

Program EVALPLOT  
(Version 2021-1)

by

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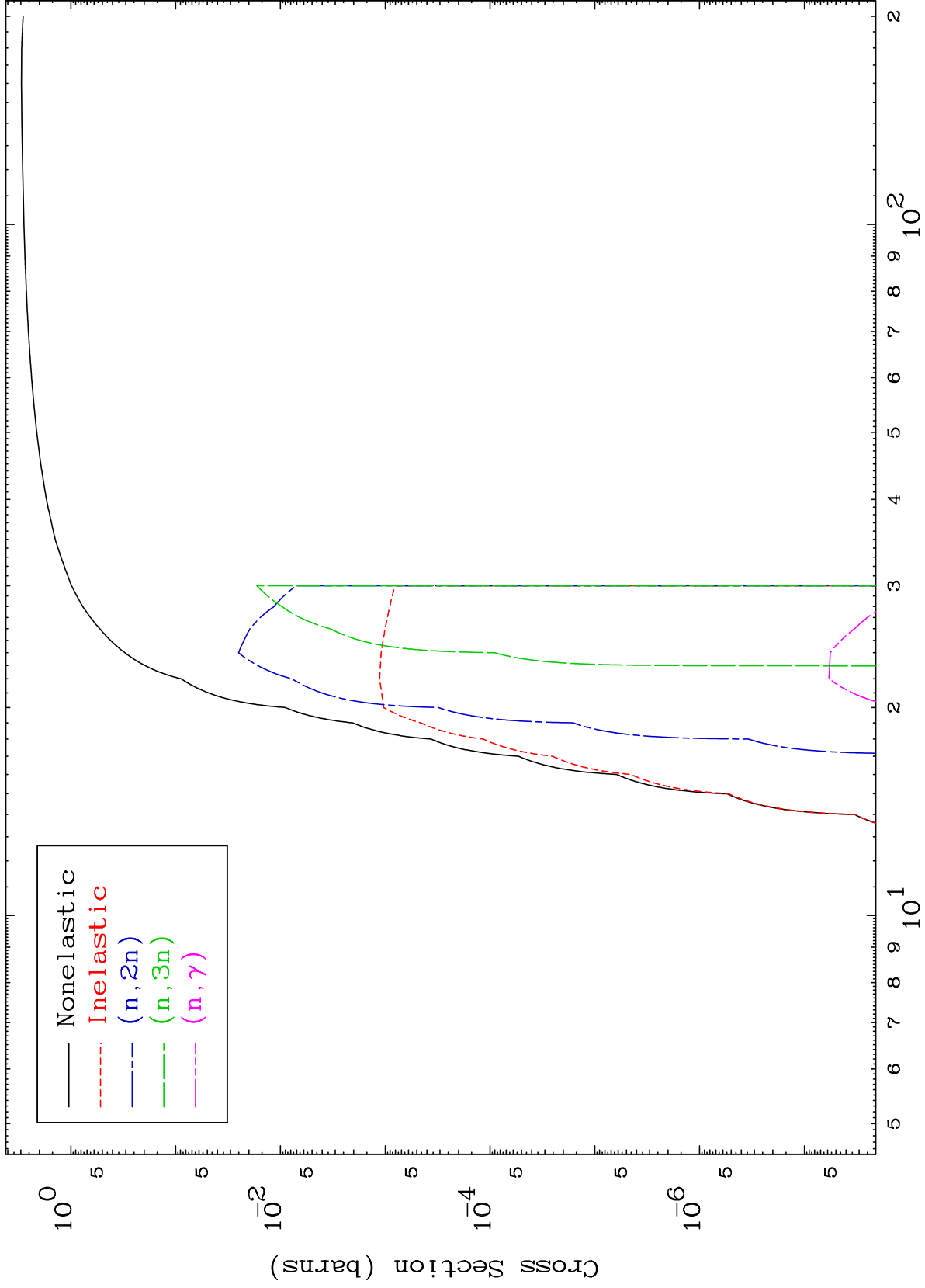
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9037

