

Program EVALPLOT  
(Version 2021-1)

by

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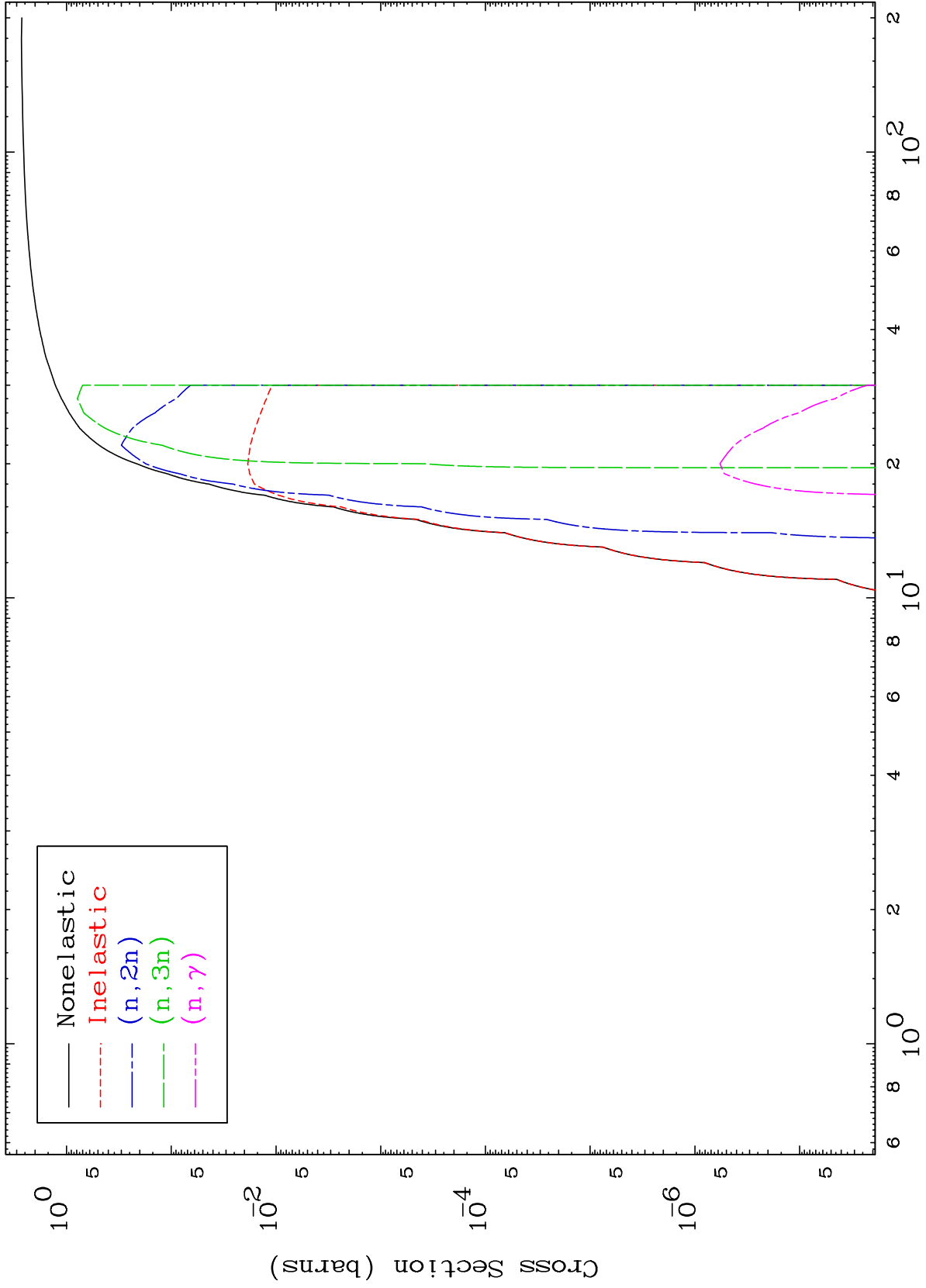
Press Mouse Button to Start

MAT 7343

$\alpha$  Major

73-Ta-186

0 Kelvin Cross Sections



1

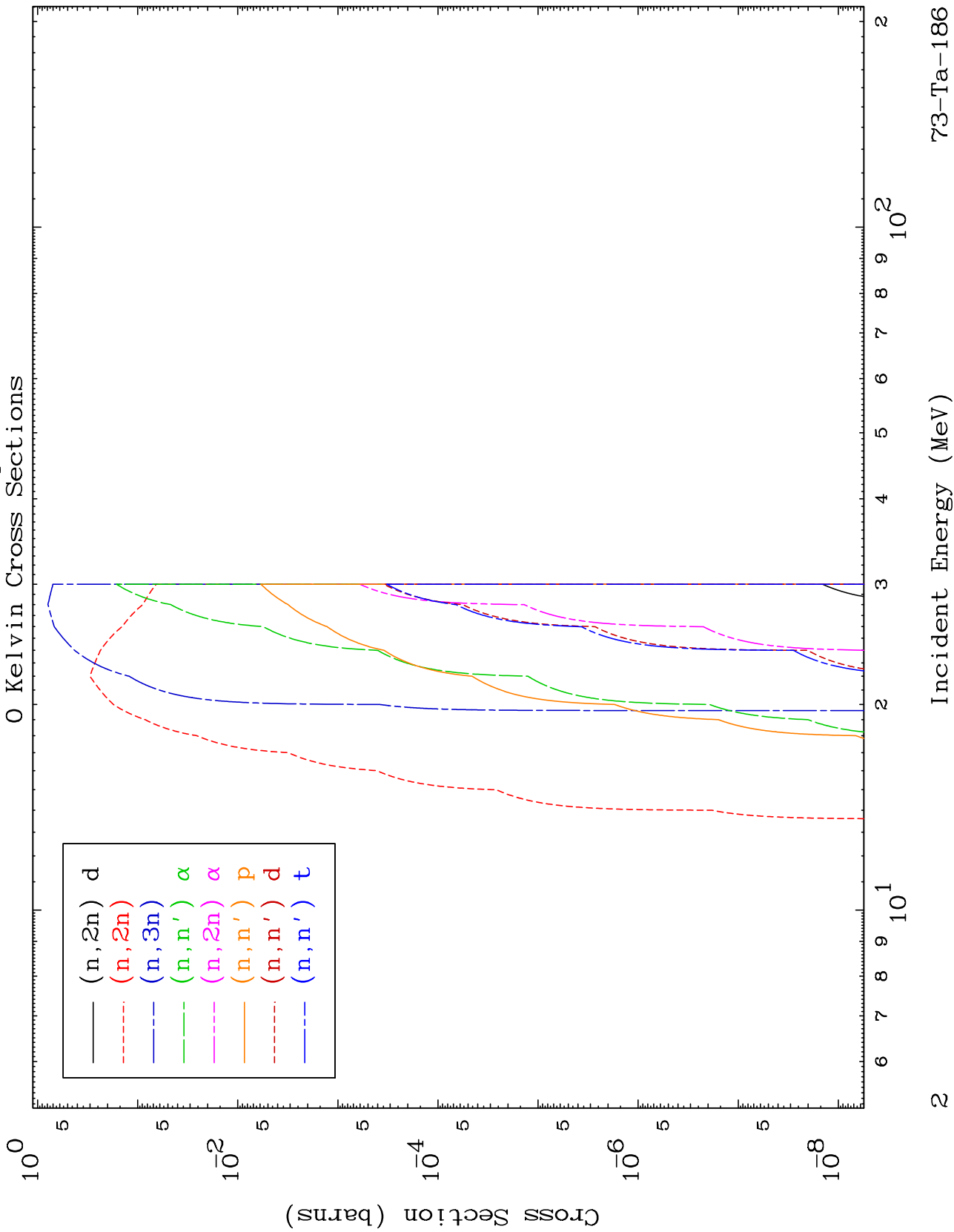
Incident Energy (MeV)

