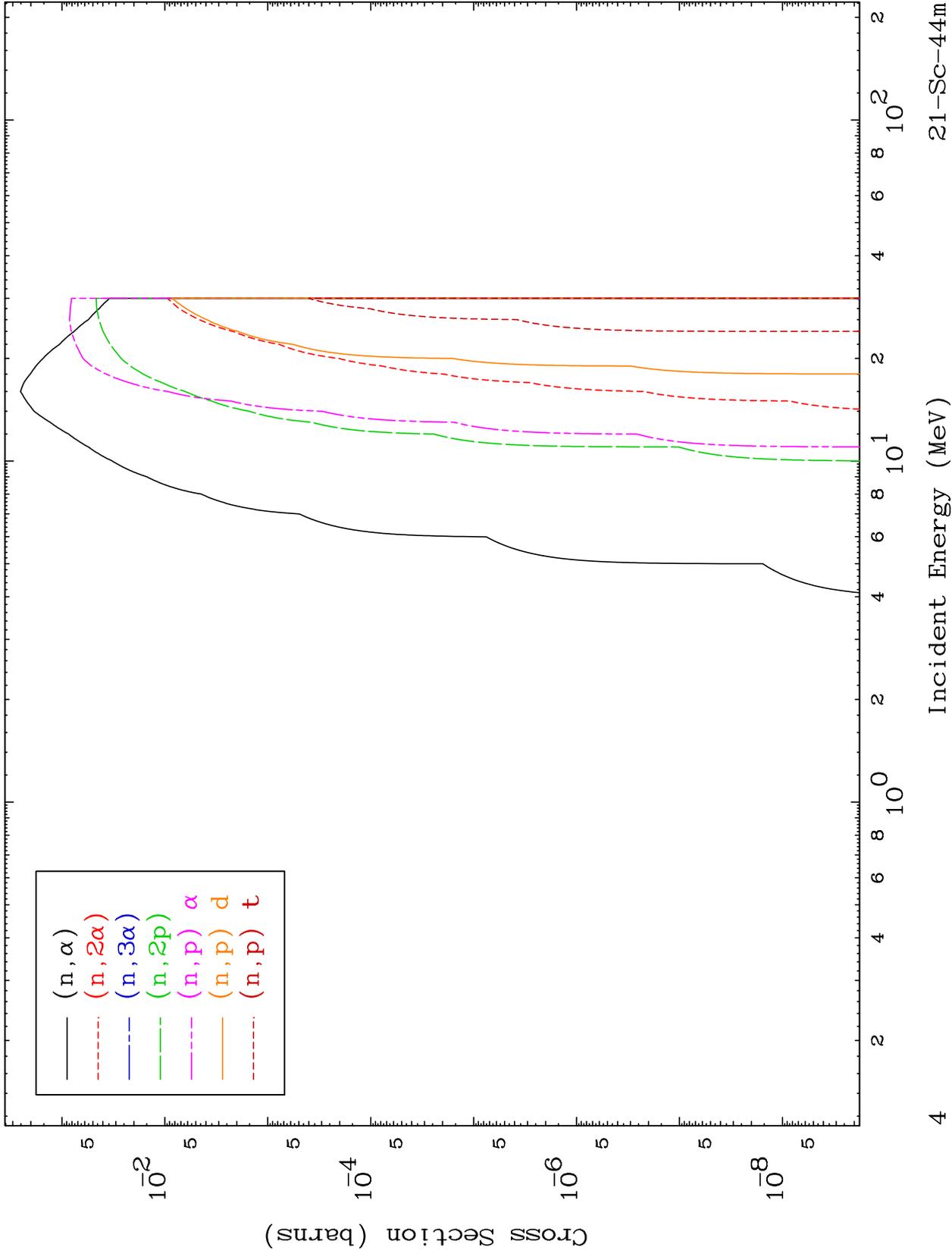


2

Incident Energy (MeV)