0 Kelvin Cross Sections

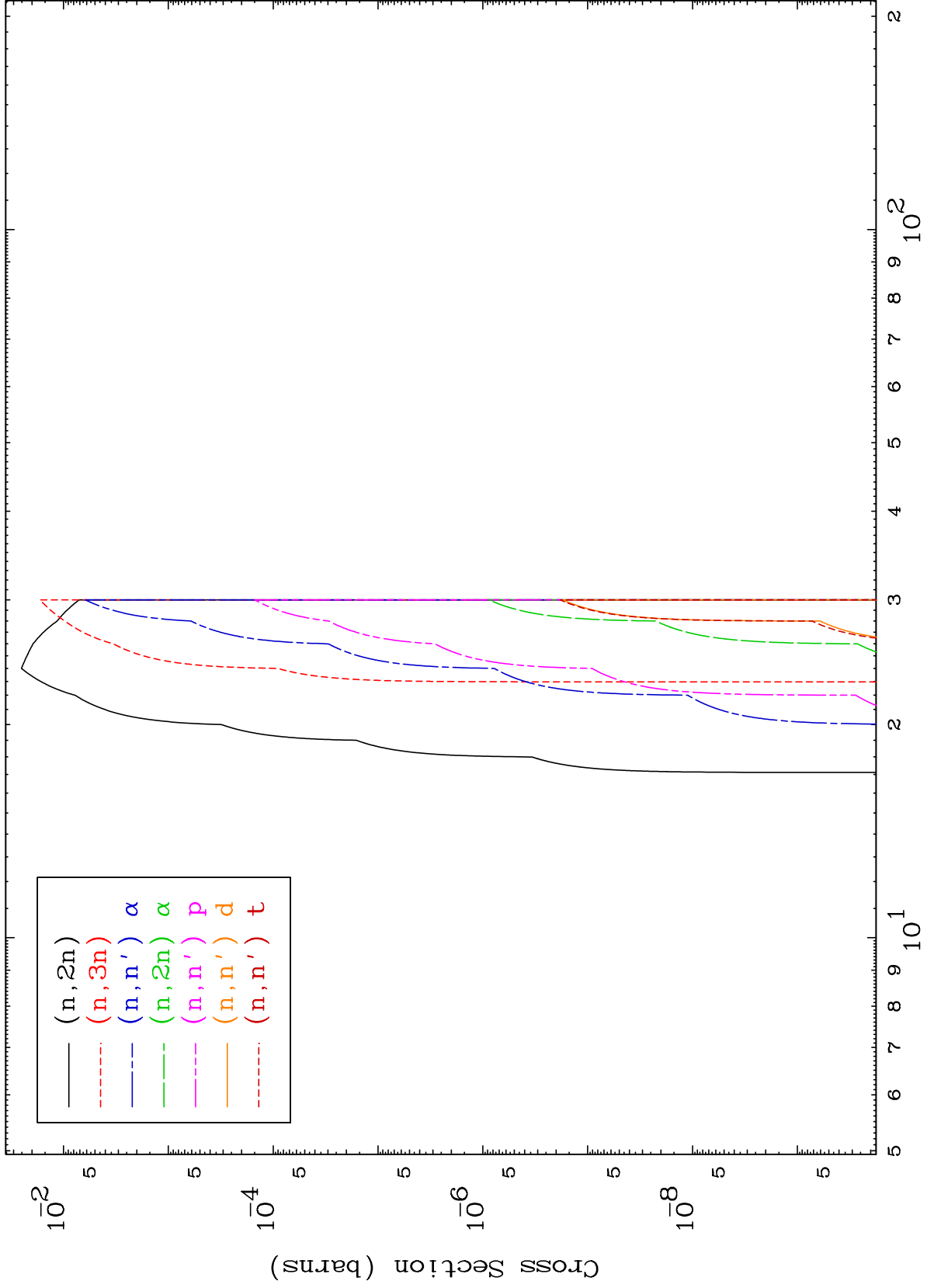
90-Th-231



MAT 9037