73-Ta-186

MAT 7343

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

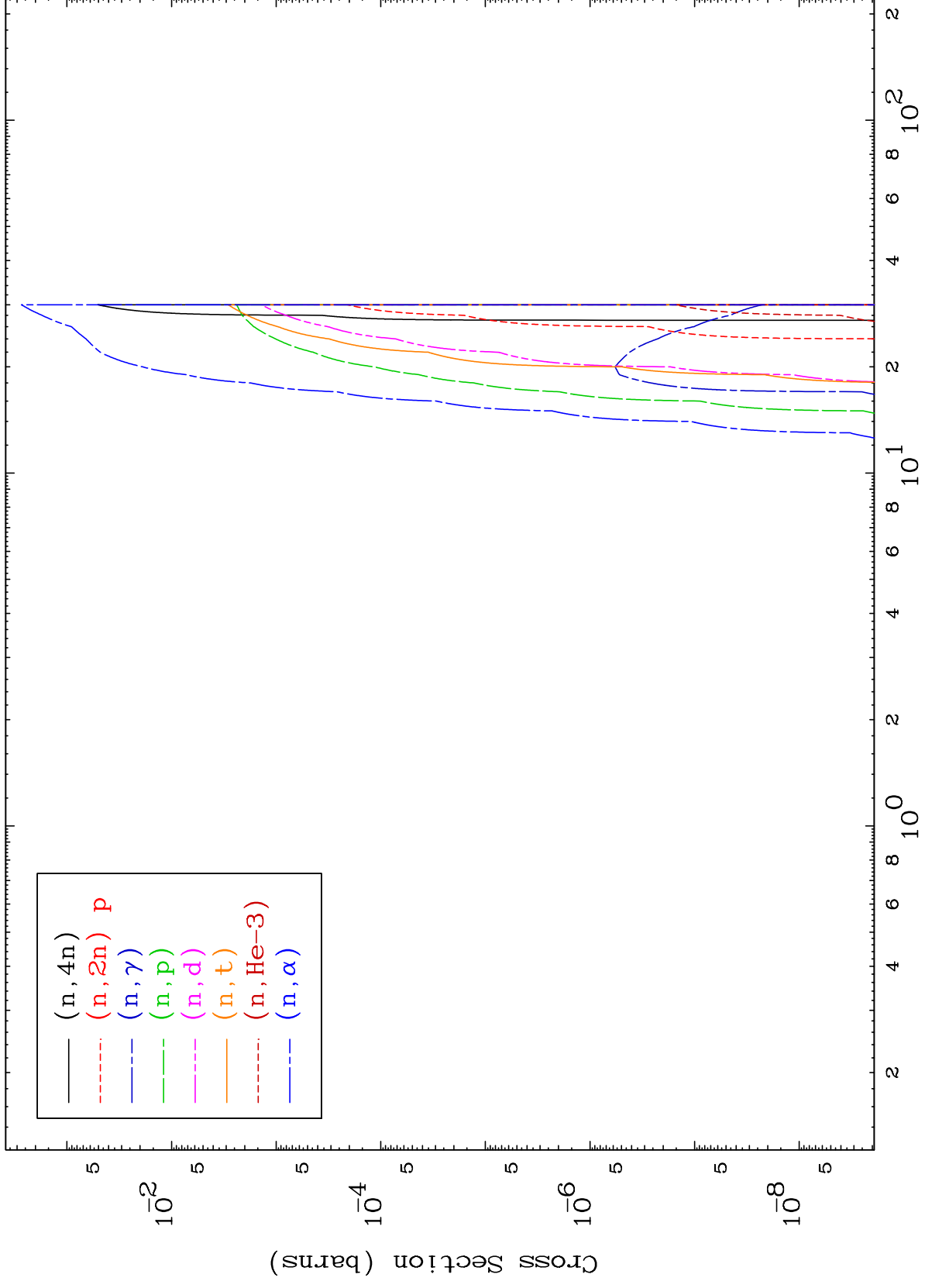
73-Ta-186



MAT 7343

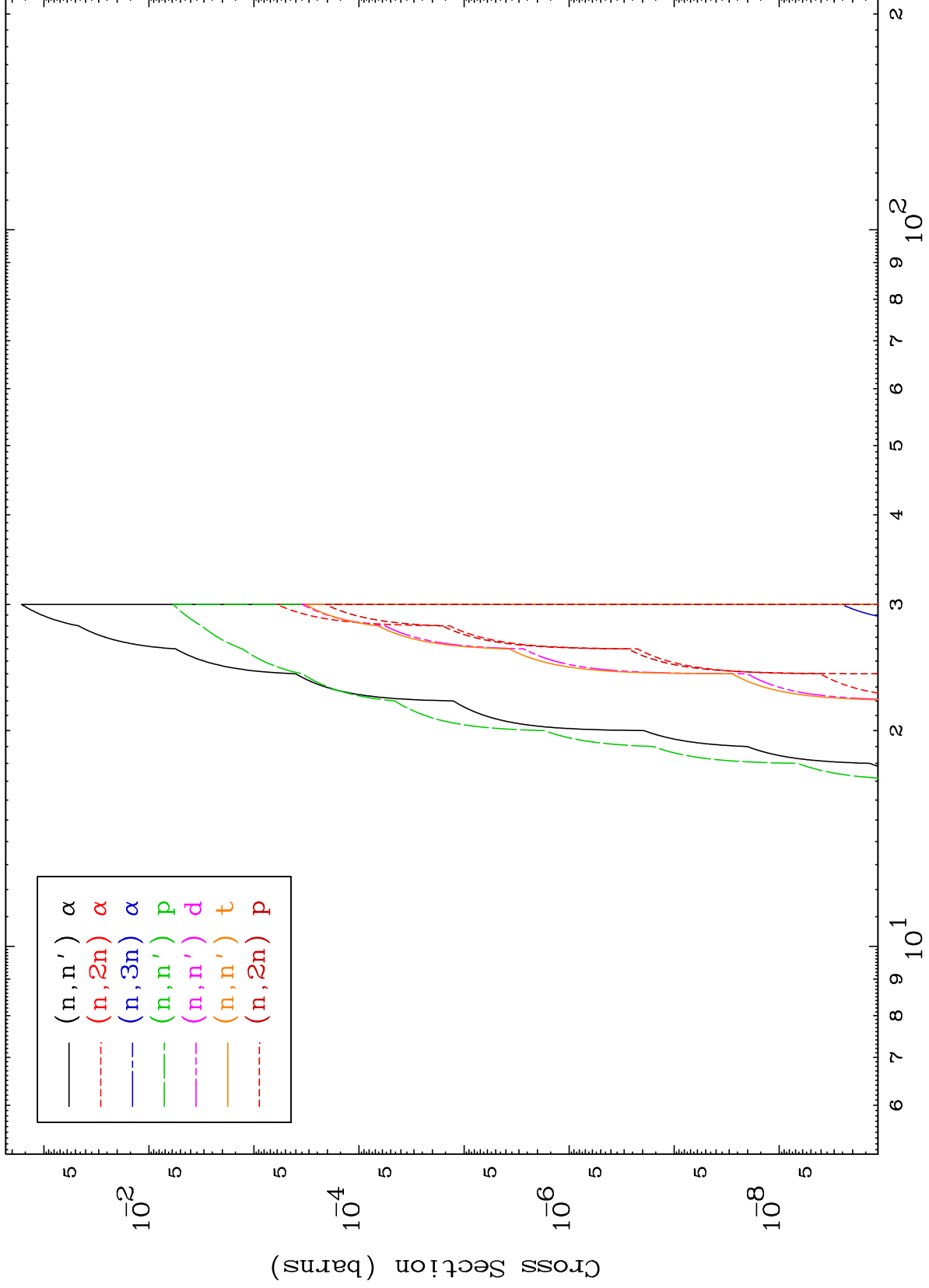
$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