21-Sc-44m

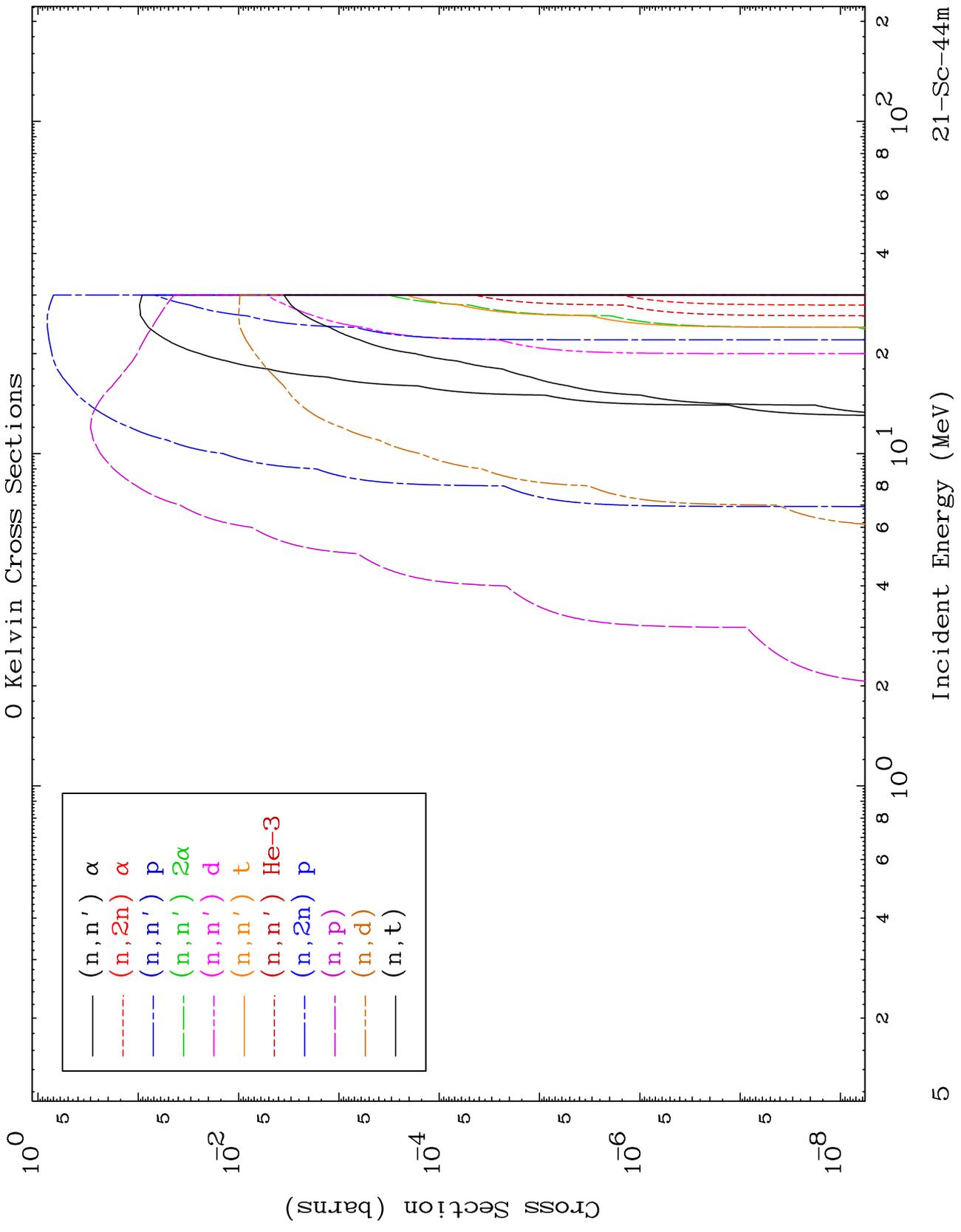


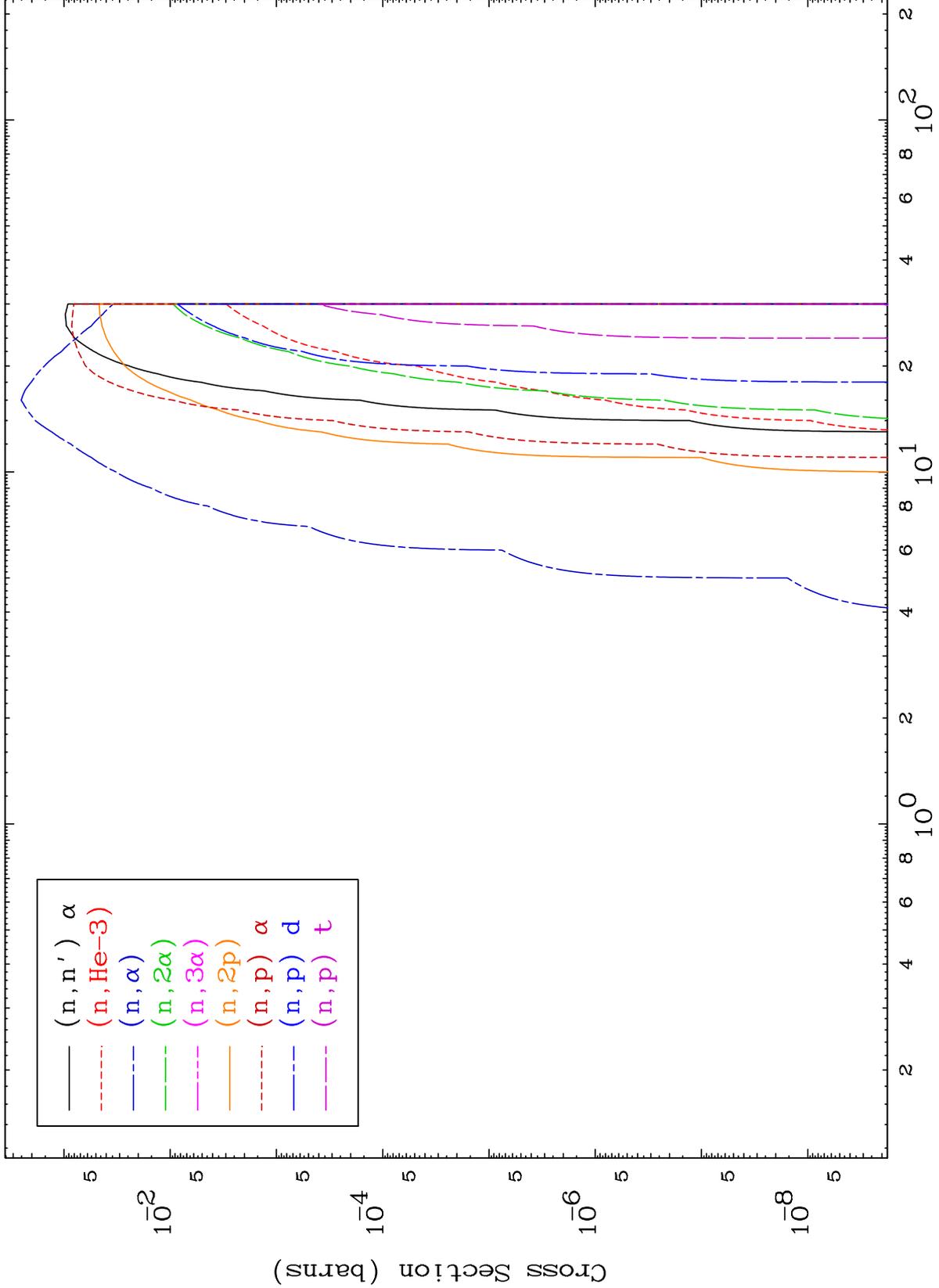


MAT 2123

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

21-Sc-44m



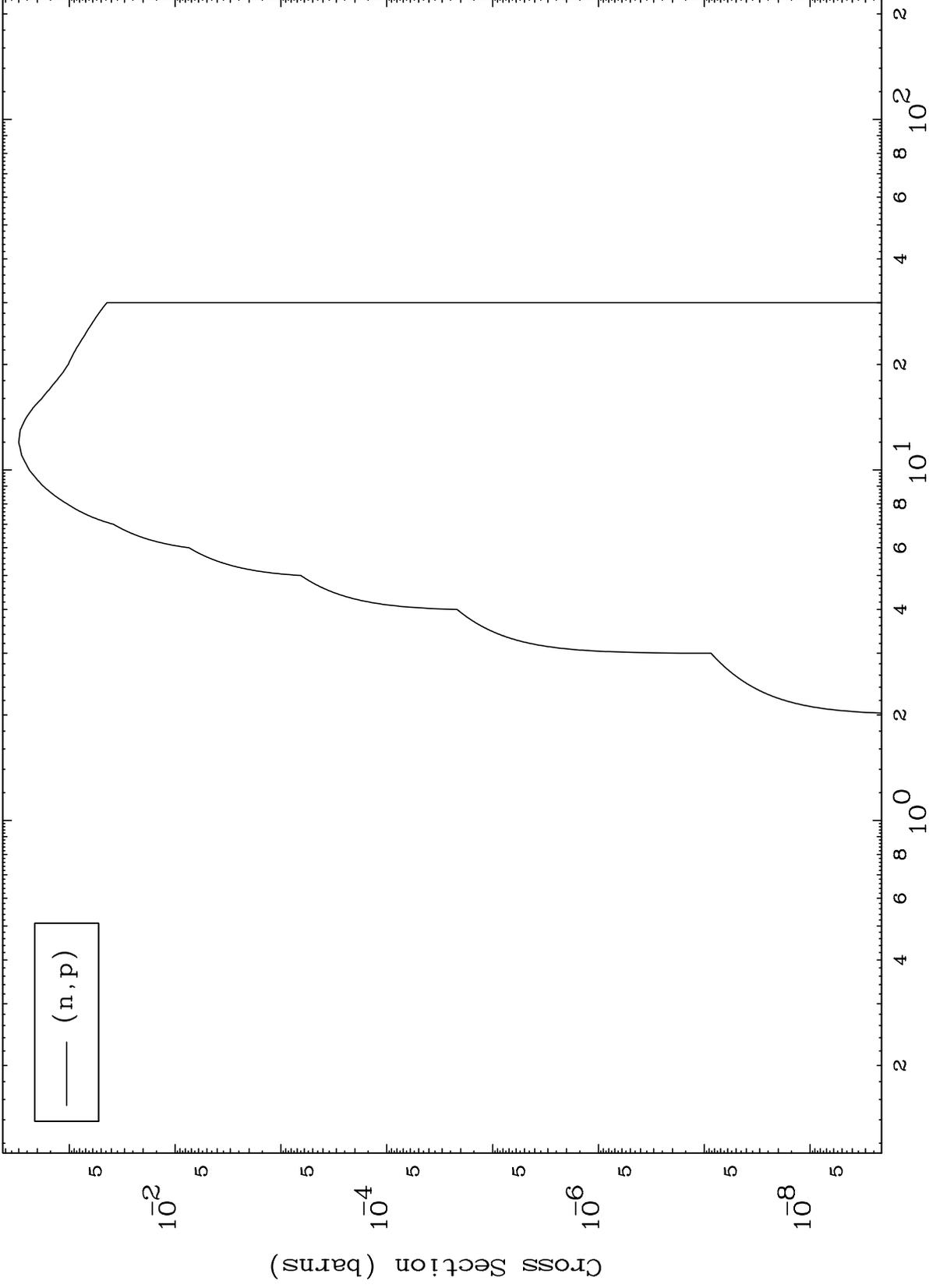


MAT 2123