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

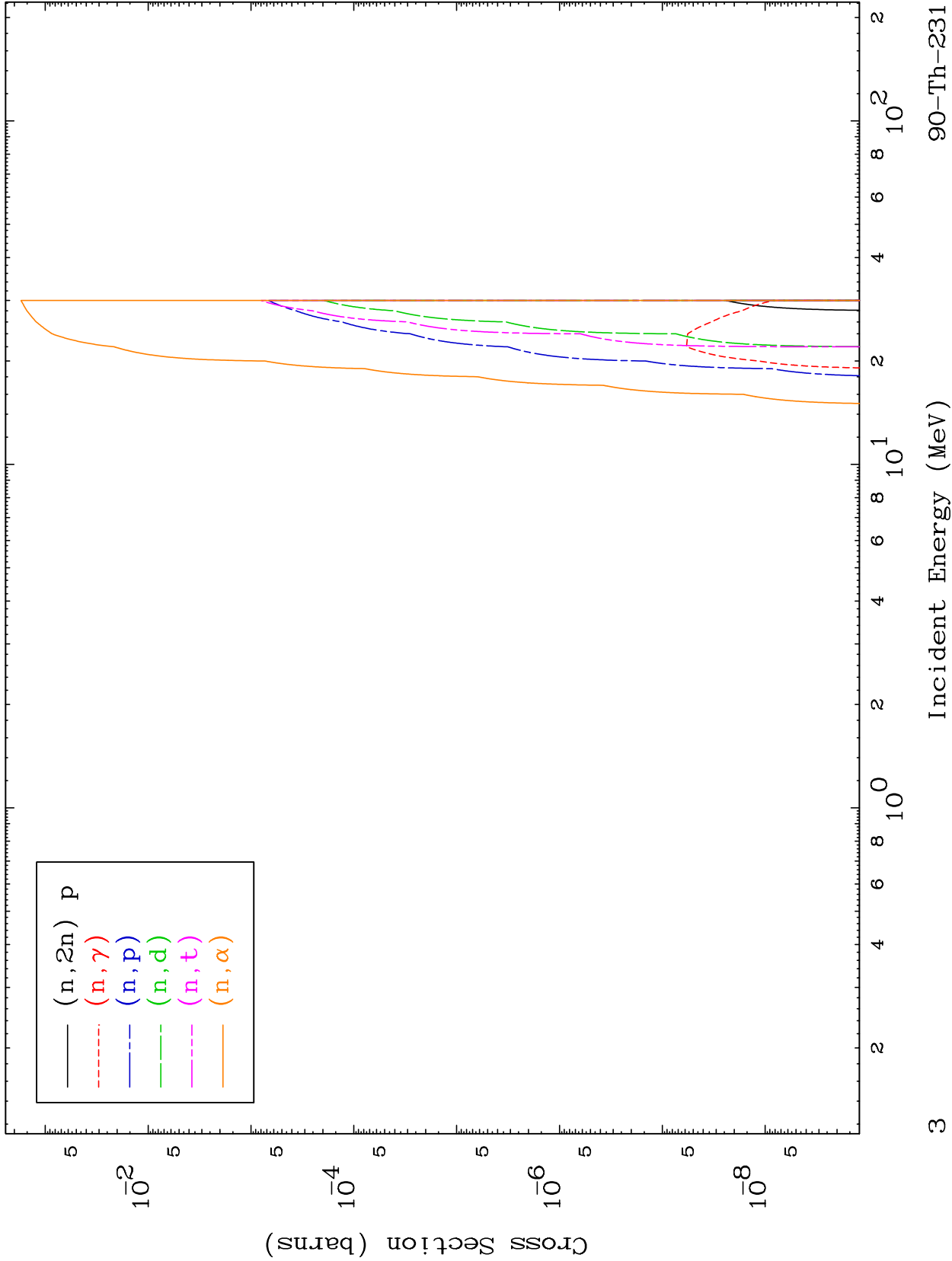
90-Th-231