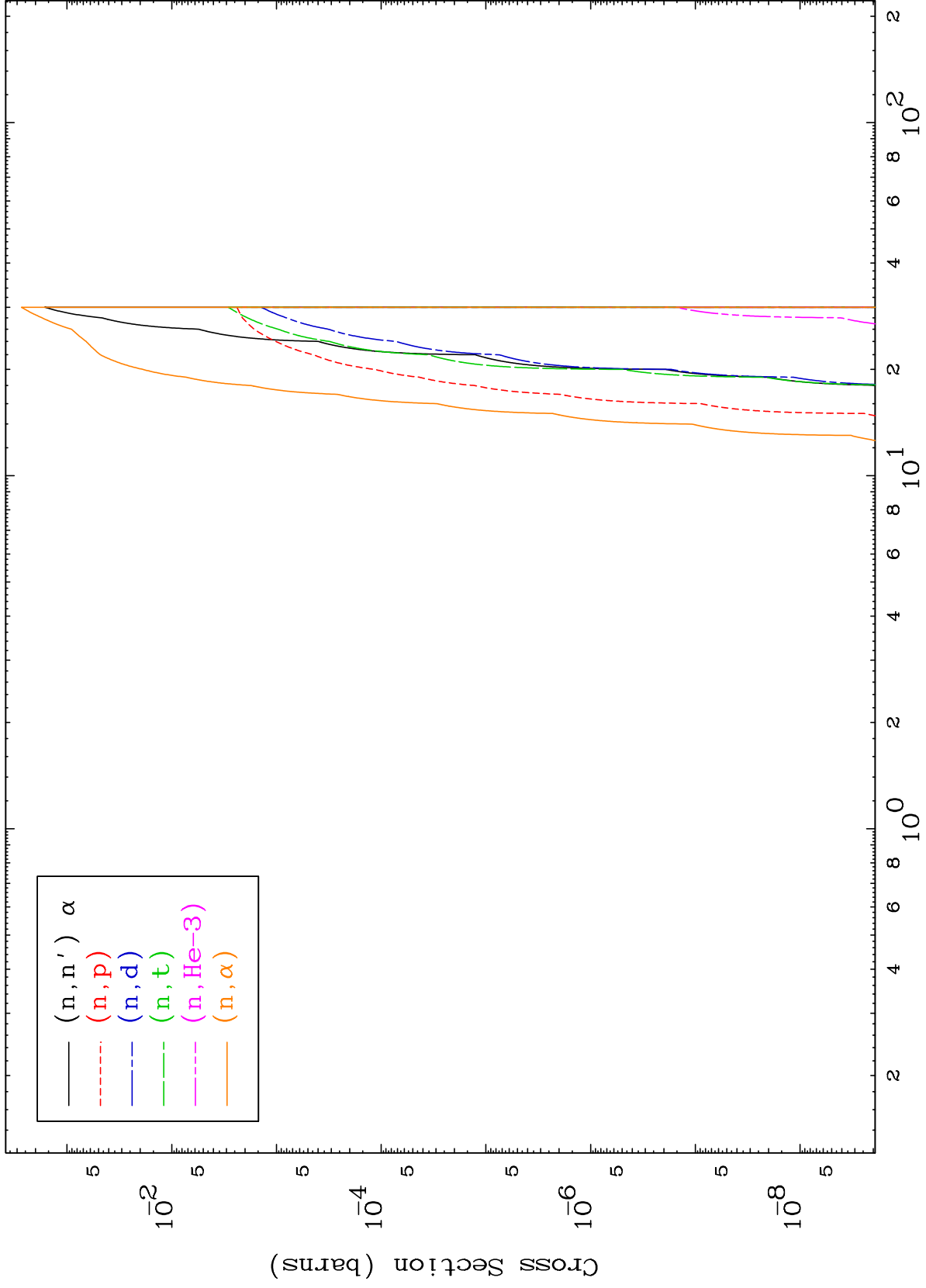
73-Ta-186



73-Ta-186

Incident Energy (MeV)