( $\alpha, p$ ) Levels

21-Sc-44m

0 Kelvin Cross Sections

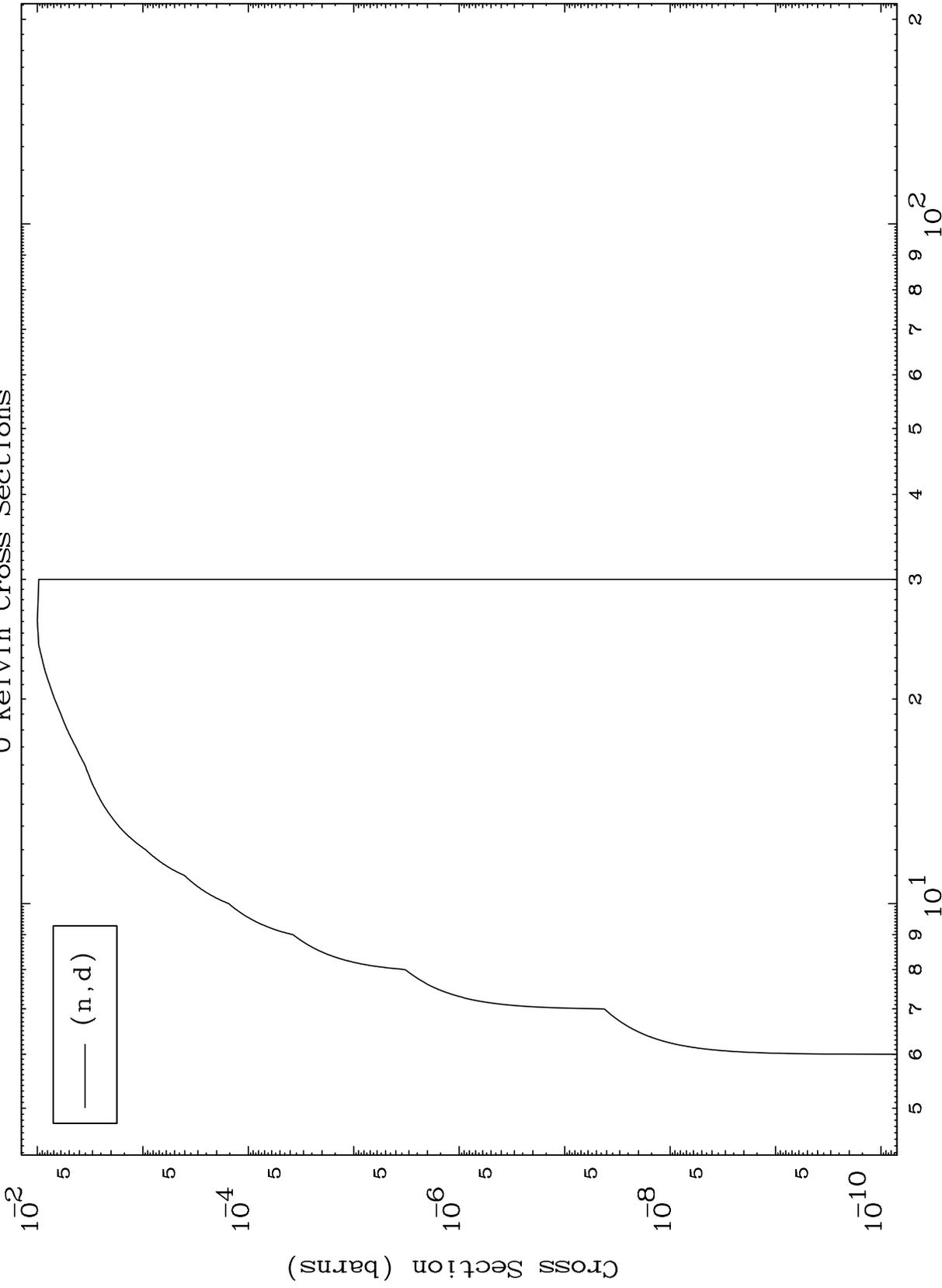


MAT 2123

( $\alpha, d$ ) Levels

21-Sc-44m

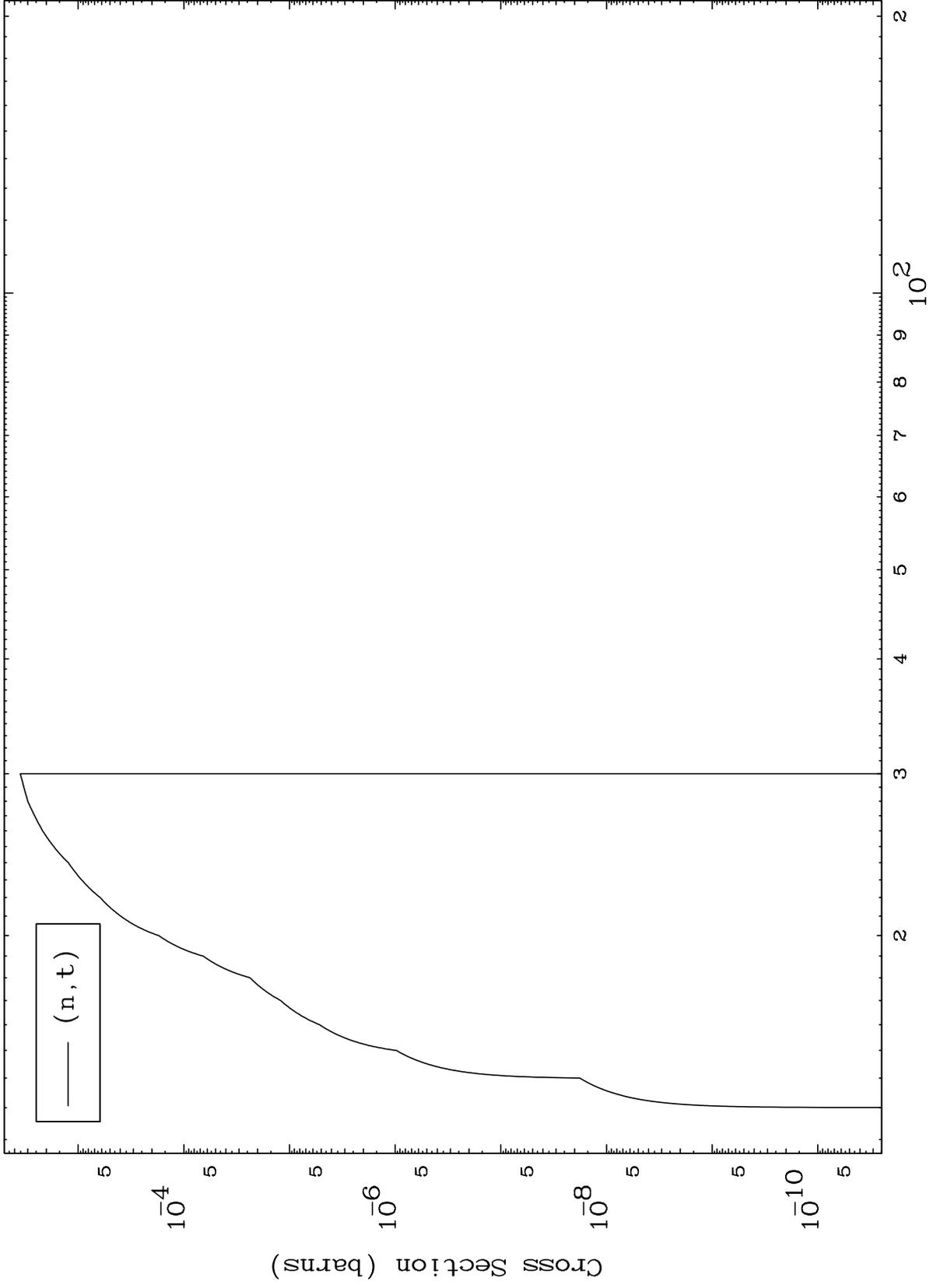
0 Kelvin Cross Sections



8

Incident Energy (MeV)