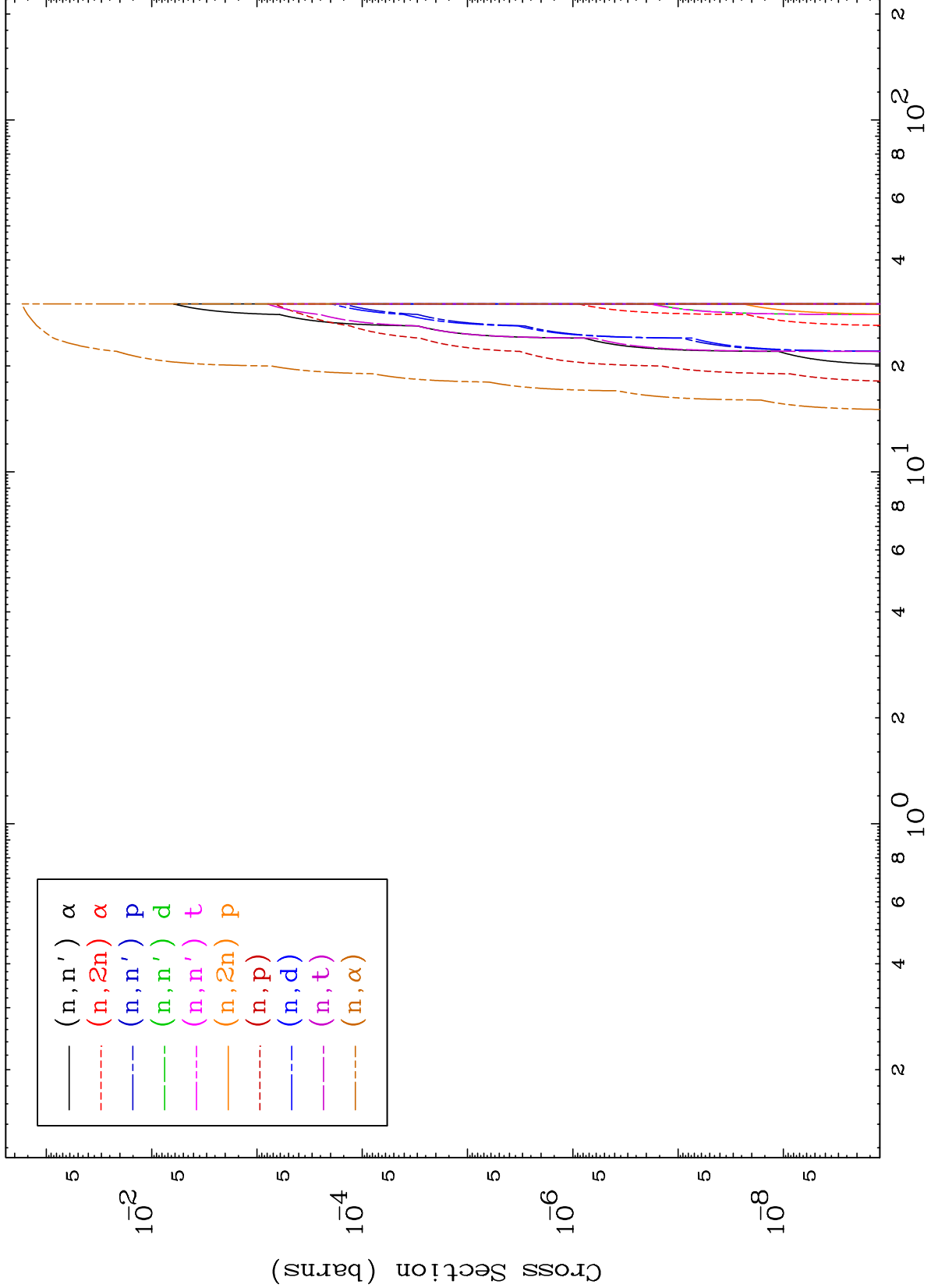


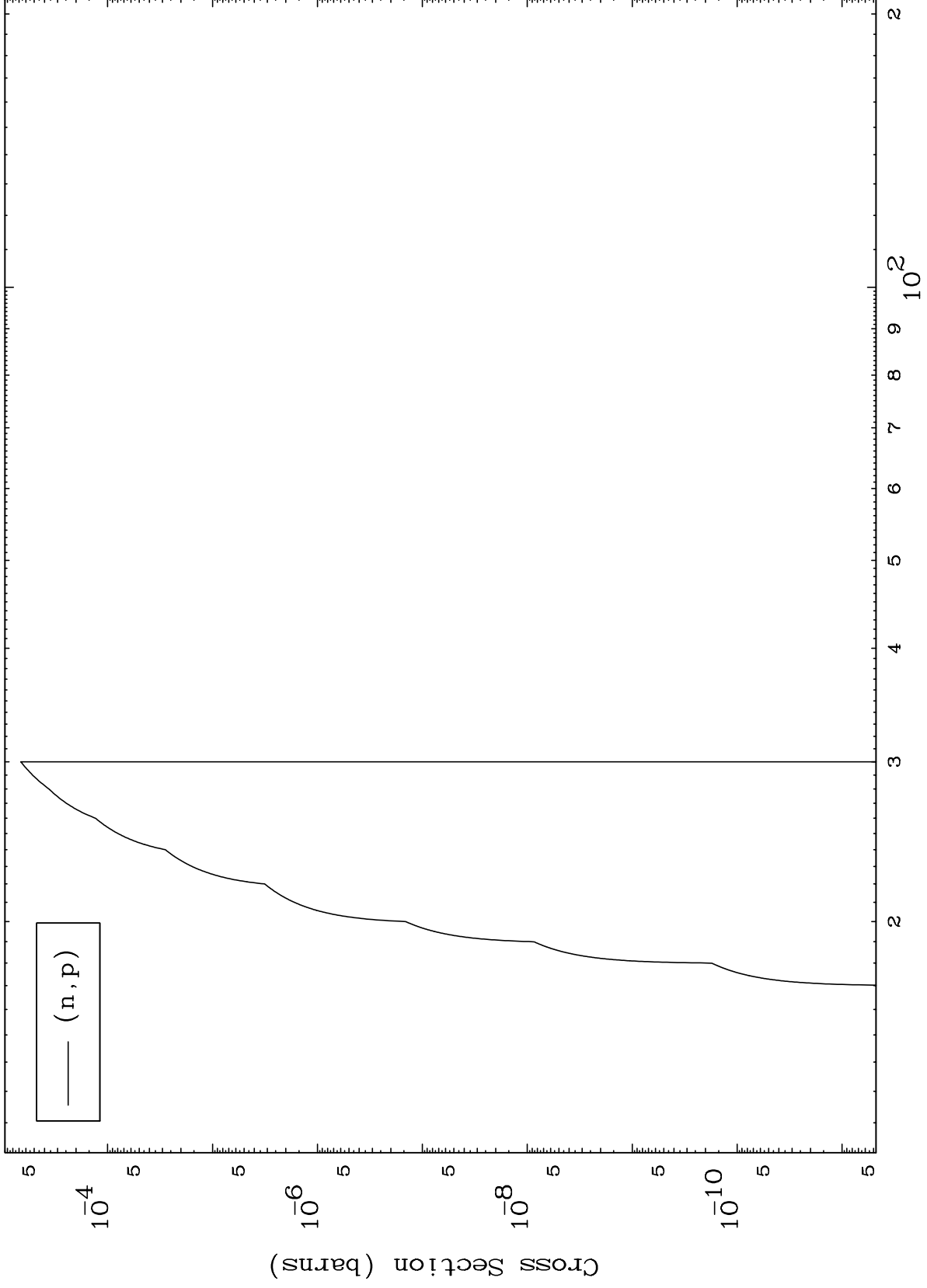
2

Incident Energy (MeV)