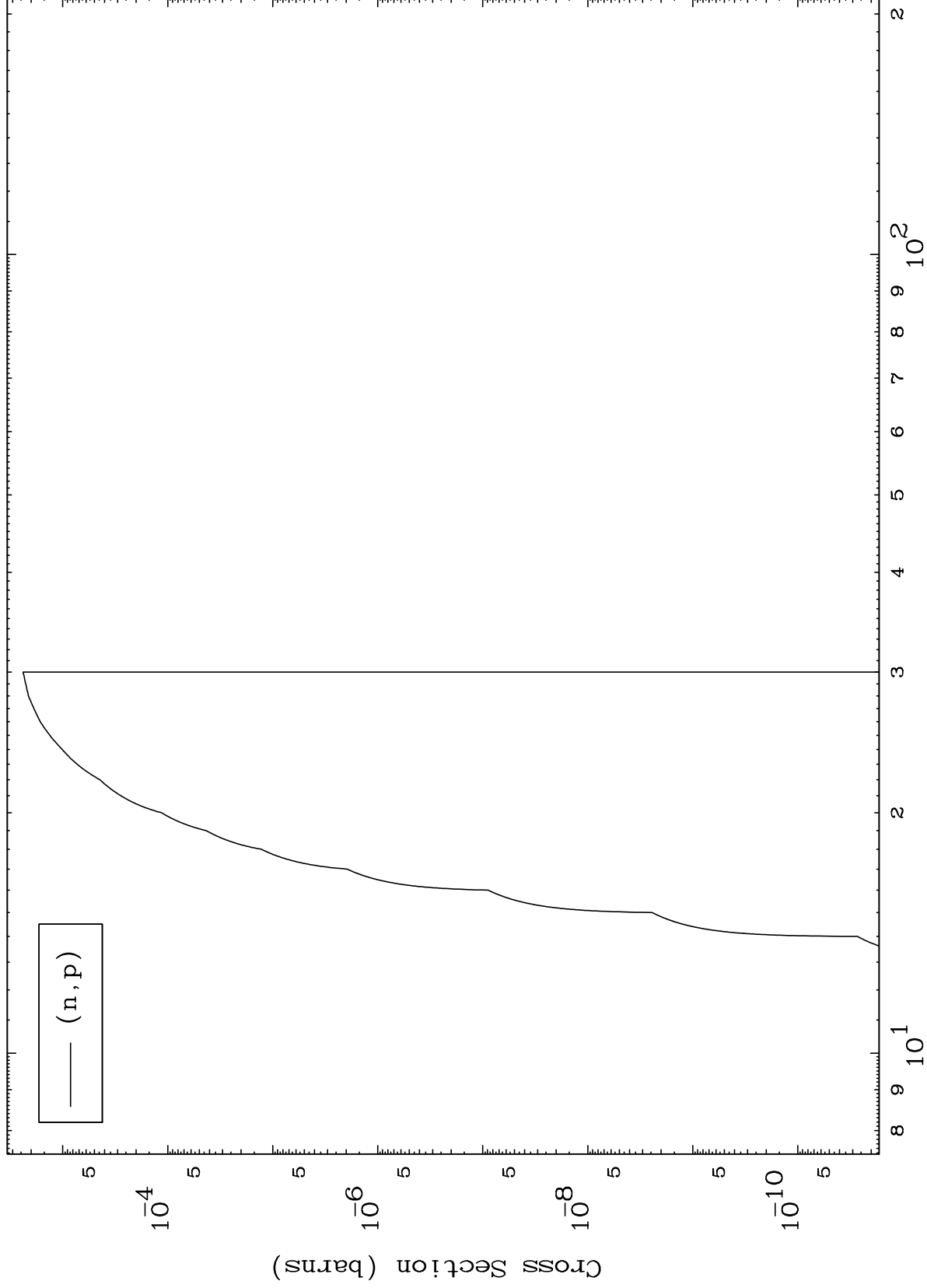


MAT 7343

( $\alpha, p$ ) Levels

$^{73}\text{Ta-186}$

0 Kelvin Cross Sections



Incident Energy (MeV)

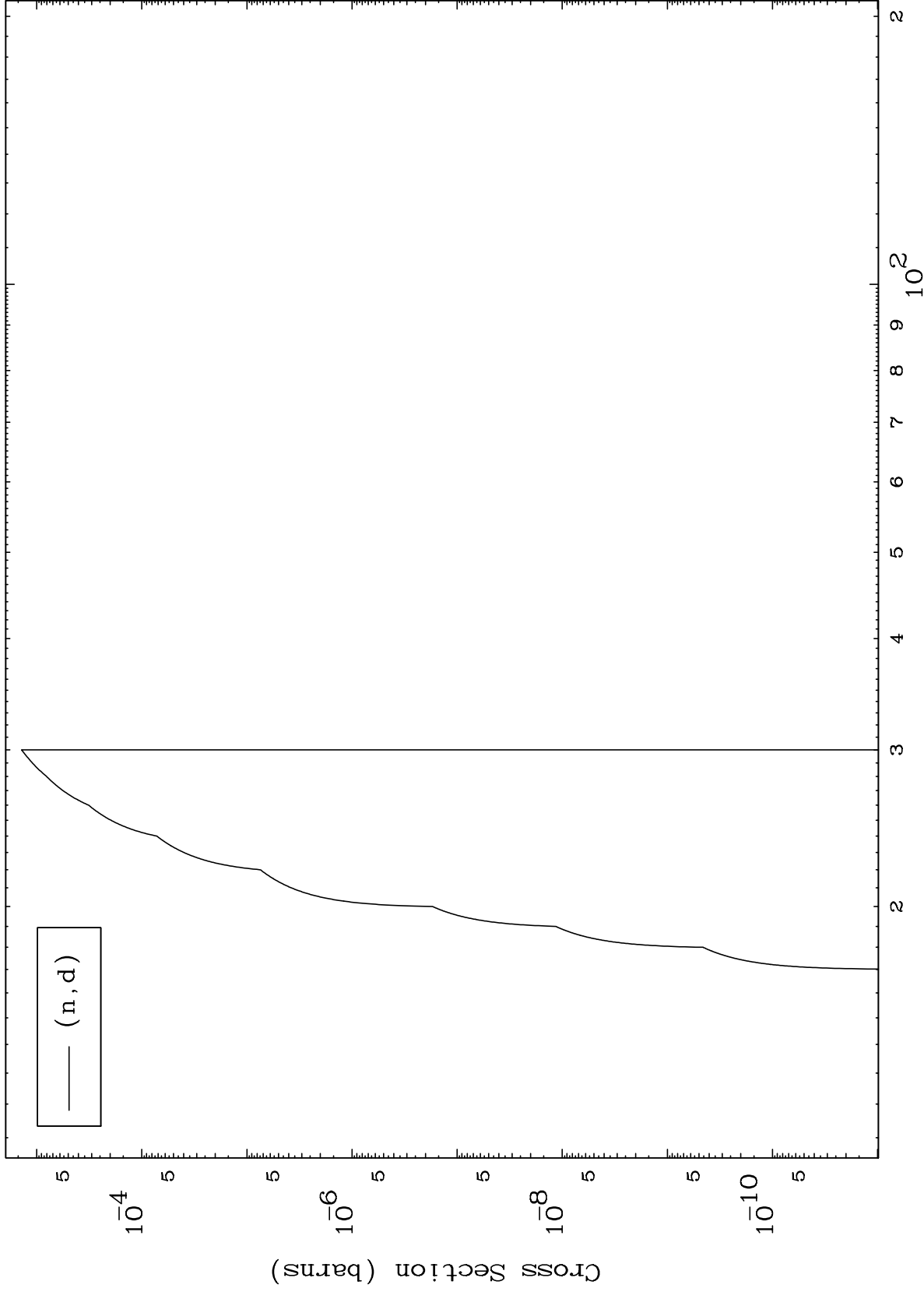
$^{73}\text{Ta-186}$

6

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( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