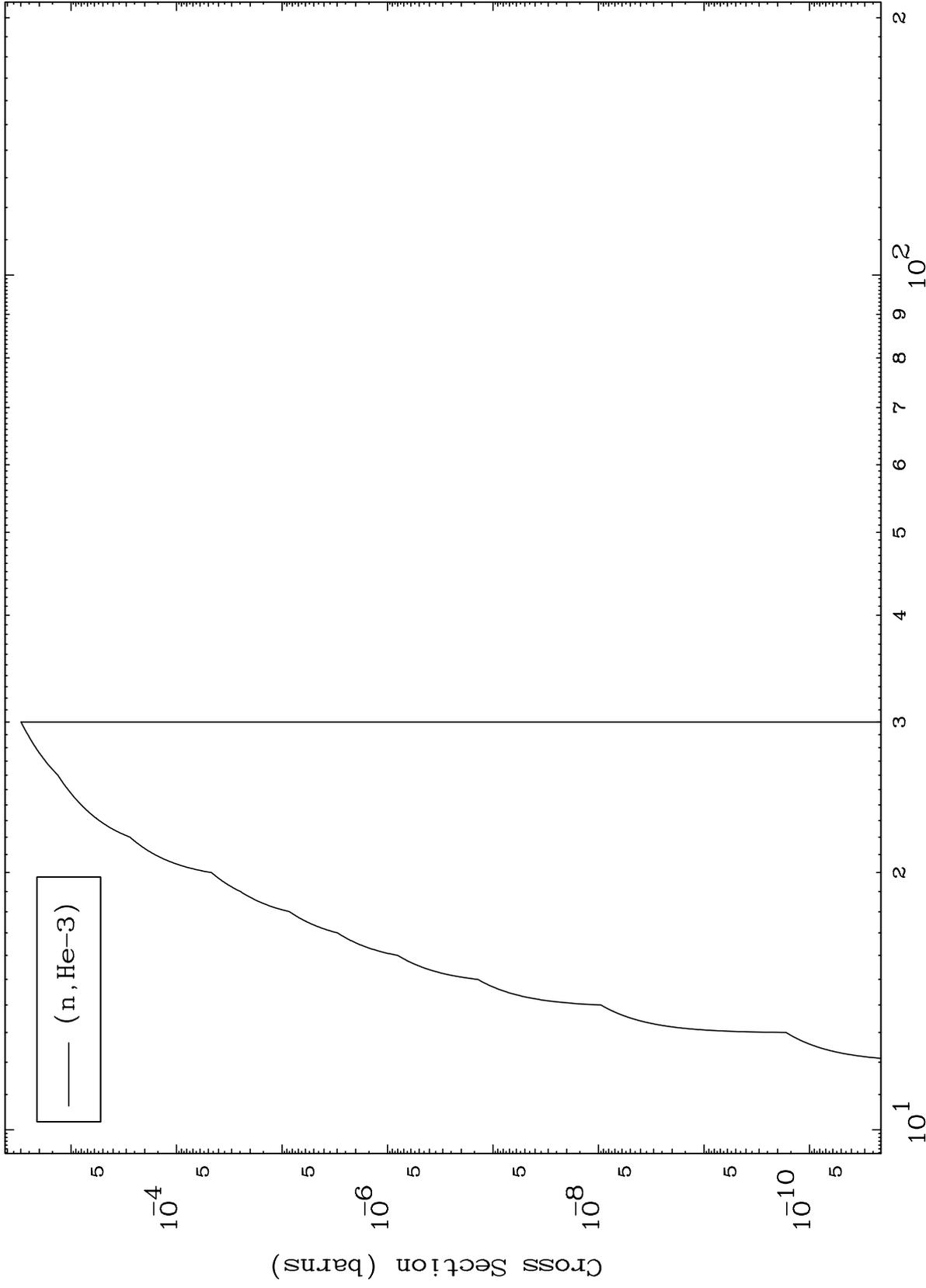
21-Sc-44m



MAT 2123

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

21-Sc-44m



Incident Energy (MeV)