90-Th-231



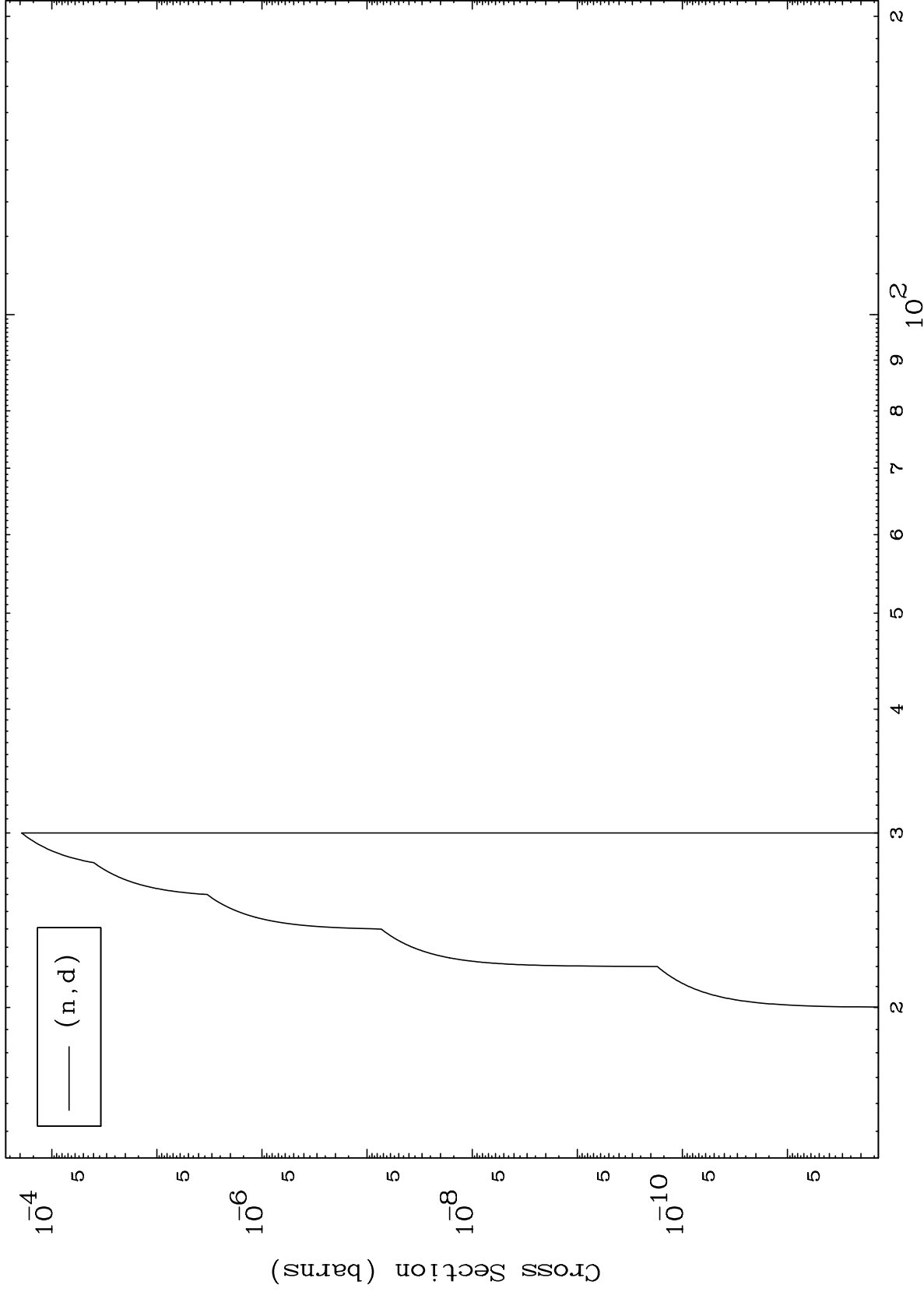




MAT 9037

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