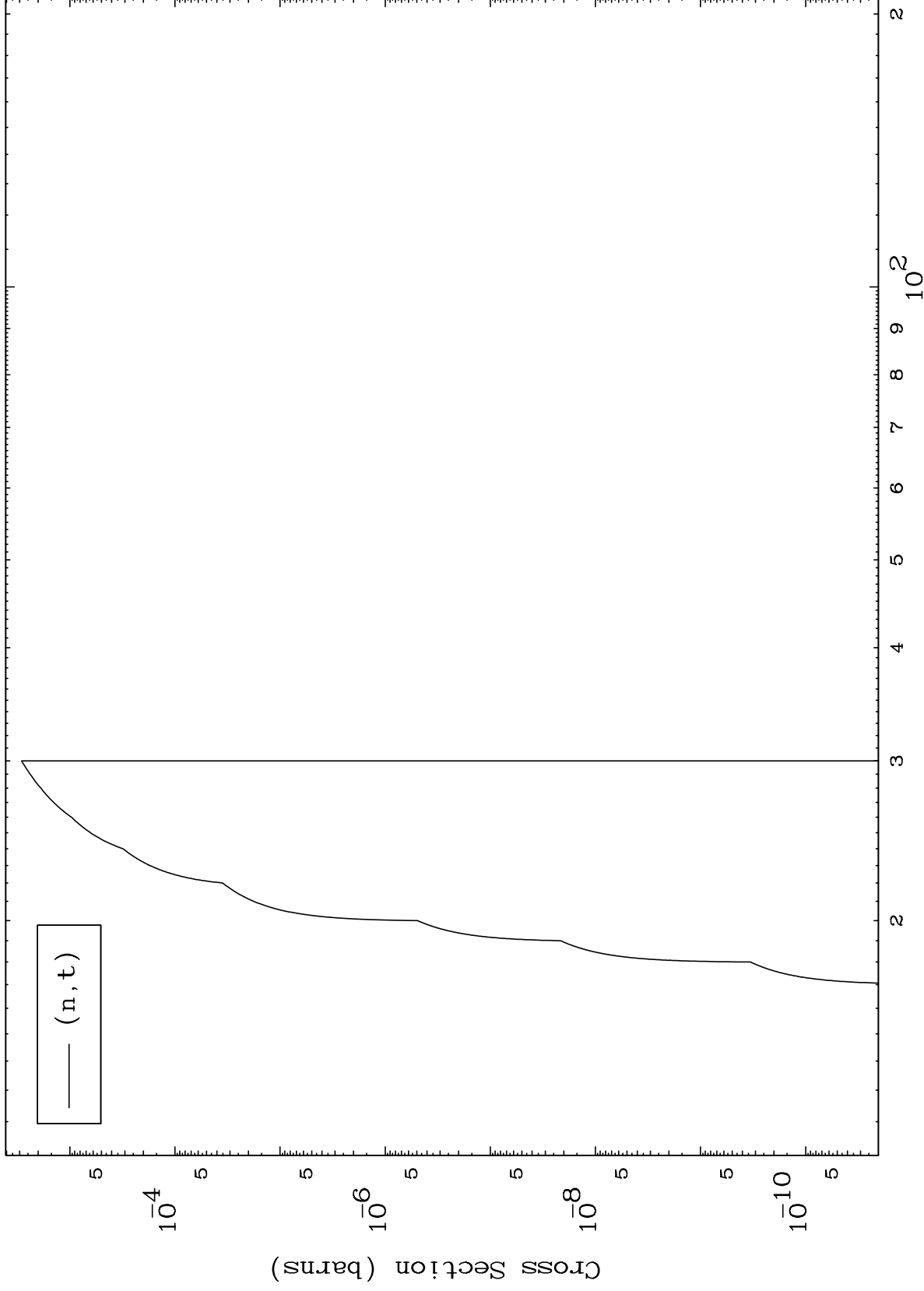
$^{73}\text{Ta-186}$



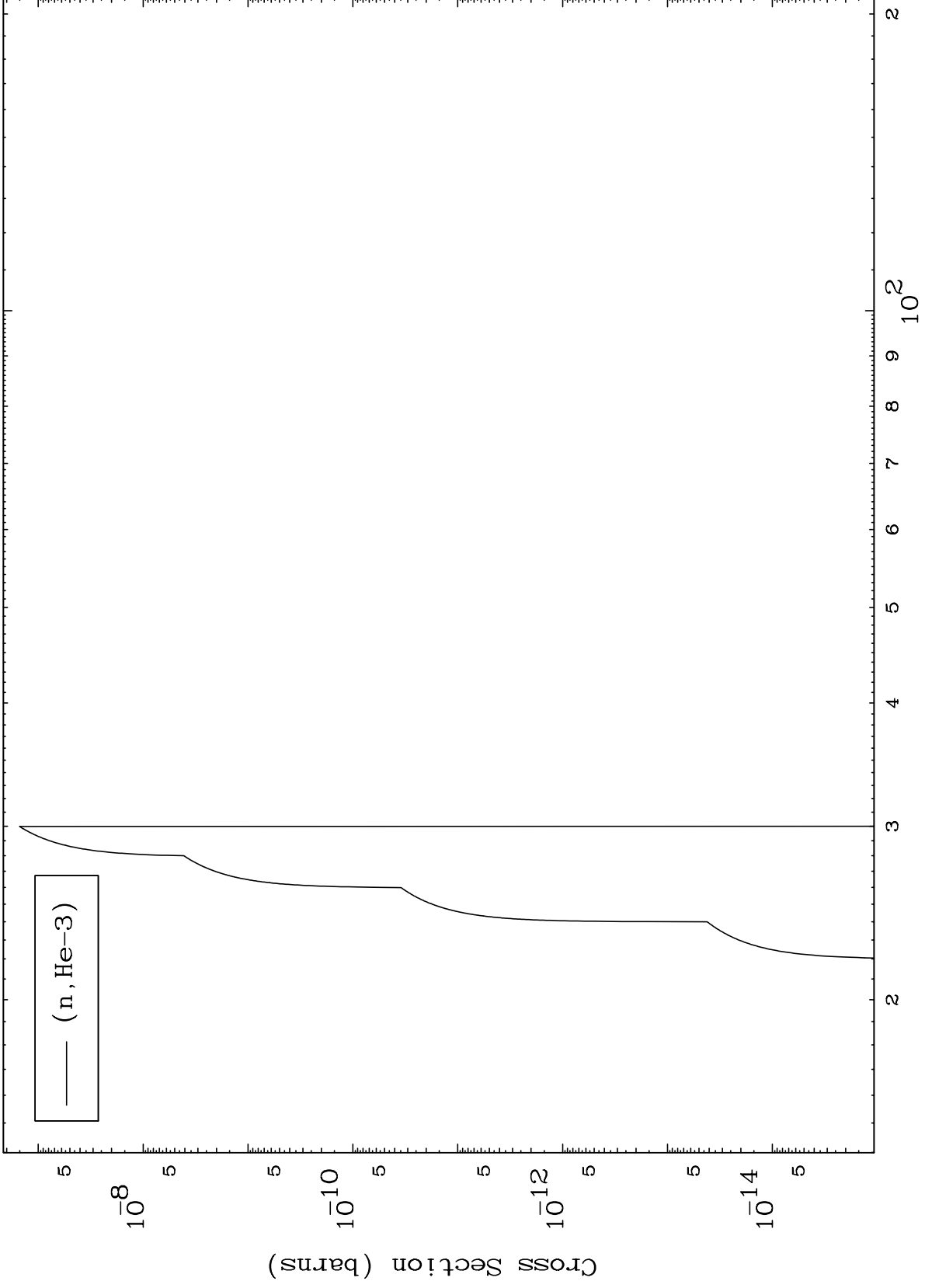
7

Incident Energy (MeV)