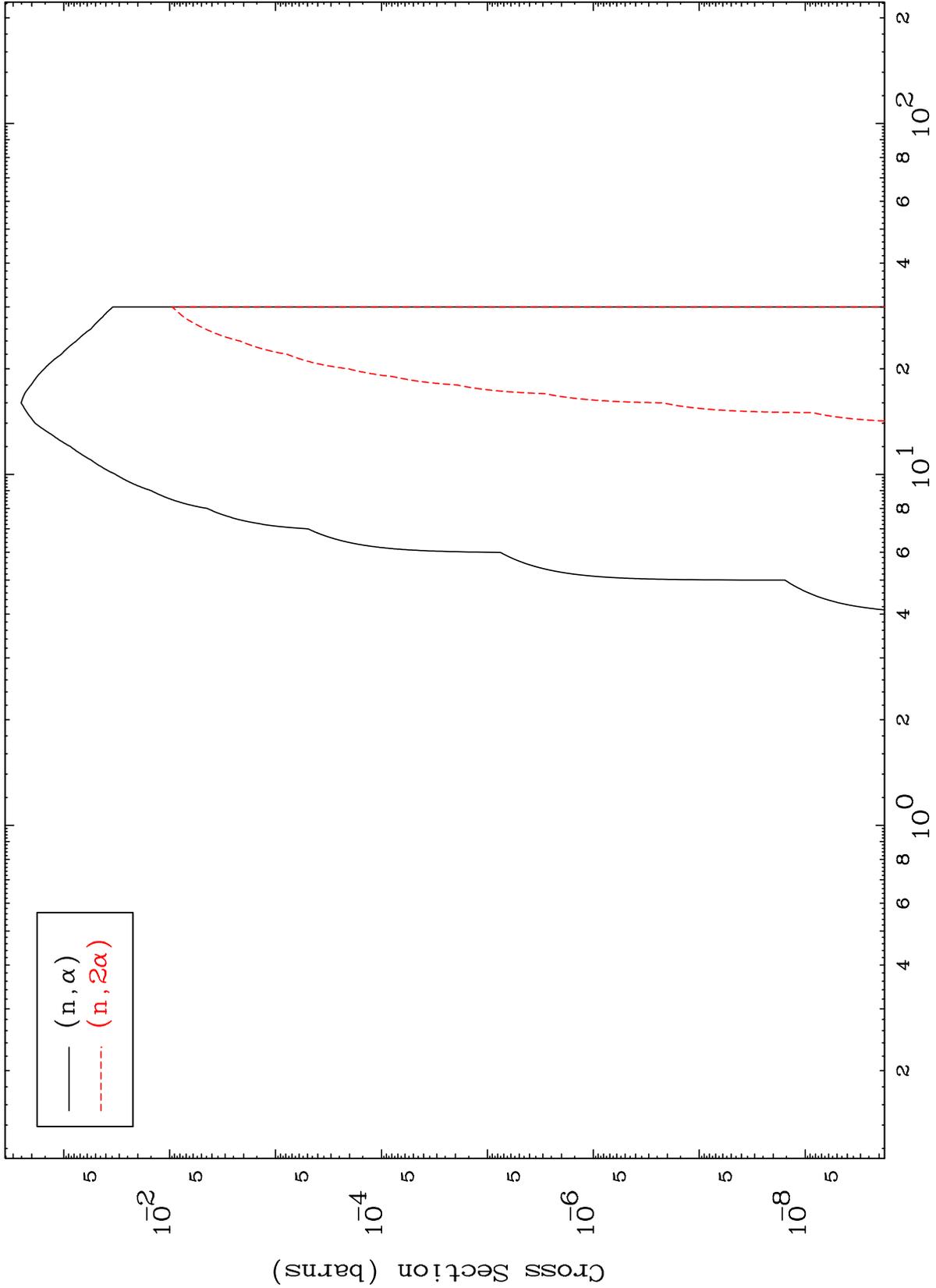
21-Sc-44m

MAT 2123

( $\alpha, \alpha$ ) Levels

21-Sc-44m

0 Kelvin Cross Sections

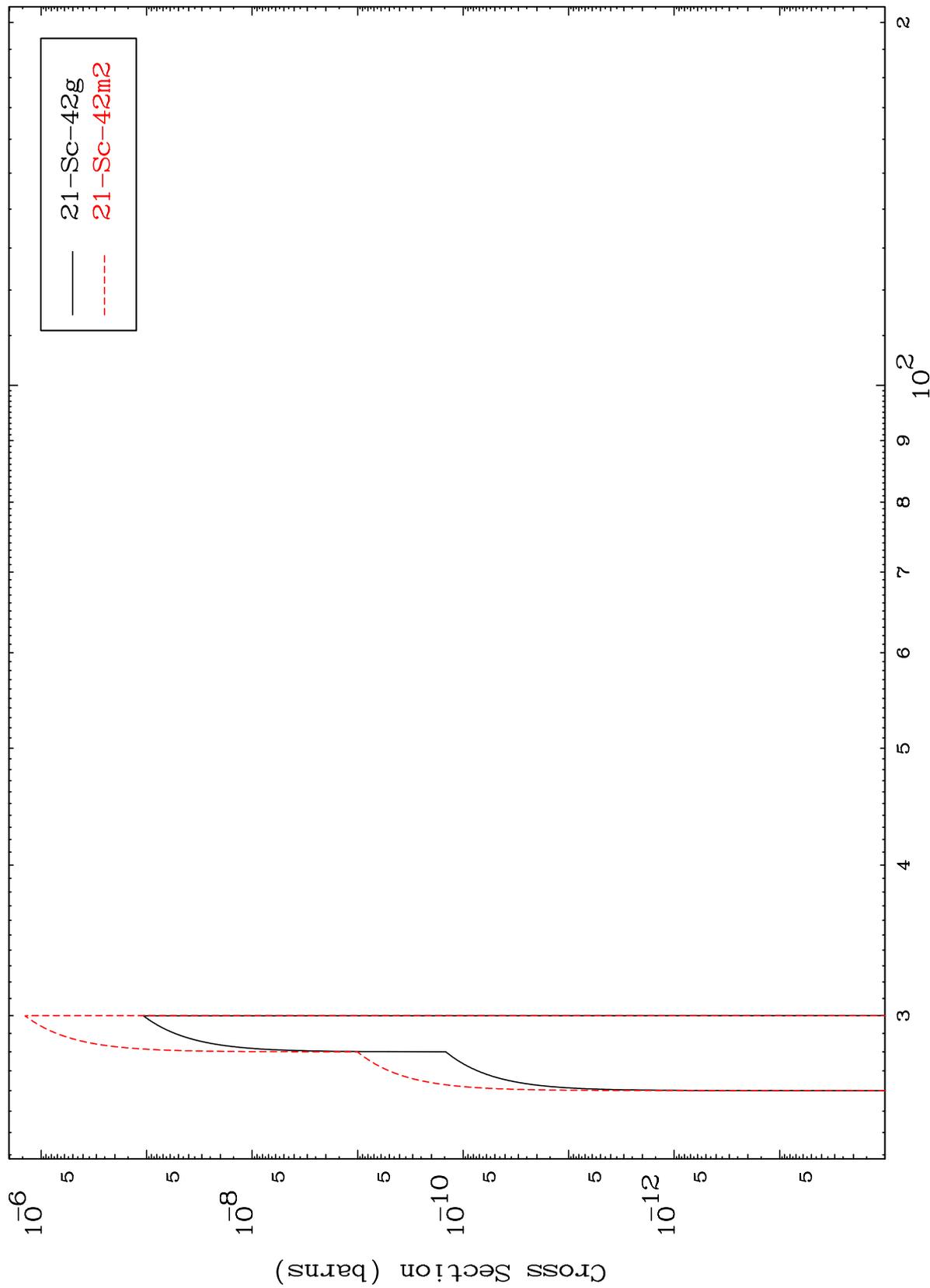


MAT 2123

(n,2n)  $\alpha$

21-Sc-44m

Radionuclide Production Cross Section



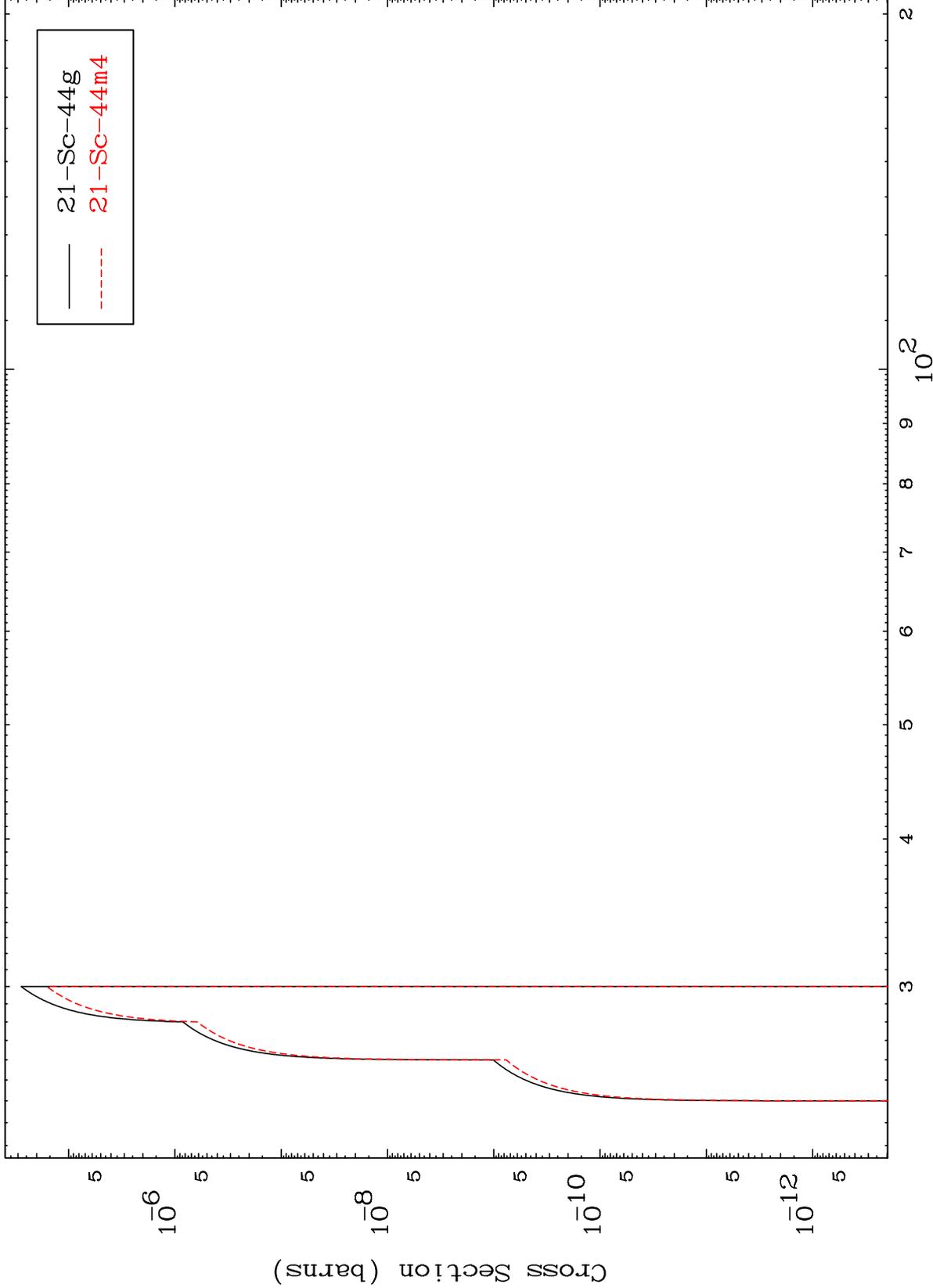
21-Sc-42g  
21-Sc-42m2

12

Incident Energy (MeV)