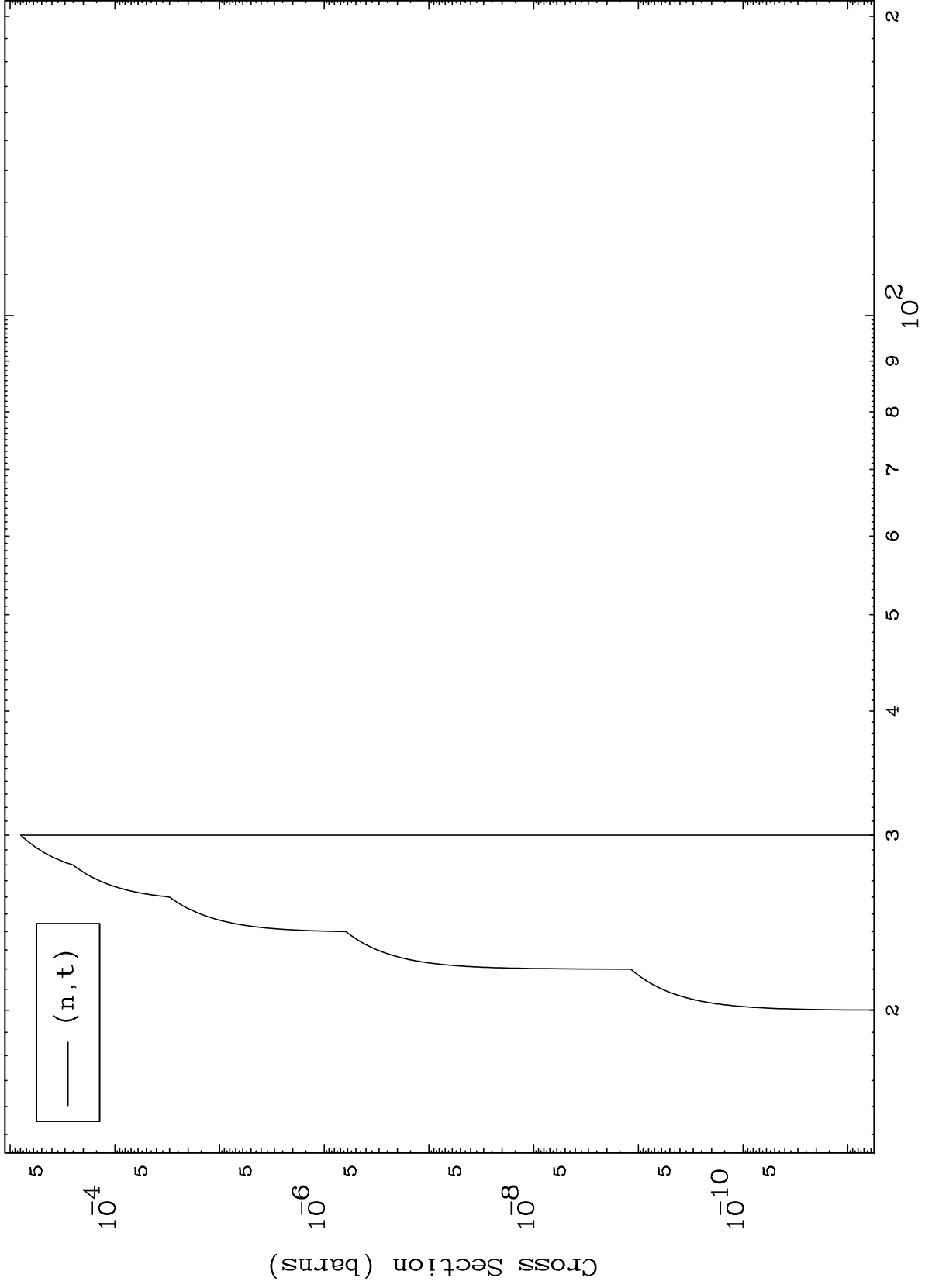
90-Th-231



6

Incident Energy (MeV)