$^{73}\text{Ta-186}$



0 Kelvin Cross Sections

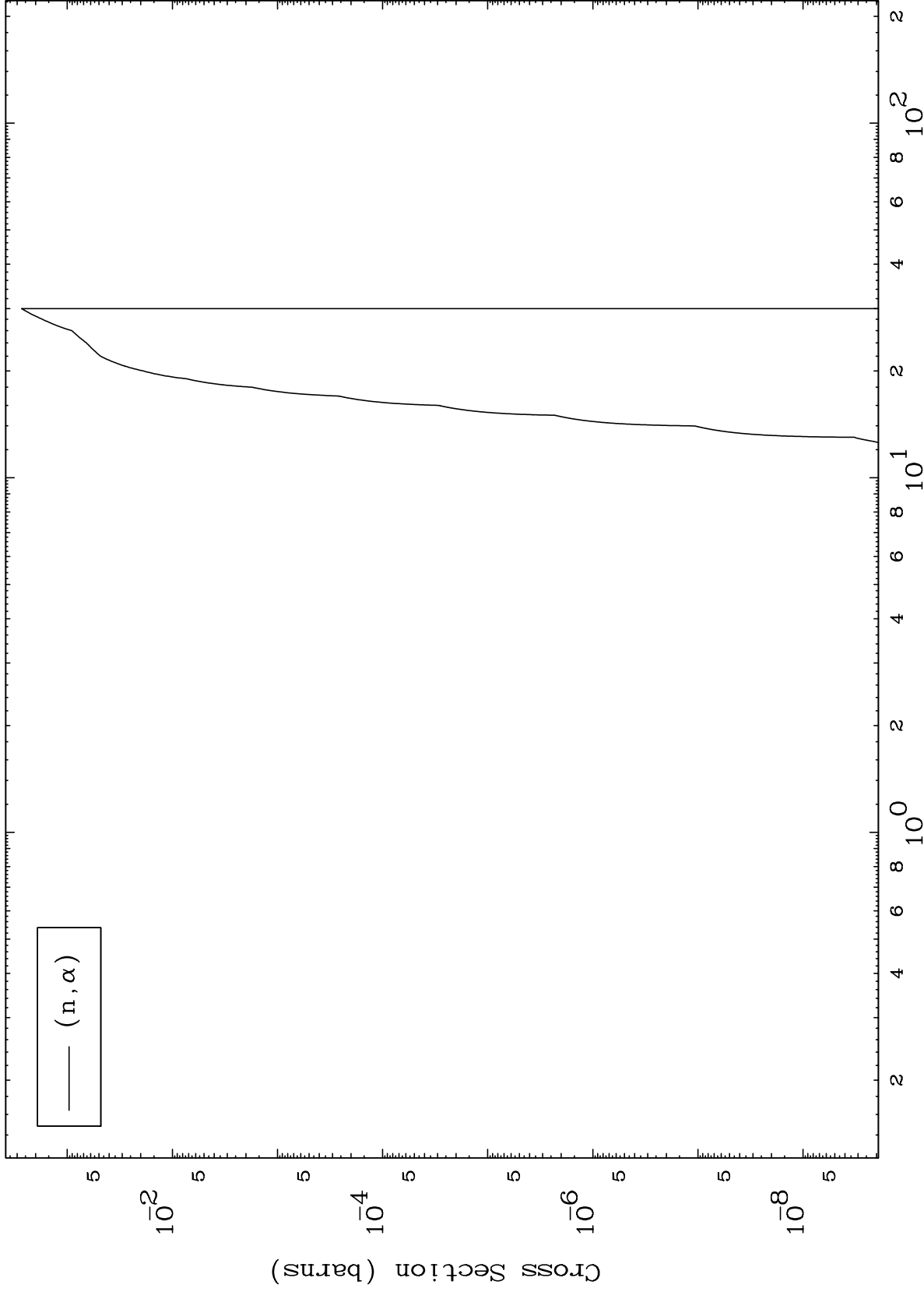


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( $\alpha, \alpha$ ) Levels

73-Ta-186

0 Kelvin Cross Sections



10

Incident Energy (MeV)