21-Sc-44m

Radionuclide Production Cross Section

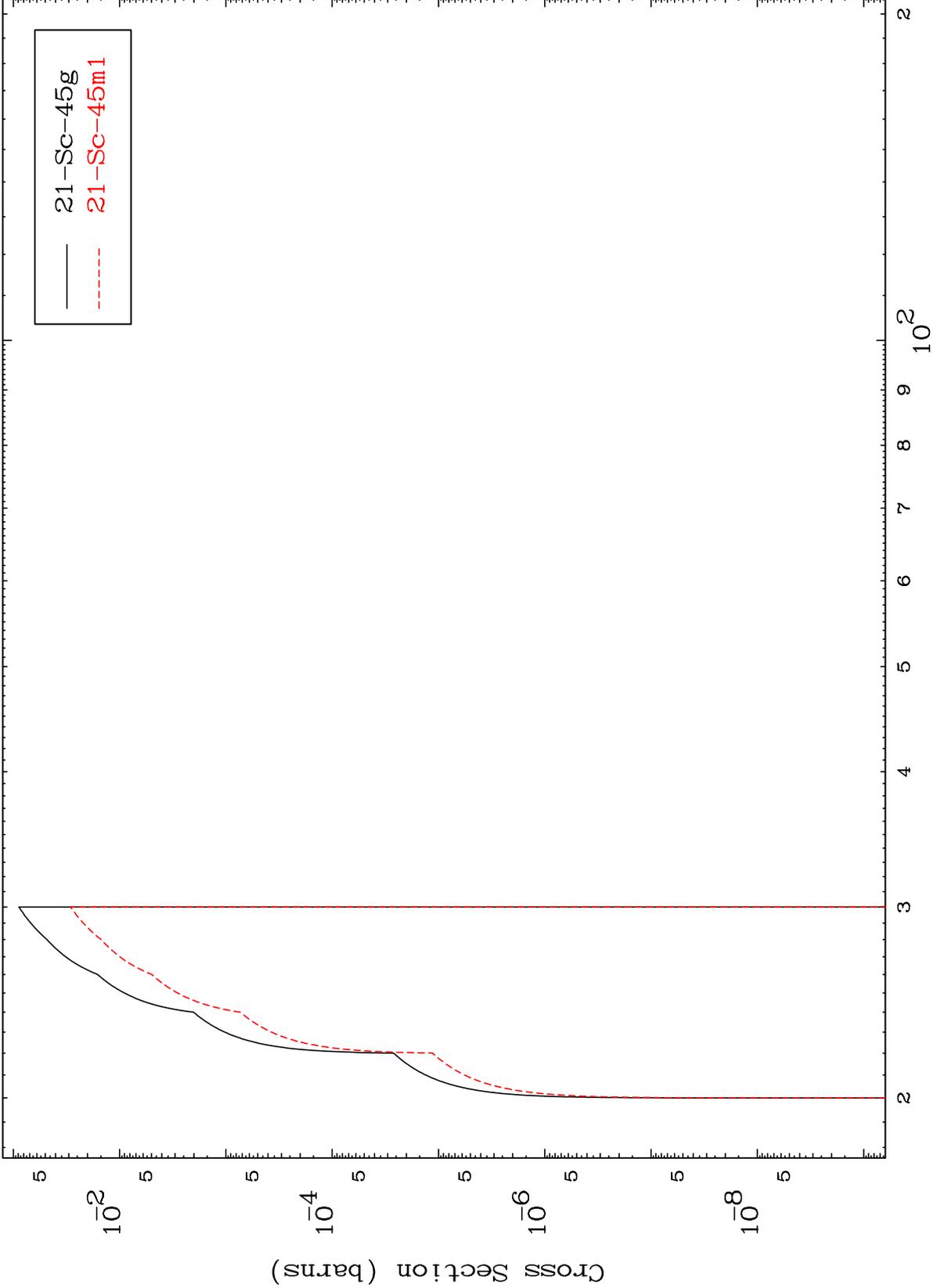


MAT 2123

(n,2n) p

21-Sc-44m

Radionuclide Production Cross Section



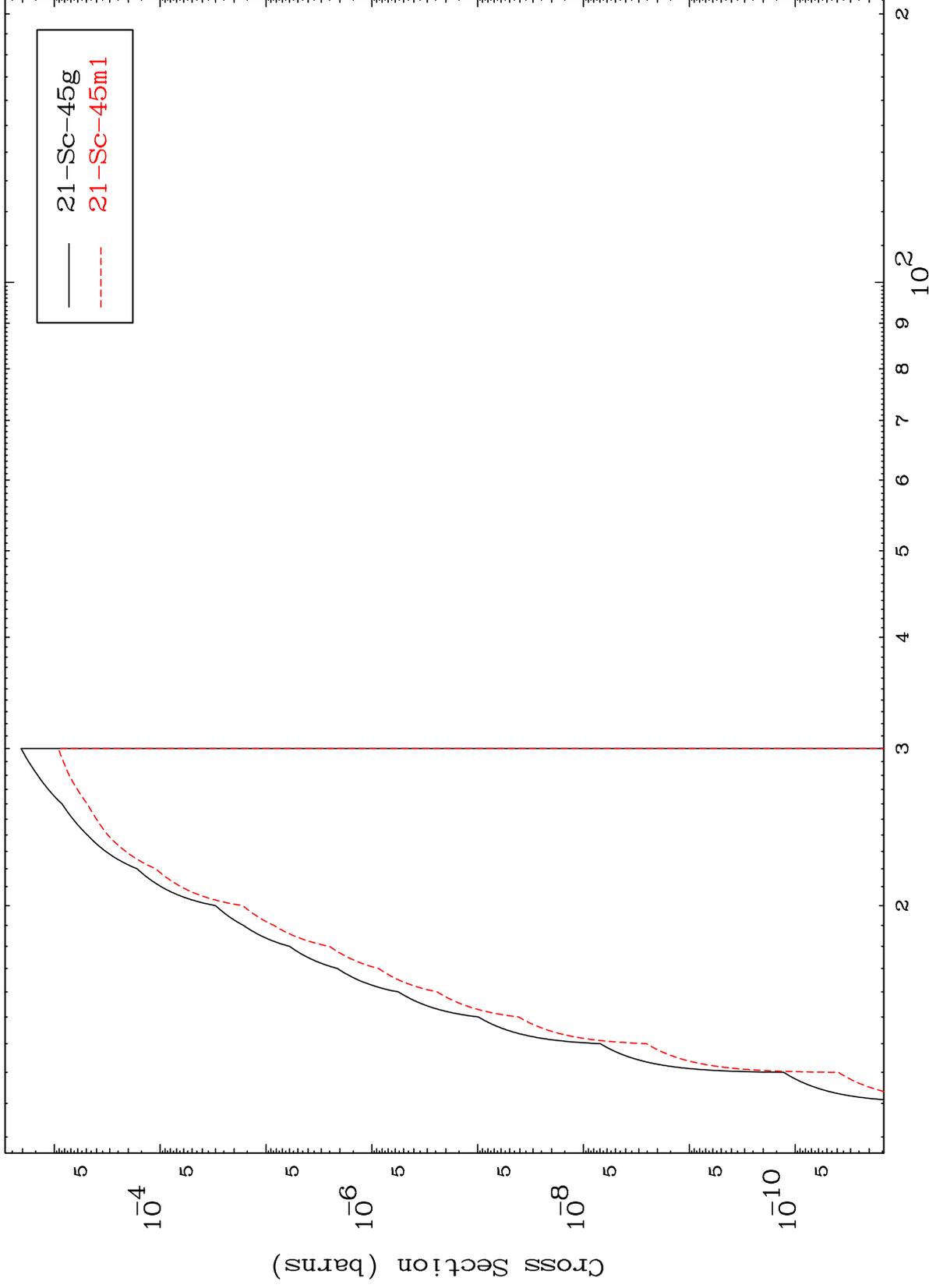
21-Sc-45g  
21-Sc-45m1

14

Incident Energy (MeV)