90-Th-231

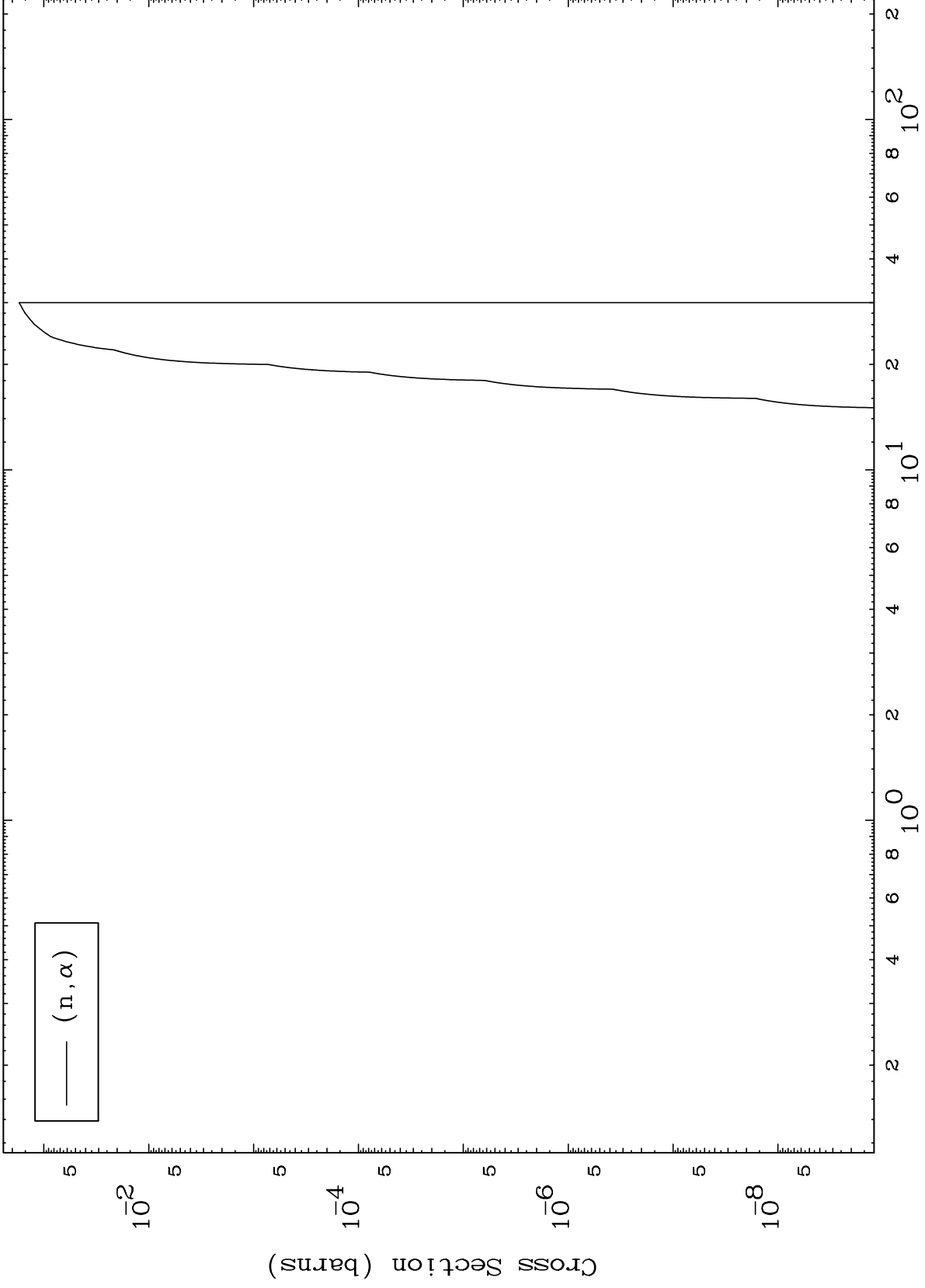


MAT 9037

( $\alpha, \alpha$ ) Levels

90-Th-231

0 Kelvin Cross Sections

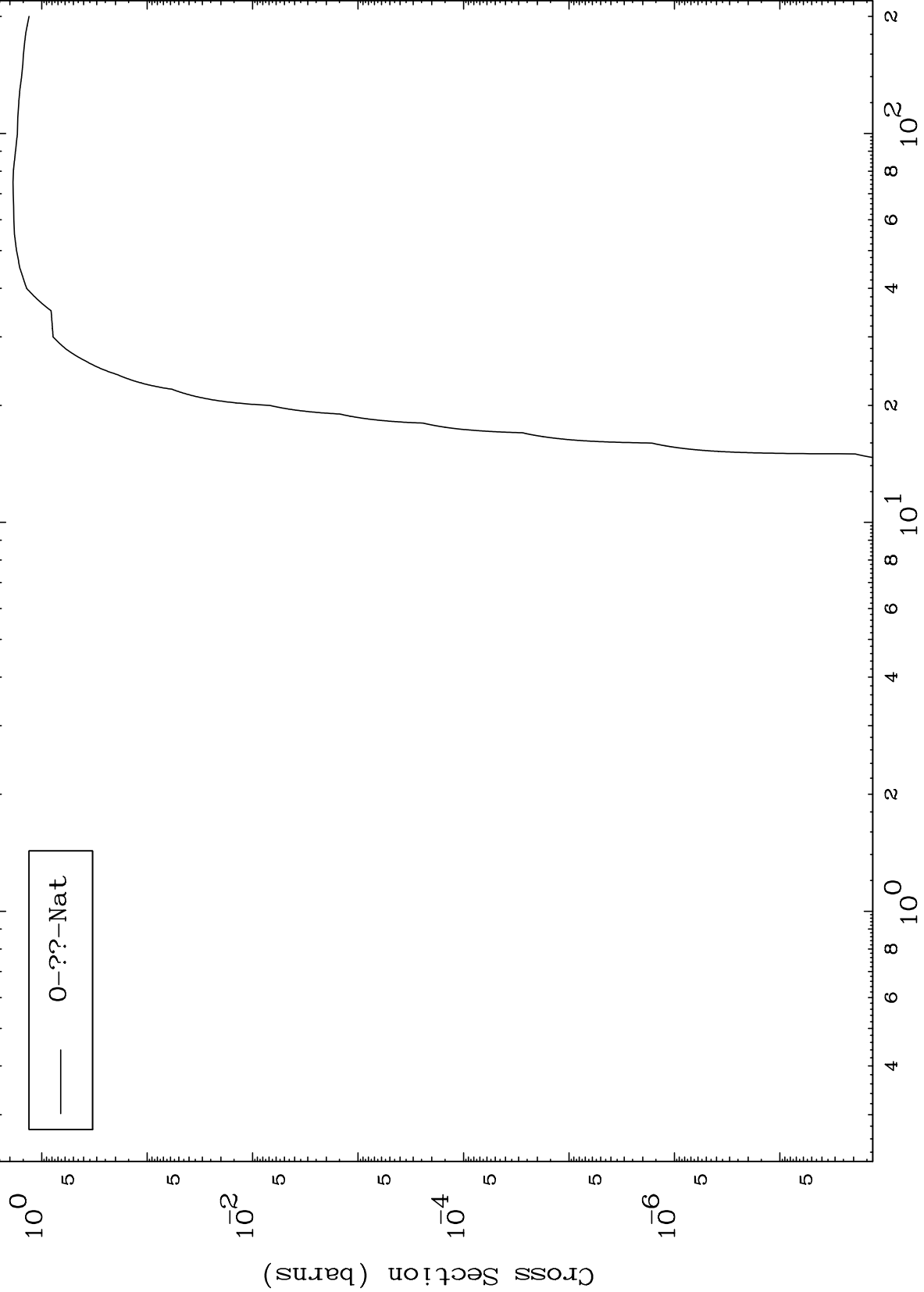


MAT 9037

Fission

90-Th-231

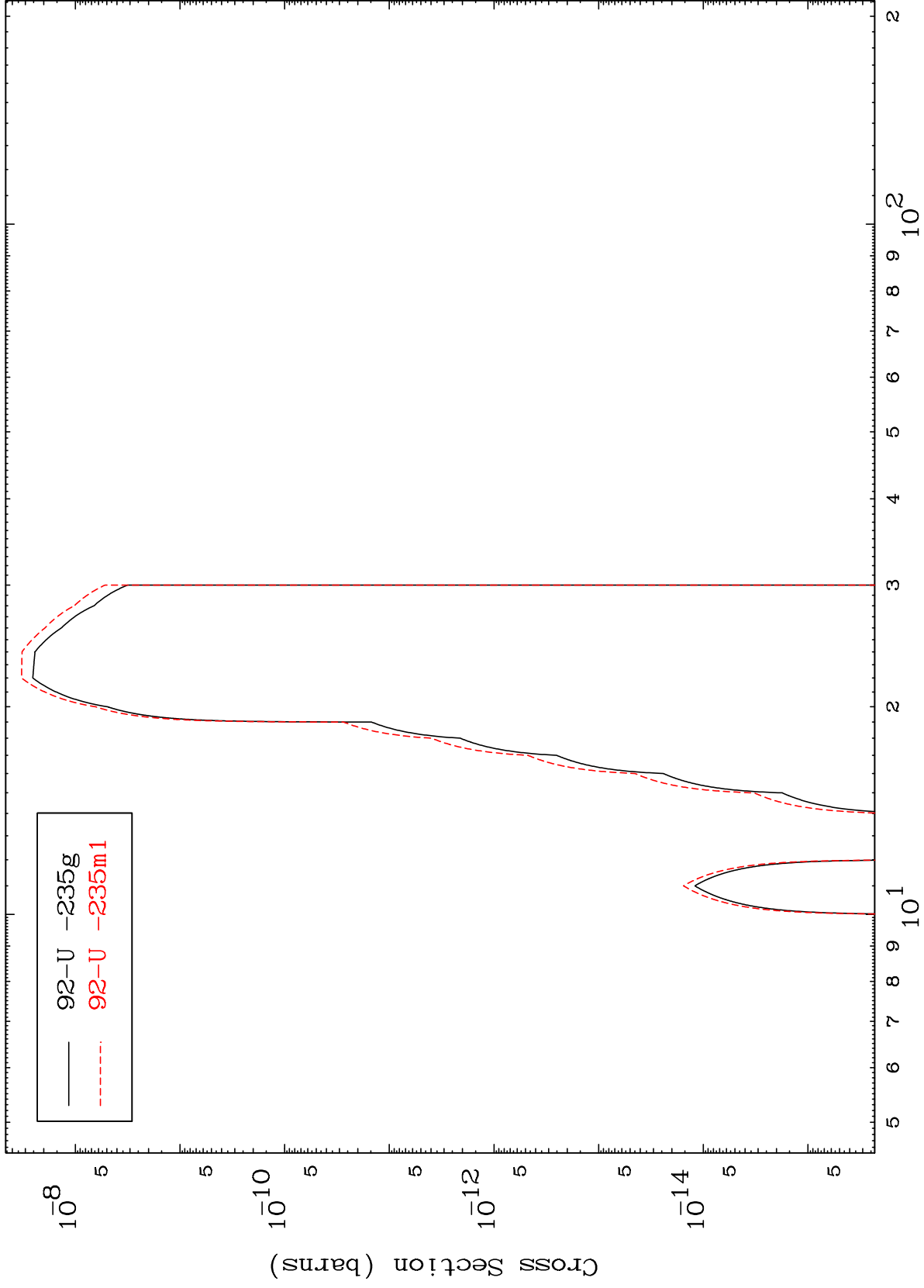
Radionuclide Production Cross Section



MAT 9037

90-Th-231

(n,  $\gamma$ )  
Radionuclide Production Cross Section



90-Th-231

Incident Energy (MeV)

10

MAT 9037

90-Th-231

(n,p)  
Radionuclide Production Cross Section