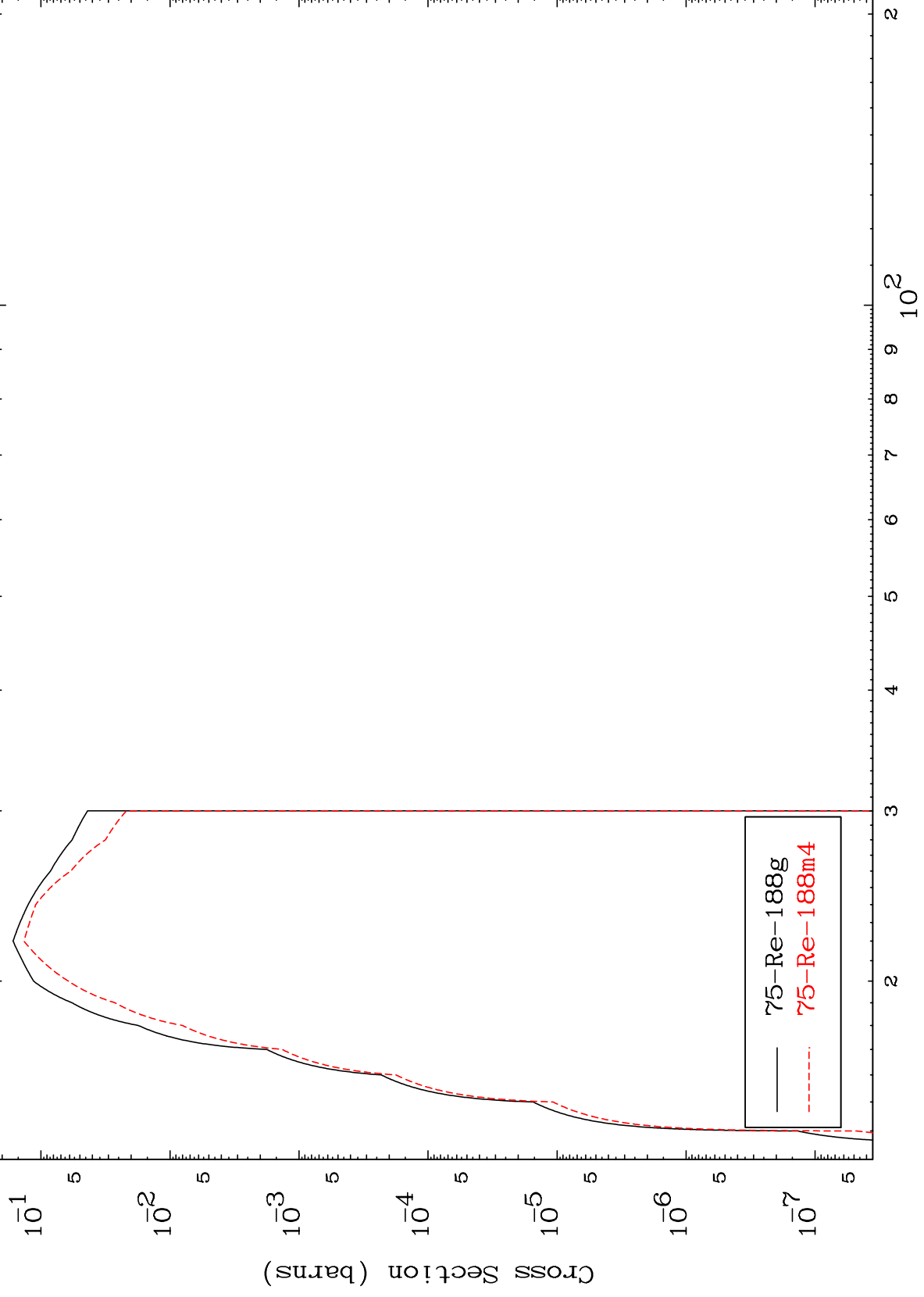
73-Ta-186

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(n,2n)

<sup>73</sup>Ta-186

Radionuclide Production Cross Section

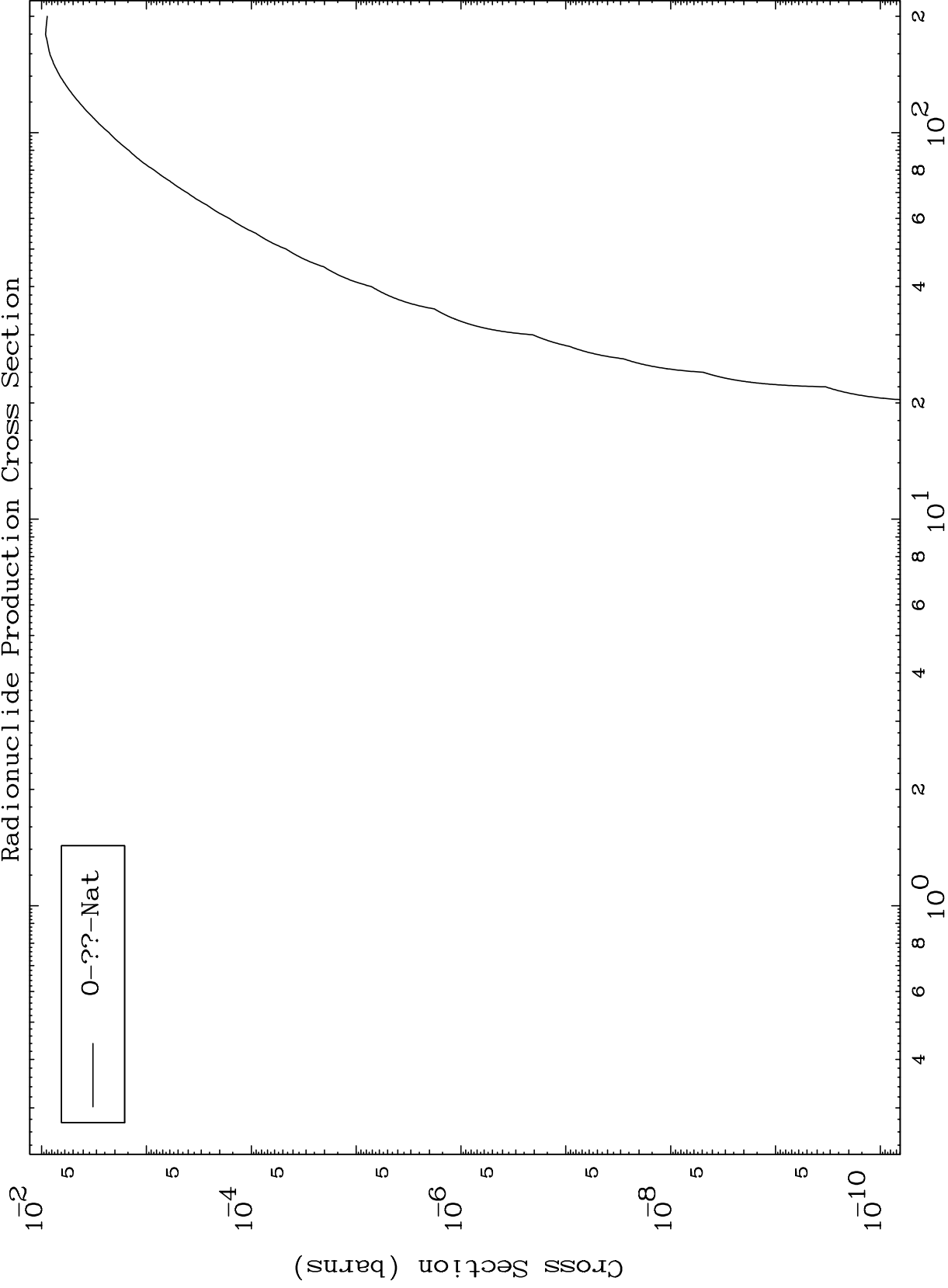


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Fission

<sup>73</sup>Ta-186

Radionuclide Production Cross Section



12

Incident Energy (MeV)