21-Sc-44m

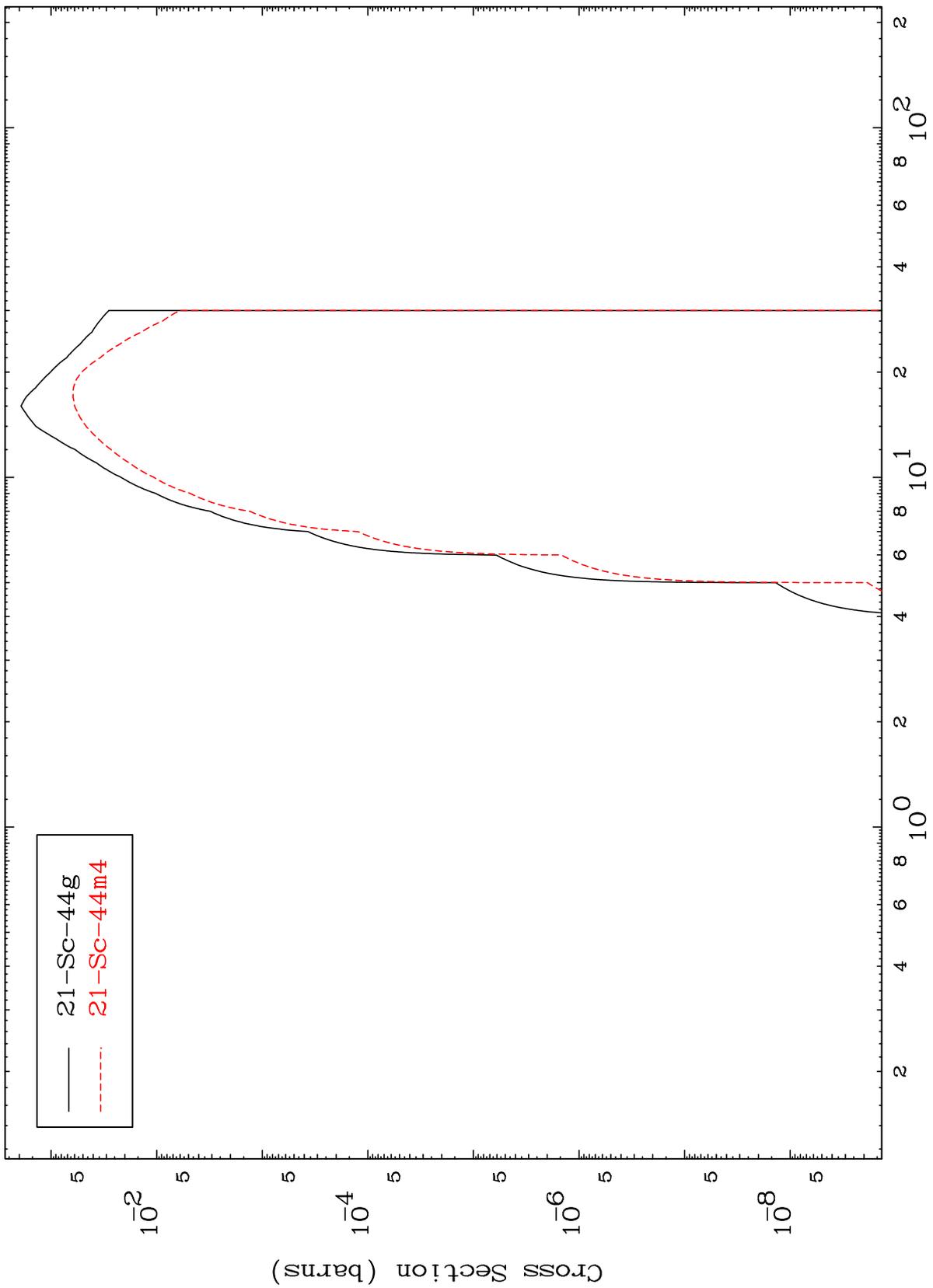
Radionuclide Production Cross Section



MAT 2123

21-Sc-44m

Radionuclide Production Cross Section



21-Sc-44m

Incident Energy (MeV)

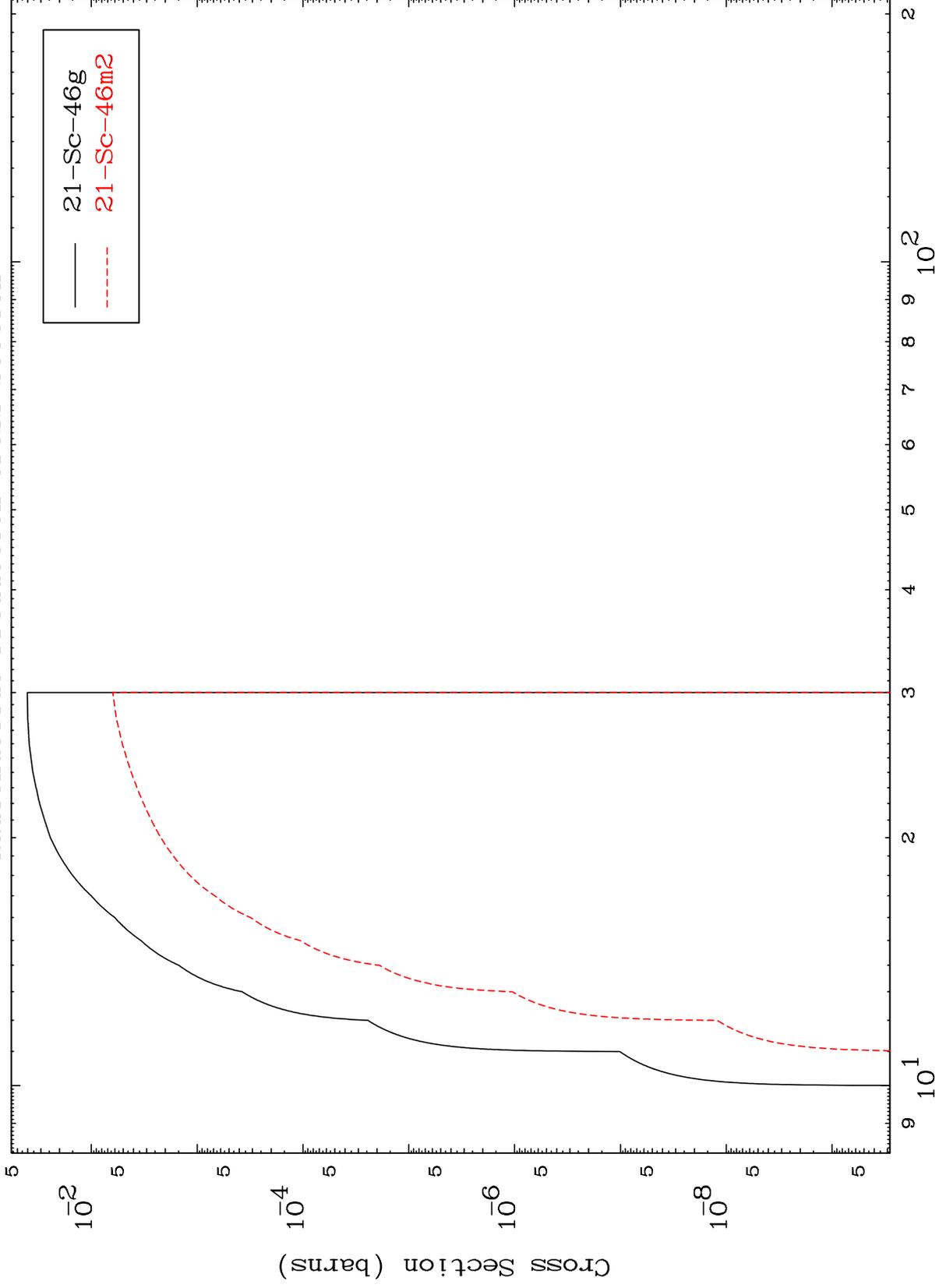
16

MAT 2123

(n,2p)

21-Sc-44m

Radionuclide Production Cross Section



17

Incident Energy (MeV)