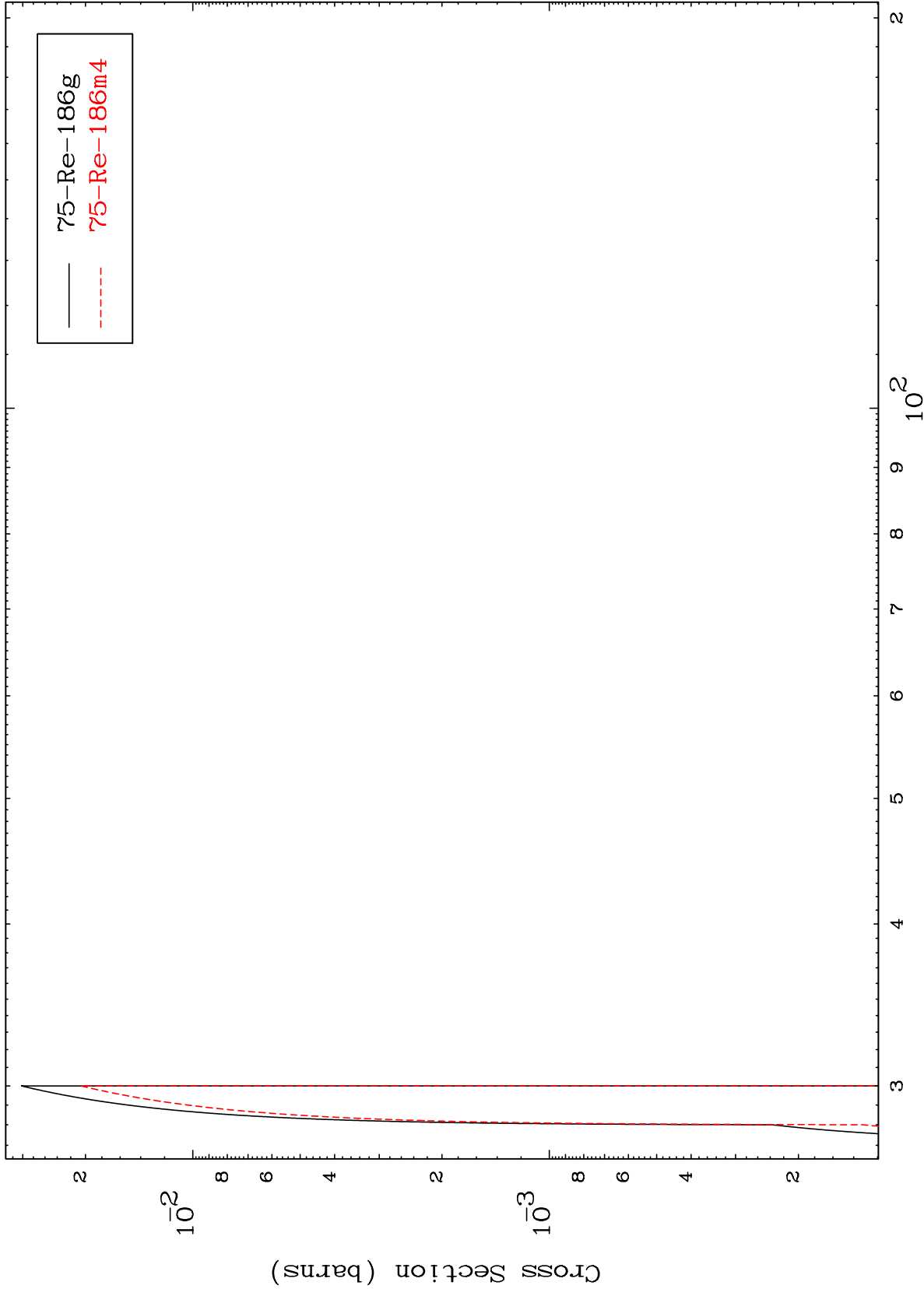
<sup>73</sup>Ta-186

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(n,4n)

73-Ta-186

Radionuclide Production Cross Section



13

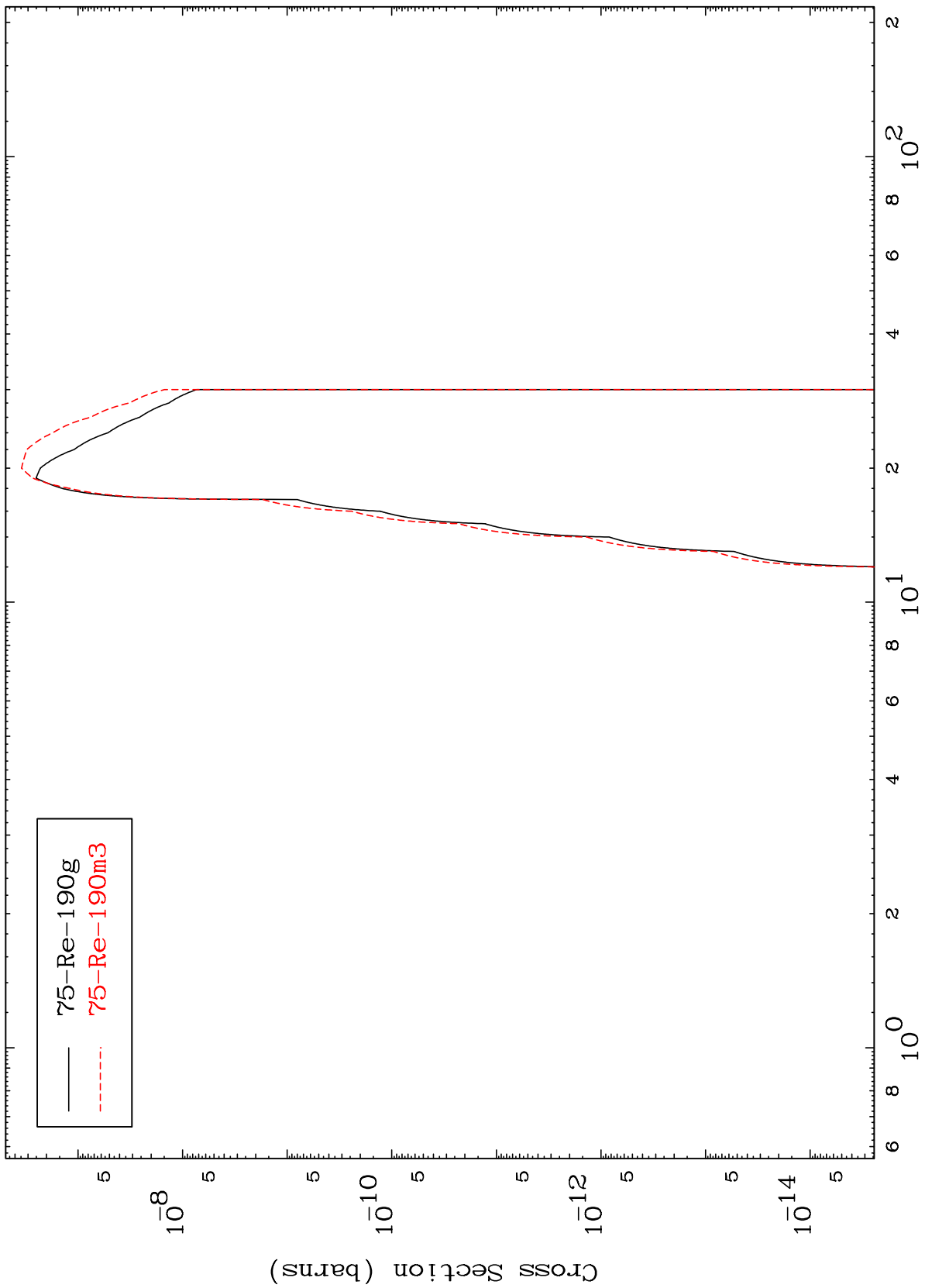
Incident Energy (MeV)

73-Ta-186

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73-Ta-186

(n,γ)  
Radionuclide Production Cross Section



73-Ta-186

Incident Energy (MeV)

14