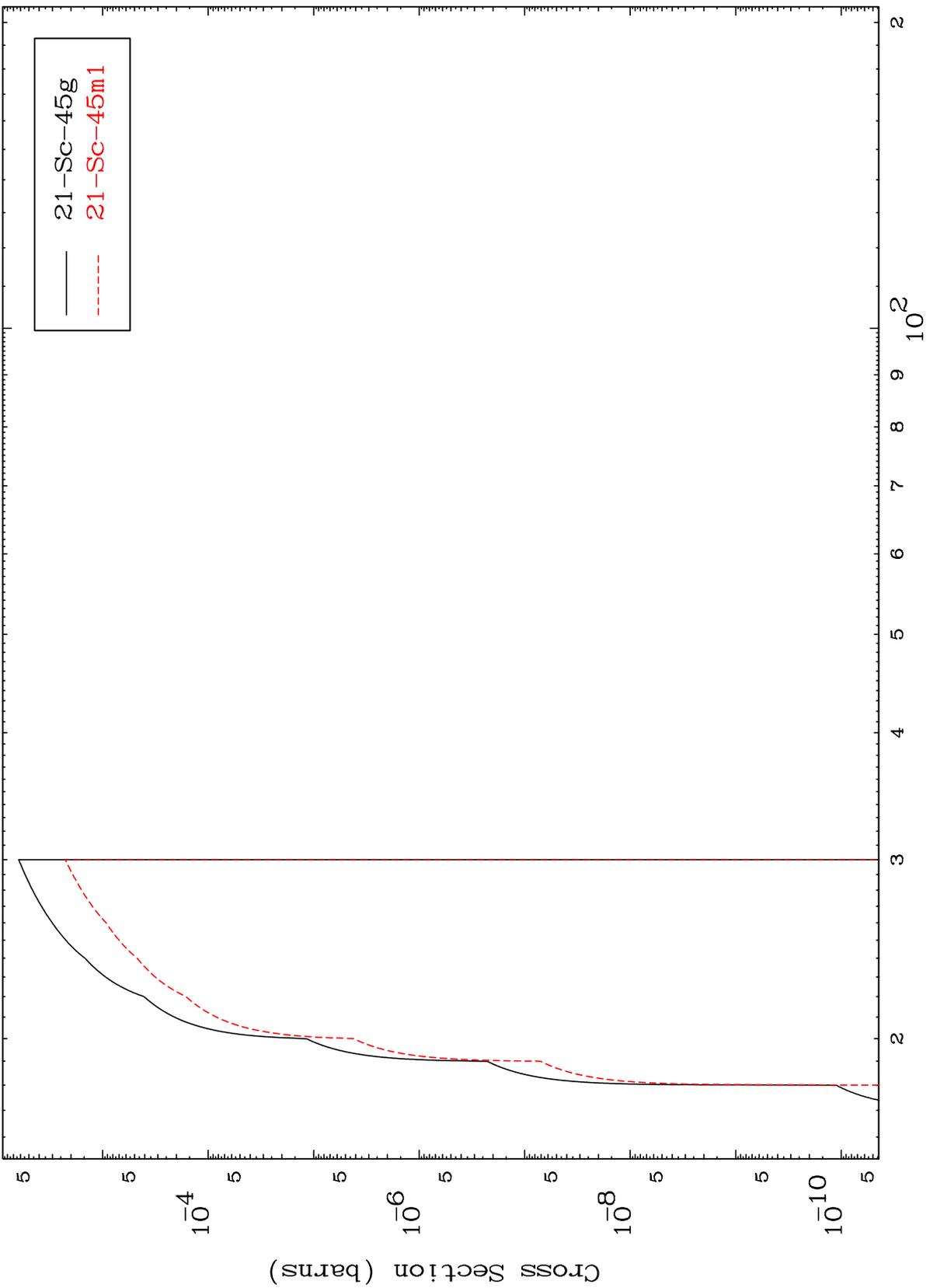
21-Sc-44m

MAT 2123

(n,p) d

21-Sc-44m

Radionuclide Production Cross Section

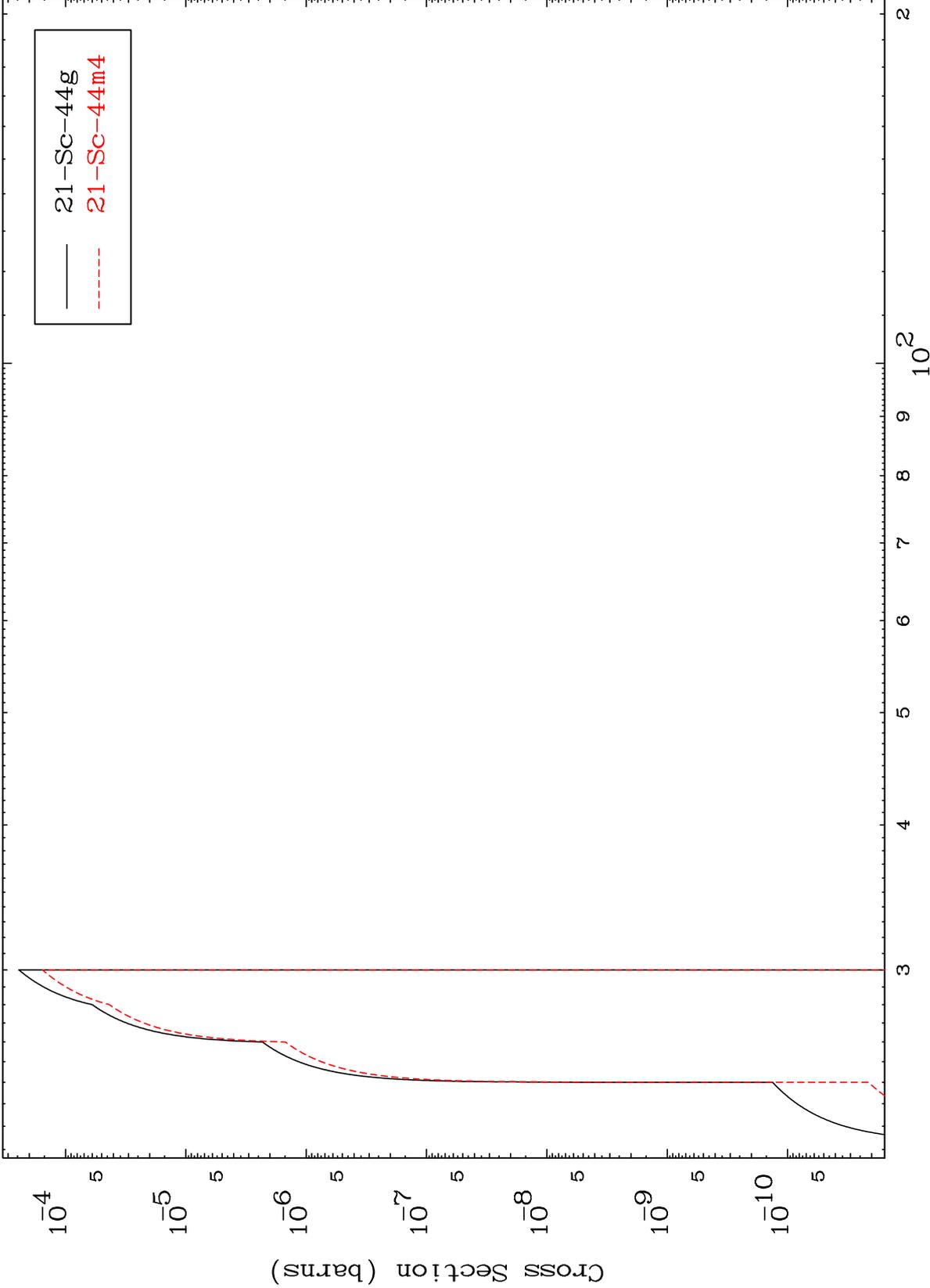


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Incident Energy (MeV)

21-Sc-44m

Radionuclide Production Cross Section



— 21-Sc-44g  
- - - 21-Sc-44m4